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Vol. 4, No. 4

OPERATING REACTORS LICENSING ACTIONS SUMMARY

UNITED STATES NUCLEAR REGULATORY COMMISSION



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WASHINGTON, D.C. 20555



FOREWORD

THE OPERATING REACTORS LICENSING ACTIONS SUMMARY IS DESIGNED TO PROVIDE THE MANAGEMENT OF THE NUCLEAR REGULATORY COMMISSION (NRC) WITH AN OVERVIEW OF LICENSING ACTIONS DEALING WITH OPERATING POWER AND NONPOWER REACTORS. THESE REPORTS UTILIZE DATA COLLECTED FROM THE DIVISION OF LICENSING IN THE OFFICE OF NUCLEAR REACTOR REGULATION AND ARE PREPARED BY THE OFFICE OF RESOURCE MANAGEMENT.

THIS SUMMARY REPORT IS PUBLISHED PRIMARILY FOR INTERNAL NRC USE IN MANAGING THE OPERATING REACTORS LICENSING ACTIONS PROGRAM. ITS CONTENT WILL CHANGE BASED ON NRC MANAGEMENT INFORMATIONAL REQUIREMENTS.

Definitions

I. Licensing Actions

- A. Active Licensing Action - an item which has been initiated but not completed.
 - 1. Multi-Plant Licensing Action - a licensing action topic that applies to a class of reactors.
 - 2. Plant Specific Licensing Action - a licensing action associated with a certain facility because of the uniqueness of the technical specifications or situation, e.g., reduce the number of incore detectors.
- B. Types of Licensing Actions
 - 1. Amendment - action that results in changes to facility licenses including technical specification changes.
 - 2. Letters to Licensees - letters sent to licensee requesting information that will resolve or aid in the resolution of technical issues. Examples of letters sent to licensees are: fuel handling inside containment, assymetric loads or reactor vessel materials surveillance.
 - 3. Orders/Exemptions - actions on operating plant that result in an Order for Modification of a License or an exemption to the regulation.
- C. Categories of Multiplant Actions
 - 1. A-Series: actions directly attributed to a specific regulation requirement, e.g., inservice inspection/testing requirements 10 CFR 50.55a(g). (Related to Regulations).
 - 2. B-Series: action directly attributed to an occurrence of accumulative experience at licensed reactors, e.g., fire protection. (Related to Operating Experience).

3. C-Series: action resulting from NRC staff reviews of license applications, generic topics and operating reactor licensing actions, e.g., reporting requirements, fuel handling accidents, or filter tech spec requirements. (A result of internal review.)
4. D-Series: action required as a result of current data which shows that previously accepted requirements are no longer acceptable or are overly restrictive, e.g., ECCS input errors, asymmetric loads, equipment qualification. This new information may result from technological advancements, research or advanced analytical studies. (A result of new information.)
5. E-Series: action requested by the licensee that does not fit any of the other sources of licensing actions. Usually these actions are requested because of the licensee's desire to improve operational performance, e.g., core reloads, spent fuel pool modifications, or (N-1) loop operation. (A result of licensee requests.)
6. F-Series: multiplant functions which are the direct result of NUREG-0737 concerning TMI Lessons Learned.
7. G-Series: multiplant actions which are a direct result of NUREG-0660, a precursor to NUREG-0737.

II. Significant Dates

- A. Initiation - This is the date an action was actually started or implementation was begun on a plant. It is the date that a letter was issued to the licensee initiating the action, the date of an amendment request, or the date a TAC was issued for an item arising out of an operating experience or new information event.
- B. PM-Target - project manager's anticipated completion date for active items.
- C. Licensing Action Complete - date on which all work, both technical and administrative was completed and sent out of the agency.

III. Definition of Priorities (Applicable to Multi-Plants or Plant Specific Actions)

A. High (1)

1. Required by Rule, Order or NRC Letter
2. Safety-related items that use definite improvements in safety or would reduce the likelihood of transients or accidents.
3. Hearing-related matters.
4. High public interest in issue.
5. Actions which, if not completed, will affect power level or scheduled restart of a facility.
6. §2.206 petitions.

B. Medium (2)

1. Items which, if not completed could result in an otherwise unnecessary IE enforcement action.
2. Spent fuel pool expansions.
3. Task should be completed within six months.

C. Low (3)

1. Licensee requests for amendments that have low safety significance.
2. Licensee requests for amendments which may have safety significance but for which NRC action is not required during the next six months (e.g., in early submittal of a core reload).

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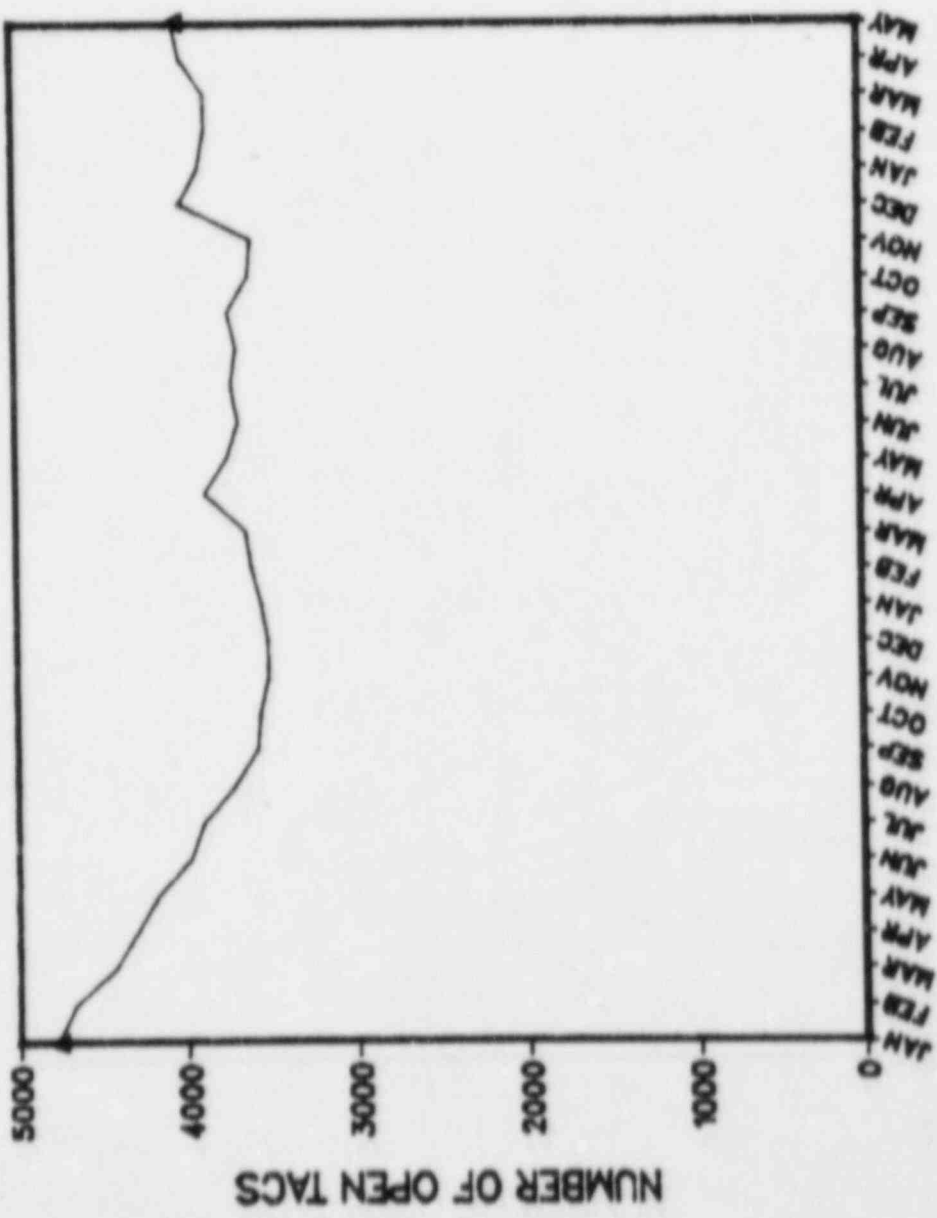
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SECTION I

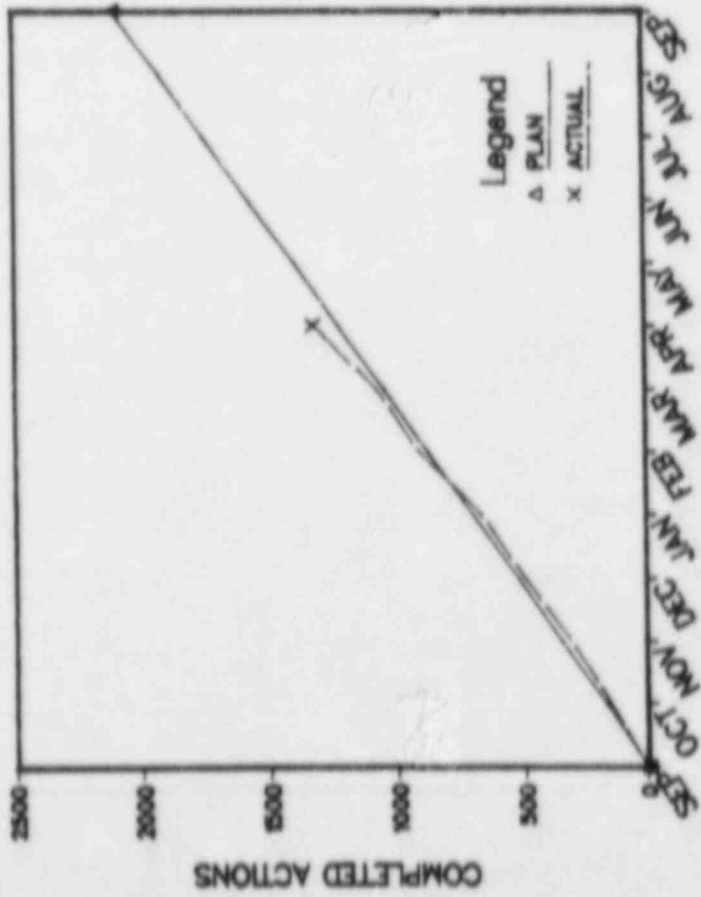
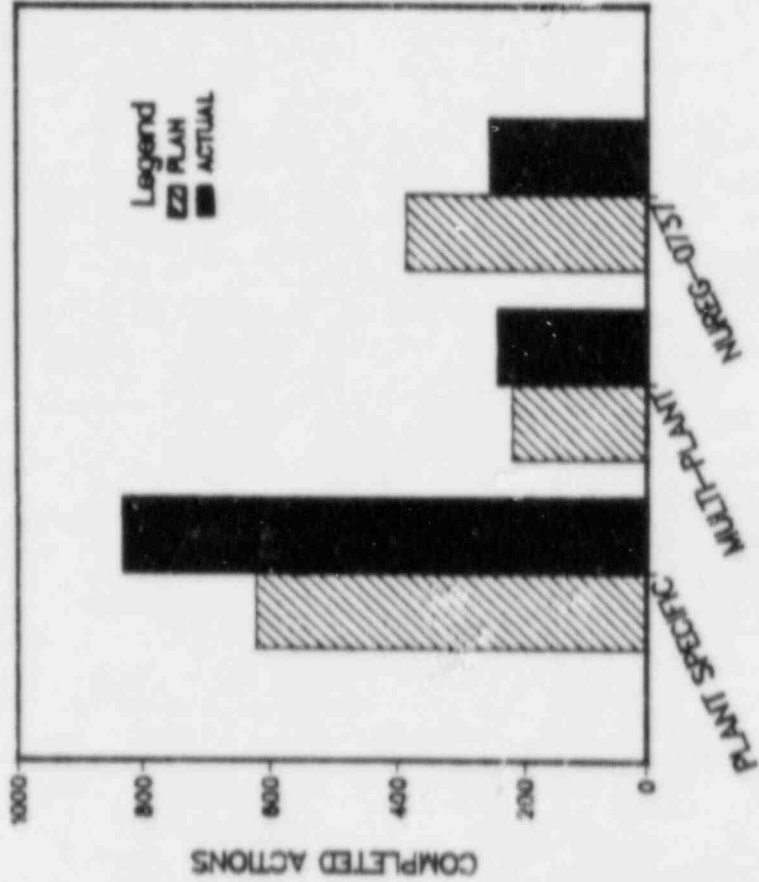


MANAGEMENT OVERVIEW REPORT

HISTORICAL VIEW OF LICENSING ACTION INVENTORY
start date 12/31/81 end date 04/30/84



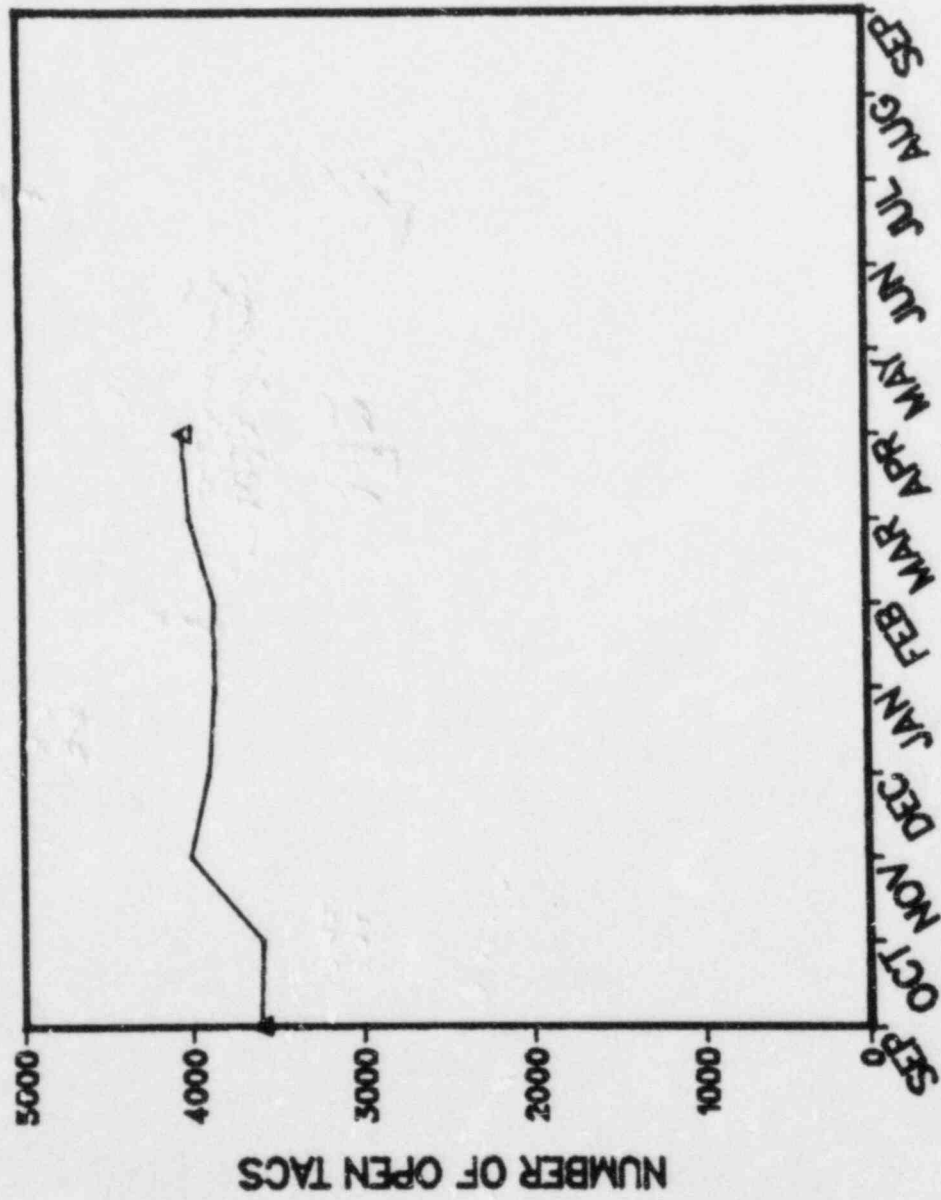
LICENSING ACTIONS COMPLETED VS FY84 OPERATING PLAN



Plant Specific	Mult Plant	NUREG 0737
193	25	19
828	241	254
620	218	386
+208	+23	-132

	OCT	NOV	DEC	JAN	FEB	MAR	APR	MAY	JUN	JUL	AUG	SEP
ACTUAL	153	309	497	668	875	1076	1224	1399	1574	1749	1924	2099
Plan	153	309	497	668	875	1076	1224	1399	1574	1749	1924	2099
Variance	-22	-41	-28	-32	+35	+26	+99					

TOTAL LICENSING ACTION INVENTORY DURING FY84



REPORT DATE: 05/21/84

FY TO DATE MULTI-PLANT & PLANT SPECIFIC ACTION SUMMARY

10/01/83 - 04/30/84

	<u>NEW LICENSING ACTIONS</u>	<u>COMPLETED LICENSING ACTIONS</u>	<u>ACTIVE LICENSING ACTIONS</u>
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MULTI-PLANT ACTIONS

REGULATIONS	0	25	145
OPERATING EXPERIENCE	471	114	792
INTERNAL REVIEWS	0	29	144
NEW INFORMATION	0	73	52
LICENSE REQUIREMENTS	0	0	12
NUREG-0737 ACTIONS	14	254	1,057
NUREG-0660 ACTIONS	0	0	50
SUBTOTALS	485	495	2,252

<u>PLANT SPECIFIC ACTIONS</u>	1,129	828	1,778
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TOTAL	1,614	1,323	4,030
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FISCAL YEAR: 1984

COMPLETED LICENSING ACTIONS BY FEE CLASS

REPORT DATE: 05/21/84

FEE CLASS	COMPLETED ACTIONS		MANHOURS		AVG HOURS/ACTION		AVG COST/ACTION	
	APR. 1984	FY TO DATE	TOTAL FY 84	TOTAL TO DATE	FOR FY 84	TO DATE	FIXED BY REG	ACTUAL @ \$40/HR
NONE	120	880	16,057.1	63,824.9	18.2	72.5	N/A	\$2,900
1	22	135	3,803.0	5,947.1	28.1	44.0	400	\$1,760
2	13	84	1,756.5	3,334.6	20.9	39.6	1,200	\$1,584
3	34	191	7,256.5	13,226.7	37.9	69.2	4,000	\$2,768
4	7	35	1,950.5	4,107.1	55.7	117.3	12,300	\$4,692
5	0	1	8.0	35.0	8.0	35.0	25,800	\$1,400
6	0	0	0.0	0.0	0.0	0.0	45,900	\$0
TOTAL	196	1,326	30,831.6	90,475.4	23.2	68.2		\$15,104

REPORT DATE: 05/21/84

MANPOWER SUMMARY BY FACILITY

FOR 04/01/84 - 04/30/84

FACILITY	MANHOURS EXPENDED LAST MONTH ON				MANHOURS EXPENDED TO DATE FY 1984	AVG HOURS/COMPLETED ACTION FOR		
	OR ROUTINE ACTIONS	HEARINGS	TMI	TOTAL		GENERIC	PLANT SPECIFIC	TOTAL
ARKANSAS 1	154.0	0.0	16.0	170.1	1,103.9	281.8	89.3	123.3
ARKANSAS 2	79.3	0.0	21.0	100.4	1,419.1	62.6	81.2	78.8
BEAVER VALLEY 1	77.3	0.0	37.0	115.4	1,562.0	31.0	34.5	33.9
BIG ROCK POINT 1	41.0	0.0	14.0	55.5	787.5	31.1	47.4	40.1
BROWNS FERRY 1	39.0	0.0	16.0	55.0	1,049.0	6.3	91.7	52.3
BROWNS FERRY 2	40.0	0.0	8.0	48.0	547.5	1.0	11.3	6.1
BROWNS FERRY 3	17.0	0.0	7.0	24.0	314.0	130.5	4.6	78.0
BRUNSWICK 1	48.0	0.0	88.0	136.0	1,262.0	75.3	62.5	67.5
BRUNSWICK 2	56.0	0.0	24.0	80.0	1,561.0	48.0	52.8	51.3
CALVERT CLIFFS 1	36.3	0.0	1.0	37.4	990.5	14.3	43.9	38.0
CALVERT CLIFFS 2	36.3	0.0	1.0	37.4	268.0	11.1	8.2	8.8
COOK 1	6.3	0.0	61.0	67.4	1,186.9	25.3	117.0	71.1
COOK 2	130.3	0.0	0.0	130.4	465.4	12.7	28.8	20.2
COOPER STATION	42.0	0.0	25.0	67.5	760.0	93.4	44.6	69.0
CRYSTAL RIVER 3	74.0	0.0	45.0	119.1	1,161.6	130.4	20.6	75.5
DAVIS-BESSE 1	160.0	0.0	18.0	178.1	923.1	116.2	42.7	65.3
DIABLO CANYON 1	215.3	172.0	19.0	406.3	6,234.3	0.0	0.0	0.0
DRESDEN 1	2.0	0.0	0.0	2.0	2.0	0.0	2.0	2.0
DRESDEN 2	74.5	0.0	15.0	89.5	1,704.0	22.2	28.5	26.2
DRESDEN 3	44.0	0.0	2.0	46.0	1,107.0	17.9	76.9	55.4
DUANE ARNOLD	97.0	0.0	42.0	139.0	1,156.5	49.1	54.0	52.0
FARLEY 1	94.3	0.0	2.0	96.4	1,318.7	20.0	44.7	39.4
FARLEY 2	16.3	0.0	0.0	16.4	458.1	8.9	27.7	22.7
FITZPATRICK	87.0	0.0	15.0	102.0	1,579.1	77.3	90.3	83.8
FORT CALHOUN 1	54.3	0.0	11.0	65.4	2,232.9	84.6	113.4	97.7
FORT ST VRAIN	25.0	0.0	68.0	93.0	763.0	49.5	25.5	32.1
GINNA	154.3	0.0	25.0	179.4	1,134.4	649.7	23.2	165.5
GRAND GULF 1	925.0	0.0	0.0	925.0	3,977.0	0.0	66.7	66.7
HADDAM NECK	53.3	0.0	17.0	70.4	1,417.9	104.0	48.6	76.3
HATCH 1	55.0	0.0	9.0	64.0	792.2	63.7	54.6	58.6
HATCH 2	30.5	0.0	9.0	39.5	825.8	35.1	52.6	45.9
HUMBOLDT BAY 3	7.0	0.0	0.0	7.0	122.5	0.0	0.0	0.0
INDIAN POINT 2	62.3	0.0	11.0	73.4	1,174.9	40.7	54.0	44.7
INDIAN POINT 3	20.3	0.0	0.0	20.4	1,716.9	5.1	216.6	137.3
KEWAUNEE	60.3	0.0	3.0	63.4	517.9	18.9	56.4	39.1

REPORT DATE: 05/21/84

MANPOWER SUMMARY BY FACILITY

FOR 04/01/84 - 04/30/84

FACILITY	MANHOURS EXPENDED LAST MONTH ON				MANHOURS EXPENDED TO DATE FY 1984	AVG HOURS/COMPLETED ACTION FOR		
	OR ROUTINE ACTIONS	HEARINGS	TMI	TOTAL		GENERIC	PLANT SPECIFIC	TOTAL
LA CROSSE	114.5	0.0	23.0	138.0	1,482.8	114.5	24.6	54.6
LASALLE 1	24.0	0.0	0.0	24.0	213.0	4.0	27.8	23.8
LASALLE 2	0.0	0.0	0.0	0.0	54.0	0.0	10.8	10.8
MAINE YANKEE	78.3	0.0	32.0	110.4	1,547.5	71.1	58.0	63.6
MCGUIRE 1	124.3	0.0	0.0	124.3	829.3	0.0	61.5	61.5
MCGUIRE 2	31.0	0.0	0.0	31.0	494.0	0.0	36.7	36.7
MILLSTONE 1	54.0	0.0	3.0	57.0	1,336.0	13.5	20.3	18.8
MILLSTONE 2	16.3	0.0	4.0	20.4	1,141.4	13.1	159.0	131.6
MONTECELLO	24.0	0.0	0.0	24.0	1,817.5	70.1	87.9	77.4
NINE MILE POINT 1	103.0	0.0	8.0	111.0	1,388.5	120.3	179.6	146.7
NORTH ANNA 1	56.3	0.0	2.0	58.4	907.9	21.8	33.8	29.4
NORTH ANNA 2	54.3	0.0	0.0	54.4	856.9	15.0	14.3	14.6
OCONEE 1	29.0	0.0	35.0	64.6	553.1	47.6	18.2	39.2
OCONEE 2	14.0	0.0	8.0	22.1	468.1	61.1	69.5	63.3
OCONEE 3	32.0	0.0	7.0	39.1	289.6	18.7	18.1	18.5
OYSTER CREEK 1	191.0	0.0	18.0	209.0	2,472.5	12.7	153.0	94.5
PALISADES	225.8	0.0	3.0	228.9	1,370.4	48.4	73.5	57.2
PEACH BOTTOM 2	66.0	0.0	12.0	78.0	1,679.5	62.6	114.0	96.9
PEACH BOTTOM 3	42.0	0.0	8.0	50.0	493.5	26.3	27.7	27.2
PILGRIM 1	95.0	0.0	7.0	102.0	1,347.5	47.8	82.1	67.1
POINT BEACH 1	66.3	0.0	0.0	67.9	602.9	33.4	18.3	24.0
POINT BEACH 2	37.3	0.0	0.0	38.9	524.4	19.6	26.6	23.8
PRAIRIE ISLAND 1	36.8	0.0	19.0	55.9	606.4	50.7	85.3	71.5
PRAIRIE ISLAND 2	22.8	0.0	4.0	26.9	439.4	50.2	77.6	65.4
QUAD CITIES 1	89.5	0.0	4.0	94.0	446.0	31.5	22.1	26.3
QUAD CITIES 2	23.5	0.0	7.0	31.5	766.0	22.1	60.5	47.7
RANCHO SECO 1	334.0	0.0	30.0	364.1	1,499.1	104.5	228.6	156.7
ROBINSON 2	197.3	0.0	35.0	232.4	1,696.9	98.0	455.2	262.9
SALEM 1	9.3	0.0	8.0	17.4	625.4	28.2	53.7	39.5
SALEM 2	19.3	0.0	0.0	19.4	396.9	30.5	47.0	38.7
SAN ONOFRE 1	70.8	0.0	8.0	78.9	1,419.9	34.3	198.6	121.2
SAN ONOFRE 2	91.8	0.0	20.0	111.8	1,283.3	49.2	16.6	18.9
SAN ONOFRE 3	24.3	0.0	0.0	24.3	537.3	17.5	17.0	17.1
SEQUOYAH 1	91.3	0.0	0.0	91.3	662.3	12.4	14.6	13.7
SEQUOYAH 2	48.3	0.0	0.0	48.3	643.8	12.0	10.2	10.6

REPORT DATE: 05/21/84

MANPOWER SUMMARY BY FACILITY

FOR 04/01/84 - 04/30/84

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	OR ROUTINE ACTIONS	HEARINGS	TMI	TOTAL		GENERIC	PLANT SPECIFIC	TOTAL
ST LUCIE 1	14.3	0.0	60.0	74.4	1,116.9	103.6	175.1	139.4
ST LUCIE 2	32.0	0.0	0.0	32.0	201.0	0.0	26.0	26.0
SUMMER 1	156.5	0.0	7.0	163.5	1,511.0	9.3	74.2	61.2
SURRY 1	22.3	0.0	26.0	48.4	623.9	59.0	77.2	66.7
SURRY 2	0.3	0.0	0.0	0.4	203.9	45.1	25.2	36.9
SUSQUEHANNA 1	69.0	0.0	4.0	73.0	1,087.5	6.0	58.5	54.1
SUSQUEHANNA 2	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0
THREE MILE ISLAND 1	361.0	55.0	25.0	441.0	5,959.5	58.1	604.1	416.4
TROJAN	30.3	0.0	0.0	30.4	1,562.1	35.6	34.7	34.9
TURKEY POINT 3	23.3	0.0	76.0	99.3	1,055.3	106.5	81.3	94.4
TURKEY POINT 4	19.3	0.0	20.0	39.3	692.8	62.9	50.2	56.8
VERMONT YANKEE 1	73.0	0.0	0.0	73.0	974.0	24.0	47.1	38.7
WASHINGTON NUCLEAR	1.0	0.0	0.0	1.0	7.0	0.0	0.0	0.0
YANKEE-ROWE 1	72.3	0.0	10.0	82.4	1,252.5	90.7	59.7	77.1
ZION 1	131.3	0.0	3.0	134.4	2,360.9	33.7	124.5	96.1
ZION 2	16.3	0.0	0.0	16.4	600.4	9.1	52.0	38.6
GRAND TOTAL	6,674.1	227.0	1,177.6	8,078.7	95,698.3	58.1	74.2	68.2

TABLE G

OPERATING POWER REACTORS HEARING

Facility	Project Manager	Hearing Attorney	Subject	Initiation Date	Status
Big Rock Point	R. Emch	M. Karman	Mixed Oxide Fuel	8/75	In indefinite suspension as a result of GESMO decision.
Big Rock Point	R. Emch	R. Goddard R. Bachman	Spent Fuel Pool Expansion	9/25/79	Evidentiary hearings held 6/7/82-6/12/82 and 10/25/83-11/4/83 covering all contentions - emergency preparedness, criticality, pool structural integrity, crane seismic capability, cask drop, aircraft overflights, shielding, EIA, and reliability of pool make-up line. NRC's proposed findings of fact and conclusions of law filed on Feb. 3, 1984. NRC and Licensee are being deposed by Intervenors. Deposition on High Range Stack Gas Monitor on 6/4/84. All other issues are before Board for decision.
Ginna	G. Dick	M. Young	Provisional Operating License Conversion to a Full Term Operating License	6/8/73	FTL SER issued October 1983; ACRS letter issued April 9, 1984, ASLB issued notice on 12/2/83. There was one response notifying the ASLB of a possible nontimely filing of petition for leave to intervene. Prehearing conference scheduled for June 13, 1984.

Facility	Project Manager	Hearing Attorney	Subject	Initiation Date	Status
Humboldt Bay	P. Erickson	M. Young	Deletion of Seismic Conditions	7/29/77	The ASLB ordered that six months after the Commission reaches a decision on safety goals and issues guidelines the licensee must report to the Board (a) readiness to take actions to permit resumed operation; or (b) commit to submit a decommissioning plan. By letter dated July 7, 1983, the licensee committed to submit a decommissioning plan to the Commission. The licensee has also requested a meeting with the staff to discuss decommissioning. A meeting was held at plant with licensee on 10/18/83. Licensee plans to submit decommissioning plan in June 1984.
Indian Point 2/3	P. Polk	J. Moore	Risk Assessment of Plant		Comments from interested parties received 2/7/84. Comments on Board findings have been received by Commission. Commission has received comments and NRR is awaiting Commission direction.

Facility	Project Manager	Hearing Attorney	Subject	Initiation Date	Status
LaCrosse	R. Dudley	C. Woodhead	Full Term Operating License (with seismic considerations)	6/23/78	The FTOL proceeding is contingent upon the completion of the SEP for the facility. Seismic considerations were resolved by ASLB Decision dated 4/21/83; affirmed by Appeal Board on 7/13/83 and finalized by Commission decision on September 15, 1983.
North Anna 1/2	L. Engle	D. Swanson	Turbine Missile Risk	2/27/78	On June 2, 1982, the ASLAB issued ASAB-676 wherein the Appeal Board delineated its decision resolving the North Anna Turbine Missile issue. All issues in this proceeding are terminated other than the radon emissions in the milling & mining of uranium fuel. The radon issues will be addressed following resolution in other hearings which are contested.

Facility	Project Manager	Hearing Attorney	Subject	Initiation Date	Status
North Anna 1/2	L. Engle	D. Swanson	Spent Fuel Pool Capacity Expansion	12/8/82	Pursuant to 10 CFR §2.751a, a notice of hearing was issued on December 8, 1982 by the ASLB and a prehearing conference was held February 16, 1983 to consider VEPCO's proposed amendments to the NA 1/2 Operating License regarding the transfer and storage of Surry spent fuel at NA and the expansion of the NA spent fuel storage capacity. On February 18, 1983 the ASLB issued two Orders. One admitted the "County" as an intervenor and the other deferred ruling on other contentions pending briefings on two issues. Interrogatories now underway. By Order dated June 10, 1983 the ASLB ruled that it had jurisdiction to consider the health and safety as well as environmental impact regarding the transshipment of spent fuel from Surry to North Anna. On July 28, 1982 VEPCO submitted a motion for directed certification to 10 CFR §2.787(a) regarding the ASLB ruling of 6/10/83 and the ASLAB was convened on 8/10/83 and the ASLAB on 9/15/83 (ALAB-741) denied VEPCO's motion for

Facility	Project Manager	Hearing Attorney	Subject	Initiation Date	Status
					directed certification regarding the ASLB ruling to consider health and safety and environmental impact of transshipment of fuel from Surry to North Anna. On 11/15/83, the time provided within which the Commission may act to review ASAB-741 expired and accordingly, the decision (ALAB-741) became final agency action. On 4/27/84, the County of Louisa filed a Notice of Withdrawal from the proceeding. By Order dated 5/23/84, the ASLB granted Louisa County requests for withdrawal as an intervening party and dismissed its contentions. The staff's SER and EIA regarding VEPCO's proposed amendments are presently scheduled for issuance in early June 1984.
Three Mile Island 1	J. VanVliet	R. Rawson J. Goldberg M. Wagner	TMI-1 Restart	8/79	All Partial Initial Decisions have been issued in favor of restart; ASLAB has found in favor of restart on all issues except management, which it is presently reviewing. The Commission has completed final agency action on emergency planning issues and is reviewing the remaining issues.

Facility	Project Manager	Hearing Attorney	Subject	Initiation Date	Status
Robinson 2	G. Requa	S. Treby	Radon	11/30/82	Partial Initial Decision issued on ASLB; affirmed by ASAB on 10/31/79 (ASAL-569). Pending before ASLAB on radon only.
Robinson 2	G. Requa	M. Karman	Steam Generator Repair	1/5/83	Hearing held 2/7/84. Hearing concluded 2/9/84. Board issued Decision 2/10/84. Hearing completed since intervenor withdrew all contentions.
Salem 1/2	D. Fischer	J. Moore	Integrated Leak Rate	10/21/83	State of Delaware filed motion to withdraw on 1/20/84. Withdrawal granted on 1/25/84.
Turkey Pt. 3/4	D. McDonald	M. Young	Reload	11/7/83	Board established. Prehearing conference held 2/28/84. Issued Prehearing Order dated 5/17/84. Hearing will be held and discovery initiated.
Zion 1/2	J. Norris		Containment Leakage Testing	2/13/84	Notice of Withdrawal of Application signed April 18, 1984.

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NOTE: * UNDER EST. COMP. DATE INDICATES SCHEDULE SLIPPED

RE- QSTG OFCE	TIA #	*****SUBJECT*****	REQUEST **DATE**	EST. COMP. DATE*	ACTUAL COMP. **DATE**	PA / TAC # / ****PM****	NRR BRANCHES
R-3	82-21	PALISADES - IST PROGRAM FOR PUMPS AND VALVES	04/23/82	07/84 *	XX/XX/XX	1163 / 11214 / WAMBACH	MEB, ORB-5
R-2	82-22	SEQUOYAH - FIRE PROTEC- TION FEATURES	03/12/82	08/83 *	XX/XX/XX	1162 / 48599 / STAHL	CHEB, LB-4
R-2	82-33	SEQUOYAH - ROD DROP PRO- TECTION FOR WESTINGHOUSE PLANTS	03/08/82	08/83 *	XX/XX/XX	1262 / 48581 / STAHL	CPB, LB-4
R-2	82-35	MCGUIRE - LOSS OF HIGH HEAD INJECTION - CONCERN THAT TECHNICAL SPECIFICA- TION ACTION STATEMENT MAY NOT BE CONSERVATIVE	04/12/82	12/83 *	XX/XX/XX	1162 / 48395 / BIRKEL	SPEB
R-3	82-43	QUAD CITIES - ELECTRIC POWER TECHNICAL SPECIFI- CATIONS	07/26/82	08/84 *	XX/XX/XX	1163 / 48667 / 48668 / BEVAN	ORAB, ORB-2
R-3	82-52	FERMI 2 - CABLE TRAY SEPARATION	08/13/82	12/83 *	XX/XX/XX	---- / 141415 / LYNCH	CHEB, LB-1
IE	82-57	HATCH 1/FITZPATRICK/PIL- GRIM - TARGET ROCK SRVS	09/23/82	10/84 *	XX/XX/XX	---- / 48771 / RIVENBARK	MEB, ORB-4
I&E	82-59	BWR CRD SYSTEM HYDRAULIC LOADS	08/31/82	07/84 *	XX/XX/XX	---- / 48843 / HOUSTON	LB-2, MEB, RSB
R-2	82-72	ROBINSON/CRYSTAL RIVER - AFW CHECK VALVE LEAKAGE	11/26/82	03/84 *	XX/XX/XX	1162 / 43064 / 49065 / REQUA	MEB, ORB-1
R-5	82-75	ALLEGED DESIGN DEFICIEN- CIES IN CLASS I PIPING	11/24/82	02/84 *	XX/XX/XX	1165 / 49242 / MARTIN	MEB, LB-2
R-5	83-01	DIABLO/PALO VERDE/WNP 1-4 CABLE TRAY & CONDUIT SUPPORTS	12/16/82	03/84 *	XX/XX/XX	1165 / 49313 / BUCKLEY	SEB, MEB, LB-3, LB-4

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RE- QSTG OFCE	TIA #	*****SUBJECT*****	REQUEST **DATE**	EST. COMP. DATE*	ACTUAL COMP. **DATE**	PA/ TAC # / ****PM****	NRR BRANCHES
R-5	83-08	SAN ONOFRE S,3/ANO-2 - ESFAS DESIGN DISCREPANCY	12/29/82	03/84 *	XX/XX/XX	---- / 49485, 49486, 49320 / ROOD, R. LEE	ICSB, ORAB, LB-3
R-4	83-15	FORT ST. VRAIN - INSTRU- MENTATION SETPOINTS	01/14/83	09/84 *	XX/XX/XX	---- / 49587 / COLBURN	ICSB, ORB-3
R-3	83-19 83-19A	SURVEILLANCE TESTING OF FIRE PUMPS	02/14/83	06/84 *	XX/XX/XX	1163 / 49720 / WAMBACH	CHEB, LB-3
R-1	83-20	SALEM 1 - LONGTERM ACTION PLAN	03/02/83	12/84 *	XX/XX/XX	1161 / 49795 / FISCHER	ICSB, EQB, LQB, ORB-1
R-1	83-22	CALVERT CLIFFS - SPURIOUS ACTUATION OF RPS/SFAS	02/06/83	06/84 *	XX/XX/XX	1161 / 49584 / JAFFE	ICSB, ORAB, ORB-
NRR	83-24	GINNA - SEP ACTION ITEMS	02/22/83	06/84	XX/XX/XX	---- / 49803 / DICK, PERSINKO	SEP, ORB-5
R-3	83-26	QUAD CITIES - TECH SPEC INTERPRET ON DC POWER SYSTEMS	02/16/83	08/84 *	XX/XX/XX	1263 / 49893, 49894 / BEVAN	SSPB, PSB, ORB-2
NRR	83-28	PALISADES SEP ACTION ITEMS	03/07/83	12/84 *	XX/XX/XX	---- / 42199 / WAMBACH, MICHAELS	SEP, ORB-5
R-3	83-33	LASALLE/BYRON/BRAIDWOOD - PIPE WHIP RESTRAINTS	03/21/83	05/84 *	XX/XX/XX	---- / 141415, 141413 / STEVENS	MEB, LB-1

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R-3	83-35	POINT BEACH - AFW SYSTEM OPERABILITY	03/22/83	11/84 *	XX/XX/XX	1163 / 51042 / 51043 / COLBURN	ASB, SSPB, ORB-3
R-5	83-38	SAN ONOFRE 2&3 - NUISANCE ALARMS	03/24/83	09/84 *	XX/XX/XX	1165 / 51315 / 51317 / ROOD	HFEB, LQB, LB-3
R-3	83-40	MIDLAND 1&2 - SOILS ISSUES	03/16/83	06/84	XX/XX/XX	1163 / 51341 / HOOD	SGEB, LB-4
NRR	83-41	MILLSTONE 1 SEP ACTION ITEMS	04/06/83	12/84	XX/XX/XX	---- / 49786 / SHEA, PERSINKO	ORB-5 SEPB
R-3	83-42	BIG ROCK POINT - FIRE DAMPERS TECH SPEC	04/13/83	09/84 *	XX/XX/XX	1163 / 51394 / EMCH	CHEB, SSPB ORB-5
R-2	83-43	SUMMER - PIPING ANALYSIS REVIEW	04/14/83	04/84 *	XX/XX/XX	1162 / 51411 / HOPKINS	LB-1, MEB
NRR	83-45	DRESDEN 2 - SEP ACTION ITEMS	04/18/83	12/84	XX/XX/XX	---- / 52436 / GILBERT, CHERY	ORB-5 SEPB
NRR	83-46	OYSTER CREEK - SEP ACTION ITEMS	03/29/83	12/84	XX/XX/XX	---- / 49809 / LOMBARDO, CHERY	SEPB, ORB-5
R-5	83-49	RANCHO SECO - CORE BARREL BOLTS	04/27/83	06/84 *	XX/XX/XX	---- / 51432 / MINER	MTEB, MEB, ORB-4
R-3	83-52	LASALLE - SCRAM RESPONSE TIME TESTING	04/22/83	06/84 *	12/20/83	---- / 51494 / BOURNIA	ICSB, SSPB LB-2
R-2	83-54	ST. LUCIE 2 - SEISMIC ANALYSIS OF PIPING	05/02/83	07/84 *	XX/XX/XX	1162 / 51554 / SELLS	MEB, ORB-3

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R-4	83-55	ANO-1 - CORE BARREL BOLTS	05/12/83	06/84 *	XX/XX/XX	---- / 51553 / VISSING	MTEB, MEB, ORB-4
IE	83-56	FIRE PROTECTION INSPEC- TION ASSISTANCE APPENDIX R	05/19/83	06/84 *	XX/XX/XX	VARIOUS / 51395. 51396. 51397. 51398. 51399. 52144 / WAMBACH	CHEB, ORB-5
R-3	83-57	BIG ROCK/QUAD CITIES - CILRT	05/11/83	09/84 *	XX/XX/XX	1163 / 51557. 51611. 51612 / EMCH	CSB, ORB-5
R-1	83-58	OYSTER CREEK - INSOLATION CONDENSER TECH SPECS	02/10/84	10/84 *	XX/XX/XX	1161 / 51403 / LOMBARDO	SSPB, RSB, ORB-5
R-2	83-60	FARLEY/SUMMER - SCRAM BREAKER PROBLEMS	05/04/83	06/84 *	XX/XX/XX	---- / 51537. 51536. 51568 / HOPKINS	ORAB, EQB, ORB-1
R-2	83-61	CRYSTAL RIVER/OCONEE 1,2, 3/DAVIS BESSE - CORE BARREL BOLTS	05/23/83	06/84 *	XX/XX/XX	1142 / 51694 / KADAMBI	MTEB, MEB, RSB, ORB-4
R-3	83-68	FIRE PROTECTION TECHNICAL SPECIFICATIONS	07/12/83	06/84 *	XX/XX/XX	1057 / 40002 / WAMBACH	CHEB, ORB-5
NRR	83-69	YANKEE ROWE - SEP ACTION ITEMS	07/14/83	12/84	XX/XX/XX	---- / 51903 / ERICKSON, BOYLE	SEP, ORB-5
R-3	83-70	QUAD CITIES - ISI TEST PROGRAM	04/22/83	07/84 *	XX/XX/XX	1163 / 52080. 52081 / BEVAN	MEB, ORB-2
R-5	83-73	PALO VERDE - RCS DAMAGE	08/04/83	05/84 *	XX/XX/XX	---- / 141435 / LICITRA	MEB, EQB

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RE- QSTG OFCE	TIA #	*****SUBJECT*****	REQUEST **DATE**	EST. COMP. DATE*	ACTUAL COMP. **DATE**	PA/ TAC # / *****PM*****	NRR BRANCHES
R-1	83-74	BEAVER VALLEY 2, HOPE CREEK, LIMERICK - HEAVY WALLED PIPING	07/26/83	03/84 *	XX/XX/XX	---- / 141431 / LAZO	MEB, LB-3
R-3	83-78	MIDLAND/CLINTON - DEFI- CIENCIES IN HVAC	08/04/83	01/84	XX/XX/XX	---- / 141433 / HOOD	MEB, MTEB, LB-4
NRR	83-79	LA CROSSE - SEP ACTION ITEMS	08/12/83	12/84	XX/XX/XX	---- / 51880 / DUDLEY, BROWN	ORB-3, SEPB
R-3	83-80	ZION - ACTIVE FIRE BARRIERS	07/14/83	06/84 *	XX/XX/XX	1163 / 52120, 52121 / NORRIS	CHEB, SEPB, ORB-
R-3	83-81	MONTICELLO - LOSS OF OFFSITE POWER EVENT	08/11/83	08/84 *	XX/XX/XX	---- / 52089 / ROONEY	ORAB, PSB, ORB-2
R-1	83-82	SHOREHAM - D/G CRANKSHAFT FAILURE	08/24/83	09/84 *	XX/XX/XX	---- / 141431 / CARUSO	PSB, MEB, MTEB, LB-2 ORAB
R-3	83-83	DAVIS BESSE - FIRE PRO- TECTION DEFICIENCIES	08/02/84	02/84 *	XX/XX/XX	1163 / 52156 / DEAGAZIO	CHEB, ORB-4
R-5 R-3	83-87	EMERGENCY LIGHTING REQUIREMENTS	07/29/83	06/84 *	XX/XX/XX	1163 / 52038 / WAMBACH	CHEB, PSB ORB-5
NRR	83-89	HADDAM NECK - SEP ACTION ITEMS	08/15/83	10/85	XX/XX/XX	---- / 52010 / LYONS, BROWN	ORB-3 SEPB
R-2	83-90	BRUNSWICK - REFUELING TECHNICAL SPECIFICATIONS	08/30/83	07/84 *	XX/XX/XX	1162 / 52365, 52366 / GROTENHUIS	SSPB, ORB-2

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RE- QSTG OFCE	TIA #	*****SUBJECT*****	REQUEST **DATE**	EST. COMP. DATE*	ACTUAL COMP. **DATE**	PA/ TAC # / ****PM****	NRR BRANCHES
R-5	83-91	SAN ONOFRE 2,3 - SNUBBER ALLEGATION	09/13/83	01/84	XX/XX/XX	1165 / 52491 / 52492 / ROOD	MEB, LB-3
IE	83-92	RUDEL-GOULD MODE SWITCH MALFUNCTIONS	09/29/83	07/84 *	XX/XX/XX	1562 / 40011 / PERCH	ICSB, ORAB, LB-2
R-5	83-93	WNP-2 - CONSTRUCTION DEFICIENCIES	08/19/83	06/84 *	XX/XX/XX	---- / 141435 / AULUCK	MEB, SGEB LB-2
R-2	83-94	GRAND GULF - DELAVEL D/G EVALUATION	09/23/83	06/84 *	XX/XX/XX	---- / 52411 / HOUSTON	PSB, LB-2
R-5	83-96	SONGS 2/3 - REACTOR TRIP BREAKERS	10/07/83	03/84 *	XX/XX/XX	---- / 52527 / ROOD	EQB, LB-3
R-5	83-98	SAN ONOFRE 2,3 - RTB UV COIL ARMATURE PROBLEM	11/04/83	03/84 *	XX/XX/XX	---- / 52624 / 52627 / ROOD	EQB, ORAB, LB-3
R-5	83-100	DIABLO CANYON 1 - STRUC- TURAL ALLEGATIONS	10/02/84	02/84 *	XX/XX/XX	1165 / 52622 / SCHIERLING	SGEB, MEB, LB-3
R-5	83-101	DIABLO CANYON - RHR ALLE- GATIONS	10/26/83	02/84 *	XX/XX/XX	1165 / 52623 / SCHIERLING	ICSB, ASB, LB-3
R-4	83-105	FT. ST. VRAIN - FIRE PRO- TECTION DEFICIENCIES	11/02/83	03/84 *	XX/XX/XX	1164 / 52647 / COLBURN	CHEB, ASB, ORB-3
R-3	83-106	DC COOK 2 - CONTAINMENT SPRAY ADDITIVE SYSTEM	11/04/83	01/84	XX/XX/XX	1163 / 52673 / WIGGINTON	CSB, ORB-1
R-3	83-107	DRESDEN 3/QUAD CITIES 2 - BWR PIPE CRACKS	11/04/83	06/84 *	XX/XX/XX	---- / 52670 / 52094 / BEVAN	MTEB, RAB, ORB-2

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RE- QSTG OFCE	TIA #	*****SUBJECT*****	REQUEST **DATE**	EST. COMP DATE*	ACTUAL COMP. **DATE**	PA/ TAC # / ****PM****	NRR BRANCHES
R-1	83-108	TMI-1&2 - FIRE BRIGADE TRAINING	09/26/83	01/84	XX/XX/XX	1161 / 52489 / VANVLIET	LQB, ORB-4
R-2	83-109	ROBINSON 2 - REACTOR TRIP SYSTEM	11/07/83	07/84	XX/XX/XX	1162 / 53222 / REQUA	ICSB, ORB-1
R-2	83-111	GRAND GULF ET.AL. - DEFINI- TION OF CORE ALTERATIONS	11/21/83	08/84 *	XX/XX/XX	1162 / 53231 / HOUSTON	SSPB, LB-2
R-1	83-112	VERMONT YANKEE - LOW LEVEL RADIOACTIVE RELEASE	11/07/83	06/84 *	XX/XX/XX	1161 / 53220 / ROONEY	METB, ORB-2
R-2	83-114	ROBINSON 2 - CONTAINMENT RECIRCULATION SPRAY	11/04/83	07/84 *	XX/XX/XX	1162 / 53223 / REQUA	CSB, ORB-1
R-2	83-115	BELLEFONTE - FIRE PUMP/ SPRINKLER SYSTEMS REQUIRE- MENTS	11/14/83	01/84	XX/XX/XX	---- / 141432 / STAHLE	CMEB, LB-4
R-3	83-116	LASALLE - TECH.SPEC. IN- TERPRETATION - SNUBBERS	11/01/83	06/84 *	XX/XX/XX	1163 / 52714 / BOURNIA	SSPB,MEB, LB-2
R-3	83-117	CALLAWAY - CLOSURE VERIFI- CATION OF CATEGORY C CHECK VALVES	11/08/83	04/84	XX/XX/XX	XXXX / 141433 / HOLONICH	MEB, LB-1
R-1	83-118	SUSQUEHANNA 1 - MOV THER- MAL OVERLOAD DESIGN	10/20/83	02/84	XX/XX/XX	1161 / 52609 / PERCH	PSB, LB-2
R-3	83-120	LACROSSE - BWR PIPE CRACKS	12/02/83	07/84 *	XX/XX/XX	---- / 53284 / DUDLEY	METB, RAB, ORB-5
R-1	83-121	MAINE YANKEE - MULTIPLE RTB UV CO. 1 FAILURE	12/09/84	09/84 *	XX/XX/XX	1161 / 53267 / HEITNER	ORAB, EQB, ORB-3

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RE- QSTG OFCE	TIA #	*****SUBJECT*****	REQUEST **DATE**	EST. COMP DATE*	ACTUAL COMP **DATE**	PA/ TAC # / *****PM*****	NRR BRANCHES
IE	83-122	FIRE PROTECTION INSPEC- TION ASSISTANCE - APPEN- DIX R	10/11/83	12/84	XX/XX/XX	---- / VARIOUS / WAMBACH	CHEB, ASB, ORB-5
R-5	83-123	SONGS 2/3 - POST-ACCIDENT SAMPLING	11/09/83	02/84	XX/XX/XX	1165 / 53304, 53305 / ROOD	METB, LB-3
R-3	83-129	MIDLAND - DIESEL GENERATOR BUILDING	11/03/83	02/84	XX/XX/XX	---- / 141433 / HOOD	PSB, SGEB, MEB, LB-4
R-5	84-02	PALO VERDE - FOUNDATION SOILS ALLEGATION	12/16/83	03/84	XX/XX/XX	---- / 141435 / LICITRA	SGEB, LB-3
R-5	84-03	PALO VERDE - KAMAN RADIA- TION MONITORS	12/20/83	03/84	XX/XX/XX	---- / 141435 / LICITRA	MTEB, LB-3
R-3	84-05	LASALLE/DRESDEN - MA- TERIALS OF INDETERMINATE QUALITY	12/05/83	06/84 *	XX/XX/XX	1163 / 53421, 53422 / BOURNIA	EQB, LB-2
R-3	84-06	LASALLE - VENTILATION SYSTEM TECH SPECS	12/29/83	06/84 *	XX/XX/XX	1163 / 53397 / BOURNIA	METB, LB-2
R-3	84-07	PALISADES - STEAM GENERA- TOR TUBE DEGRADATION	01/18/84	07/84 *	XX/XX/XX	---- / 53508 / PAULSON	METB, MEB, RAB, ORB-5
R-5	84-08	SAN ONOFRE 3 - HIGH PRI- MARY COOLANT ACTIVITY	01/19/84	06/84 *	XX/XX/XX	1165 / 53463 / H. ROOD	CPB, METB, RAB, LB-3
R-2	84-09	HATCH 2 - VENT HEADER CRACK	02/15/84	12/84 *	XX/XX/XX	1162 / 54150 / RIVENBARK	CSB, MTEB, ORAB, ORB-4
R-3	84-10	DAVIS BESSE - 50.59 REVIEWS	01/10/84	06/84	XX/XX/XX	1163 / 53448 / DEAGAZIO	ICSB, MEB, SGEB, EQB, AEB, ORB-4

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R-3	84-11	QUAD CITIES/DRESDEN - FIRE PROTECTION ISSUES	12/28/83	08/84 *	XX/XX/XX	1163 / 53501, 53502, 53499, 53500 / GILBERT	METB, MEB, RAB, ORB-5
R-3	84-12	LASALLE - EQUIPMENT EXPO- SURE TO HIGH TEMPERATURES	01/30/84	07/84	XX/XX/XX	1163 / 54153 / BOURNIA	EQB, LB-2
R-5	84-13	PALO VERDE - LPSI PUMP PROBLEMS	02/06/84	04/84	XX/XX/XX	---- 141435 / LICITRA	EQB, RSB, LB-3
R-1	84-14	SEABROOK - TORNADO MIS- SILE PATH	02/07/84	11/84 *	XX/XX/XX	---- / 141431 / NERSES	ASB, LB-3
R-3	84-15	DAVIS BESSE - CLASS 1E POWER SOURCES	02/21/84	06/84	XX/XX/XX	1163 / 54262 / DEAGAZIO	PSB, ORB-4
R-3	84-16	MONTICELLO - PIPE CRACKS	02/29/84	09/84	XX/XX/XX	1163 / 54455 / ROONEY	MTEB, RAB, ORB-2
R-3	84-17	DAVIS BESSE - SAFETY RELIEF VALVE	03/09/84	06/84	XX/XX/XX	1163 / 54375 / 54764 / DEAGAZIO	MEB, RSB, PSRB, ORB-
R-3	84-18	POINT BEACH 1 - CONTROL ROD GUIDE TUBE PIN FAILURES	03/09/84	03/84	XX/XX/XX	1163 / 54356 / COLBURN	MEB, CPB, MTEB, ORB-
R-4	84-19	FT. ST. VRAIN - STEAM GENERATOR TUBE LEAKS	03/06/84	05/84	XX/XX/XX	1164 / 54598 / COLBURN	MTEB, ORB-3
R-5	84-20	SALEM - FIRE PROTECTION	02/22/84	06/84	XX/XX/XX	1161 / 54507, 54508 / FISCHER	CHEB, ASB, ORB-1
R-1	84-21	VERMONT YANKEE - FIRE PROTECTION	02/22/84	03/84	XX/XX/XX	1161 / 54496 / ROONEY	CHEB, ASB, ORB-2

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R-4	84-22	FT. ST. VRAIN - CRACKED FUEL ELEMENTS	03/07/84	06/84	XX/XX/XX	1164 / 54494 / COLBURN	CPB, ORB-3
R-1	84-23	SUSQUEHANNA 1/2 - POWER SUPPLY TECH SPECS	02/14/84	05/84	XX/XX/XX	1161 / 54490 / PERCH	PSB, LB-2
R-5	84-24	RANCHO SECO - HYDROGEN EXPLOSION, FIRE, LOSS OF NNI	04/13/84	07/84	XX/XX/XX	1163 / 54487 / MINER	ICSB, PSRB ORB-4
R-1	84-26	NINE MILE POINT 1 - CRD STUB TUBES CRACKS	04/03/84	07/84	XX/XX/XX	1161 / 54628 / HERMANN	MTEB, ORB-2
R-3	84-27	DUANE ARNOLD - MATERIALS OF INDETERMINATE QUALITY	04/03/84	08/84	XX/XX/XX	1163 / 54679 / THADANI	EQB, ORB-2
R-2	84-28	ST. LUCIE - FIRE PROTEC- TION FEATURES	04/06/84	09/84	XX/XX/XX	1162 / 54746 / SELLS	CHEB, ORB-3
R-3	84-30	LASALLE - ESF RESET CON- TROLS FOR MSIVS	04/13/84	06/84	XX/XX/XX	1163 / 54611 / BOURNIA	CSB, ICSB, LB-2
R-3	84-31	BYRON - FIRE PROTECTION EVALUATIONS	04/19/84	06/84	XX/XX/XX	---- / 141415 / OLSHAN	CHEB, ASB, LB-1
R-4	84-32	FT. ST. VRAIN - PCRV TEN- DONS	04/13/84	08/84	XX/XX/XX	1164 / 54639 / COLBURN	SGEB, ORB-3
R-3	84-33	CALLAWAY - ELECTRICAL TEST REQUIREMENTS	04/17/84	05/84	XX/XX/XX	---- / 141416 / HOLONICH	PSRB, PSB, LB-1
R-3	84-34	QUAD CITIES 1 - PIPE CRACKS	04/25/84	08/84	XX/XX/XX	---- / 54955 / BEVAN	MTEB, RAB, ORB-2
R-3	84-35	PALISADES - WATER HAMMER DAMAGE TO AFW SYSTEM	04/18/84	07/84	XX/XX/XX	1163 / 54856 / PAULSON	ASB, MTEB, ORB-5, MEB

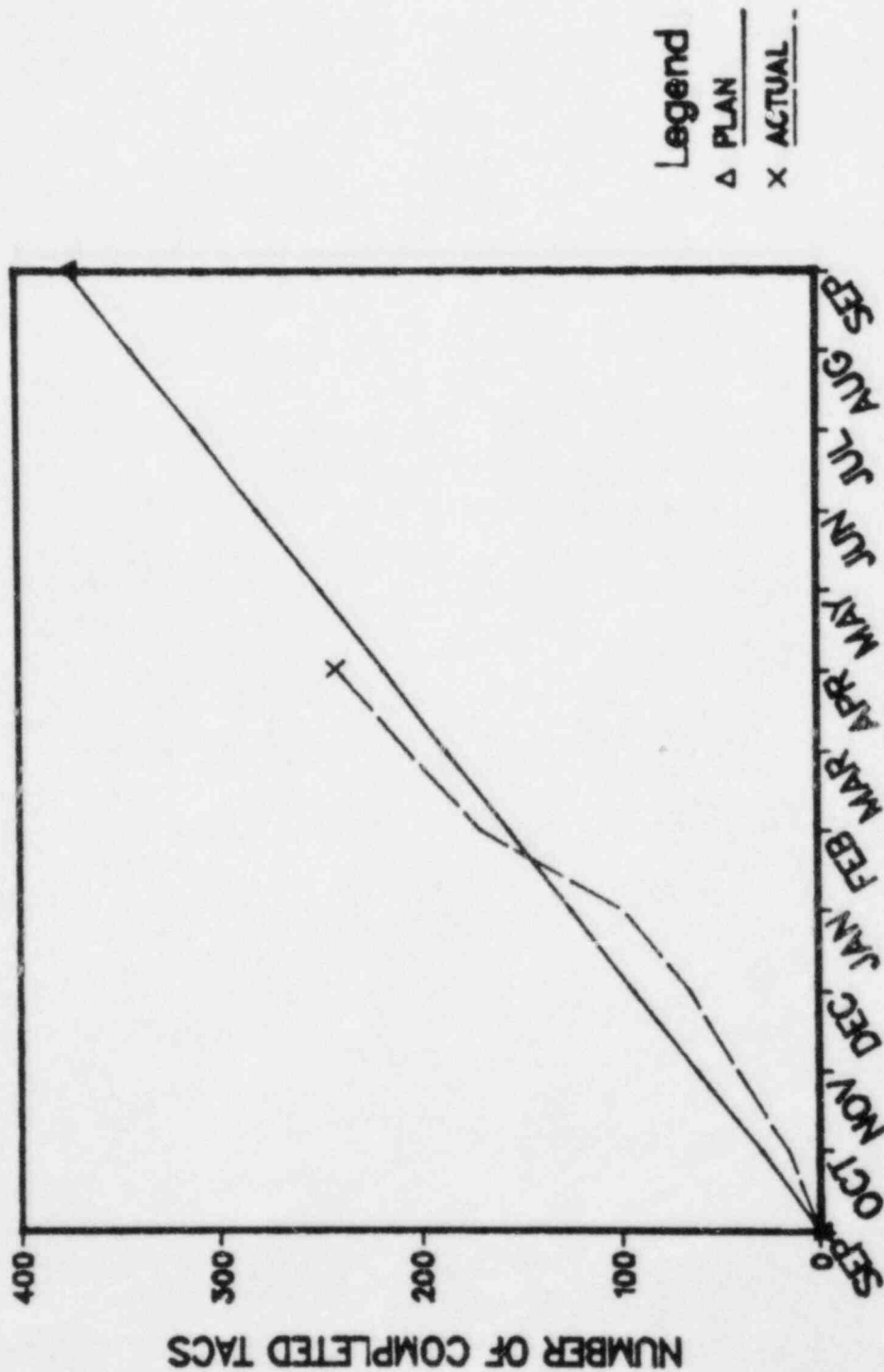
REQUESTS FOR NRR ASSISTANCE - STATUS REPORT
 DIVISION OF LICENSING
~~5/15~~ 04/20/84
 OPEN ITEMS

RE- QSTG OFCE	TIA #	*****SUBJECT*****	REQUEST **DATE**	EST. COMP. DATE*	ACTUAL COMP. **DATE**	PA/ TAC # / *****	NRR BRANCHES
R-2	84-36	MCGUIRE - FIRE PROTECTION INSPECTION	04/19/84	08/84	XX/XX/XX	1162 / 54878 / 54879 / BIRKEL	ASB, CHEB, LB-4
R-5	84-37	TROJAN - MAIN STEAM VALVES PROBLEMS	05/04/84	07/84	XX/XX/XX	1165 / 54988 / XXXXXX	MEB, RSB, ORB-2
R-3	84-XX	BYRON/BRAIDWOOD - DC SYS- TEM	05/01/84	08/84	XX/XX/XX	---- / 141433 / STEVENS	CMEB, ASB, PSB, LB-1
R-3	84-XX	QUAD CITIES - DC BATTERY 8-HR RATING	05/16/84	07/84	XX/XX/XX	1163 / 55008 / BEVAN	PSB, ORB-2
R-5	84-XX	RANCHO SECO - EFFLUENT REPORT	05/18/84	07/84	XX/XX/XX	1165 / 55038 / MIMER	METB, RAB, ORB-4
R-3	84-XX	DAVIS BESSE - EVALUATION OF S/G TUBE VIBRATION	05/18/84	11/84	XX/XX/XX	---- / XXXXX / XXXXXX	MEB, RSB, ORB-4

SECTION II

MULTIPLANT ACTION OVERVIEW

MULTIPLANT ACTIONS VS PLANNED COMPLETIONS FOR FY84 does not include NUREG 0737 items



TECHNICAL ASSIGNMENT CONTROL SYSTEM
MULTI-PLANT ISSUES SUMMARY REPORT

05/21/84

CODE	GENERIC SUBJECT	ACTIVE	FUTURE	COMPL. CURRENT FY (84)	COMPL. PRIOR FY	TOTAL
A-01	10 CFR 50.55 A(G) - ISI	0	0	2	63	70
A-02	APPENDIX I - ALARA	45	0	9	16	70
A-03	SECURITY REVIEWS-MODIFIED AMENDMENT PLANS	0	0	0	73	73
A-04	APPENDIX J - CONTAINMENT LEAK TESTING	6	0	4	43	53
A-05	GE MARK I CONTAINMENT TECH SPECS-SHORT TERM	0	0	0	20	20
A-06	RESPIRATORY PROTECTION SYSTEM	0	0	0	12	12
A-07	APPENDIX G - FRACTURE TOUGHNESS	0	0	1	8	9
A-08	ECCS EVALUATION-GENERIC PER 50.46 COMPLIANCE	0	0	0	8	8
A-09	PRESSURE VESSEL BELTLINE MATERIAL SURVEILLANCE	0	0	0	67	67
A-10	CONTINGENCY PLANNING	0	0	0	67	67
A-11	GUARD TRAINING PLANS	0	0	0	68	68
A-12	VITAL AREA ANALYSIS	0	0	0	68	68
A-13	NON POWER REACTOR SAFEGUARDS PLANS	0	0	0	0	0
A-14	10CFR 50.55 A(G) - INSERVICE TESTING	16	0	6	29	51
A-15	QUALITY ASSURANCE REQUEST REGARDING DIESEL GENERATOR FUEL OIL	0	0	1	36	37
A-16	QUALIFICATIONS OF EXAMINATION, INSPECTION, TESTING AND AUDIT PERSONNEL	0	0	0	57	57
A-17	INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)	78	0	2	0	80
B-01	DIESEL GENERATOR LOCKOUT	0	0	0	26	26
B-02	FIRE PROTECTION	0	0	0	65	65
B-03	PWR MODERATOR DILUTION	0	0	0	41	41
B-04	REACTOR VESSEL OVERPRESSURE PROTECTION	4	0	0	34	38
B-05	STRESS CORROSION CRACKING - BWR RCSPB	21	0	3	25	49
B-06	BWR RELIEF VALVE	0	0	0	23	23
B-07	STEAM GENERATOR FEEDWATER FLOW INSTABILITY	0	0	0	26	26
B-08	PWR HPSI-LPSI FLOW RESISTANCE	0	0	0	15	15
B-09	CHARGING SYSTEMS PIPE VIBRATIONS	0	0	0	15	15
B-10	BURNABLE POISON ROD FAILURE - B&W	0	0	0	2	2
B-11	FLOOD OF EQUIPMENT IMPORTANT TO SAFETY	0	0	0	10	10
B-12	STEAM GENERATOR TUBE INSPECTION	0	0	0	12	12
B-13	FUEL ROD BOW	0	0	0	8	8
B-14	CEA GUIDE TUBE WEAR	0	0	0	39	39
B-15	C-E POISON ROD GROWTH	0	0	0	1	1
B-16	EMERGENCY PLANNING AND REVISIONS	2	0	0	64	66
B-17	TECH SPEC SURVEILLANCE FOR HYDRAULIC SNUBBERS	18	0	1	46	65
B-18	WORTHINGTON RHR PUMP SHAFT INTEGRITY	0	0	0	2	2
B-19	NEUTRON SHIELDING - CE REACTORS	0	0	0	2	2
B-20	CONTAINMENT LEAKAGE DUE TO SEAL DETERIORATION	0	0	0	57	57
B-21	LOSS OF 125-V DC BUS VOLTAGE WITH LOSS OF ANNUNCIATOR SYSTEM	0	0	0	66	66
B-22	TECH SPEC SURVEILLANCE REQUIREMENTS FOR MECHANICAL SNUBBERS	16	0	4	50	70
B-23	DEGRADED GRID VOLTAGE	5	0	1	61	67
B-24	VENTING AND PURGING CONTAINMENTS WHILE AT FULL POWER AND EFFECT ON LOCA	18	0	15	101	134
B-25	BWR FEEDWATER NOZZLE CRACKING	5	0	2	16	23
B-26	INADVERTANT SAFETY INJECTION DURING COOLDOWN	0	0	0	40	40
B-27	REVIEW RESPONSES TO IE BULLETIN 78-03 (OFFGAS EXPLOSIONS)	0	0	0	28	28
B-28	BWR JET PUMP FLOW INDICATION ELIMINATION	0	0	0	5	5
B-29	BWR FEEDWATER PUMP TRIP	0	0	0	5	5
B-30	STEAM GENERATOR REPLACEMENT PROGRAM	0	0	0	8	8
B-31	LONG SHAFT LHSI & OUTSIDE RECIRC. PUMP DEGRADATION	0	0	0	3	3
B-32	BLOCKED SI SIGNAL DURING COOLDOWN	1	0	7	15	23
B-33	IODINE SPIKING	0	0	0	0	0
B-34	BWR JET PUMP INTEGRITY ASSURANCE	0	0	0	0	0

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MULTI-PLANT ISSUES SUMMARY REPORT

CODE	GENERIC SUBJECT	ACTIVE	FUTURE	COMPL. CURRENT FY (84)	COMPL. PRIOR FY	TOTAL
B-35	ORIFICE ROD ASSEMBLY INTEGRITY - B&W	0	0	0	6	6
B-36	RESISTANCE TEMPERATURE DETECTOR (RTD) RESPONSE - CE	0	0	0	7	7
B-37	STEAM GENERATOR TUBE DENTING AND SUPPORT PLATE MODIFICATIONS - CE	0	0	0	11	11
B-38	TENDON SURVEILLANCE - BECHTEL CONTAINMENTS	0	0	0	3	3
B-39	PWR PRESSURE - TEMPERATURE LIMIT TECH SPECS	0	0	0	40	40
B-40	PIPE SUPPORT BASE PLATES	0	0	0	1	1
B-41	FIRE PROTECTION - FINAL TECH SPECS (INCLUDES SER SUPPLEMENTS)	25	0	12	121	158
B-42	TMI FOLLOW UP - ALL PLANTS	0	0	0	87	87
B-43	PWR FEEDWATER LINE CRACKS	0	0	0	71	71
B-44	LESSONS LEARNED IMPLEMENTATION	0	0	0	809	809
B-45	WASH 1400 EVENT V, "PRIMARY COOLANT SYSTEM PRESSURE ISOLATION VALVES"	0	0	0	65	65
B-46	ANALYSIS OF TURBINE DISC CRACKS	1	0	0	69	70
B-47	ECCS; CLAD SWELLING AND RUPTURE	0	0	0	45	45
B-48	ADEQUACY OF STATION ELECTRIC DISTRIBUTION VOLTAGE	7	0	4	58	69
B-49	PWR CONTROL ROD MISALIGNMENT	0	0	0	9	9
B-50	AUXILIARY FEEDWATER SYSTEM EVALUATION	0	0	0	0	0
B-51	EVALUATION OF BULLETIN 79-06 AND 79-08	0	0	0	6	6
B-52	REVIEW OF SAFETY ASPECT OF INADVERTENT SAFETY ACTIONS DURING SUR. TEST	0	0	0	28	28
B-53	LESSONS LEARNED CATEGORY B (ITEMS)	0	0	0	5	5
B-54	LESSONS CATEGORY A TECH SPEC	0	0	0	19	19
B-55	B&O REPORT ON BWRs	0	0	0	3	3
B-56	CONTROL RODS FAILURE TO INSERT. BWR	0	0	0	5	5
B-57	DHR CAPABILITY	7	0	2	22	31
B-58	SDV CAPABILITY	0	0	3	38	41
B-59	MASONRY WALL DESIGN	48	0	5	17	70
B-60	ENVIRONMENTAL QUALIFICATION	74	0	0	6	80
B-61	LOSS OF NON CLASS IE I&C POWER	0	0	0	66	66
B-62	ESF RESET CONTROL DESIGN DEFICIENCY	0	0	1	66	67
B-63	INTERIM PROCEDURES FOR SHORT TERM BLACKOUT	0	0	0	71	71
B-64	B&C INDUCED FLUX ERRORS	0	0	0	8	8
B-65	SAFETY CONCERNS ASSOCIATED WITH PIPE BREAKS IN BWR SCRAM SYSTEM	17	0	0	9	26
B-66	NATURAL CIRCULATION COOLDOWN	9	0	16	24	49
B-67	THERMAL SHOCK	0	0	0	16	16
B-68	ECCS PUMP DEADHEADING	0	0	0	0	0
B-69	IEB 80-4 MAINSTEAM LINE BREAK WITH CONTINUED FEEDWATER ADDITION	2	0	1	46	49
B-70	FATIGUE TRANSIENT LIMIT TS	0	0	0	47	47
B-71	120 VAC VITAL INSTRUMENT BUSES AND INVERTER TECHNICAL SPECS.	0	0	0	0	0
B-72	NUREG 0737 TECH SPECS (GENERIC LETTER 82-16)	42	0	26	8	76
B-73	PLANS FOR PREVENTING EXCEEDING PTS SCREENING CRITERION	0	0	8	0	8
B-74	THERMAL SHIELD FOLLOW UP ANALYSIS	3	0	1	0	4
B-75	REVIEW OF B&W CORE BARREL BOLTS CRACKING	0	0	0	0	0
B-76	ITEM 1.1 - POST TRIP REVIEW; PROGRAM DESCRIPTION & PROCEDURES	81	0	0	0	81
B-77	ITEM 2.1 - EQUIPMENT CLASSIFICATION & VENDOR INTERFACE - RTS COMPONENTS	81	0	0	0	81
B-78	ITEMS 3.1.1 & 3.1.2 -POST MAINTENANCE TEST PROCEDURES & VENDOR RECOMM.	80	0	0	0	80
B-79	ITEM 3.1.3 - POST MAINTENANCE TESTING - CHANGES TO TECH SPECS - RTS COMP.	80	0	0	0	80
B-80	ITEM 4.1 - REACTOR TRIP SYSTEM RELIABILITY - VENDOR RELATED MODS	53	0	0	0	53
B-81	ITEMS 4.2.1 & 4.2.2 -PREVENTATIVE MAINT PROG FOR REACTOR TRIP BREAKERS	53	0	0	0	53
B-82	ITEM 4.3 - AUTOMATIC ACTUATION OF SHUNT TRIP ATTACH. FOR WEST & B&W	39	0	2	0	41
C-01	PWR SECONDARY WATER CHEMISTRY MONITORING REQUIREMENTS	0	0	1	37	38
C-02	BWR-RECIRC. PUMP TRIP (ATWS)	0	0	0	23	23
C-03	QUALIFICATIONS OF RADIATION PROTECTION MANAGER	0	0	0	14	14

TECHNICAL ASSIGNMENT CONTROL SYSTEM
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CODE	GENERIC SUBJECT	ACTIVE	FUTURE	COMPL. CURRENT FY (84)	COMPL. PRIOR FY	TOTAL
C-04	FILTER TECH SPECS	1	0	2	12	15
C-05	CONVERSION TO STANDARD TECH SPECS	0	0	0	5	5
C-06	PUMP SUPPORT-LAMELLAR TEARING	0	0	0	38	38
C-07	FUEL HANDLING ACCIDENT INSIDE CONTAINMENT	2	0	2	35	39
C-08	BWR POST LOCA H2 CONTROL	0	0	0	8	8
C-09	PWR AUX FW PUMPS	0	0	0	7	7
C-10	CONTROL OF HEAVY LOADS OVER SPENT FUEL POOL	43	0	17	20	80
C-11	RPS POWER SUPPLY	9	0	3	12	24
C-12	BORON SOLUBILITY DURING LONG TERM COOLING FOLLOWING LOCA	0	0	0	6	6
C-13	LOSS OF OFFSITE POWER	0	0	0	2	2
C-14	AUXILIARY FEEDWATER SEISMIC QUALIFICATION	9	0	3	35	47
C-15	CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPA# C-10)	80	0	1	0	81
D-01	GE MARK I CONTAINMENT EVALUATION - LONG TERM	14	0	8	0	22
D-02	ECCS ZIRC CLAD MODEL ERROR-COMPLIANCE WITH 10 CFR-46	0	0	0	20	20
D-03	PRESSURIZER HEATUP RATE ERROR	0	0	0	14	14
D-04	PWR REACTOR VESSEL CAVITY SEAL RING MISSILE POTENTIAL	0	0	4	14	18
D-05	PLANT UPI MODEL PROBLEM	6	0	0	0	6
D-06	PEAKING MODEL CHANGE FOR CE REACTOR CORE	0	0	0	6	6
D-07	BWR POWER LEVEL FOR RWM	0	0	0	3	3
D-08	DEFICIENCY IN CHEM ADDITION TO CONTAINMENT SPRAYS	0	0	0	3	3
D-09	GE ECCS INPUT ERRORS	0	0	0	1	1
D-10	ASYMMETRIC LOCA LOADS	15	0	19	5	39
D-11	FISSION GAS RELEASE	0	0	0	65	65
D-12	NON-JET PUMP BWR CORE SPRAY PERFORMANCE	0	0	0	6	6
D-13	B&W SMALL BREAK ERROR	0	0	0	7	7
D-14	REACTOR VESSEL WELD - WIRE DEFICIENCY	0	0	0	11	11
D-15	HIGH ENERGY LINE BREAK & CONSEQUENTIAL SYSTEM FAILURE	5	0	39	22	66
D-16	REVIEW OF CORPORATE MANAGEMENT CAPABILITY	0	0	0	18	18
D-17	DEFINITION OF OPERABLE	12	0	2	56	70
D-18	NUREG 0630 CLADDING MODELS (B&W PLANTS)	0	0	1	7	8
E-01	SPENT FUEL POOL EXPANSIONS	0	0	0	39	39
E-02	FUEL CASK DROP	0	0	0	8	8
E-03	CORE RELOADS REQUIRING PRIOR NRC APPROVAL	0	0	0	37	37
E-04	BWR SINGLE LOOP OPERATION	8	0	0	11	19
E-05	W N-1 LOOP OPERATION	4	0	0	5	9
E-06	CEA POSITION INDICATION FAILURES - CE	0	0	0	7	7
E-07	REACTOR PROTECTION SYSTEM LOGIC - CE	0	0	0	0	5
F-01	I.A.1.1 SHIFT TECHNICAL ADVISOR	0	0	0	0	71
F-02	I.A.1.3 SHIFT MANNING	2	0	12	0	69
F-03	I.A.2.1 UPGRADING OF RO AND SRD TRAINING	2	0	6	0	71
F-04	I.C.1.2 INADEQUATE CORE COOLING GUIDELINES AND PROCEDURES	66	0	0	6	72
F-05	I.C.1.3 ABNORMAL TRANSIENT OPERATOR GUIDELINES AND PROCEDURES	69	0	0	4	73
F-06	I.C.5 FEEDBACK OF OPERATING EXPERIENCE	1	0	0	72	73
F-07	I.C.6 CORRECT PERFORMANCE OF OPERATING ACTIVITIES	1	0	0	70	71
F-08	I.D.1 CONTROL ROOM DESIGN REVIEW	81	0	0	0	81
F-09	I.D.2 SAFETY PARAMETER DISPLAY SYSTEM	80	0	1	0	81
F-10	II.B.1 RCS HIGH POINT VENTS	0	0	16	58	74
F-11	II.B.2 PLANT SHIELDING	7	0	13	50	70
F-12	II.B.3 POST ACCIDENT SAMPLING	45	0	23	6	74
F-13	II.B.4 TRAINING FOR MITIGATING CORE DAMAGE	1	0	6	64	71
F-14	II.D.1 RV AND SV TRAINING	67	0	6	12	85

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F-15	II.E.1.1 AFW SYSTEM EVALUATION	2	0	5	38	45
F-16	II.E.1.2.1 AFW SYSTEM INITIATION	2	0	0	43	45
F-17	II.E.1.2.2 AFW SYSTEM FLOW INDICATION	2	0	0	44	46
F-18	II.E.4.1 DEDICATED HYDROGEN PENETRATION	1	0	0	70	71
F-19	II.E.4.2 CONTAINMENT ISOLATION DEPENDABILITY	10	0	4	57	71
F-20	II.F.1.1 NOBLE GAS MONITOR	2	0	0	70	72
F-21	II.F.1.2 IODINE/PARTICULATE SAMPLING	1	0	0	71	72
F-22	II.F.1.3 CONTAINMENT HIGH RANGE MONITOR	1	0	0	70	71
F-23	II.F.1.4 CONTAINMENT PRESSURE INSTRUMENT	7	0	4	60	71
F-24	II.F.1.5 CONTAINMENT WATER LEVEL INSTRUMENT	8	0	3	60	71
F-25	II.F.1.6 CONTAINMENT HYDROGEN MONITOR	9	0	3	59	71
F-26	II.F.2 INSTRUMENTS FOR DETECTION ON INADEQUATE CORE COOLING	72	0	6	2	80
F-27	II.K.2.9 FMEA ON ICS	0	0	0	8	8
F-28	II.K.2.10 SAFETY GRADE ARTS	0	0	0	7	7
F-29	II.K.2.11 CONTINUED OPERATOR TRAINING AND DRILLING	0	0	0	7	7
F-30	II.K.2.13 THERMAL-MECHANICAL REPORT	48	0	2	4	54
F-31	II.K.2.14 LIFT FREQUENCY OF PORVS AND SVS	0	0	0	8	8
F-32	II.K.2.16 RCP SEAL DAMAGE	0	0	0	7	7
F-33	II.K.2.17 POTENTIAL FOR VOIDING IN RCS	2	0	38	9	49
F-35	II.K.2.20 SYSTEM RESPONSE TO SB LOCA	0	0	0	7	7
F-36	II.K.3.1 AUTO PORV ISOLATION	3	0	15	28	46
F-37	II.K.3.2 REPORT ON PORV FAILURES	2	0	15	29	46
F-38	II.K.3.3 REPORTING SV AND RV FAILURES AND CHALLENGES	1	0	0	70	71
F-39	II.K.3.5 AUTO TRIP OF RCPS	1	0	0	50	51
F-40	II.K.3.9 PID CONTROLLER	0	0	0	30	30
F-41	II.K.3.10 ANTICIPATORY TRIP MODIFICATIONS	0	0	0	30	30
F-42	II.K.3.12 ANTICIPATORY TRIP ON TURBINE TRIP	0	0	0	25	25
F-43	II.K.3.13 HPCI AND RCIC INITIATION LEVELS	0	0	0	8	8
F-44	II.K.3.14 ISOLATION CONDENSER ISOLATION MODIFICATION	0	0	0	19	19
F-45	II.K.3.15 ISOLATION OF HPCI AND RCIC MODIFICATIONS	0	0	0	3	25
F-46	II.K.3.16 CHALLENGES AND FAILURES OF RELIEF VALVES	22	0	0	62	69
F-47	II.K.3.17 ECCS OUTAGES	0	0	7	8	26
F-48	II.K.3.18 ADS ACTUATION	18	0	0	24	25
F-49	II.K.3.19 INTERLOCK RECIRCULATION PUMP MODIFICATION	1	0	0	24	24
F-50	II.K.3.21 RESTART OF CSS AND LPCI	0	0	0	9	17
F-51	II.K.3.22 RCIC SUCTION	5	0	3	19	19
F-52	II.K.3.24 SPACE COOLING FOR HPCI/RCIC MODIFICATION	0	0	0	56	61
F-53	II.K.3.25 POWER ON PUMP SEALS	5	0	0	25	25
F-54	II.K.3.27 COMMON REFERENCE LEVEL	0	0	0	5	22
F-55	II.K.3.28 QUALIFICATION OF ADS ACCUMULATORS	16	0	1	7	8
F-56	II.K.3.29 PERFORMANCE OF ISOLATION CONDENSERS	0	0	1	3	73
F-57	II.K.3.30 SB LOCA METHODS	46	0	24	3	75
F-58	II.K.3.31 COMPLIANCE WITH 10 CFR 50.46	50	0	22	23	25
F-59	II.K.3.44 ANTICIPATED TRANSIENTS WITH SINGLE FAILURES	0	0	2	22	25
F-60	II.K.3.45 MANUAL DEPRESSURIZATION	1	0	0	0	0
F-61	II.K.3.46 MICHELSON CONCERNS	0	0	0	0	0
F-62	II.K.3.57 MANUAL ACTUATION OF ADS	0	0	0	2	75
F-63	III.A.1.2 TECHNICAL SUPPORT CENTER	73	0	0	2	74
F-64	III.A.1.2 OPERATIONAL SUPPORT CENTER	72	0	0	4	74
F-65	III.A.1.2 EMERGENCY OPERATIONS FACILITY	70	0	0	72	72
F-66	III.A.1.2 NUCLEAR DATA LINK	0	0	0		

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<u>CODE</u>	<u>GENERIC SUBJECT</u>	<u>ACTIVE</u>	<u>FUTURE</u>	<u>COMPL. CURRENT FY (84)</u>	<u>COMPL. PRIOR FY</u>	<u>TOTAL</u>
F-67	III.A.2.1 EMERGENCY PLAN UPGRADE TO MEET RULE	3	0	5	65	73
F-68	III.A.2.2 METEOROLOGICAL DATA UPGRADE	70	0	1	1	72
F-69	III.D.3.3 IMPLANT RADIATION MONITORING	0	0	0	71	71
F-70	III.D.3.4 CONTROL ROOM HABITABILITY	9	0	8	55	72
G-01	REACTOR COOLANT PUMP TRIP (D660 - II.K.3.5)	50	0	0	0	50
TOTAL GENERIC ISSUES		2,252	0	495	6,203	8,950

MPA No.	Multi-plant Action Title	Projected FY83 Carryover (Inputs/LAs)	Priority	Review Branch	Schedule for Technical Inputs to DL					Remaining Inputs to be Completed	FY 1985	FY 1986	Schedule for Actions Issued by DL			FY1984 Technical Review Performed by		
					FY 1984								FY 1984	FY 1985	FY 1986	In- House Staff	Under Contract	
					1st	2nd	3rd	4th	Total								DL	Under Div.
A-10	Contingency Planning		Complete															
A-11	Guard Training Plans		Complete															
A-12	Vital Area Analysis		Complete															
A-13	Non-Power Reactors Safeguards		Not Active															
A-14	10 CFR 50.55 A(G) - Inservice Testing (5 yr. old)	(13/22)	1	REG MEB	7 3	3 0	0 0	0 0	10 3	0 0	0 0	0 0	22 22	0 0	0 0	13 13		
A-15	Quality Assurance Request Diesel Generator Fuel Oil		Complete															
A-16	Qualification of Inspection Examination, Testing, and Audit Personnel		Complete	QAB RAB ICSB RSB														
A-17	Instrumentation to Follow		1	AEB ASB CPB	5	10	15	10	40	40	25	15	35	35	10	40		
B-01	Diesel Generator Lockout		Complete															
B-02	Fire Protection		Complete															
B-03	PWR Moderator Dilution		Complete															
B-04	Reactor Vessel Overpressure Protection (USI: A-26) (5 yr. old)	(2/4)	2	RSB	2	0	0	0	2	0	0	0	4	0	0			
B-05	Stress Corrosion Cracking BWR RCSPB (USI: A-42)	(24/24)	1	MTEB	6	6	6	6	24	0	0	0	20	4	0	24		

MPA No.	Multi-plant Action Title	Projected FY83 Carryover (Inputs/LAs)	Priority	Review Branch	Schedule for Technical Inputs to DL					Remaining Inputs to be Completed	FY 1985	FY 1986	Schedule for Actions Issued by DL			FY1984 Technical Review Performed by		
					FY 1984								FY 1984	FY 1985	FY 1986	In- House Staff	Under DL	Under Contract Div.
					1st	2nd	3rd	4th	Total									
B-30	Steam Generator Replacement Program		Complete															
B-31	Long Shaft LHSI & Outside Recirc. Pump Degradation		Complete															
B-32	Blocked SI Signal During Cooldown	(0/9)		ICSB	0	0	0	0	0	0	0	9	0	0	0			
B-33	Iodine Spiking		Complete															
B-34	BWR-Weld Failure of Jet Pump Retainer Bolt		Complete															
B-35	Orifice Rod Assembly Integrity B&W		Complete															
B-36	Resistance Temperature Detector (RID) Response - CE		Complete															
R-37	Steam Generator Tube Denting and Support Plate Modifications - CE		Complete															
B-38	Tendon Surveillance-Bechtel Cont.		Complete															
B-39	PWR Pressure - Temperature Limit Tech Specs		Complete															
B-40	Pipe Support Base Plates		Complete															
B-41	Fire Protection App. R	(19/23)	1	ASB CHEB	4 6	7 5	8 8	0 0	12 19	0	0	0	35					

MPA No.	Multi-plant Action Title	Projected FY83 Carryover (Inputs/LAs)	Priority	Review Branch	Schedule for: Technical Inputs to DL							Schedule for Actions Issued by DL			FY1984 Technical Review Performed by			
					FY 1984					Remaining Inputs to be Completed	FY 1985	FY 1986	FY 1984	FY 1985	FY 1986	In- House Staff	Under DL	Under Contract Div.
					1st	2nd	3rd	4th	Total									
B-68	ECCS Pump Deadheading			Not Yet Active														
B-69	PWR MSLB with Continued Feedwater Addition	(3/3)	1	CSB	1	2	0	0	3	0	0	0	3			3		
B-70	Monitoring Fatigue transients limits for RCCS			Complete														
B-72	NUREG 0737 TS		2	REG	17	19	19	19	74	0	0	0	40	34	0	54		(+20 in RII)
B-73	Plans for presenting Exceeding PTS Screening Criteria		Complete	MTEB	8	8	0	0	8							8		
B-74	Thermal Shield Support Structure Review		Complete	MTEB	2	2	0	0	4	0	0	0	4			4		
B-75	Cracking of Core Barrel		1	MTEB														
C-01	PWR Secondary Water Chemistry Monitoring Requirements (5 yr. old)	(2/4)	Complete		2	0	0	0	2	0	0	0	4		2			
C-02	BWR-Ricirc. Pump Trip (ATWS)		Complete															
C-03	Qualifications of Radiation Protection Manager		Complete															
C-04	Filter Tech Specs (5 yr. old)	(0/3)	3	ETSB	0	0	0	0	0	0	0	0	3		0			
C-05	Conversion to Standard Tech Spec		Complete															
C-06	Pump Support Lamellar Tearing		Complete															

MPA No.	Multi-plant Action Title	Projected FY83 Carryover (Inputs/LAs)	Priority	Review Branch	Schedule for Technical Inputs to DL							Schedule for Actions Issued by DL			FY1984 Technical Review Performed by			
					FY 1984					Remaining Inputs to be Completed	FY 1985	FY 1986	FY 1984	FY 1985	FY 1986	In- House Staff	Contract	
					1st	2nd	3rd	4th	Total								Under DL	Under Div.
D-04	PWR Reactor Vessel Cavity Seal Ring Missile Potential	(4/4)	Complete	MEB	1	1	2	0	4	0	0	0	4			4		
D-05	Plant UPI Model Problem	(6/6)	3	RSB	0	0	6	0	6	0	0	0	6				6	
D-06	Peaking Model Change for CE Reactor Core		Complete															
D-07	BWR Power Level for RWM		Complete															
D-08	Deficiency in Chem. Addition to Containment Sprays		Complete															
D-09	GE ECCS Input Errors		Complete															
D-10	Asymmetric LOCA Loads (USI:A-21)	(4/7)	2	MEB	8	4	4	4	20	0	0	0	23				20	
D-11	Fission Gas Release		Complete															
D-12	Non-Jet Pump BWR Core Spray Performance		Complete															
D-13	B&W Small Break Error		Complete															
D-14	Reactor Vessel Weld - Wire Deficiency		Complete															
D-15	High Energy Line Break & Consequential System Failure	(0/71)	Complete	RRAB	0	0	0	0	0	0	0	0	71				0	

MPA No.	Multi-plant Action Title	Projected FY83 Carryover (Inputs/LAs)	Priority	Review Branch	Schedule for Technical Inputs to DL							Schedule for Actions Issued by DL			FY1984 Technical Review Performed by			
					FY 1984					Remaining Inputs to be Completed	FY 1985	FY 1986	FY 1984	FY 1985	FY 1986	Review In- House Staff	Contract	
					1st	2nd	3rd	4th	Total								Under DL	Under Div.
F-10	II.B.1 RCS High Point Vents II.B.1.1 Final Design & Procedures	(0/12)	Complete	RSB	0	0	0	0	0	0	0	0	12	0	0	0		
F-11	II.B.2 Plant Shielding II.B.2.2 Modifications	(13/20)	Post	REG	12	1	0	0	13	0	0	0	20				13	
F-12	II.B.3 Post Accident Sampling II.B.3.2 Upgraded System		Post	CEB	13	13	12	12	50	10	10	0	50	10			50	
F-13	II.B.4 Training for Mitigating Core Damage	(4/4)	Post	REG	4	0	0	0	4				0	0	0		4	
F-14	II.D.1 RV & SV Testing II.D.1.1 BWR Test Report II.D.1.2.a BWR Plant Unique Analysis II.D.1.2.b PWR Test Report II.D.1.3 PWR Plant Unique	(15/17) (52/52)	BWR Post Pre Pre	EQB EQB MEB MEB	4 4	4 6	5 7	2 7	15 24	(BWR) 28(PWR)	28		17 20		32			15
F-15	II.E.1.1 AFW Sys Eval		Pre	ASB	2	1	1	4			4		4					
F-16	II.E.1.2.1 AFW System Initiation II.E.1.2.1.b Safety Grade		Post	ICSB	2						2		2					
F-17	II.E.1.2.2 AFW System Flow Indicat. II.E.1.2.2.b Safety Grade		Post	ICSB	2						2		2					
F-18	II.E.4.1 Dedicated H2 Penetration		Post	ICSB	2						2		2					

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MPA No.	Multi-plant Action Title	Projected FY83 Carryover (Inputs/LAs)	Priority	Review Branch	Schedule for Technical Inputs to DL					Schedule for Actions Issued by DL			FY1984 Technical Review Performed by					
					FY 1984					Remaining Inputs to be Completed	FY 1985	FY 1986	FY 1984	FY 1985	FY 1986	In- House Staff	Contract	
					1st	2nd	3rd	4th	Total								Under DL	Under Div.
F-42	II.K.3.12 Anticip Trip on II II.K.3.12a Design		Complete															
F-43	II.K.3.13 HPCI/RCIC Init Lvi II.K.3.13a Design II.K.3.13b Modify		Complete Post	ICSB	Complete													
F-44	II.K.3.14 IC Mods		Complete															
F-45	II.K.3.15 Isol of HPCI/RCIC		Complete	RSB														
F-46	II.K.3.16 Chall/Fail of RVS Report		Post	RSB				0				23						
F-47	II.K.3.17 ECCS Outage (0/6) Rpt		Complete	RRAB				0				6						
F-48	II.K.3.18 ADS Actuation II.K.3.18a Study II.K.3.18b Design II.K.3.18c Modifications		Complete Post Pre	ICSB RSB								24						
F-49	II.K.3.19 Interlock RCPs		Post	RSB			1	1				1						
F-50	II.K.3.21 Restart LPCI/CSS II.K.3.21a Design		Complete	RSB ICSB	Complete													
F-51	II.K.3.22 RCIC Suction II.K.3.22b Modify		Post	JCSB	1	2	4	7				7			7			
F-52	II.K.3.24 Space Cooling for RCOC/HPCI		Complete	ASB														
F-53	II.K.3.25 PWR Pump Seals II.K.3.24a BWR Study II.K.3.25a CE&W Study		Post Post	RSB RSB				6				6			6			

NPA No.	Multi-plant Action Title	Projected FY83 Carryover (Inputs/LAs)	Priority	Review Branch	Schedule for Technical Inputs to DL					Schedule for Actions Issued by DL			FY1984 Technical Review Performed by					
					FY 1984					Remaining Inputs to be Completed	FY 1985	FY 1986	FY 1984	FY 1985	FY 1986	In- House Staff	Contract	
					1st	2nd	3rd	4th	Total								Under DL	Under Div.
F-67	III.A.2.1 Emerg Plan to Upgrade Rule		Post	IE	4	4	0	0	8				8			8		
F-68	III.A.2.2 Met Data		Post	Reg. AEB														
F-69	III.D.3.3 Inplant Rad. Mon		Complete															
F-70	III.D.3.4 CR Habibility III.D.3.4.1 Design		Post	AEB	2	2	3	2	9	2	2	9	2					
G-1	RCP Trip		1	RSB	0	10	15	26	51	0		51				51		
TOTALS		(/)							1193									

TABLE L

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MULTIPLANT ACTIONS - LEAD ASSIGNMENTS AND CONTRACT INFORMATION

Data as of 04/08/83

Multiplant Action Number	Title	Status*	Lead Project Manager	Review Contracted		Contract Information			
				Yes	No	Funding Division	Contract Manager	FIN No.	Contractor
A-01	10 CFR 50.55 A(g) - ISI	C							
A-02	Appendix I - ALARA	A	G. Gears	X		DL/DSI	M. Carrington	B-6891	FRC II/ES&G
A-03	Security Reviews - Modified Amendment Plans	C							
A-04	Appendix J - Containment Leak Testing	A	G. Dick	X		DL	S. Dajwa	B-6590	FRC I(completed)
A-05	GE Mark I Containment Tech Specs - Short Term	C							
A-06	Respiratory Protection System	C							
A-07	Appendix G - Fracture Toughness	C							
A-08	ECCS Evaluation - Generic Per 50.46 Compliance	C							
A-09	Pressure Vessel Beltline Material Surveillance	C							
A-10	Contingency Planning	C							
A-11	Guard Training Plans	C							
A-12	Vital Area Analysis	C							
A-13	Non Power Reactor Safeguards Plan	I							
A-14	10 CFR 50.55 A (G) - Inservice Testing	A	P. Tam		X				
A-15	Quality Assurance Request Regarding Diesel Generator Fuel Oil	C							
A-16	Qualification of Inspection, Examination, Testing and Audit Personnel	C							
A-17	Instrumentation to Follow the Course of an Accident (R.G. 1.97)	A	J Shea						

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MULTIPLANT ACTIONS - LEAD ASSIGNMENTS AND CONTRACT INFORMATION

Multiplant Action Number	Title	Status*	Lead Project Manager	Review Contracted		Contract Information			
				Yes	No	Funding Division	Contract Manager	FIN No.	Contractor
				B-01	Diesel Generator Lockout	C			
B-02	Fire Protection	C							
B-03	PWR Moderator Dilution	C							
B-04	Reactor Vessel Overpressure Protection	A	A. DeAgazio	X		DSI			EG&G
B-05	Stress Corrosion Cracking - BWR RCSPB	A	R. Hermann	X		DL	J. Donohew	A-6429	INEL
B-06	BWR Relief Valve	C							
B-07	Steam Generator Feedwater Flow Instability	C							
B-08	PWR HPSI-LPSI Flow Resistance	C							
B-09	Charging Systems Pipe Vibrations	C							
B-10	Burnable Poison Rod Failure - B&W	C							
B-11	Flood of Equipment Important to Safety	C							
B-12	Steam Generator Tube Inspection	C							
B-13	Fuel Rod Bow	C							
B-14	CEA Guide Tube Wear	C							
B-15	C-E Poison Rod Growth	C							
B-16	Emergency Planning and Revisions	C							
B-17	Tech Spec Surveillance for Hydraulic Snubbers	A	L. Engle	X	X**	Region I	J. Donohew	A-0250	LLL
B-18	Worthington RHR Pump Shaft Integrity	C							
B-19	Neutron Shielding - CE Reactors	C							
B-20	Containment Leakage Due to Seal Deterioration	C							
B-21	Loss of 125-V DC Bus Voltage with Loss of Annunciator System	C							

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 **review not contracted for other Regions.

MULTIPLANT ACTIONS - LEAD ASSIGNMENTS AND CONTRACT INFORMATION

Multiplant Action Number	Title	Status*	Lead Project Manager	Review Contracted		Contract Information			
				Yes	No	Funding Division	Contract Manager	FIN No.	Contractor
B-22	Tech Spec Surveillance Requirements for Mechanical Snubbers	A	L. Engle	X	X***	Region I	J. Donohew	A-0250	LLL
B-23	Degraded Grid Voltage	A	R. Dudley	X		DL	J. Donohew	A-0250 A-6429	LLL & INEL
B-24	Venting and Purging Containment While at Full Power and Effect on LOCA	A	E. Reeves	X		DL	S. Bajwa	B-6590	FRC I (completed)
B-25	BWR Feedwater Nozzle Cracking	A	R. Gilbert		X				
B-26	Inadvertant Safety Injection During Cooldown	C							
B-27	Review Responses to IE Bulletin 78-03 (Offgas Explosions)	C							
B-28	BWR Jet Pump Flow Indication Elimination	C							
B-29	BWR Feedwater Pump Trip	C							
B-30	Steam Generator Replacement Program**	C							
B-31	Long Shift LHSI and Outside Recirculation Degradation	C							
B-32	Blocked SI Signal During Cooldown	A	D. McDonald						
B-33	Iodine Spiking	C							
B-34	BWR-Weld Failure of Jet Pump Retainer Bolt	C							
B-35	Grifce Rod Assembly Integrity - B&W	C							
B-36	Resistance Temperature Detector (RTD) Response - CE	C							
B-37	Steam Generator Tube Denting and Support Plate Modifications - CE	C							
B-38	Tendon Surveillance - Bechtel Containments	C							
B-39	PWR Pressure - Temperature Limit Tech Specs	C							

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** All new and pending reviews are being handled as plant specific actions.

***Review not contracted for other Regions.

MULTIPLANT ACTIONS - LEAD ASSIGNMENTS AND CONTRACT INFORMATION

Multiplant Action Number	Title	Status*	Lead Project Manager	Review Contracted		Contract Information			
				Yes	No	Funding Division	Contract Manager	FIN No.	Contractor
B-40	Pipe Support Base Plates	C							
B-41	Fire Protection - Final Tech Specs (Includes SER Supplements)	A	T. Wambach	X		DSI			BNL
B-42	TMI Follow Up-All Plants	C							
B-43	PWR Feedwater Line Cracks	C							
B-44	Lessons Learned Implementation	C							
B-45	WASH 1400 Event V, "Primary Coolant Pres. Isolation Valves"	C							
B-46	Analysis of Turbine Disc Cracks	C							
B-47	ECCS; Clad Swelling and Rupture	C							
B-48	Adequacy of Station Electric Distrib. Voltage	A	R. Dudley	X		DL	J. Donohew	A-0250 A-6429	LLL & INEL
B-49	PWR Control Rod Misalignment	C							
B-50	Auxiliary Feedwater System Evaluation	C							
B-51	Evaluation of Bulletin 79-05, 79-06, 79-08	C							
B-52	Review of Safety Aspect of Inadvertent Safety Actions During Surveillance Test	C							
B-53	Lessons Learned Category B (Items)	C							
B-54	Lessons Learned Category A Tech Spec	C							
B-55	B&O Report on BWRs	C							
B-56	Control Rods Fail to Insert BWR	C							
B-57	DHR Capability	A	A. DeAgazio	X		DL	J. Donohew	A-6429	INEL (completed)
B-58	SDV Capability	C							
B-59	Masonry Wall Design	A	C. Trammell	X		DL	M. Carrington	B-5891	FRC II

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MULTIPLANT ACTIONS - LEAD ASSIGNMENTS AND CONTRACT INFORMATION

Multiplant Action Number	Title	Status*	Lead Project Manager	Review Contracted		Contract Information			
				Yes	No	Funding Division	Contract Manager	FIN No.	Contractor
B-60	Environmental Qualification of Electrical Equipment	A	R. Karsch	X		DL	S. Bajwa	B-6890	FRC I
B-61	Loss of Non Class IE Instr and Control Power	C			X				
B-62	ESF Reset Design Deficiency-IE Bulletin 80-06	C					J. Donohew	A-0250	LLL (completed)
B-63	Interim procedures for Station Blackout	C			X				
B-64	B&W Induced Flux Errors	C							
B-65	Safety Concerns Assoc. with Pipe Breaks in the BWR Scram System	A	V. Rooney		X				
B-66	Nat. Cir. Cool-down	A	K. Heitner						
B-67	Thermal Shock	C							
B-68	ECCS Pump Deadheading	I							
B-69	IEB 80-4 Mainsteam Line Break with Continued Feedwater Addition	A	E. Tourigny	X		DL	M. Carrington	B-6891	FRC II
B-70	Monitoring of Fatigue Transient Limits for ECCS	I							
B-71	120 VAC Vital Instrument Busses and Inverter Technical Specifications	I**	G. Rivenbark						
B-72	NUREG-0737 Tech Specs	A	C. Patel		X				
B-73	Plans for Preventing Exceeding PTS Screening Criterion	C							
B-74	Thermal Shield Support Structure Review	C							
B-75	B&W Reactor Vessel Internals Bolting	A	N. Kadambi		X				

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** Action inactive pending CRGR review.

MULTIPLANT ACTIONS - LEAD ASSIGNMENTS AND CONTRACT INFORMATION

Multiplant Action Number	Title	Status*	Lead Project Manager	Review Contracted		Contract Information			
				Yes	No	Funding Division	Contract Manager	FIN No.	Contractor
B-76	Item 1.1 - Post-Trip Review - Program Description and Procedures	A	A. Bournia		X				
B-77	Item 2.1 - Equipment Classification and Vendor Interface - RTS Components	A	H. Abelson		X				
B-78	Items 3.1.1 and 3.1.2 - Post-Maintenance Testing Procedures and Vendor Recommendations - RTS Components	A	R. Birkel		X				
B-79	Item 3.1.3 - Post-Maintenance Testing - Changes to Tech Specs - RTS Components	A	R. Birkel		X				
B-80	Item 4.1 - Reactor Trip System Reliability Vendor Related Modifications	A	R. Birkel		X				
B-81	Items 4.2.1 and 4.2.2 - Preventative Maintenance Program for Reactor Trip Breakers - Maintenance and Testing	A	R. Pearch		X				
B-82	Item 4.3 - Automatic Actuation of Shunt Trip Attachment for W and B&W Plants	A	T. Kenyon		X				
B-83	Review of the Technical Specifications for NUREG-0737 (Generic Letters 83-36 and 83-37)	A	C. Patel		X				

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MULTIPLANT ACTIONS - LEAD ASSIGNMENTS AND CONTRACT INFORMATION

Multiplant Action Number	Title	Status*	Lead Project Manager	Review Contracted		Contract Information			
				Yes	No	Funding Division	Contract Manager	FIN No.	Contractor
C-01	PWR Secondary Water Chemistry Monitoring Requirements	C							
C-02	BWR - Recirc. Pump Trip (ATWS)	C							
C-03	Qualifications of Radiation Protection Manager	C							
C-04	Filter Tech Specs	A	J. Lombardo		X				
C-05	Conversion to Standard Tech Specs	C							
C-06	Pump Support - Lamellar Tearing	C							
C-07	Fuel Handling Accident Inside Containment	A	J. Lombardo		X				
C-08	BWR Post LOCA H-2 Control	C							
C-09	PWR Aux. FW Pumps	C							
C-10	Control of Heavy Loads Over Spent Pool (Phase 1)	A	D. Neighbors	X		DL	M. Carrington	B-6891	FRC II
C-11	RPS Power Supply	A	M. Thadani	X		DL	J. Donohew	A-0250	LLL
C-12	Boron Solubility During Long Term Cooling Following LOCA	C							
C-13	Loss of Offsite Power	C							
C-14	Aux. Feedwater Seismic Qualification	A	J. Beard	X		DL	J. T. Beard	A-0429	LLL
C-15	Heavy Loads Phase 2	A	D. Neighbors	X					

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MULTIPLANT ACTIONS - LEAD ASSIGNMENTS AND CONTRACT INFORMATION

Multiplant Action Number	Title	Status*	Lead Project Manager	Review Contracted		Contract Information			
				Yes	No	Funding Division	Contract Manager	FIN No.	Contractor
D-01	GE Mark I Containment Evaluation Long Term	A	B. Siegel	X		DL	B. Barnhart	A-3713	BNL
D-02	ECCS Circ. Clad Model Error - Compliance with 10 CFR 46	C							
D-03	Pressurizer Heatup Rate Error	C							
D-04	PWR Reactor Vessel Cavity Seal Ring Missile Potential	C							
D-05	Plant UPI Model Problem	A	S. MacKay		X	DSI			SNL
D-06	Peaking Model Change for CE Reactor Core	C							
D-07	PWR Power Level for RWM	C							
D-08	Deficiency in Chem Addition to Containment Sprays	C							
D-09	GE ECCS Input Errors	C							
D-10	Asymmetric LOCA Loads	A	J. Shea		X				
D-11	Fission Gas Release	C							
D-12	Non-Jet Pump BWR Core Spray Performance	C							
D-13	B&W Small Break Error	C							
D-14	Reactor Vessel Weld-Wire Deficiency	C							
D-15	High Energy Line Break and Consequential System Failure	C							
D-16	Review of Corporate Management Capability	C							
D-17	Definition of Operable	A	R. Lee	X		DL	J. Donohew	A-6429	INEL
D-18	NUREG-0630 Cladding Models - B&W Plants	C							

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MULTIPLANT ACTIONS - LEAD ASSIGNMENTS AND CONTRACT INFORMATION

Multiplant Action Number	Title	Status*	Lead Project Manager	Review Contracted		Contract Information			
				Yes	No	Funding Division	Contract Manager	FIN No.	Contractor
				E-01	Spent Fuel Pool Expansions**	C			
E-02	Fuel Cask Drop	C							
E-03	Core Reloads Requiring Prior NRC Approval**	C							
E-04	BWR Single Loop Operation	A	R. Bevan	X		DL	J. Donohew	A-0250	LLL
E-05	WN-1 Loop Operation	A	D. Wigginton	X		DL	J. Donohew	A-0250	LLL(completed)
E-06	CEA Position Indication Failures - CE	C							
E-07	Reactor Protection System Logic - CE	C							

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**All new and pending reviews are being handled as plant specific actions

MULTIPLANT ACTIONS - LEAD ASSIGNMENTS AND CONTRACT INFORMATION

L10

Multiplant Action Number	Title	Status*	Lead Project Manager	Review Contracted		Contract Information			
				Yes	No	Funding Division	Contract Manager	FIN No.	Contractor
F-01	I.A.1.1 Shift Technical Advisor	C							
F-02	I.A.1.3 Shift Manning	A	D. Fischer		X				
F-03	I.A.2.1 Upgrading of RO and SRO Training	A	D. Wigginton	X		DL	S. Bajwa	B-8011	SAI
F-04	I.C.1.2 Inadequate Core Cooling Guidelines and Procedures	A	J. Lyons		X				
F-05	I.C.1.3 Abnormal Transient Operator Guidelines and Procedures	A	J. Lyons	X		DSI			
F-06	I.C.5 Feedback of Operating Experience	C							
F-07	I.C.6 Correct Performance of Operating Activities	A	J. Hopkins		X				
F-08	I.D.1 Control Room Design Review	A	G. Dick		X				
F-09	I.D.2 Safety Parameter Display System	A	G. Dick		X				
F-10	II.B.1 RCS High Point Vents	C							
F-11	II.B.2 Plant Shielding	A	E. Tourigny		X				
F-12	II.B.3 Post accident Sampling	A	J. Norris						
F-13	II.B.4 Training for mitigating Core Damage	A	D. Wigginton	X		DL	S. Bajwa	B-8011	SAI
F-14	II.B.1 RV and SV Testing	A	M. Caruso	X		DE			
F-15	II.E.1.1 AFW System Evaluation	A	R. Hernan						
F-16	II.E.1.2.1 AFW System Initiation	A	C. Patel	X		DL	S. Bajwa	B-6590	FRC I (completed)
F-17	II.E.1.2.2 AFW System Flow Indication	A	C. Patel	X		DL	S. Bajwa	B-6590	FRC I (completed)
F-18	II.E.4.1 Dedicated Hydrogen Penetrations	A	R. Hernan						
F-19	II.E.4.2 Containment Isolation Dependability	A	E. Reeves						
F-20	II.F.1.1 Noble Gas Monitor	A	H. Nicolaras						
F-21	II.F.1.2 Iodine/Particulate Sampling	A	H. Nicolaras						

* A = Active, C = Complete, I = Inactive.

MULTIPLANT ACTIONS - LEAD ASSIGNMENTS AND CONTRACT INFORMATION

Multiplant Action Number	Title	Status*	Lead Project Manager	Review Contracted		Contract Information			
				Yes	No	Funding Division	Contract Manager	FIN No.	Contractor
F-22	II.F.1.3 Containment High Range Monitor	A	R. Hernan						
F-23	II.F.1.4 Containment Pressure Instrument	A	H. Nicolaras						
F-24	II.F.1.5 Containment Water Level Instrument	A	H. Nicolaras						
F-25	II.F.1.6 Containment Hydrogen Monitor	A	H. Nicolaras						
F-26	II.F.2 Instruments for Detection of Inadequate Core Cooling	A	J. Shea	X		DSI			ORNL
F-27	II.K.2.9 FMEA on ICS	C							
F-28	II.K.2.10 Safety Grade ARTS	C							
F-29	II.K.2.11 Continued Operator Training and Drilling	C							
F-30	II.K.2.13 Thermal-Mechanical Report	A	G. Vissing		X				
F-31	II.K.2.14 Lift Frequency of PORVs and SVs	C							
F-32	II.K.2.16 RCP Seal Damage	C							
F-33	II.K.2.17 Potential for Voiding in RCS	C							
F-35	II.K.2.20 System Response to SB LOCA	C							
F-36	II.K.3.1 Auto PORV Isolation	A	T. Colburn		X				
F-37	II.K.3.2 Report on PORV Failures	A	T. Colburn	X		DL	M. Carrington	B-6891	FRC II
F-38	II.K.3.3 Reporting SV and RV Failures and Challenges	A	T. Colburn		X				
F-39	II.K.3.5 Auto Trip of RCPs	C							
F-40	II.K.3.9 PID Controller	C							
F-41	II.K.3.10 Anticipatory Trip Modifications	C			X				
F-42	II.K.3.12 Anticipatory Trip on Turbine Trip	C							
F-43	II.K.3.13 HPCI and RCIC Initiation Levels	C							

* A = Active, C = Complete, I = Inactive.

MULTIPLANT ACTIONS - LEAD ASSIGNMENTS AND CONTRACT INFORMATION

Multiplant Action Number	Title	Status*	Lead Project Manager	Review Contracted		Contract Information			
				Yes	No	Funding Division	Contract Manager	FIN No.	Contractor
F-44	II.K.3.14 Isolation Condenser Isolation Modification	C							
F-45	II.K.3.15 Isolation of HPCI and RCIC Modification	C							
F-46	II.K.3.16 Challenges and Failures of Relief Valves	A	V. Rooney		X				
F-47	II.K.3.17 ECCS Outages	C							
F-46	II.K.3.18 ADS Actuation	A	R. Bevan		X				
F-49	II.K.3.19 Interlock Recirculation Pump Modification	A	K. Eccleston		X				
F-50	II.K.3.21 Restart of CSS and LPCI	C							
F-51	II.K.3.22 RCIC Suction	A	R. Bevan						
F-52	II.K.3.24 Space Cooling for HPCI/RCIC Modifications	C			X				
F-53	II.K.3.25 Power on Pump Seals	A	R. Hermann		X				
F-54	II.K.3.27 Common Reference Level	C			X				
F-55	II.K.3.28 Qualification of ADS Accumulators	A	R. Hermann		X				
F-56	II.K.3.29 Performance of Isolation Condensers	C							
F-57	II.K.3.30 SB LOCA Methods	A	D. Wigginton		X	DSI			LANL
F-58	II.K.3.31 Compliance with 10 CFR 50.46	A	B. Siegel		X				
F-59	II.K.3.44 Anticipated Transients with Single Failures	A	J. Lombardo		X				
F-60	II.K.3.45 Manual Depressurization	A	R. Bevan						

* A = Active, C = Complete, I = Inactive.

MULTIPLANT ACTIONS - LEAD ASSIGNMENTS AND CONTRACT INFORMATION

Multiplant Action Number	Title	Status*	Lead Project Manager	Review Contracted		Contract Information			
				Yes	No	Funding Division	Contract Manager	FIN No.	Contractor
F-51	II.K.3.46 Michelson Concerns	C							
F-52	II.K.3.47 Manual Actuation of ADS	C							
F-53	III.A.1.2 Technical Support Center	A	W. Paulson		X				
F-54	III.A.1.2 Operational Support Center	A	W. Paulson		X				
F-65	III.A.1.2 Emergency Operations Facility	A	W. Paulson		X				
F-57	III.A.2.1 Emergency Plan Upgrade to Meet Rule	A	P. Erickson		X				
F-58	III.A.2.2 Meteorological Data Upgrade	A	J. Lombardo		X				
F-59	III.D.3.3 Inplant Radiation Monitoring	C							
F-70	III.D.3.4 Control Room Habitability	A	R. Gilbert		X				
G-1	Reactor Coolant Pump Trip	A	D. Jaffe						

* A = Active, C = Complete, I = Inactive.

IMPLEMENTATION OF RESOLVED USIs

M-1

Resolved USIs which result in changes on operating reactors become multiplant actions (MPAs). The progress can be readily tracked by referring to the appropriate MPA. Below is a correlation between USI numbers and MPA numbers.

<u>JSI #</u>	<u>MPA #</u>	<u>Title</u>
A-2	D-10	Asymmetric LOCA Loads
A-7	D-01	GE Mark I Containment Evaluation - Long Term
A-10	B-25	BWP Feedwater Nozzle Cracking
A-24	B-60	Environmental Qualification
A-26	B-04	Reactor Vessel Overpressure Protection
A-36	C-10	Control of Heavy Loads Over Spent Fuel Pool
A-42	B-05	Stress Corrosion Cracking - BWR RCSPB

The following USIs have been resolved but not backfitted on operating reactors:

A-9	ATWS - Involved in rulemaking
A-31	Residual Heat Removal Requirements

DIVISION OF LICENSING POINTS OF CONTACT

N-1

FOR UNRESOLVED SAFETY ISSUES

	USI	CONTACT
A-1	Water Hammer	J. Neighbors
A-3/4/5	Steam Generator Tube Integrity	R. Martin
A-8	Mark II Containment Pool Dynamic Loads	T. Bournia
A-9	Anticipated Transients Without Scram	V. Rooney
A-11	Reactor Vessel Material Toughness	A. DeAgazio
A-12	Steam Generator and Reactor Coolant Pump Supports	A. DeAgazio
A-17	Systems Interaction	T. Michaels
A-39	SRV Pool Dynamic Loads	B. Siegel
A-40	Seismic Design Criteria - Short Term Program	T. Cheng
A-43	Containment Emergency Sump Performance	D. Jaffe
A-44	Station Blackout	L. Engle
A-45	Shutdown Decay Heat Removal Requirements	D. DiIanni
A-46	Seismic Qualification of Equipment	P. Y. Chen
A-47	Safety Implications of Control Systems	J. T. Beard
A-48	Hydrogen Control Measures	V. Rooney
A-49	Pressurized Thermal Shock	G. Vissing

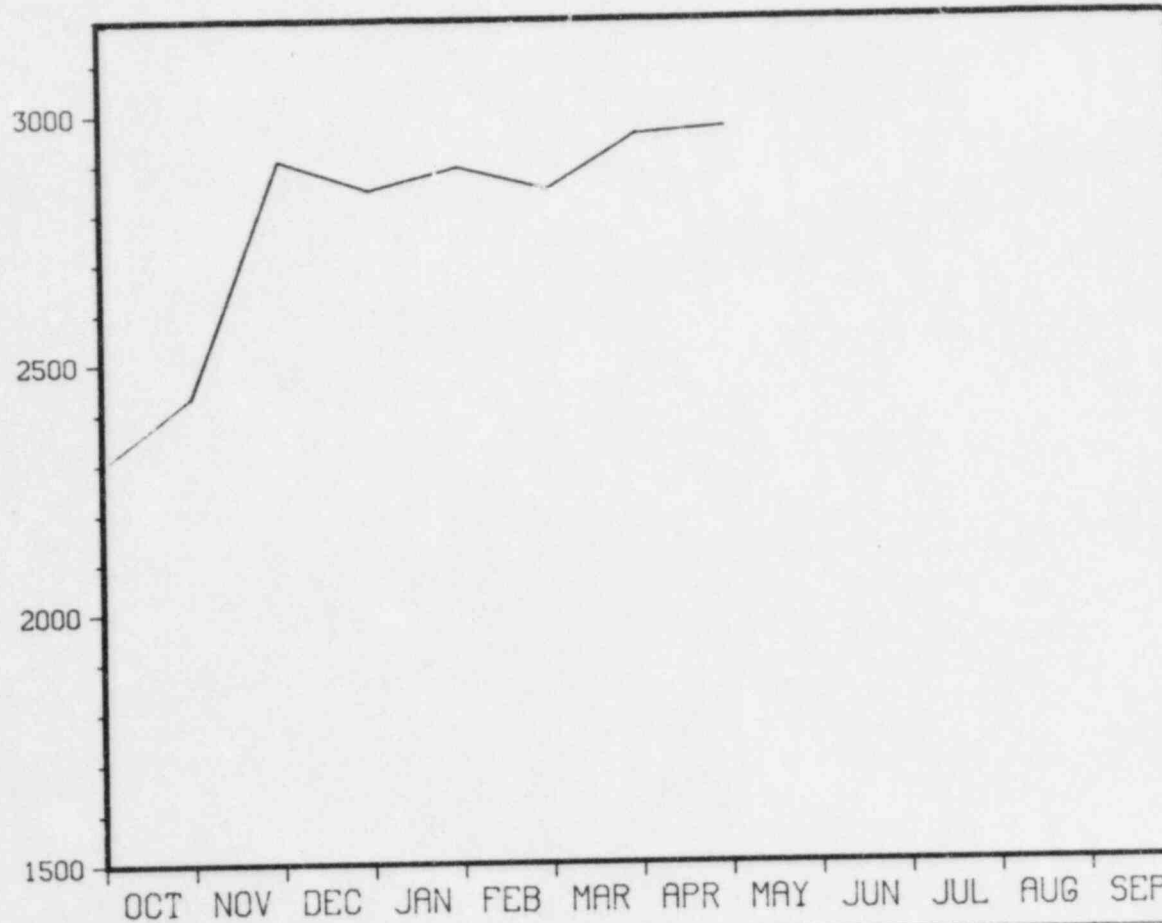
SECTION III
ROUTINE (NON-NUREG 0737)
LICENSING ACTION SUMMARY

TOTAL ACTIVE ROUTINE ACTIONS

0-1

DIVISION OF LICENSING

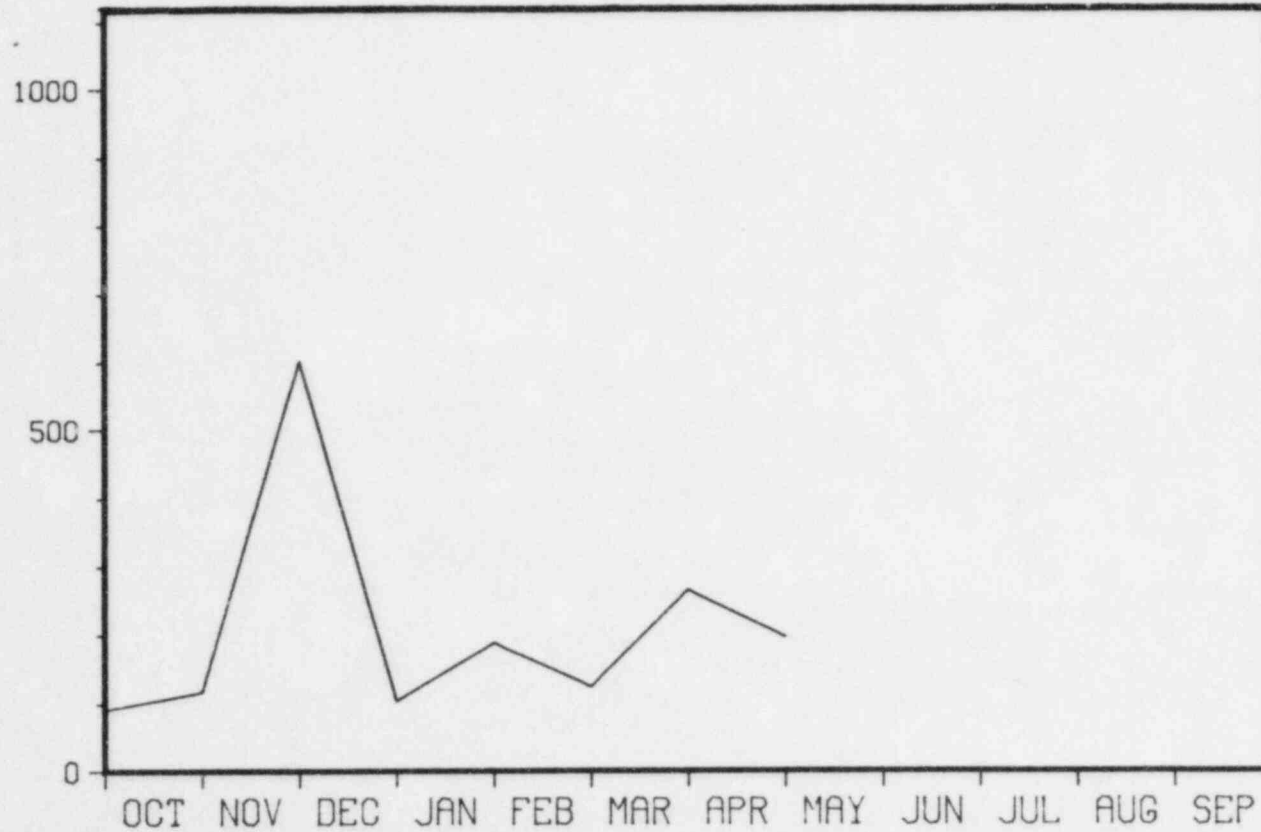
(EXCLUDES NUREG-0737 ITEMS)



TOTAL	2432	2907	2847	2893	2848	2959	2973					
ACTIONS	2429	2904	2841	2887	2840	2950	2968					
HEARINGS	03	03	06	06	08	09	05					

NEW ROUTINE ACTIONS BY MONTH

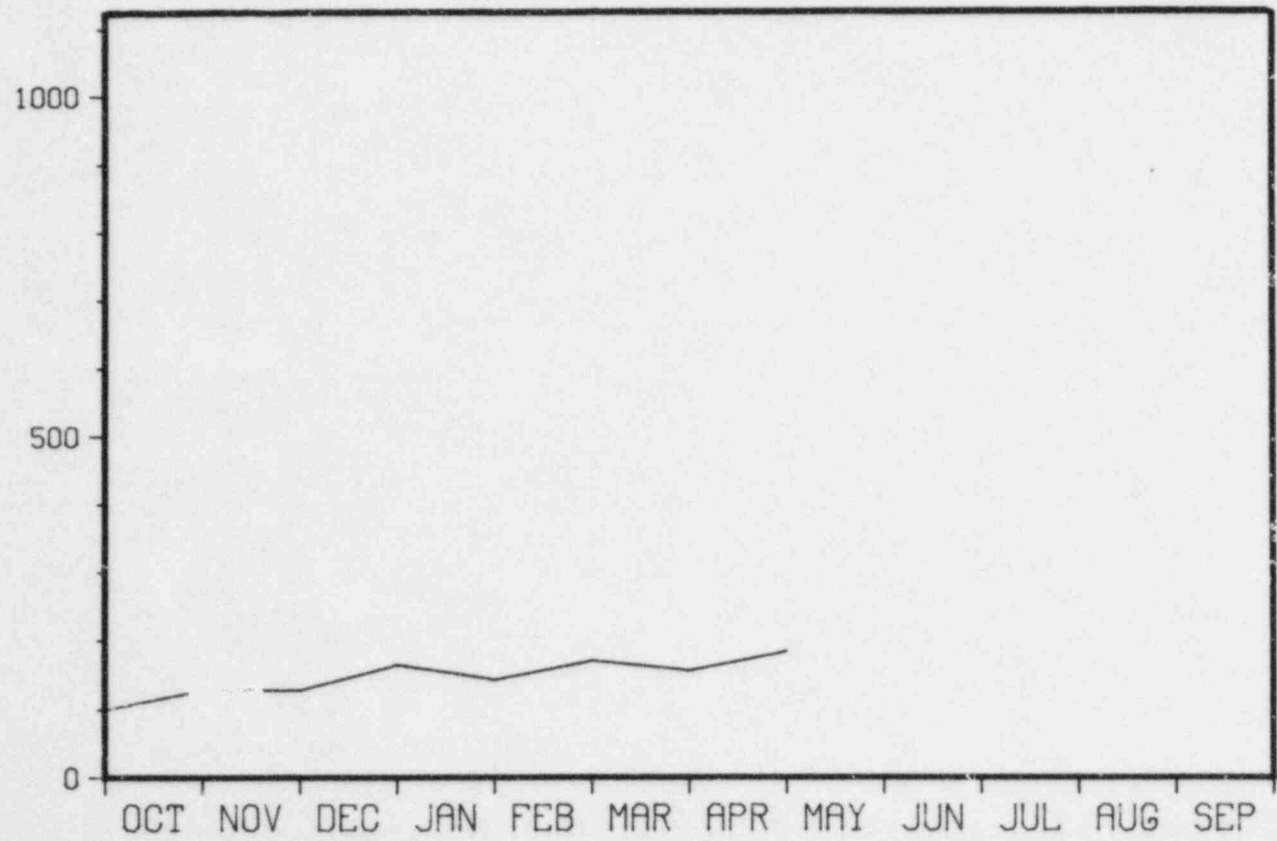
DIVISION OF LICENSING



NEW ACTIONS	118	602	105	188	125	265	197					
CUMULATIVE	118	720	825	1013	1138	1403	1600					

COMPLETED ROUTINE ACTIONS BY MONTH

DIVISION OF LICENSING



COMPLETED	128	128	164	143	170	156	183					
CUMULATIVE	128	256	420	563	733	889	1072					

FISCAL YEAR: 1984

ROUTINE LICENSING ACTIONS BY MONTH

REPORT DATE: 05/21/84

<u>FACILITY NAME</u>	<u>OCT.</u>	<u>NOV.</u>	<u>DEC.</u>	<u>JAN.</u>	<u>FEB.</u>	<u>MAR.</u>	<u>APR.</u>	<u>MAY</u>	<u>JUNE</u>	<u>JULY</u>	<u>AUG.</u>	<u>SEPT.</u>
ARKANSAS 1	30	38	37	37	37	39	39					
ARKANSAS 2	33	36	36	35	34	32	30					
BEAVER VALLEY 1	27	37	36	35	35	34	33					
BIG ROCK POINT 1	31	34	34	31	31	30	29					
BROWNS FERRY 1	55	63	57	57	58	59	61					
BROWNS FERRY 2	47	55	54	54	54	54	54					
BROWNS FERRY 3	46	53	52	66	66	66	66					
BRUNSWICK 1	39	43	32	30	31	29	29					
BRUNSWICK 2	47	48	37	34	35	33	35					
CALVERT CLIFFS 1	22	25	25	24	26	24	29					
CALVERT CLIFFS 2	17	26	28	27	29	27	22					
COOK 1	22	22	22	22	21	22	20					
COOK 2	27	27	27	25	24	25	23					
COOPER STATION	18	21	22	19	19	19	18					
CRYSTAL RIVER 3	29	38	39	38	39	41	41					
DAVIS-BESSE 1	42	50	54	52	52	56	53					
DIABLO CANYON 1	25	33	33	33	33	35	35					
DRESDEN 1	1	1	1	1	1	2	3					
DRESDEN 2	37	38	36	36	33	36	37					
DRESDEN 3	31	32	30	29	29	29	30					
DUANE ARNOLD	28	32	31	29	33	30	31					
FARLEY 1	29	34	30	25	24	30	31					
FARLEY 2	36	44	41	38	39	43	43					
FITZPATRICK	32	35	33	36	34	36	34					
FORT CALHOUN 1	22	28	28	25	25	28	28					
FORT ST VRAIN	25	28	28	28	27	26	26					
GINNA	45	51	54	57	56	62	58					
GRAND GULF 1	60	61	63	68	67	78	75					
HADDAM NECK	38	45	45	46	42	47	46					
HATCH 1	27	33	34	33	33	32	30					
HATCH 2	27	34	37	35	31	30	29					
HUMBOLDT BAY 3	2	2	3	3	3	3	3					
INDIAN POINT 2	35	38	38	38	38	40	41					
INDIAN POINT 3	32	40	38	37	38	40	38					
KEWAUNEE	23	28	28	30	28	32	30					
LA CROSSE	32	37	35	37	38	39	39					
LASALLE 1	8	12	12	13	12	12	12					
LASALLE 2	2	2	2	2	5	2	1					
MAINE YANKEE	35	38	36	40	40	39	39					
MCGUIRE 1	16	26	26	27	24	25	23					

TECHNICAL ASSIGNMENT CONTROL SYSTEM
ROUTINE LICENSING ACTIONS BY MONTH

FISCAL YEAR: 1984

REPORT DATE: 05/21/84

<u>FACILITY NAME</u>	<u>OCT.</u>	<u>NOV.</u>	<u>DEC.</u>	<u>JAN.</u>	<u>FEB.</u>	<u>MAR.</u>	<u>APR.</u>	<u>MAY</u>	<u>JUNE</u>	<u>JULY</u>	<u>AUG.</u>	<u>SEPT.</u>
MCGUIRE 2	18	28	29	28	25	24	21					
MILLSTONE 1	44	46	44	47	46	45	48					
MILLSTONE 2	30	36	32	32	32	34	32					
MONTICELLO	42	45	43	37	36	36	37					
NINE MILE POINT 1	24	33	32	36	33	34	33					

NORTH ANNA 1	22	32	32	39	35	38	37					
NORTH ANNA 2	22	31	32	36	34	38	36					
OCONEE 1	26	34	35	32	34	34	36					
OCONEE 2	27	34	35	32	34	35	37					
OCONEE 3	27	35	36	32	35	36	37					

OYSTER CREEK 1	41	46	54	50	47	47	47					
PALISADES	30	35	35	38	37	37	38					
PEACH BOTTOM 2	38	42	43	42	38	36	38					
PEACH BOTTOM 3	36	41	39	38	35	34	36					
PILGRIM 1	29	33	32	31	31	34	31					

POINT BEACH 1	20	30	29	27	27	33	31					
POINT BEACH 2	21	30	29	27	26	29	28					
PRAIRIE ISLAND 1	11	18	16	17	16	16	16					
PRAIRIE ISLAND 2	10	17	16	17	16	16	16					
QUAD CITIES 1	23	27	27	27	27	30	27					

QUAD CITIES 2	27	31	30	28	25	27	24					
RANCHO SECO 1	43	51	50	49	49	52	52					
ROBINSON 2	39	45	44	45	43	44	45					
SALEM 1	20	30	30	29	28	30	31					
SALEM 2	18	30	30	29	29	32	33					

SAN ONOFRE 1	26	34	33	35	31	34	34					
SAN ONOFRE 2	40	45	45	44	46	52	76					
SAN ONOFRE 3	31	37	36	36	38	42	68					
SEQUOYAH 1	39	47	38	40	40	42	43					
SEQUOYAH 2	31	39	28	30	30	32	32					

ST LUCIE 1	29	35	35	34	34	30	29					
ST LUCIE 2	19	26	26	26	26	27	29					
SUMMER 1	21	28	29	32	30	30	26					
SURRY 1	28	35	36	33	28	30	30					
SURRY 2	29	36	37	34	30	31	32					

SUSQUEHANNA 1	25	28	28	30	31	29	29					
SUSQUEHANNA 2				1	2	2	6					
THREE MILE ISLAND 1	40	48	54	55	59	58	51					
TROJAN	26	30	30	31	30	30	35					
TURKEY POINT 3	24	28	27	27	26	29	29					

FISCAL YEAR: 1984

ROUTINE LICENSING ACTIONS BY MONTH

REPORT DATE: 05/21/84

<u>FACILITY NAME</u>	<u>OCT.</u>	<u>NOV.</u>	<u>DEC.</u>	<u>JAN.</u>	<u>FEB.</u>	<u>MAR.</u>	<u>APR.</u>	<u>MAY</u>	<u>JUNE</u>	<u>JULY</u>	<u>AUG.</u>	<u>SEPT.</u>
TURKEY POINT 4	24	28	27	27	26	28	28					
VERMONT YANKEE 1	31	36	35	33	35	39	38					
WASHINGTON NUCLEAR 2			2	4	5	14	14					
YANKEE-ROWE 1	36	42	42	41	38	42	42					
ZION 1	33	39	36	62	61	61	56					

ZION 2	32	38	35	61	60	60	55					
GRAND TOTAL	2,432	2,907	2,847	2,893	2,848	2,959	2,973					

REPORT DATE: 05/21/84

ROUTINE LICENSING ACTIONS

POWER REACTORS

FOR 04/01/84 - 04/30/84

FACILITY	NEW ACTIONS FOR THIS REPORTING PERIOD	NEW ACTIONS TO DATE IN FY 84	COMPLETED ACTIONS FOR THIS REPORTING PERIOD	COMPLETED ACTIONS TO DATE IN FY 84	NUMBER OF ROUTINE ACTIONS	NUMBERS OF HEARINGS	TOTAL ACTIONS
ARKANSAS 1	0	20	0	16	39	0	39
ARKANSAS 2	0	15	2	21	30	0	30
BEAVER VALLEY 1	2	26	3	21	33	0	33
BIG ROCK POINT 1	1	13	2	12	29	0	29
BROWNS FERRY 1	2	14	0	8	61	0	61
BROWNS FERRY 2	4	14	4	7	54	0	54
BROWNS FERRY 3	3	27	3	7	66	0	66
BRUNSWICK 1	1	16	1	23	29	0	29
BRUNSWICK 2	2	23	0	30	35	0	35
CALVERT CLIFFS 1	10	31	5	22	29	0	29
CALVERT CLIFFS 2	1	25	6	19	22	0	22
COOK 1	0	11	2	13	20	0	20
COOK 2	0	12	2	14	23	0	23
COOPER STATION	0	9	1	9	18	0	18
CRYSTAL RIVER 3	3	20	3	7	41	0	41
DAVIS-BESSE 1	1	22	4	11	53	0	53
DIABLO CANYON 1	0	13	0	0	34	1	35
DRESDEN 1	2	3	1	1	3	0	3
DRESDEN 2	3	13	2	14	37	0	37
DRESDEN 3	3	15	2	17	30	0	30
DUANE ARNOLD	2	17	1	15	31	0	31
FARLEY 1	2	26	1	25	31	0	31
FARLEY 2	0	21	0	16	43	0	43
FITZPATRICK	0	12	2	10	34	0	34
FORT CALHOUN 1	2	16	2	10	28	0	28
FORT ST VRAIN	1	17	1	17	26	0	26
GINNA	4	29	8	20	58	0	58
GRAND GULF 1	5	33	8	16	75	0	75
HADDAM NECK	0	18	1	9	46	0	46
HATCH 1	0	14	2	14	30	0	30
HATCH 2	1	16	2	17	29	0	29
HUMBOLDT BAY 3	0	2	0	0	3	0	3
INDIAN POINT 2	1	12	0	6	41	0	41
INDIAN POINT 3	1	12	3	6	38	0	38
KEWAUNEE	0	17	2	11	30	0	30

REPORT DATE: 05/21/84

ROUTINE LICENSING ACTIONS

POWER REACTORS

FOR 04/01/84 - 04/30/84

FACILITY	NEW ACTIONS FOR THIS REPORTING PERIOD	NEW ACTIONS TO DATE IN FY 84	COMPLETED ACTIONS FOR THIS REPORTING PERIOD	COMPLETED ACTIONS TO DATE IN FY 84	NUMBER OF ROUTINE ACTIONS	NUMBERS OF HEARINGS	TOTAL ACTIONS
LA CROSSE	3	13	3	15	39	0	39
LASALLE 1	0	9	0	6	12	0	12
LASALLE 2	0	4	1	5	1	0	1
MAINE YANKEE	0	20	0	16	38	1	39
MCGUIRE 1	0	14	2	9	23	0	23
MCGUIRE 2	0	12	3	10	21	0	21
MILLSTONE 1	3	15	0	17	48	0	48
MILLSTONE 2	1	17	3	14	32	0	32
MONTICELLO	2	9	1	12	37	0	37
NINE MILE POINT 1	2	23	3	14	33	0	33
NORTH ANNA 1	4	39	5	18	37	0	37
NORTH ANNA 2	3	36	5	17	36	0	36
OCONEE 1	2	17	0	6	36	0	36
OCONEE 2	2	17	0	7	37	0	37
OCONEE 3	1	18	0	6	37	0	37
OYSTER CREEK 1	0	14	0	8	47	0	47
FALISADES	4	22	3	12	38	0	38
PEACH BOTTOM 2	4	20	2	20	38	0	38
PEACH BOTTOM 3	4	15	2	16	36	0	36
PILGRIM 1	1	16	4	11	31	0	31
POINT BEACH 1	0	27	2	18	31	0	31
POINT BEACH 2	0	22	1	17	28	0	28
PRAIRIE ISLAND 1	0	11	0	7	16	0	16
PRAIRIE ISLAND 2	0	11	0	6	16	0	16
QUAD CITIES 1	0	10	3	6	27	0	27
QUAD CITIES 2	0	6	3	9	24	0	24
RANCHO SECO 1	1	28	1	16	52	0	52
ROBINSON 2	1	25	0	13	45	0	45
SALEM 1	1	15	0	5	31	0	31
SALEM 2	1	19	0	4	33	0	33
SAN ONOFRE 1	2	18	2	11	34	0	34
SAN ONOFRE 2	34	53	10	27	76	0	76
SAN ONOFRE 3	35	51	9	16	68	0	68
SEQUOYAH 1	1	13	0	9	43	0	43
SEQUOYAH 2	0	12	0	11	32	0	32

REPORT DATE: 05/21/84

ROUTINE LICENSING ACTIONS

POWER REACTORS

FOR 04/01/84 - 04/30/84

FACILITY	NEW ACTIONS FOR THIS REPORTING PERIOD	NEW ACTIONS TO DATE IN FY 84	COMPLETED ACTIONS FOR THIS REPORTING PERIOD	COMPLETED ACTIONS TO DATE IN FY 84	NUMBER OF ROUTINE ACTIONS	NUMBERS OF HEARINGS	TOTAL ACTIONS
ST LUCIE 1	0	11	1	9	29	0	29
ST LUCIE 2	2	17	0	6	29	0	29
SUMMER 1	0	17	4	14	26	0	26
SURRY 1	2	16	2	16	30	0	30
SURRY 2	3	15	2	14	32	0	32
SUSQUEHANNA 1	1	18	1	11	29	0	29
SUSQUEHANNA 2	4	6	0	0	6	0	6
THREE MILE ISLAND 1	0	33	7	25	48	3	51
TROJAN	5	23	0	11	35	0	35
TURKEY POINT 3	5	24	5	20	29	0	29
TURKEY POINT 4	5	23	5	20	28	0	28
VERMONT YANKEE 1	0	15	1	8	38	0	38
WASHINGTON NUCLEAR 2	0	14	0	0	14	0	14
YANKEE-ROWE 1	1	15	1	12	42	0	42
ZION 1	0	39	5	14	56	0	56
ZION 2	0	39	5	14	55	0	55
GRAND TOTAL	197	1,600	183	1,072	2,968	5	2,973

REPORT DATE: 05/21/84

ACTIVE LICENSING ACTIONS

NON-POWER REACTORS

FOR 04/01/84 - 04/30/84

FACILITY	NEW ACTIONS FOR THIS REPORTING PERIOD	NEW ACTIONS TO DATE IN FY 84	COMPLETED ACTIONS FOR THIS REPORTING PERIOD	COMPLETED ACTIONS TO DATE IN FY 84	NUMBER OF ROUTINE ACTIONS	NUMBERS OF HEARINGS	TOTAL ACTIONS
MICHIGAN STATE	0	0	0	0	1	0	1
GRAND TOTAL	0	0	0	0	1	0	1

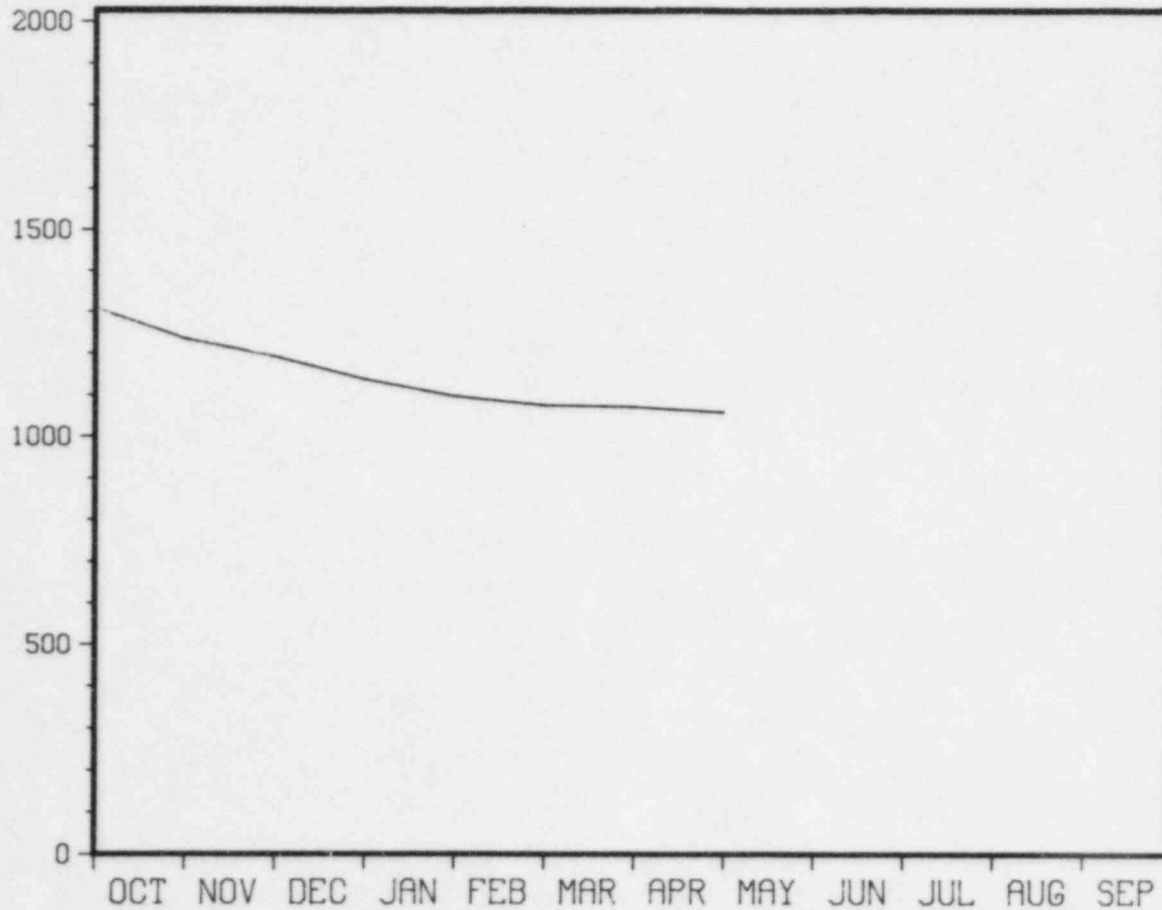
SECTION IV

NUREG-0737 LICENSING ACTIONS

TOTAL ACTIVE NUREG-0737 ACTIONS

U-1

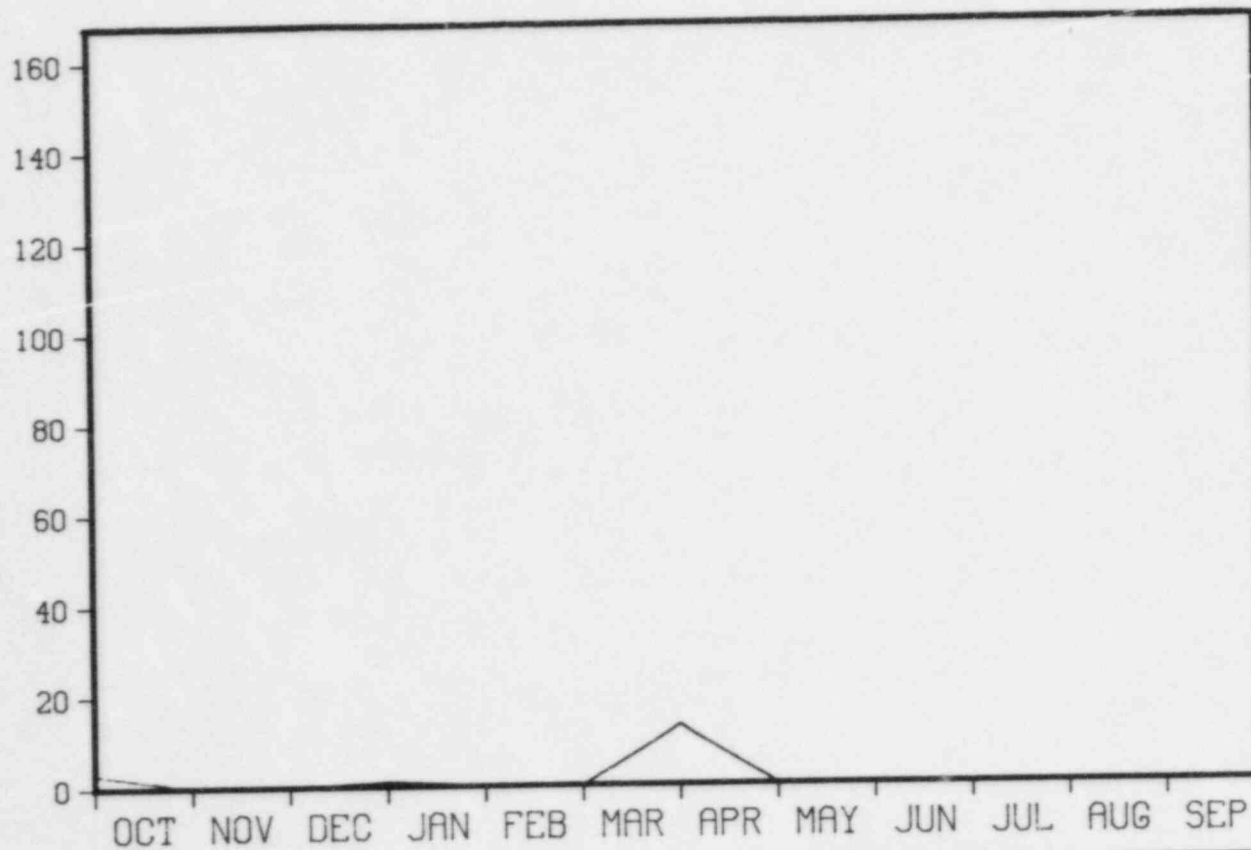
DIVISION OF LICENSING



TOTAL	1234	1191	1137	1096	1074	1070	1057					
ACTIONS	1234	1191	1137	1096	1074	1070	1057					
HEARINGS	00	00	00	00	00	00	00					

NEW NUREG-0737 ACTIONS BY MONTH

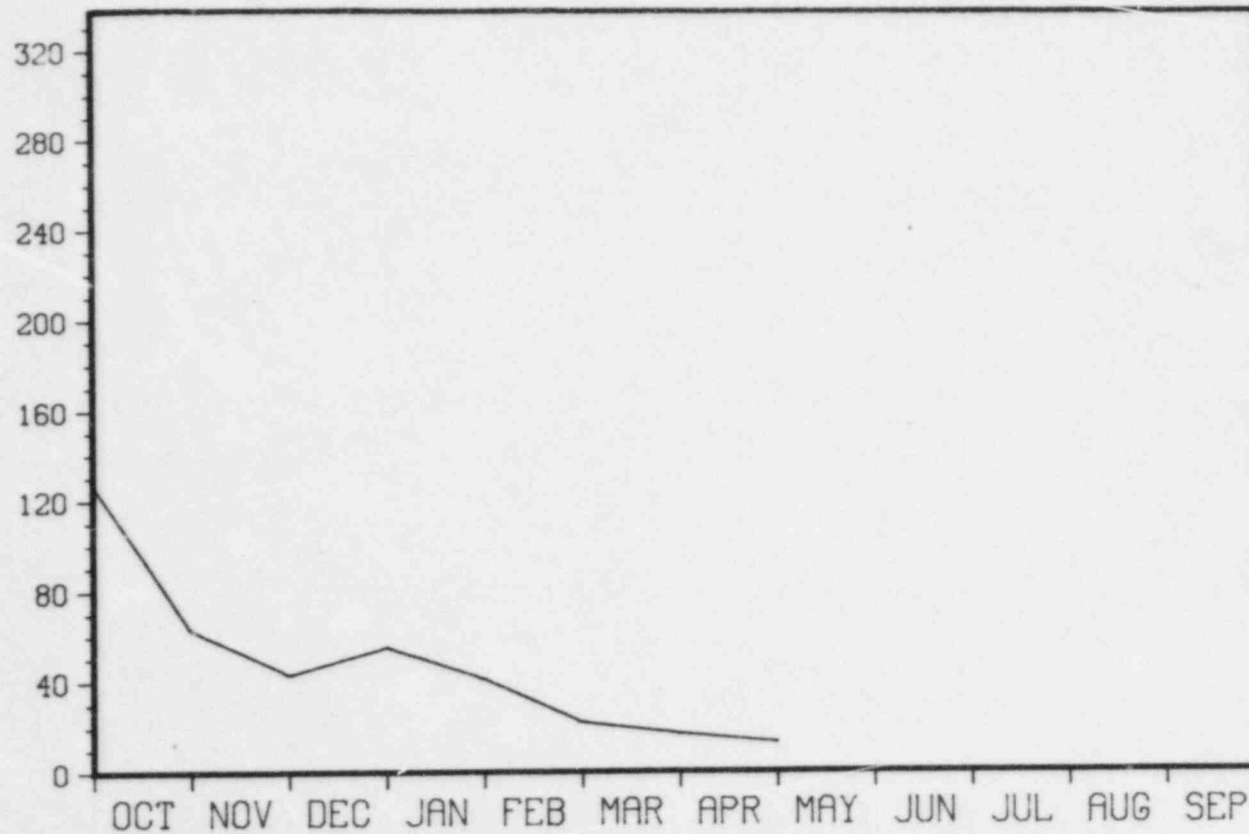
DIVISION OF LICENSING



NEW ACTIONS	00	00	01	00	00	13	00					
CUMULATIVE	0	0	1	1	1	14	14					

COMPLETED NUREG-0737 ACTIONS BY MONTH

DIVISION OF LICENSING



COMPLETED	63	43	55	41	22	17	13					
CUMULATIVE	63	106	161	202	224	241	254					

FISCAL YEAR: 1984

NUREG - 0737 LICENSING ACTIONS BY MONTH

REPORT DATE: 05/21/84

<u>FACILITY NAME</u>	<u>OCT.</u>	<u>NOV.</u>	<u>DEC.</u>	<u>JAN.</u>	<u>FEB.</u>	<u>MAR.</u>	<u>APR.</u>	<u>MAY</u>	<u>JUNE</u>	<u>JULY</u>	<u>AUG.</u>	<u>SEPT.</u>
ARKANSAS 1	14	14	14	14	14	14	14					
ARKANSAS 2	15	14	14	14	14	14	14					
BEAVER VALLEY 1	15	15	14	14	14	14	14					
BIG ROCK POINT 1	19	19	16	16	16	15	13					
BROWNS FERRY 1	18	18	16	16	15	15	15					

BROWNS FERRY 2	18	18	16	16	15	15	15					
BROWNS FERRY 3	18	18	16	16	15	15	15					
BRUNSWICK 1	18	18	17	16	16	16	16					
BRUNSWICK 2	18	18	17	16	16	16	16					
CALVERT CLIFFS 1	16	16	16	16	16	16	16					

CALVERT CLIFFS 2	16	16	16	16	16	16	16					
COOK 1	15	15	14	14	14	14	14					
COOK 2	15	15	14	14	14	14	14					
COOPER STATION	14	14	11	11	11	11	11					
CRYSTAL RIVER 3	18	17	17	17	17	17	16					

DAVIS-BESSE 1	18	18	18	18	17	17	17					
DIABLO CANYON 1	5	5	5	5	5	5	5					
DRESDEN 2	19	19	18	15	15	15	15					
DRESDEN 3	18	18	17	14	14	14	14					
DUANE ARNOLD	20	20	18	17	17	17	16					

FARLEY 1	15	14	13	12	12	12	12					
FARLEY 2	15	14	13	12	12	12	12					
FITZPATRICK	18	18	16	16	16	16	16					
FORT CALHOUN 1	13	13	13	13	13	13	13					
FORT ST VRAIN	11	11	11	11	11	11	11					

GINNA	19	19	18	18	18	18	17					
HADDAM NECK	20	20	19	19	19	19	19					
HATCH 1	17	17	16	13	13	13	13					
HATCH 2	17	17	16	13	13	13	13					
INDIAN POINT 2	17	17	17	16	16	15	15					

INDIAN POINT 3	18	18	18	17	17	17	17					
KEWAUNEE	16	16	16	15	15	15	14					
LA CROSSE	17	17	17	17	15	14	14					
LASALLE 1	3	3	3	3	3	4	4					
LASALLE 2						1	1					

MAINE YANKEE	15	15	15	15	15	15	15					
MCGUIRE 1	4	4	4	4	4	4	4					
MCGUIRE 2	3	3	3	3	3	4	4					
MILLSTONE 1	17	17	15	15	15	15	15					
MILLSTONE 2	16	16	16	16	15	15	15					

FISCAL YEAR: 1984

NUREG - 0737 LICENSING ACTIONS BY MONTH

REPORT DATE: 05/21/84

<u>FACILITY NAME</u>	<u>OCT.</u>	<u>NOV.</u>	<u>DEC.</u>	<u>JAN.</u>	<u>FEB.</u>	<u>MAR.</u>	<u>APR.</u>	<u>MAY</u>	<u>JUNE</u>	<u>JULY</u>	<u>AUG.</u>	<u>SEPT.</u>
MONTICELLO	18	17	15	15	15	13	13					
NINE MILE POINT 1	18	18	17	15	15	15	14					
NORTH ANNA 1	17	16	16	15	14	13	13					
NORTH ANNA 2	18	17	16	15	14	13	13					
OCONEE 1	20	14	14	13	13	13	13					
OCONEE 2	20	14	14	13	13	13	13					
OCONEE 3	20	14	14	13	13	13	13					
OYSTER CREEK 1	20	19	17	17	16	16	16					
PALISADES	18	17	17	17	17	17	16					
PEACH BOTTOM 2	17	16	16	16	13	13	13					
PEACH BOTTOM 3	17	16	16	16	13	13	13					
PILGRIM 1	21	21	19	19	19	16	16					
POINT BEACH 1	16	14	14	13	13	13	13					
POINT BEACH 2	16	14	14	13	13	13	13					
PRAIRIE ISLAND 1	15	14	13	13	13	13	13					
PRAIRIE ISLAND 2	15	14	13	13	13	13	13					
QUAD CITIES 1	16	15	13	13	13	13	13					
QUAD CITIES 2	16	15	13	13	13	13	13					
RANCHO SECO 1	14	14	14	14	14	14	14					
ROBINSON 2	19	19	19	19	19	19	19					
SALEM 1	16	15	15	14	14	14	14					
SALEM 2	15	14	14	13	13	13	13					
SAN ONOFRE 1	20	20	19	19	17	17	14					
SAN ONOFRE 2	7	7	7	7	7	9	9					
SAN ONOFRE 3	7	7	7	7	7	9	9					
SEQUOYAH 1	16	16	12	12	12	12	12					
SEQUOYAH 2	17	17	15	15	15	15	15					
ST LUCIE 1	13	13	13	13	13	13	13					
ST LUCIE 2	4	4	4	4	4	4	4					
SUMMER 1	6	6	6	6	5	5	5					
SURRY 1	15	15	15	14	13	13	13					
SURRY 2	15	15	15	14	13	13	13					
SUSQUEHANNA 1	3	3	4	4	4	4	4					
SUSQUEHANNA 2						1	1					
THREE MILE ISLAND 1	20	20	20	20	20	16	14					
TROJAN	15	15	15	13	13	13	13					
TURKEY POINT 3	15	13	13	12	12	12	12					
TURKEY POINT 4	16	14	14	13	13	13	13					
VERMONT YANKEE 1	17	17	17	15	14	14	14					
WASHINGTON NUCLEAR 2						1	1					

FISCAL YEAR: 1984

NUREG - 0737 LICENSING ACTIONS BY MONTH

REPORT DATE: 05/21/84

<u>FACILITY NAME</u>	<u>OCT.</u>	<u>NOV.</u>	<u>DEC.</u>	<u>JAN.</u>	<u>FEB.</u>	<u>MAR.</u>	<u>APR.</u>	<u>MAY</u>	<u>JUNE</u>	<u>JULY</u>	<u>AUG.</u>	<u>SEPT.</u>
YANKEE-ROWE 1	16	16	13	13	13	13	13					
ZION 1	14	14	14	13	13	13	13					
ZION 2	13	13	13	12	12	12	12					
GRAND TOTAL	1,234	1,191	1,137	1,096	1,074	1,070	1,057					

REPORT DATE: 05/21/84

NUREG - 0737 LICENSING ACTIONS

POWER REACTORS

FOR 04/01/84 - 04/30/84

FACILITY	NEW ACTIONS FOR THIS REPORTING PERIOD	NEW ACTIONS TO DATE IN FY 84	COMPLETED ACTIONS FOR THIS REPORTING PERIOD	COMPLETED ACTIONS TO DATE IN FY 84	NUMBER OF ROUTINE ACTIONS	NUMBERS OF HEARINGS	TOTAL ACTIONS
ARKANSAS 1	0	0	0	1	14	0	14
ARKANSAS 2	0	0	0	2	14	0	14
BEAVER VALLEY 1	0	0	0	1	14	0	14
BIG ROCK POINT 1	0	0	2	6	13	0	13
BROWNS FERRY 1	0	0	0	5	15	0	15
BROWNS FERRY 2	0	0	0	5	15	0	15
BROWNS FERRY 3	0	0	0	5	15	0	15
BRUNSWICK 1	0	0	0	5	16	0	16
BRUNSWICK 2	0	0	0	5	16	0	16
CALVERT CLIFFS 1	0	0	0	3	16	0	16
CALVERT CLIFFS 2	0	0	0	3	16	0	16
COOK 1	0	0	0	1	14	0	14
COOK 2	0	0	0	1	14	0	14
COOPER STATION	0	0	0	3	11	0	11
CRYSTAL RIVER 3	0	0	1	3	16	0	16
DAVIS-BESSE 1	0	0	0	2	17	0	17
DIABLO CANYON 1	0	0	0	0	5	0	5
DRESDEN 2	0	0	0	5	15	0	15
DRESDEN 3	0	0	0	5	14	0	14
DUANE ARNOLD	0	0	1	4	16	0	16
FARLEY 1	0	0	0	3	12	0	12
FARLEY 2	0	0	0	3	12	0	12
FITZPATRICK	0	0	0	2	16	0	16
FORT CALHOUN 1	0	0	0	1	13	0	13
FORT ST VRAIN	0	0	0	1	11	0	11
GINNA	0	0	1	2	17	0	17
GRAND GULF 1	0	1	0	0	6	0	6
HADDAM NECK	0	0	0	1	19	0	19
HATCH 1	0	0	0	4	13	0	13
HATCH 2	0	0	0	4	13	0	13
INDIAN POINT 2	0	0	0	4	15	0	15
INDIAN POINT 3	0	0	0	2	17	0	17
KEWAUNEE	0	0	1	2	14	0	14
LA CROSSE	0	0	0	3	14	0	14
LASALLE 1	0	1	0	0	4	0	4

REPORT DATE: 05/21/84

NUREG - 0737 LICENSING ACTIONS

POWER REACTORS

FOR 04/01/84 - 04/30/84

FACILITY	NEW ACTIONS FOR THIS REPORTING PERIOD	NEW ACTIONS TO DATE IN FY 84	COMPLETED ACTIONS FOR THIS REPORTING PERIOD	COMPLETED ACTIONS TO DATE IN FY 84	NUMBER OF ROUTINE ACTIONS	NUMBERS OF HEARINGS	TOTAL ACTIONS
LASALLE 2	0	1	0	0	1	0	1
MAINE YANKEE	0	0	0	3	15	0	15
MCGUIRE 1	0	0	0	0	4	0	4
MCGUIRE 2	0	1	0	0	4	0	4
MILLSTONE 1	0	0	0	2	15	0	15
MILLSTONE 2	0	0	0	2	15	0	15
MONTICELLO	0	0	0	5	13	0	13
NINE MILE POINT 1	0	0	1	4	14	0	14
NORTH ANNA 1	0	0	0	7	13	0	13
NORTH ANNA 2	0	0	0	8	13	0	13
OCONEE 1	0	0	0	8	13	0	13
OCONEE 2	0	0	0	8	13	0	13
OCONEE 3	0	0	0	8	13	0	13
OYSTER CREEK 1	0	0	0	4	16	0	16
PALISADES	0	0	1	5	16	0	16
PEACH BOTTOM 2	0	0	0	4	13	0	13
PEACH BOTTOM 3	0	0	0	4	13	0	13
PILGRIM 1	0	0	0	5	16	0	16
POINT BEACH 1	0	0	0	3	13	0	13
POINT BEACH 2	0	0	0	3	13	0	13
PRAIRIE ISLAND 1	0	0	0	3	13	0	13
PRAIRIE ISLAND 2	0	0	0	3	13	0	13
QUAD CITIES 1	0	0	0	3	13	0	13
QUAD CITIES 2	0	0	0	3	13	0	13
RANCHO SECO 1	0	0	0	3	14	0	14
ROBINSON 2	0	0	0	0	19	0	19
SALEM 1	0	0	0	4	14	0	14
SALEM 2	0	0	0	4	13	0	13
SAN ONOFRE 1	0	0	3	6	14	0	14
SAN ONOFRE 2	0	3	0	1	9	0	9
SAN ONOFRE 3	0	3	0	1	9	0	9
SEQUOYAH 1	0	0	0	4	12	0	12
SEQUOYAH 2	0	0	0	2	15	0	15
ST LUCIE 1	0	0	0	3	13	0	13
ST LUCIE 2	0	0	0	0	4	0	4

REPORT DATE: 05/21/84

NUREG - 0737 LICENSING ACTIONS

POWER REACTORS

FOR 04/01/84 - 04/30/84

FACILITY	NEW ACTIONS FOR THIS REPORTING PERIOD	NEW ACTIONS TO DATE IN FY 84	COMPLETED ACTIONS FOR THIS REPORTING PERIOD	COMPLETED ACTIONS TO DATE IN FY 84	NUMBER OF ROUTINE ACTIONS	NUMBERS OF HEARINGS	TOTAL ACTIONS
SUMMER 1	0	0	0	1	5	0	5
SURRY 1	0	0	0	3	13	0	13
SURRY 2	0	0	0	3	13	0	13
SUSQUEHANNA 1	0	2	0	1	4	0	4
SUSQUEHANNA 2	0	1	0	0	1	0	1
THREE MILE ISLAND 1	0	0	2	7	14	0	14
TROJAN	0	0	0	2	13	0	13
TURKEY POINT 3	0	0	0	3	12	0	12
TURKEY POINT 4	0	0	0	3	13	0	13
VERMONT YANKEE 1	0	0	0	3	14	0	14
WASHINGTON NUCLEAR 2	0	1	0	0	1	0	1
YANKEE-ROWE 1	0	0	0	4	13	0	13
ZION 1	0	0	0	2	13	0	13
ZION 2	0	0	0	2	12	0	12
GRAND TOTAL	0	14	13	254	1,057	0	1,057

STATUS OF NUREG-0737 REQUIREMENTS SCHEDULE

Z-1

	DEVELOPMT COMPLTD	ISSUED TO OPERATG REACTORS	PRE/POST IMPLMNTN	TECHNL REVIEW ORGN TZN	ORDERS ISSUED	SUBMTLS RECEIVED	TECHNICL REVIEW COMPLETE	SER ISSUED	TECHNL SPECS ISSUED	LICENSEE SCHEDULE CODE	NRC COMPLTN SCHEDULE	COMPLETE
I.A.1.1.1 STA ON DUTY	Y	Y	POST	DL	65(65)	65(65)	65(65)	65(65)	65(65)	A	10/81	65(65)
I.A.1.1.3 STA TRAINED PER LL CAT B (F-01)	Y	Y	POST	LQB	65(65)	65(65)	65(65)	65(65)	65(65)	B	06/82	65(65)
I.A.1.1.4 STA LONG TERM PROGRAM	Y	Y	POST	LQB	65(65)	65(65)	65(65)	NA	NA	B	06/82	65(65)
I.A.1.2 SHIFT SUPERVSR DUTIES	Y	Y	POST	DL	65(65)	65(65)	65(65)	65(65)	NA	A	05/80	65(65)
I.A.1.3.1 LIMIT OVERTIME (F-01)	Y	Y	POST	IE	65(65)	65(65)	65(65)	65(65)	NA	B	12/81	65(65)
I.A.1.3.2 MIN SHIFT CREW (F-02)	Y	Y	POST	LQB	NE	65(65)	65(65)	65(65)	65(65)	D	09/84	65(65)
I.A.2.1.1 SRO EXPERIENCE	Y	Y	POST	OLB	NE	NA	65(65)	NA	NA	B	03/80	65(65)
I.A.2.1.2 SRO BE RD 1 YR	Y	Y	POST	OLB	NE	NA	65(65)	NA	NA	B	03/80	65(65)
I.A.2.1.3 3 MO TRAINING ON SHIFT	Y	Y	POST	OLB	NE	NA	65(65)	NA	NA	B	03/80	65(65)
I.A.2.1.4 MODIFY TN PROGRAM (F-03)	Y	Y	POST	OLB	65(65)	65(65)	65(65)	65(65)	NA	D	09/83	65(65)
I.A.2.1.5 FACILITY CERTI	Y	Y	POST	OLB	NE	NA	65(65)	NA	NA	C	03/80	65(65)
I.A.2.3 INSTR COMPLETE SRO TRAINING	Y	Y	POST	OLB	NE	NA	65(65)	NA	NA	B	03/80	65(65)

LICENSEE SCHEDULE CODE KEYS: A- STILL
 B- THROUGH 6/30/81 EXCLUDING STILL
 C- 6/30/81 TO 03/01/82
 D- AFTER 03/01/82
 E- SUPPLEMENT 1

OTHER KEYS: XX(XX)- NO. COMPLETE(TOTAL NO. TO BE COMPLETED)
 Y- COMPLETE
 NA- NOT APPLICABLE
 NE- NO ENTRY
 TBD- TO BE DETERMINED

STATUS OF NUREG-0737 REQUIREMENTS SCHEDULE

Z-2

	DEVELOPMT COMPLTD	ISSUED TO OPERATG REACTORS	PRE/POST IMPLMNTN	TECHNL REVIEW ORGNZTN	ORDERS ISSUED	SUBMITLS RECEIVED	TECHNICL REVIEW COMPLETE	SER ISSUED	TECHNL SPECS ISSUED	LICENSEE SCHEDULE CODE	NRC COMPLTN SCHEDULE	COMPLETE
I.A.3.1.1 SCOPE OF LICENSING EXAM	Y	Y	OLB	NA	NE	NA	65(65)	NA	NA	B	03/80	65(65)
I.A.3.1.2 PASSING GRADE	Y	Y	NA	OLB	NE	NA	65(65)	NA	NA	B	03/80	65(65)
I.A.3.1.3 SIMULATOR EXAM	Y	Y	NA	OLB	65(65)	65(65)	65(65)	65(65)	NA	C	10/81	65(65)
I.C.1.1 SB LOCA PROCED	Y	Y	POST	RSB	NA	65(65)	65(65)	65(65)	NA	A	05/80	65(65)
I.C.1.2A ICC GUIDELINES (F-04)	Y	Y	PRE	PTB	65(65)	65(65)	65(65)	65(65)	NA	B	12/81	65(65)
I.C.1.2B ICC PROCEDURES (F-04)	Y	Y	PRE	PTB/FRC	0(65)	0(65)	0(65)	0(65)	NA	E	09/84	0(65)
I.C.1.3A TRANSIENT/ACC GUIDELINES (F-05)	Y	Y	PRE	RSB	65(65)	65(65)	65(65)	65(65)	NA	B	09/82	65(65)
I.C.1.3B TRANS/ACCIDENT PROCEDURES (F-05)	Y	Y	PRE	PTB/FRC	0(65)	0(65)	0(65)	0(65)	NA	E	09/84	0(65)
I.C.2 SHIFT/RELIEF TURNOVER PROCD	Y	Y	POST	DL	65(65)	65(65)	65(65)	65(65)	NA	A	05/80	65(65)
I.C.3 SHIFT SUP RESP	Y	Y	POST	DL	65(65)	65(65)	65(65)	65(65)	NA	A	05/80	65(65)
I.C.4 CONT RM ACCESS	Y	Y	POST	DL	65(65)	65(65)	65(65)	65(65)	NA	A	05/80	65(65)
I.C.5 FEEDBACK OF OPERATG EXPERC	Y	Y	POST	IE	65(65)	NA	65(65)	65(65)	NA	B	12/81	65(65)
I.C.6 PERF OF OPERTG ACTIVITIES	Y	Y	POST	IE	65(65)	NA	64(65)	64(65)	NA	C	09/83	64(65)

STATUS OF NUREG-0737 REQUIREMENTS SCHEDULE

Z-3

	DEVELOPMT COMPLTD	ISSUED TO OPERATG REACTORS	PRE/POST IMPLMNTN	TECHNL REVIEW ORGNTZN	ORDERS ISSUED	SUBMITTLS RECEIVED	TECHNICAL REVIEW COMPLETE	SER ISSUED	TECHNL SPECS ISSUED	LICENSEE SCHEDULE CODE	NRC COMPLTN SCHEDULE	COMPLETE
I.D.1 CONTROL ROOM DESIGN REVIEW (F-08)	NE	NE	NE	HFEB/LLL	0(65)	1(65)	0(65)	0(65)	NA	E	TBD	0(65)
I.D.2.1 SPDS DESCRIPTN	NE	NE	NE	HFEB/LLL	0(65)	0(65)	0(65)	0(65)	NA	E	TBD	0(65)
I.D.2.2 SPDS INSTALLED (F-09)	NE	NE	NE	HFEB/LLL	NE	0(65)	0(65)	0(65)	0(65)	E	TBD	0(65)
II.B.1.1 RCS VENT DESGN (F-10)	Y	Y	POST	RSB/LLL	NE	65(65)	65(65)	65(65)	NA	C	09/83	65(65)
II.B.1.2 RCS VENT INSTL (F-10)	Y	Y	PRE	RSB/LLL	NE	65(65)	65(65)	65(65)	18(65)	D	09/84	18(65)
II.B.1.3 RCS VENT PROCD (F-10)	Y	Y	PRE	RSB/LLL	NE	65(65)	65(65)	65(65)	NA	D	09/83	65(65)
II.B.2.1 SHIELDG REVIEW (F-11)	Y	Y	POST	DL/EGG	65(65)	65(65)	65(65)	65(65)	NA	A	05/80	65(65)
II.B.2.2 SHIELDG MODS (F-11)	Y	Y	POST	RAB/EGG	65(65)	65(65)	61(65)	61(65)	NA	C	09/84	61(65)
II.B.2.3 EQUIP QUAL (B-60)	Y	Y	POST	EQB/FRC	NE	65(65)	65(65)	NA	NA	D	09/83	65(65)
II.B.3.1 ACC SAMP INTER SYSTEM	Y	Y	POST	DL	65(65)	65(65)	65(65)	65(65)	NA	A	05/80	65(65)
II.B.3.2 UPGRADE ACC SAMP (F-12)	Y	Y	POST	ETB/INEL	65(65)	65(65)	26(65)	26(65)	1(65)	C	09/84	1(65)

LICENSEE SCHEDULE CODE KEYS: A- STILL
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 C- 6/30/81 TO 03/01/82
 D- AFTER 03/01/82
 E- SUPPLEMENT 1

OTHER KEYS: XX(XX)- NO. COMPLETE(TOTAL NO. TO BE COMPLETED)
 Y- COMPLETE
 NA- NOT APPLICABLE
 NE- NO ENTRY
 TBD- TO BE DETERMINED

STATUS OF NUREG-0737 REQUIREMENTS SCHEDULE

Z-4

	DEVELOPMT COMPLTD	ISSUED TO OPERATG REACTORS	PRE/POST IMPLMNTN	TECHNL REVIEW ORGN TZN	ORDERS ISSUED	SUBMITTLS RECEIVED	TECHNICL REVIEW COMPLETE	SER ISSUED	TECHNL SPECS ISSUED	LICENSEE SCHEDULE CODE	NRC COMPLTN SCHEDULE	COMPLETE
II.B.4 TRG P20G CORE DAM (F-13)	Y	Y	POST	DLB	65(65)	65(65)	65(65)	65(65)	NA	C	09/83	65(65)
II.D.1.1 RV/SV TEST PROGRAM (F-14)	Y	Y	POST	DL/EGG	65(65)	65(65)	65(65)	65(65)	NA	A	05/80	65(65)
II.D.1.2A TEST PROGRAM COMP, BWR'S (F-14)	Y	Y	POST	MEB/EGG	NE	24(24)	24(24)	NA	NA	C	07/82	24(24)
II.D.1.2A* TEST PROGRAM COMP, PWR'S (F-14)	Y	Y	POST	MEB/EGG	NE	40(40)	40(40)	NA	NA	C	09/83	40(40)
II.D.1.2.B* PLANT SPEC RPTS, PWR'S (F-14)	Y	Y	PRE	MEB/EGG	40(40)	40(40)	0(40)	0(40)	NA	D	09/84	0(40)
II.D.1.2B PSPEC RPT, BWR (F-14)	Y	Y	PRE	EQB	24(24)	24(24)	16(24)	16(24)	NA	D	09/84	16(24)
II.D.1.3 BK VALVE TESTING (F-14)	Y	Y	PRE	MEB/EGG	40(40)	40(40)	0(40)	0(40)	NA	D	09/84	0(40)
II.D.3.1 VALVE POS INDIC INSTL	Y	Y	POST	DL	65(65)	65(65)	65(65)	65(65)	65(65)	A	09/81	65(65)
II.D.3.3 VALVE POS ENVIR QUAL (B-60)	Y	Y	POST	EQB/FRC	NE	65(65)	65(65)	NA	NA	D	09/83	65(65)
II.E.1.1 AFS EVAL (F-15)	Y	Y	PRE	ASB	NE	40(40)	36(40)	36(40)	34(40)	C	09/83	34(40)
II.E.1.2.1A AFS INIT CONT GRADE (F-16)	Y	Y	PRE	DL	40(40)	40(40)	40(40)	40(40)	40(40)	A	08/81	40(40)

STATUS OF NUREG-0737 REQUIREMENTS SCHEDULE

	DEVELOPMT COMPLTD	ISSUED TO OPERATG REACTORS	PRE/POST IMPLMNTN	TECHNL REVIEW ORGNTZN	ORDERS ISSUED	SUBMITLS RECEIVED	TECHNICAL REVIEW COMPLETE	SER ISSUED	TECHNL SPECS ISSUED	LICENSEE SCHEDULE CODE	NRC COMPLTN SCHEDULE	COMPLETE
II.E.1.2.1.B AFS INIT SAFE GRADE (F-16)	Y	Y	POST	ICSB	40(40)	40(40)	38(40)	38(40)	18(40)	C	09/83	18(40)
II.E.1.2.2.A AFS FLOW CONT GRADE (F-17)	Y	Y	POST	DL	40(40)	40(40)	40(40)	40(40)	40(40)	A	08/81	40(40)
II.E.1.2.2.B AFS FLOW SAFE GRADE (F-17)	Y	Y	POST	ICSB	40(40)	40(40)	38(40)	38(40)	20(40)	C	09/83	20(40)
II.E.3.1 EMERG PWR FOR HEATERS	Y	Y	POST	DL	40(40)	40(40)	40(40)	40(40)	40(40)	A	09/81	40(40)
II.E.4.1.1 DED H2 PENET DESIGN (F-18)	Y	Y	PRE	DL	65(65)	65(65)	65(65)	65(65)	NA	A	05/80	65(65)
II.E.4.1.2 DED H2 PENET INSTALL (F-18)	Y	Y	POST	CSB	NE	65(65)	65(65)	65(65)	NA	C	12/81	65(65)
II.E.4.2.1-4 DIVERSE ISO SIG (F-19)	Y	Y	POST	DL	65(65)	65(65)	65(65)	65(65)	65(65)	A	09/81	65(65)
II.E.4.2.5A CONT PRESS SETPT (F-19)	Y	Y	PRE	CSB/LLL	65(65)	65(65)	65(65)	65(65)	NA	B	09/81	65(65)
II.E.4.2.5B CONT PRESS MODIFY (F-19)	Y	Y	POST	CSB/LLL	10(10)	10(10)	10(10)	10(10)	10(10)	C	09/83	10(10)

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 NE- NO ENTRY
 TBD- TO BE DETERMINED

STATUS OF NUREG-0737 REQUIREMENTS SCHEDULE

Z-6

	DEVELOPMT COMPLTD	ISSUED TO OPERATG REACTORS	PRE/POST IMPLMNTN	TECHNL REVIEW ORGNZTN	ORDERS ISSUED	SUBMITLS RECEIVED	TECHNICL REVIEW COMPLETE	SER ISSUED	TECHNL SPECS ISSUED	LICENSEE SCHEDULE CODE	NRC COMPLTN SCHEDULE	COMPLETE
II.E.4.2.6 CONT PURGE VALVES (F-19)	Y	Y	POST	CSB/LLL	65(65)	65(65)	57(65)	57(65)	NA	C	09/83	56(65)
II.E.4.2.7 RAD SIG ON PURGE VAL (F-19)	Y	Y	POST	CSB/LLL	65(65)	65(65)	63(65)	56(65)	7(65)	C	09/83	7(65)
II.F.1.1 NOBLE GAS MONITOR (F-20)	Y	Y	POST	ETSB/ INEL	65(65)	65(65)	65(65)	65(65)	2(65)	C	09/84	2(65)
II.F.1.2 IODINE/PART MONITOR (F-21)	Y	Y	POST	ETSB/ INEL	65(65)	65(65)	65(65)	65(65)	3(65)	C	09/84	3(65)
II.F.1.3 CONT HI-RANGE MONITOR (F-22)	Y	Y	POST	ETSB/ INEL	65(65)	65(65)	65(65)	65(65)	2(65)	C	09/83	2(65)
II.F.1.4 CONT PRESS MONITOR (F-23)	Y	Y	POST	CSB/LLL	65(65)	65(65)	60(65)	60(65)	2(65)	C	09/83	2(65)
II.F.1.5 CONT H2O LEVEL (F-24)	Y	Y	POST	CSB/LLL	65(65)	65(65)	61(65)	61(65)	1(65)	C	09/83	1(65)
II.F.1.6 CONT HYDROGEN (F-25)	Y	Y	POST	CSB/ LLL	65(65)	65(65)	59(65)	59(65)	1(65)	C	09/83	1(65)
II.F.2.1 SUB COOL METER	Y	Y	POST	DL	40(40)	40(40)	40(40)	40(40)	40(40)	A	10/81	40(40)
II.F.2.3 INSTRUMENTATN (F-26)	Y	Y	POST	CPB	07(65)	65(65)	1(65)	1(65)	1(65)	D	09/84	1(65)

STATUS OF NUREG-0737 REQUIREMENTS SCHEDULE

	DEVELOPMT COMPLTD	ISSUED TO OPERATG REACTORS	PRE/POST IMPLMNTN	TECHNL REVIEW ORGNZTN	ORDERS ISSUED	SUBMTTLS RECEIVED	TECHNICL REVIEW COMPLETE	SER ISSUED	TECHNL SPECS ISSUED	LICENSEE SCHEDULE CODE	NRC COMPLTN SCHEDULE	COMPLETE
II.G.1 UPGRADE POWER PORV, LVL INST	Y	Y	POST	DL	40(40)	40(40)	40(40)	40(40)	40(40)	A	09/81	40(40)
II.K.1 IE BULLETINS	Y	Y	POST	DL	NA	65(65)	65(65)	65(65)	NA	B	06/81	65(65)
II.K.2.8 UPGRADE AFW	Y	Y	PRE	ASB	07(07)	7(07)	7(07)	7(07)	7(07)	B	08/81	7(07)
II.K.2.9 FEMA ON ICS (F-27)	Y	Y	POST	ICSB	07(07)	7(07)	7(07)	7(07)	NA	B	06/82	7(07)
II.K.2.10 SAFE GRADE TRIP (F-28)	Y	Y	POST	ICSB	07(07)	7(07)	7(07)	7(07)	7(07)	C	06/82	7(07)
II.K.2.11 OPER TNG & DRILLING (F-29)	Y	Y	POST	DLB	07(07)	7(07)	7(07)	7(07)	NA	B	11/81	7(07)
II.K.2.13A THERM MECH RPT B&W (F-30)	Y	Y	POST	RSB/LLL	07(07)	7(07)	0(07)	0(07)	0(07)	C	09/84	0(07)
II.K.2.13B THERM MECH RPT CE&W (F-30)	Y	Y	POST	RSB/LLL	NE	33(33)	0(33)	0(33)	0(33)	C	09/84	0(33)
II.K.2.14 LIFT FREQ PORV &SV (F-31)	Y	Y	POST	RRAB	07(07)	7(07)	7(07)	7(07)	NA	B	06/82	7(07)
II.K.2.15 SLUG FLOW	Y	Y	POST	RSB	07(07)	7(07)	7(07)	7(07)	NA	B	01/81	7(07)
II.K.2.16 RCP SEAL DAMAGE (F-32)	Y	Y	POST	RSB/EGG	07(07)	7(07)	7(07)	7(07)	NA	B	12/81	7(07)

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STATUS OF NUREG-0737 REQUIREMENTS SCHEDULE

Z-8

	DEVELOPMT COMPLTD	ISSUED TO OPERATG REACTORS	PRE/POST IMPLMNTN	TECHNL REVIEW ORGTZN	ORDERS ISSUED	SUBMTLS RECEIVED	TECHNICL REVIEW COMPLETE	SER ISSUED	TECHNL SPECS ISSUED	LICENSEE SCHEDULE CODE	NRC COMPLTN SCHEDULE	COMPLETE
II.K.2.17 VOIDING IN RCS (F-33)	Y	Y	POST	RSB	07(07)	40(40)	40(40)	40(40)	NA	C	09/84	40(40)
II.K.2.19A SEQ AFS FLOW, B&W (F-34)	Y	Y	POST	RSB	07(07)	7(07)	7(07)	7(07)	NA	C	02/81	7(07)
II.K.2.19B SEQ AFS FLOW, CE&W (F-34)	Y	Y	POST	RSB	NE	33(33)	33(33)	33(33)	NA	C	07/81	33(33)
II.K.2.20 SYS RESP TO SB LOCA (F-35)	Y	Y	POST	RSB	07(07)	7(07)	7(07)	7(07)	NA	B	TBD	7(07)
II.K.3.1 PORV ISOL (F-36)	Y	Y	POST	ICSB	NE	40(40)	40(40)	40(40)	NA	C	09/84	40(40)
II.K.3.2 RPT PORV FAILURES (F-37)	Y	Y	POST	RRAB/FRC	40(40)	40(40)	40(40)	40(40)	NA	C	09/84	40(40)
II.K.3.3 RPT SV & RV FAILURES (F-38)	Y	Y	POST	RRAB	65(65)	65(65)	65(65)	65(65)	18(65)	C	09/83	18(65)
II.K.3.5A RCP AUTO TRIP DESIGN (F-39)	Y	Y	POST	RSB	NE	40(40)	40(40)	40(40)	NA	C	09/82	40(40)
II.K.3.5B RCP AUTO TRIP MODIFY (F-39)	Y	Y	PRE	RSB	NE	40(40)	40(40)	40(40)	NA	C	09/82	40(40)
II.K.3.9 PID CONTROLLER (F-40)	Y	Y	POST	IE/DL	25(25)	25(25)	25(25)	25(25)	NA	B	09/81	25(25)

STATUS OF NUREG-0737 REQUIREMENTS SCHEDULE

	DEVELOPMT COMPLTD	ISSUED TO OPERATG REACTORS	PRE/POST IMPLMNTN	TECHNL REVIEW ORGNZTN	ORDERS ISSUED	SUBMITTL RCEIVED	TECHNICL REVIEW COMPLETE	SER ISSUED	TECHNL SPECS ISSUED	LICENSEE SCHEDULE CODE	NRC COMPLTN SCHEDULE	COMPLETE
II.K.3.10 ANTICIP TRIP (F-41)	Y	Y	PRE	ICSB	NE	25(25)	25(25)	25(25)	NA	B	06/82	25(25)
II.K.3.11 USE OF CERTAIN PORV	Y	Y	POST	RSB	NA	NA	65(65)	NA	NA	B	09/81	65(65)
II.K.3.12 ANTICIP TRIP ON IT (F-42)	Y	Y	POST	ICSB	25(25)	25(25)	25(25)	25(25)	NA	C	09/81	25(25)
II.K.3.13A HPCI/RCIC INI LEV DESIGN (F-43)	Y	Y	POST	RSB	17(17)	17	17(17)	17(17)	NA	B	10/81	17(17)
II.K.3.13B HPCI/RCIC MODS (F-43)	Y	Y	POST	ICSB	NE	17(17)	17(17)	17(17)	5(17)	C	09/83	5(17)
II.K.3.14 ISO COND MODS (F-44)	Y	Y	POST	CSB	NE	7(07)	7(07)	7(07)	NA	C	12/81	7(07)
II.K.3.15 ISO OF HPCI/RCIC (F-45)	Y	Y	POST	RSB	19(19)	19(19)	19(19)	19(19)	12(19)	C	09/83	12(19)
II.K.3.16A RV FAIL REPORT (F-46)	Y	Y	POST	RRAB FRC	24(24)	24(24)	24(24)	24(24)	NA	B	09/83	24(24)
II.K.3.16B RV FAIL MODS (F-46)	Y	Y	PRE	RSB/FRC	NE	24(24)	0(24)	0(24)	0(24)	D	09/83	0(24)

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STATUS OF NUREG-0737 REQUIREMENTS SCHEDULE

Z-10

	DEVELOPMT COMPLTD	ISSUED TO OPERATG REACTORS	PRE/POST IMPLMNTN	TECHNL REVIEW ORGN TZN	ORDERS ISSUED	SUBMTLS RECEIVED	TECHNICL REVIEW COMPLETE	SER ISSUED	TECHNL SPECS ISSUED	LICENSEE SCHEDULE CODE	NRC COMPLTN SCHEDULE	COMPLETE
II.K.3.17 ECCS OUTAGE REPORT (F-47)	Y	Y	POST	RRAB/FRC	65(65)	65(65)	65(65)	65(65)	NA	C	09/83	65(65)
II.K.3.18A ADS ACT STUDY (F-48)	Y	Y	POST	RSB	24(24)	24(24)	24(24)	24(24)	NA	B	04/82	24(24)
II.K.3.18B ADS ACT DESIGN (F-48)	Y	Y	POST	ICSB/RSB	24(24)	24(24)	24(24)	24(24)	2(24)	D	09/84	2(24)
II.K.3.18C ACS ACT MODS (F-48)	Y	Y	PRE	ICSB/RSB	NE	24(24)	2(24)	2(24)	2(24)	D	09/84	2(24)
II.K.3.19 INTERLOCK RECIRC (F-49)	Y	Y	POST	RSB	3(03)	3(03)	2(03)	2(03)	2(03)	C	09/83	2(03)
II.K.3.20 LOSS SVC WATER AT BRP	Y	Y	POST	ASB	NA	1(01)	1(01)	1(01)	NA	C	06/81	1(01)
II.K.3.21A RESTART LPCI/ CSS DESIGN (F-50)	Y	Y	POST	RSB/EGG	24(24)	24(24)	24(24)	24(24)	NA	B	09/82	24(24)
II.K.3.21B RESTART LPCI/ CSS MODS (F-50)	Y	Y	NE	ICSB/EGG	NE	24(24)	24(24)	24(24)	NA	C	09/83	24(24)
II.K.3.22A RCIC SUCTION PROCEDURES (F-51)	Y	Y	POST	IE	17(17)	NA	17(17)	17(17)	NA	B	08/81	17(17)
II.K.3.22B RCIC SUCTION MODS (F-51)	Y	Y	POST	ICSB	17(17)	17(17)	13(17)	13(17)	3(17)	C	09/83	3(17)
II.K.3.24 SPACE COOLING FOR RCIC/HPCI (F-52)	Y	Y	POST	RSB	19(19)	19(19)	19(19)	19(19)	NA	C	09/83	19(19)

STATUS OF NUREG-0737 REQUIREMENTS SCHEDULE

Z-11

	DEVELOPMT COMPLTD	ISSUED TO OPERATG REACTORS	PRE/POST IMPLMNTN	TECHNL REVIEW ORGN TZN	ORDERS ISSUED	SUBMITLS RECEIVED	TECHNICL REVIEW COMPLETE	SER ISSUED	TECHNL SPECS ISSUED	LICENSEE SCHEDULE CODE	NRC COMPLTN SCHEDULE	COMPLETE
II.K.3.25A PUMP SEAL STUDY (BWR) (F-53)	Y	Y	POST	RSB	NE	24(24)	24(24)	24(24)	NA	C	12/81	24(24)
II.K.3.25A PUMP SEAL STUDY (CE&W) (F-53)	Y	Y	POST	RSB	NE	33(33)	33(33)	33(33)	NA	C	06/82	33(33)
II.K.3.25B PUMP SEAL MODS (BWR) (F-53)	Y	Y	PRE	IE	NE	24(24)	24(24)	24(24)	NA	D	09/83	24(24)
II.K.3.25B PUMP SEAL MODS (CE&W) (F-53)	Y	Y	PRE	IE	NE	33(33)	28(33)	28(33)	NA	D	09/83	27(33)
II.K.3.27 COMMON REF. LEVEL (F-54)	Y	Y	POST	ORB2		24(24)	24(24)	24(24)	24(24)	C	01/82	24(24)
II.K.3.28 QUAL OF ADS ACC (F-55)	Y	Y	POST	EQB	NE	21(21)	5(21)	5(21)	0(21)	C	09/83	0(21)
II.K.3.29 PERF OF ISO COND (F-56)	Y	Y	POST	RSB		07(07)	7(07)	7(07)	NA	B	10/81	7(07)
II.K.3.30A SB LOCA SCHEDULE (F-57)	Y	Y	POST	RSB		65(65)	65(65)	65(65)	NA	B	04/81	65(65)
II.K.3.30B SB LOCA METHOD (F-57)	Y	Y	PRE	RSB/EGG		0(65)	65(65)	24(65)	24(65)	NA	09/84	24(65)

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STATUS OF NUREG-0737 REQUIREMENTS SCHEDULE

Z-12

	DEVELOPMT COMPLTD	ISSUED TO OPERATG REACTORS	PRE/POST IMPLMNTN	TECHNL REVIEW ORGNTZN	ORDERS ISSUED	SUBMTLS RECEIVED	TECHNICL REVIEW COMPLETE	SER ISSUED	TECHNL SPECS ISSUED	LICENSEE SCHEDULE CODE	NRC COMPLTN SCHEDULE	COMPLETE
II.K.3.31 3B LOCA PUA (F-58)	Y	Y	PRE	RSB	0(65)	22(65)	22(65)	22(65)	22(65)	D	09/84	22(65)
II.K.3.44 TRANS W/SINGLE FAILURE (F-59)	Y	Y	POST	RSB	24(24)	24(24)	22(24)	22(24)	NA	C	09/83	22(24)
II.K.3.45 MANUAL DEPRESS (F-60)	Y	Y	POST	RSB/EGG	24(24)	24(24)	24(24)	24(24)	NA	C	09/83	24(24)
II.K.3.46 MICHELSON CONCERNS (F-61)	Y	Y	POST	RSB	NE	24(24)	24(24)	24(24)	NA	B	08/81	24(24)
II.K.3.57 MAN ACT OF ADS (F-62)	Y	Y	POST	RSB	NE	24(24)	24(24)	NA	NA	B	TBD	24(24)
III.A.1.1 EMERG PLAN SHORT TERM	Y	Y	POST	DL	65(65)	65(65)	65(65)	65(65)	NA	A	05/80	65(65)
III.A.1.2.1 INTERIM FACILITIES	Y	Y	POST	DL	65(65)	65(65)	65(65)	65(65)	NA	A	05/80	65(65)
III.A.1.2.2 DESIGN UPGRD FACIL (F-63,64)	Y	Y	POST	EPLB/PNL	NE	0(65)	0(65)	0(65)	0(65)	E	09/84	0(65)
III.A.2.1 UPGRADE EMER PLAN (F-67)	Y	Y	POST	IE	NE	65(65)	62(65)	62(65)	NA	D	TBD	62(65)
III.A.2.2 METEOROLOGICAL DATA (F-68)	Y	Y	POST	IE		0(65)	0(65)	0(65)	0(65)	E	09/84	0(65)
III.D.1.1 PRIMARY COOLANT LEAK	Y	Y	POST	DL	65(65)	65(65)	65(65)	65(65)	NA	A	09/81	65(65)

STATUS OF NUREG-0737 REQUIREMENTS SCHEDULE

	DEVELOPMT COMPLTD	ISSUED TO OPERATG REACTORS	PRE/POST IMPLMNTN	TECHNL REVIEW ORGNTZN	ORDERS ISSUED	SUBMTLS RECEIVED	TECHNICL REVIEW COMPLETE	SER ISSUED	TECHNL SPECS ISSUED	LICENSEE SCHEDULE CODE	NRC COMPLTN SCHEDULE	COMPLETE
III.D.3.3.1 I DETECT SHORT TERM	Y	Y	POST	DL	65(65)	65(65)	65(65)	65(65)	NA	A	05/80	65(65)
III.D.3.3.2 I DETECT UPGRADE(F-69)	Y	Y	POST	IE/INEL	65(65)	65(65)	65(65)	65(65)	NA	B	03/82	65(65)
III.D.3.4.1 CR HABIT REVIEW (F-70)	Y	Y	POST	AEB/PNL	65(65)	65(65)	65(65)	65(65)	NA	B	09/83	65(65)
III.D.3.4.2 CR HABIT MODS (F-70)	Y	Y	POST	AEB/PNL	65(65)	65(65)	58(65)	58(65)	0(65)	D	09/83	0(65)

L I C E N S I N G A C T I O N T R A C K I N G S Y S T E M
N R R I M P L E M E N T A T I O N R E V I E W
C O M P L E T I O N S T A T U S

LICENSEE SCHEDULE CODE	NUMBER OF ACTIONS	SUBMITTALS RECEIVED		SER ISSUED		TECH SPEC ISSUED		ACTIONS COMPLETED
		# ACTIONS	# COMPLETED	# ACTIONS	# COMPLETED	# ACTIONS	# COMPLETED	
A	1,240	1,240	1,240	1,240	1,240	395	395	1,240
B	1,430	893	893	821	821	72	72	1,430
C	1,933	1,803	1,803	1,804	1,657	863	172	1,793
D	948	948	905	818	521	397	110	651
E	455	455	1	455	0	195	0	0
TOTALS	6,006	5,339	4,842	5,138	4,239	1,922	749	5,114

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SECTION V
CONDENSED MANAGEMENT REPORT
OF
LICENSING ACTIONS BY POWER REACTOR

DATA AS OF - 04/30/84

CONDENSED MANAGEMENT REPORT

FACILITY: ARKANSAS 1

PLANT LOCATION: 6 MI WNW OF RUSSELLVILLE, AR
 DOCKET NUMBER: 050-00313
 ARCH/ENGINEER: BECH
 IE INSPECTOR: W. JOHNSON

LICENSED POWER: 2568 MWT
 DESIGN POWER: 0850 MWE
 NSSS VENDOR: B&W

PROJECT MANAGER: G. VISSING
 BRANCH CHIEF: J. STOLZ
 LIC. ASSISTANT: R. INGRAM

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>MULTI-PLANT</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
07862	ANO 1 - PRESSURE VESSEL SUPPORT (ASYMMETRICAL LOADS)		2	<u>COMPLETE</u> 02/14/84
<u>ACTIVE ACTIONS</u>				
52206	ARKANSAS 1 - CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPA#C-10)			
52735	ARKANSAS 1 - ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION AND PROCEDURES			
52816	ARKANSAS 1 - ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RTS COMPONENTS			
52897	ARKANSAS 1 - ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING PROCEDURES/VENDOR RECOMMENDATIONS - RTS COMPONENTS			
52978	ARKANSAS 1-ITEM 3.1.3 -POST-MAINTENANCE TESTING-CHANGES TO TECH SPECS-RTS COMPONENTS			
53059	ARKANSAS 1 - REACTOR TRIP SYSTEM RELIABILITY - VENDOR RELATED MODIFICATIONS			
53112	ARKANSAS 1 - ITEMS 4.2.1 & 4.2.2 - PREVENTATIVE MAINT PROGRAM FOR REACTOR TRIP BREAKERS - MAINTENANCE & TRENDING			
53165	ARKANSAS 1 - ITEM 4.3 - AUTOAMTIC ACTUATION OF SHUNT TRIP ATTACHMENT FOR W AND B & W PLANTS		1	04/30/84
11236	ANO 1 - REVISED IST - TESTING		1	06/28/84
43632	ANO 1 - SEISMIC QUALIFICATION OF THE AUXILIARY FEEDWATER SYSTEM		3	06/30/84
08105	ANO 1 - APPENDIX I TECH SPEC IMPLEMENTATION REVIEW		2	07/30/84
07971	ARKANSAS 1 -CONTROL OF HEAVY LOADS OVER SPENT FUEL POOL (NUREG 0612)		2	09/30/84
11266	ANO 1 - MECHANICAL SNUBBERS		3	09/30/84
11267	ANO 1 - HYDRAULIC SNUBBERS		1	09/30/84
42514	ANO 1 - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21)		2	09/30/84
47181	ARKANSAS 1 NATURAL CIRCULATION COOLDOWN (GENERIC LTR 81-21)			09/30/84
51069	ARKANSAS 1 - INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)			09/30/84
49699	ARKANSAS 1 - REACTOR COOLANT PUMP TRIP			10/30/84
<u>TMI ACTIONS</u>				
<u>COMPLETED ACTIONS</u>				
42634	ARKANSAS 1 - NUREG-0737, II.E.1.1, AFW SYSTEM EVALUATION		1	<u>COMPLETE</u> 10/06/83
<u>ACTIVE ACTIONS</u>				
45112	ANO 1 - NUREG-0737 II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION		1	<u>TARGET</u> 06/30/83
43425	ANO 1 - ITEM II K.2.13 THERMAL-MECH EFFECTS OF HPI ON VESSEL		1	05/30/84
44419	ANO 1 - NUREG-0737 II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS		1	06/30/84
44560	ARKANSAS 1 - RV AND SV TESTING		1	06/30/84
45803	ANO 1 - NUREG-0737 II.K.3.30, SB LOCA OUTLINE		1	09/30/84
45923	ANO 1 - NUREG-0737 III.A.1.2, TECHNICAL SUPPORT CENTER		1	09/30/84
45994	ANO 1 - NUREG-0737 III.A.1.2, OPERATIONAL SUPPORT CENTER		1	09/30/84
46066	ANO 1 - NUREG-0737 III.A.1.2, EMERGENCY OPERATIONS FACILITY		1	09/30/84
46282	ANO 1 - NUREG-0737 III.A.2.2, METEOROLOGICAL DATA UPGRADE		1	09/30/84
51219	ARKANSAS 1 - I.D.2. SAFETY PARAMETER DISPLAY SYSTEM			09/30/84
44210	ANO 1 - NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES		1	11/30/84
42842	ARKANSAS 1 - NUREG-0737, I.C.1.3, ABNORMAL TRANSIENT OPER GUIDELINES & PROCEDURES		1	11/30/84
48144	ARKANSAS 1 -NUREG 0737 II.K.3.31 COMPLIANCE WITH 10 CFR 50.46		1	09/30/85
51359	ARKANSAS 1 - I.D.1 - CONTROL ROOM DESIGN REVIEW			11/30/85

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CONDENSED MANAGEMENT REPORT

(CONTINUATION)

ARKANSAS 1

TAC #	TAC DESCRIPTION	PLANT SPECIFIC	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				<u>COMPLETE</u>
51010	ARKANSAS NUCLEAR ONE, UNIT 1, APPENDIX R SCHEDULAR EXEMPT REQUEST FOR ONE HOUR FIRE BARRIERS			10/02/83
52091	ANO-1 TS CHANGE CONCERNING ANO& RVSP CAPSULE SCHEDULE			10/08/83
51926	ANO-1 SALP REVIEW JUL 1, 1982 TO JUNE 30, 1983			10/14/83
52162	ANO-1 - EVALUATION OF BACKUP TSC HABITABILITY			10/24/83
51575	ANO-1 - ELECT 1 AND C AND TSS RELATED TO OVER PRES. PROT.			11/07/83
51736	ANO 1: STATION DIST. VOLTAGE VERIF. TEST			12/19/83
41013	ANO 1 - IREP			12/23/83
51928	ANO-1/SOURCE RANGE NEUTRON FLUX MONITORING INSTRUMENTATION		1	01/06/84
47553	ANO-1 - REVIEW 5 YR TENDON SURVEILLANCE REPORT			01/16/84
51779	ANO-1 SHUNT TRIP DEVICE DESIGN FOR CRDTB			01/24/84
49807	ANO-1 TECH SPEC ADMINISTRATIVE CHANGES			02/01/84
53235	ANO-1 INTERPRETATION AT TECHNICAL SPECIFICATIONS			02/28/84
53425	ANO-1 EXEMPTION REQUEST FOR A FULL SCALE EMERGENCY EXERCISE			03/22/84
54272	ANA0-1 PROJECT MANAGER'S BRIEFING BOOK			03/30/84
<u>ACTIVE ACTIONS</u>				<u>TARGET</u>
51353	ARKANSAS NUCLEAR 1 UNIT 1 - APPENDIX R SCHEDULAR EXEMPTION REQUEST RELATED TO SUPPRESSION/DETECTION SYS/FIRE BARRIERS			05/30/84
54744	ANO - LICENSEE'S 1 REP RESPONSE			06/15/84
54348	ANO-1 P/T HEAT UP & COOLDOWN LIMITATIONS			06/30/84
54286	ANO-1 - STAFF REQUEST FOR T.S. CONCERNING ELECT. DIST.			06/30/84
54224	ANO-1, COMMISSION PAPER CONCERNING EOF			06/30/84
53530	ANO-1 ADMIN. CHANGE-CORRECT ERRORS			06/30/84
53259	ANO-1 REVIEW EFW ISSUE OPEN ITEMS GL-9 & GL-2			06/30/84
49722	- ARKANSAS 1 NUREG 0737 TECH SPECS (GENERIC LTR 82-16)			06/30/84
48030	ANG-1 REACTOR COOLANT PUMP SEAL DAMAGE		3	06/30/84
52382	ANO-1 - LEAK TESTING OF SEALED RADIOACTIVE SOURCES			06/30/84
12374	ANO 1 - REVISE ENVIRONMENTAL (T.S. APPENDIX B)		3	06/30/84
43595	ANO 1 - T.S. FOR OPERABILITY OF PORV		1	06/30/84
54483	ANO-1 TECH. SPEC. FOR OVERPRESSURE PROTECTION			07/20/84
54311	ANO-1 - IST RELIEF FOR CF-1A, CF-1B, BS-4A, BS-4B			09/05/84
53310	ANO-1 - BOUNDING ANALYSIS OF NUREG 0630 MODEL WITH FLESET			09/30/84
54514	ARKANSAS 1 - TECHNICAL SPECIFICATIONS COVERED BY GENERIC LETTERS 83-36, 83-37 FOR NUREG-0737			09/30/84
54516	BEAVER VALLEY 1 - TECHNICAL SPECIFICATIONS COVERED BY GENERIC LETTERS 83-36, 83-37 FOR NUREG-0737			09/30/84
54380	ARKANSAS 1- RCS VENTS TECHNICAL SPECIFICATIONS NUREG 073 ITEM II.B.1			09/30/84
49826	ARKANSAS NUCLEAR 1- SMALL BREAK LOCA PROCED & MAINTAINING PROPER SG LEVEL			09/30/84
43164	ANO 1 - WASTE GAS H/O LIMITS T.S. CHANGE		1	09/30/84
51584	ANO-1, TS CHANGES RELATED TO KEY LOCK SWITCHES FOR PURGE VALVES			12/30/84
<u>ANTICIPATED ACTIONS</u>				<u>INITIATION</u>
53567	ARKANSAS 1 - ITEM 1.2 - POST TRIP REVIEW-DATA AND INFORMATION CAPABILITY			11/01/84
53650	ARKANSAS 1 - ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS			11/01/84
53733	ARKANSAS 1- ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS			11/01/84
53816	ARKANSAS 1 - ITEM 3.2.3 - POST-MAINTENANCE TESTING -CHANGES TO TECH. SPEC. - ALL SR COMPONENTS			11/01/84
53899	-ITEMS 4.2.3 & 4.2.4-PREVENTATIVE MAINT. PROGRAM FOR REACTOR TRIP BREAKERS-LIFE TESTING & REPLACEMENT			11/01/84
53952	ARKANSAS 1 - ITEM 4.4 - IMPROVEMENTS IN MAINTENANCE AND TEST PROCEDURES FOR B & W PLANTS			11/01/84
53960	ARKANSAS 1 - ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS			11/01/84

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TECHNICAL ASSIGNMENT CONTROL SYSTEM

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(CONTINUATION)

ARKANSAS 1

TAC # TAC DESCRIPTION

PLANT SPECIFIC

PRIORITY

CRITICAL DATE

ANTICIPATED ACTIONS (CONTINUATION)

INITIATION

54043 ARKANSAS 1 - ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL TESTING - DIVERSE TRIP FEATURES

11/01/84

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CONDENSED MANAGEMENT REPORT

FACILITY: ARKANSAS 2

PLANT LOCATION: 6 MI WNW OF RUSSELLVILLE, AR
 DOCKET NUMBER: 050-00368
 ARCH/ENGINEER: BECH
 IE INSPECTOR: L. CALLAN

LICENSED POWER: 2815 MWT
 DESIGN POWER: 0912 MWE
 NSSS VENDOR: COMB

PROJECT MANAGER: R. LEE
 BRANCH CHIEF: J. MILLER
 LIC. ASSISTANT: P. KREUTZER

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>MULTI-PLANT</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
47161	ARKANSAS 2 NATURAL CIRCULATION COOLDOWN (GENERIC LTR 81-21)		2	<u>COMPLETE</u> 03/27/84
<u>ACTIVE ACTIONS</u>				
51070	ARKANSAS 2 - INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)		3	<u>TARGET</u> 06/30/84
49723	ARKANSAS 2 - "NUREG 0737 TECH SPECS (GENERIC LETTER 82-16 82-16 OR 83-02)"			07/30/84
42496	AND 2 - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21)		1	07/30/84
08159	AND 2 - APPENDIX I TECH SPEC IMPLEMENTATION REVIEW		3	07/30/84
12069	AND 2 - SURVEILLANCE FOR HYDRAULIC SNUBBERS (B-17)		3	08/30/84
12070	AND 2 - T.S. SURVEILLANCE MECHANICAL SNUBBERS (B-22) (B-22)		3	08/30/84
12542	AND 2 - MISC. CHANGES TO FIRE PROTECTION T.S.		3	08/30/84
52898	ARKANSAS 2 - ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING PROCEDURES/VENDOR RECOMMENDATIONS - RTS COMPONENTS		1	11/01/84
52979	ARKANSAS 2-ITEM 3.1.3 -POST-MAINTENANCE TESTING-CHANGES TO TECH SPECS-RTS COMPONENTS		1	11/01/84
53060	ARKANSAS 2 - REACTOR TRIP SYSTEM RELIABILITY - VENDOR RELATED MODIFICATIONS		1	11/01/84
53113	ARKANSAS 2 - ITEMS 4.2.1 & 4.2.2 - PREVENTATIVE MAINT PROGRAM FOR REACTOR TRIP BREAKERS - MAINTENANCE & TRENDING		1	11/01/84
52736	ARKANSAS 2 - ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION AND PROCEDURES		1	11/01/84
52817	ARKANSAS 2 - ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RTS COMPONENTS		1	11/01/84
49676	ARKANSAS 2 - REACTOR COOLANT PUMP TRIP			12/30/84
12946	ARKANSAS 2 -CONTROL OF HEAVY LOADS OVER SPENT FUEL POOL (NUREG 0612)		2	12/30/84
43633	AND 2 - SEISMIC QUALIFICATION OF THE AUXILIARY FEEDWATER SYSTEM		3	12/30/84
52207	ARKANSAS 2 - CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPA#C-10)			09/30/85
<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>TMI ACTIONS</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
47791	ARKANSAS 2 - NUREG 0737 ITEM II K2.17 POTENTIAL FOR VOIDING IN RCS		1	<u>COMPLETE</u> 10/18/83
44660	AND 2 - NUREG-0737 II.E.1.1, EFW SYSTEM EVALUATION EVALUATION		1	11/18/83
<u>ACTIVE ACTIONS</u>				
44420	AND 2 - NUREG-0737 II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS		1	<u>TARGET</u> 07/30/84
44561	ARKANSAS 2 - RV AND SV TESTING		1	09/30/84
46877	ARKANSAS 2 - II K2.13 THERMAL MECHANICAL REPORT		1	09/30/84
45924	AND 2 - NUREG-0737 III.A.1.2, TECHNICAL SUPPORT CENTER		1	09/30/84
45995	AND 2 - NUREG-0737 III.A.1.2, OPERATIONAL SUPPORT CENTER		1	09/30/84
46067	AND 2 - NUREG-0737 III.A.1.2, EMERGENCY OPERATIONS FACILITY		1	09/30/84
51140	ARKANSAS 2 - I.D.1 CONTROL ROOM DESIGN REVIEW			09/30/84
51220	ARKANSAS 2 - I.D.2. SAFETY PARAMETER DISPLAY SYSTEM			09/30/84
45804	AND 2 - NUREG-0737 II.K.3.30, SB LOCA OUTLINE		1	12/30/84
45113	AND 2 - NUREG-0737 II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION		1	12/30/85
44211	AND 2 - NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES		1	12/30/85
44281	AND 2 - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES		1	12/30/85
48145	ARKANSAS 2 -NUREG 0737 II.K.3.31 COMPLIANCE WITH 10 CFR 50.46		3	12/30/85
46283	AND 2 - NUREG-0737 III.A.2.2, METEOROLOGICAL DATA UPGRADE		1	09/30/86

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CONDENSED MANAGEMENT REPORT

(CONTINUATION)

ARKANSAS 2

TAC #	TAC DESCRIPTION	PLANT SPECIFIC	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				<u>COMPLETE</u>
51009	ARKANSAS NUCLEAR ONE, APPENDIX R SCHEDULAR EXEMPT REQUEST FOR ONE HOUR FIRE BARRIERS (UNIT 2)			10/07/83
51927	ANO-2 SALP REVIEW, JULY 1, 1982 TO JUNE 30, 1983			10/20/83
52161	ANO-2 - EVALUATION OF BACKUP TSC HABITABILITY			10/24/83
49447	ANO-2, APPENDIX R SCHEDULAR EXEMPTION REQUEST, FIREZONE 2100Z AND 2101AA SWITCHGEAR MOD			11/01/83
52104	ANO-2 T.S. CHANGE REQUEST - FUEL ENRICHMENT			11/07/83
51417	ANO-2 - TECH SPEC CHANGES RELATED TO INCORE DETECTORS SURVEILLANCE REQUIREMENTS & ADMINISTRATIVE CORRECTIONS			11/10/83
52356	ANO-2 - EQ DEADLINE EXTENSION			11/16/83
49502	ANO-2, CPC SOFTWARE ERROR			11/28/83
52065	ANO-2 T.S. CHANGE REQUEST - SURVEILLANCE OF EFW PUMP			12/13/83
52066	ANO-2 T.S. CHANGE REQUEST - EFW FLOW PATH VERIFICATION			12/13/83
51929	ANO-2/SOURCE RANGE NEUTRON FLUX MONITORING INSTRUMENTATION			01/06/84
53309	ANO-2 - APP. R - SCHEDULAR EXEMPTION REQUEST PERTAINING TO SOURCE RANGE MONITORING CAPABILITY			01/23/84
49808	ANO-2 TECH SPEC ADMINISTRATIVE CHANGES			02/01/84
48918	ANO 2 - FUEL SHOULDER GAP DECREASE			02/24/84
12081	ANO 2 - VERIFICATION OF TRANSIENT ANALYSIS CODE CESEC CESEC		1	03/20/84
53426	ANO-2 EXEMPTION REQUEST FOR A FULL SCALE EMERGENCY EXERCISE		1	03/22/84
52064	ANO-2 T.S. CHANGE REQUEST - INCORE DETECTION SYSTEM			03/26/84
53350	ANO-2, TECH SPEC CHANGE REQUEST - ELECTRICAL POWER SYSTEMS			03/27/84
12466	ANO 2 - T.S. CHANGE 2 SEISMIC INSTRUMENTATION		2	04/13/84
54225	ANO-2, COMMISSION PAPER CONCERNING EOF			04/25/84
<u>ACTIVE ACTIONS</u>				<u>TARGET</u>
54223	ANO-2, PM BRIEFING BOOK			02/28/84
49320	ARKANSAS NUCLEAR ONE - UNIT 2 - ESFAS DESIGN DISCREPANCY		2	05/30/84
43165	ANO 2 - WASTE GAS H/O LIMITS T.S. CHANGE		3	06/15/84
52383	ANO-2 - LEAK TESTING OF SEALED RADIOACTIVE SOURCES		3	06/30/84
52635	ANO-2, TECH SPEC CHANGE REQUESTS FOR REMOTE SHUTDOWN PANEL INSTRUMENTATION AND H-H ANALYZER CALIB.		2	06/30/84
52636	ANO-2, LICENSE AMENDMENT FOR IAEA SAFEGUARD INSPECTION PROGRAM			07/30/84
12083	ANO 2 - FIRE PROTECTION-LICENSE CONDITION RESOLUTION		3	07/30/84
54761	ANO-2, TSCR PERTAINING TO DIESEL GENERATOR SURVEILLANCE REQUIREMENTS			07/30/84
54381	ARKANSAS 2- RCS VENTS TECHNICAL SPECIFICATIONS NUREG 073 ITEM II.B.1			09/30/84
54515	ARKANSAS 2 - TECHNICAL SPECIFICATIONS COVERED BY GENERIC LETTERS 83-36, 83-37 FOR NUREG-0737			09/30/84
51585	ANO-2, TS CHANGES RELATED TO KEY LOCK SWITCHES FOR PURGE VALVES		3	09/30/84
51352	ARKANSAS NUCLEAR 1 UNIT 2 - APPENDIX R SCHEDULAR EXEMPTION REQUEST RELATED TO SUPPRESSION/DETECTION SYS/FIRE BARRIERS		3	12/30/84
12062	ANO 2 - INSERVICE TESTING PROGRAM (A-14)		3	12/30/84
<u>ANTICIPATED ACTION:</u>				<u>INITIATION</u>
53817	ARKANSAS 2 - ITEM 3.2.3 - POST-MAINTENANCE TESTING -CHANGES TO TECH. SPEC. - ALL SR COMPONENTS			11/01/84
53900	-ITEMS 4.2.3 & 4.2.4-PREVENTATIVE MAINT. PROGRAM FOR REACTOR TRIP BREAKERS-LIFE TESTING & REPLACEMENT			11/01/84
53961	ARKANSAS 2 - ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS			11/01/84
54044	ARKANSAS 2 - ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL TESTING - DIVERSE TRIP FEATURES			11/01/84
53568	ARKANSAS 2 - ITEM 1.2 - POST TRIP REVIEW-DATA AND INFORMATION CAPABILITY			11/01/84
53651	ARKANSAS 2 - ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS			11/01/84
53734	ARKANSAS 2- ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS			11/01/84

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CONDENSED MANAGEMENT REPORT

FACILITY: BEAVER VALLEY 1

PLANT LOCATION: 5 MI E OF E. LIVERPOOL, OH
 DOCKET NUMBER: 050-00334
 ARCH/ENGINEER: S&W
 IE INSPECTOR: W. TROSKOSKI

LICENSED POWER: 2660 MWT
 DESIGN POWER: 0835 MWE
 NSSS VENDOR: WEST

PROJECT MANAGER: P. TAM
 BRANCH CHIEF: S. VARGA
 LIC. ASSISTANT: C. PARRISH

TAC #	TAC DESCRIPTION	MULTI-PLANT	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				
49724	BEAVER VALLEY 1 - "NUREG 0737 TECH SPECS (GENERIC LETTER 82-16 OR 83-02)"			<u>COMPLETE</u> 10/21/83
12263	BEAVER VALLEY 1 - HELB AND CONSEQUENTIAL SYSTEM FAILURE BEAVER VALLEY 1		3	01/31/84
49417	BEAVER VALLEY 1 - BLOCKING SI SIGNAL DURING COOLDOWN		2	02/09/84
<u>ACTIVE ACTIONS</u>				
52737	BEAVER VALLEY 1 - ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION AND PROCEDURES		1	<u>TARGET</u>
52818	BEAVER VALLEY 1 - ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RTS COMPONENTS		1	
52899	BAVER VALLEY 1 - ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING PROCEDURES/VENDOR RECOMMENDATIONS - RTS COMPONENTS		1	
52980	BEAVER VALLEY 1-ITEM 3.1.3-POST-MAINT TESTING-CHNGS TO TECH SPECS-RTS COMPONENTS		1	
53061	BEAVER VALLEY 1 - REACTOR TRIP SYSTEM RELIABILITY - VENDOR RELATED MODIFICATIONS		1	
53114	BEAVER VALLEY 1 - ITEMS 4.2.1 & 4.2.2 - PREVENTATIVE MAINT PROGRAM FOR REACTOR TRIP BREAKERS - MAINTENANCE & TRENDING		1	
53166	BEAVER VALLEY 1 - ITEM 4.3 - AUTOAMTIC ACTUATION OF SHUNT TRIP ATTACHMENT FOR W AND B & W PLANTS		1	
10386	BEAVER VALLEY 1 - N-1 LOOP OPERATION		2	05/31/84
42459	BEAVER VALLEY 1 - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21)		1	09/30/84
07972	BEAVER VALLEY 1-CNTRL OF HEAVY LOADS OVER SPENT FUEL POOL (NUREG 0612)		2	10/30/84
49695	BEAVER VALLEY 1 - REACTOR COOLANT PUMP TRIP			12/30/84
52208	BEAVER VALLEY 1 - CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPA#C-10)			12/31/84
08597	BEAVER VALLEY 1 - REVIEW OF ASYMMETRIC LOCA LOADS		2	12/31/84
51071	BEAVER VALLEY 1 - INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)			03/31/85
<u>TAC # TAC DESCRIPTION IMI ACTIONS PRIORITY CRITICAL DATE</u>				
<u>COMPLETED ACTIONS</u>				
47792	BEAVER VALLEY 1 - NUREG 0737 ITEM II K2.17 POTENTIAL FOR VOIDING IN RCS		1	<u>COMPLETE</u> 12/19/83
<u>ACTIVE ACTIONS</u>				
44421	BEAVER VALLEY 1 - NUREG-0737 II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS		1	05/31/84
45805	BEAVER VALLEY 1 - NUREG-0737 II.K.3.30, SB LOCA OUTLINE		1	09/30/84
45925	BEAVER VALLEY 1 - NUREG-0737 III.A.1.2, TECHNICAL SUPPORT CENTER		1	09/30/84
45996	BEAVER VALLEY 1 - NUREG-0737 III.A.1.2, OPERATIONAL SUPPORT CENTER		1	09/30/84
46068	BEAVER VALLEY 1 - NUREG-0737 III.A.1.2, EMERGENCY OPERATIONS FACILITY		1	09/30/84
51141	BEAVER VALLEY 1 - I.D.1 CONTROL ROOM DESIGN REVIEW		1	09/30/84
51221	BEAVER VALLEY 1 - I.D.2. SAFETY PARAMETER DISPLAY SYSTEM		1	09/30/84
44212	BEAVER VALLEY 1 - NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES		1	09/30/84
44282	BEAVER VALLEY 1 - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES		1	09/30/84
46878	BEAVER VALLEY 1 II K2.13 THERMAL MECHANICAL REPORT		1	10/30/84
46284	BEAVER VALLEY 1 - NUREG-0737 III.A.2.2, METEOROLOGICAL DATA UPGRADE		1	12/31/84
45114	BEAVER VALLEY 1 - NUREG-0737 II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION		1	12/31/84
44562	BEAVER VALLEY 1 - RV AND SV TESTING		1	06/30/85
48146	BEAVER VALLEY 1 -NUREG 0737 II.K.3.31 COMPLIANCE WITH 10 CFR 50.46		1	09/30/85

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BEAVER VALLEY 1

TAC #	TAC DESCRIPTION	PLANT SPECIFIC	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				<u>COMPLETE</u>
49436	BEAVER VALLEY 1 - REVIEW OF REPORT ON THIMBLE DELETION FOR TS CHANGE			10/21/83
51048	BEAVER VALLEY UNIT 1 - SURVEILLANCE REQUIREMENTS FOR HEPA FILTERS AND CHARCOAL ADSORBER UNITS			10/27/83
51818	BEAVER VALLEY UNIT 1 - TECH SPEC FOR OVERPRESSURE PROTECTION SYSTEM			10/31/83
52364	BEAVER VALLEY UNIT 1 - ROD BANK WORTH MEASUREMENT WCAP-9863		2	11/04/83
52554	BEAVER VALLEY UNIT - REVISED CYCLE 4 RELOAD REPORT			11/05/83
52068	BEAVER VALLEY UNIT 1 - ADMINISTRATIVE CHANGES, CHANGE REQUEST NO. 86		3	12/13/83
52070	BEAVER VALLEY UNIT 1 - ONSITE POWER DISTRIBUTION SYSTEMS		2	12/13/83
51577	BEAVER VALLEY UNIT 1 - RHR HEAT EXCHANGER ISI RELIEF		2	12/15/83
52184	BEAVER VALLEY UNIT 1 - RCS VENT SYSTEM SCHEDULAR EXEMPTION		1	01/12/84
52051	BEAVER VALLEY UNIT 1 - DELAY IN INSTALLATION OF NEW DIESEL FUEL TANK		2	01/31/84
53542	BEAVER VALLEY UNIT 1 - PRIVACY INFORMATION IN EMERGENCY PLAN		3	02/14/84
52183	BEAVER VALLEY UNIT 1 - MAINE FEEDWATER SYSTEM SNUBBERS			02/27/84
53415	BEAVER VALLEY UNIT 1 - BRIEFING BOOK FOR EMERGENCIES		2	02/28/84
54219	BEAVER VALLEY UNIT 1 - REVISION TO APP. R SER (USE OF CETS VS. T-HOT)			03/22/84
54358	BEAVER VALLEY JNIT 1 - LICENSEE COMMENTS ON NUREG-1021			03/26/84
52555	BEAVER VALLEY UNIT - DOSE PROJECTION USING COMPUTER CODE IRDAM		1	04/09/84
53472	BEAVER VALLEY UNIT 1 - INFRARED AERIAL PHOTOS		2	04/11/84
53207	BEAVER VALLEY UNIT 1 - ELIMINATION OF APP. B. TS RADIOLOGICAL		1	04/17/84
<u>ACTIVE ACTIONS</u>				<u>TARGET</u>
54841	BEAVER VALLEY UNIT 1 - REORGANIZATION OF OFFSITE ORGANIZATION			04/30/84
54198	BEAVER VALLEY 1 - RIVER WATER SYSTEM CLASS 3 PIPING SUPPORT - ISI		3	05/30/84
53315	BEAVER VALLEY UNIT 1 - 1983 CONTAINMENT LEAK RATE TEST		2	05/30/84
49437	BEAVER VALLEY 1 - RESPONSE TO IST SER		3	05/31/84
52530	BEAVER VALLEY UNIT 1 - REANALYSIS OF LOCA DOSES		2	05/31/84
52691	BEAVER VALLEY UNIT 1 - CLASS 3 PIPE SUPPORT RELIEF REQUEST		2	06/15/84
52531	BEAVER VALLEY 1 - REVISION TO REACTOR VESSEL ISI		1	06/22/84
54825	BEAVER VALLEY UNIT 1 - EMERGENCY EXERCISE EXEMPTION PER IN 84-05			06/30/84
53380	BEAVER VALLEY UNIT 1 - ADDITIONAL FIRE PROTECTION EXEMPTION REQUESTS		2	06/30/84
53225	BEAVER VALLEY UNIT 1 - SALP REPORT FOR 1983			06/30/84
53291	BEAVER VALLEY UNIT 1 - ELIMINATION OF MECHANICAL VACUUM PUMP TECH. SPECS.		2	06/30/84
52071	BEAVER VALLEY UNIT 1 - BYPASSES TO BLOCK TRIP SYSTEM INSTRUMENTS AND ESF CHANNELS		2	06/30/84
52027	BEAVER VALLEY UNIT 1 - NOZZLE THERMAL STRESSES DUE TO INADVERTENT SI ACTUATION		2	06/30/84
51513	BEAVER VALLEY UNIT 1 - REVISED INSERVICE INSPECTION PROGRAM		1	06/30/84
53208	BEAVER VALLEY UNIT 1 - ELIMINATION OF APPENDIX B TS, NON-RADIOLOGICAL		2	08/31/84
54258	BEAVER VALLEY UNIT 1 - EXTENSION OF ISI INTERVAL			09/15/84
54382	BEAVER VALLEY 1 - RCS VENTS TECHNICAL SPECIFICATIONS NUREG 073 ITEM II.B.1			09/30/84
52069	BEAVER VALLEY UNIT 1 - CONTAINMENT AIR LOCKS		2	12/31/84
54627	BEAVER VALLEY UNIT 1 - SALP FOR 4/84 TO 10/85			12/31/85
<u>ANTICIPATED ACTIONS</u>				<u>INITIATION</u>
55010	BEAVER VALLEY UNIT 1			05/02/84
53569	BEAVER VALLEY 1 - ITEM 1.2 - POST TRIP REVIEW-DATA AND INFORMATION CAPABILITY			11/01/84
53652	BEAVER VALLEY 1 - ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS			11/01/84
53735	BEAVER VALLEY 1 - ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS			11/01/84
53818	BEAVER VALLEY 1 - ITEM 3.2.3 - POST-MAINTENANCE TESTING -CHANGES TO TECH. SPEC. - ALL SR COMPONENTS			11/01/84
53901	-ITEMS 4.2.3 & 4.2.4-PREVENTATIVE MAINT. PROGRAM FOR REACTOR TRIP BREAKERS-LIFE TESTING & REPLACEMENT			11/01/84

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CONDENSED MANAGEMENT REPORT

(CONTINUATION)

BEAVER VALLEY 1

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>ANTICIPATED ACTIONS (CONTINUATION)</u>				
53962	BEAVER VALLEY 1 - ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS			<u>INITIATION</u> 11/01/84
54045	BEAVER VALLEY 1 - ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL TESTING - DIVERSE TRIP FEATURES			11/01/84

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CONDENSED MANAGEMENT REPORT

FACILITY: BIG ROCK POINT 1

PLANT LOCATION: 4 MI NE OF CHARLEVOIX, MICH
 DOCKET NUMBER: 050-00155
 ARCH/ENGINEER: BECH
 IE INSPECTOR: G. WRIGHT

LICENSED POWER: 0240 MWT
 DESIGN POWER: 0072 MWE
 NSSS VENDOR: GE

PROJECT MANAGER: R. EMCH
 BRANCH CHIEF: D. CRUTCHFIELD
 LIC. ASSISTANT: H. SMITH

TAC #	TAC DESCRIPTION	MULTI-PLANT	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				
08441	BIG ROCK POINT - MECHANICAL SNUBBERS		2	<u>COMPLETE</u> 01/11/84
51668	BIG ROCK PT 1 - NUREG 0737 TECH SPECS			02/10/84
<u>ACTIVE ACTIONS</u>				
52738	BIG ROCK POINT 1 - ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION AND PROCEDURES			<u>TARGET</u>
52819	BIG ROCK POINT 1 - ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RTS COMPONENTS			
52900	BIG ROCK POINT 1 - ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING PROCEDURES/VENDOR RECOMMENDATIONS - RTS COMPONENTS			
52981	BIG ROCK PT 1-ITEM 3.1.3-POST-MAINTENANCE TESTING-CHNGS TO TECH SPECS-RTS COMPONENTS		2	05/30/84
46646	BIG ROCK PT 1-CONTROL OF HEAVY LOADS OVER SPENT FUEL POOL PLANTS (NUREG 0612)		2	06/30/84
46651	BIG ROCK POINT - IMPLEMENTATION OF NUREG-0313, REV. 1		2	06/30/84
42515	BIG ROCK POINT - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21)		1	06/30/84
42907	BIG ROCK POINT - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11		2	06/30/84
08096	BIG ROCK POINT - APPENDIX I TECH SPEC IMPLEMENTATION REVIEW		3	09/01/84
43044	BIG ROCK POINT - TECHNICAL SPECIFICATIONS DEFINING OPERABILITY FOR SAFETY SYSTEMS		2	09/30/84
51072	BIG ROCK POINT 1 - INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)		1	09/30/84
52209	BIG ROCK POINT 1 - CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPA#C-10)			09/30/84
<u>TAC # TAC DESCRIPTION IMI ACTIONS PRIORITY CRITICAL DATE</u>				
<u>COMPLETED ACTIONS</u>				
48147	BIG ROCK POINT -NUREG 0737 II.K.3.31 COMPLIANCE WITH 10 CFR 50.46		1	<u>COMPLETE</u> 12/27/83
45806	BIG ROCK POINT - NUREG-0737 II.K.3.30, SB LOCA OUTLINE		1	12/27/83
46427	BIG ROCK POINT - NUREG-0737 III.D.3.4, CONTROL ROOM HABITABILITY		1	12/30/83
45875	BIG ROCK POINT - NUREG-0737 II.K.3.44, TRANSIENTS WITH SINGLE FAILURES		1	03/15/84
47581	BIG ROCK POINT - NUREG 0737 ITEM II F1.4 - CONTAINMENT PRESSURE INSTRUMENT		1	04/16/84
47652	BIG ROCK POINT - NUREG 0737 ITEM II F1.5 CONTAINMENT WATER LEVEL MONITOR		1	04/16/84
<u>ACTIVE ACTIONS</u>				
47723	BIG ROCK POINT - NUREG 0737 ITEM II F1.6 CONTAINMENT HYDROGEN MONITOR		1	<u>TARGET</u> 06/30/84
47920	BIG ROCK POINT - PLANT SHIELDING MODIFICATIONS (II.B.2.2)		1	06/30/84
45115	BIG ROCK POINT - NUREG-0737 II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION		1	06/30/84
44213	BIG ROCK POINT - NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES		1	06/30/84
44283	BIG ROCK POINT - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES		1	06/30/84
44422	BIG ROCK POINT - NUREG-0737 II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS		1	06/30/84
44563	BIG ROCK POINT 1 - RV AND SV TESTING		1	09/30/84
45926	BIG ROCK POINT - NUREG-0737 III.A.1.2, TECHNICAL SUPPORT CENTER		1	09/30/84
45997	BIG ROCK POINT - NUREG-0737 III.A.1.2, OPERATIONAL SUPPORT CENTER		1	09/30/84
46069	BIG ROCK POINT - NUREG-0737 III.A.1.2, EMERGENCY OPERATIONS FACILITY		1	09/30/84
46285	BIG ROCK POINT - NUREG-0737 III.A.2.2, METEOROLOGICAL DATA UPGRADE		1	09/30/84
51142	BIG ROCK POINT 1 - I.D.1 CONTROL ROOM DESIGN REVIEW		1	09/30/84
51222	BIG ROCK POINT 1 - I.D.2. SAFETY PARAMETER DISPLAY SYSTEM		1	09/30/84

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CONDENSED MANAGEMENT REPORT

(CONTINUATION)

BIG ROCK POINT 1

TAC #	TAC DESCRIPTION	PLANT SPECIFIC	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				<u>COMPLETE</u>
52340	BIG ROCK POINT - FOR G13 & G14 FUEL MAPLHGR		2	11/14/83
49590	BIG ROCK POINT - CONTAINMENT LEAK TEST T.S.			12/27/83
52520	BIG ROCK POINT - EXEMPTION REQUEST - 20.54(M) (2) & (3)		1	12/30/83
48708	BIG ROCK POINT - SAFETY MANAGEMENT FUNCTION, TS CHG		2	01/04/84
49304	BIG ROCK POINT - REVISION OF ADMINISTRATIVE CONTROLS T.S.		2	01/04/84
51976	BIG ROCK POINT - REVISION TO GUARD TRAINING PLAN			02/17/84
54251	BIG ROCK POINT - II.F.1(1) HIGH RANGE STACK GAS MONITOR DEVIATION			03/07/84
52657	BIG ROCK POINT - EXEMPTION TO 10CFR, APPR, II.G.3		2	03/20/84
52658	BIG ROCK POINT - REACTOR VESSEL PRESSURE TEMPERATURE LIMITS T.S.		1	04/12/84
51495	BIG ROCK POINT - SECURITY PLAN CHANGE		2	04/18/84
<u>ACTIVE ACTIONS</u>				<u>TARGET</u>
52012	BIG ROCK POINT - CONTROL ROD WITHDRAWAL RATE LIMIT T.S.			
54763	BIG ROCK POINT - DEVIATION TO II.F.1(1) STACK GAS MONITOR		1	05/15/84
49298	BIG ROCK POINT - MCPR T.S.		1	05/15/84
12979	BIG ROCK POINT - RISK ASSESSMENT			05/22/84
54250	BIG ROCK POINT - STATE EXERCISE PARTICIPATION EXEMPTION			05/30/84
53356	BIG ROCK POINT - REACTOR COOLANT IODINE T.S.			05/30/84
51974	BIG ROCK POINT - CONTAINMENT PRESSURE & WATER LEVEL MONITOR T.S.			05/30/84
51975	BIG ROCK POINT - STACK GAS MONITORING SYSTEM T.S.			05/30/84
51900	BIG ROCK POINT - VENT VALVE SEISMIC QUAL. & AIR CYLINDER			06/30/84
51988	BIG ROCK POINT - TRANSFER CASK TRIP TEST T.S.			06/30/84
52355	BIG ROCK POINT - SCRAM DISCHARGE VOLUME LEVEL INSTRUMENTATION			06/30/84
52485	BIG ROCK POINT - REQUEST FOR EXEMPTION TO 10 CFR 50.44 (C)(3)(III) - HIGH POINT VENTS		1	06/30/84
08277	BIG ROCK POINT - ATWS RECIRCULATION PUMP TRIP		1	06/30/84
49499	BIG ROCK POINT - DEBRIS SCREENS, VENT VALVE TESTS, VALVE PERCENT TIME OPEN		2	06/30/84
53379	BIG ROCK POINT - ISI RELIEF REQUESTS (REV. 3)			07/30/84
49589	BIG ROCK POINT - TEN YEAR 1ST PUMP & VALVE PROGRAM		2	09/01/84
54517	BIG ROCK POINT 1 - TECHNICAL SPECIFICATIONS COVERED BY GENERIC LETTERS 83-36, 83-37 FOR NUREG-0737			09/30/84
<u>ANTICIPATED ACTIONS</u>				<u>INITIATION</u>
53570	BIG ROCK POINT 1 - ITEM 1.2 - POST TRIP REVIEW-DATA AND INFORMATION CAPABILITY			11/01/84
53653	BIG ROCK POINT 1 - ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS			11/01/84
53736	BIG ROCK PT 1- ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS			11/01/84
53819	BIG ROCK POINT 1 - ITEM 3.2.3 - POST-MAINTENANCE TESTING -CHANGES TO TECH. SPEC. - ALL SR COMPONENTS			11/01/84
53963	BIG ROCK POINT 1 - ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS			11/01/84
54046	BIG ROCK POINT 1 - ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL TESTING - DIVERSE TRIP FEATURES			11/01/84

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CONDENSED MANAGEMENT REPORT

FACILITY: BROWNS FERRY 1

PLANT LOCATION: 10 MI NW OF DECATUR, ALA
 DOCKET NUMBER: 050-00259
 ARCH/ENGINEER: TVA
 IE INSPECTOR: J. PAULK

LICENSED POWER: 3293 MWT
 DESIGN POWER: 1065 MWE
 NSSS VENDOR: GE

PROJECT MANAGER: R. CLARK
 BRANCH CHIEF: D. VASSALLO
 LIC. ASSISTANT: S. NORRIS

TAC #	TAC DESCRIPTION	MULTI-PLANT	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				
12295	BROWN'S FERRY 1 - HELB AND CONSEQUENTIAL SYSTEM FAILURE		3	<u>COMPLETE</u> 02/01/84
<u>ACTIVE ACTIONS</u>				
51073	BROWNS FERRY 1 - INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)			<u>TARGET</u>
52192	BROWNS FERRY 1 - RPS MODIFICATION			
52210	BROWNS FERRY 1 - CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPA#C-10)			
52392	BROWNS FERRY 1 - APPENDIX R EXEMPTION REQUESTS			
52739	BROWNS FERRY 1 - ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION AND PROCEDURES			
52820	BROWNS FERRY 1 - ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RTS COMPONENTS			
52901	BROWNS FERRY 1 - ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING PROCEDURES/VENDOR RECOMMENDATIONS - RTS COMPONENTS			
52982	BROWNS FERRY 1-ITEM 3.1.3-POST-MAINTENANCE TESTING-CHNGS TO TECH SPECS-RTS COMPONENTS			
08715	BROWNS FERRY 1 - CONTAINMENT LEAK TESTING - APP. J (GENERIC)		2	09/01/83
46652	BROWNS FERRY 1 - IMPLEMENTATION OF NUREG-0313, REV. 1		2	09/28/83
42568	BROWNS FERRY 1 - LONG TERM REVIEW CONTAINMENT PURGE AND VENT (B-24)		1	09/30/83
11324	BROWNS FERRY 1 - INSERVICE TESTING		1	11/11/83
48131	BROWNS FERRY 1 - SINGLE LOOP OPERATION		3	11/21/83
07974	BROWNS FERRY 1-CONTROL OF HEAVY LOADS OVER SPENT FUEL POOL (NUREG 0612)		2	12/30/83
08435	BROWN FERRY 1 - BWR FEEDWATER & CONTROL ROD DRIVE NOZZLE CRACKING		1	02/17/84
51669	BROWNS FERRY 1 NUREG 0737 TECH SPECS			03/01/84
08133	BROWNS FERRY 1 - APPENDIX I T.S. IMPLEMENTATION REVIEW REVIEW		3	03/16/84
42873	BROWNS FERRY 1 - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11		2	03/30/84
43727	BROWNS FERRY 1 - SAFETY CONCERNS ASSOCIATED WITH PIPE BREAKS IN THE BWR SCRAM SYSTEM		1	03/30/84
07929	BROWNS FERRY 1 - MARK I CONTAINMENT LONG TERM PROGRAM IMPLEMENTATION		1	06/30/84
48134	BROWNS FERRY 1 - FIRE PROTECTION APPENDIX R		3	07/30/84
08929	BROWNS FERRY 1 - RPS POWER SUPPLY		1	08/10/84
42481	BROWNS FERRY 1 - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21)		1	09/07/84
<u>TMI ACTIONS</u>				
<u>COMPLETED ACTIONS</u>				
44143	BROWNS FERRY 1 - NUREG-0737 I.A.2.1.4, UPGRADING OF RO AND SRO TRAINING		1	<u>COMPLETE</u> 10/12/83
44493	BROWNS FERRY 1 - NUREG-0737 II.B.4.1, TRAINING FOR MITIGATING CORE		1	10/12/83
48148	BROWNS FERRY 1 -NUREG 0737 II.K.3.31 COMPLIANCE WITH 10 CFR 50.46		1	12/28/83
45807	BROWNS FERRY 1 - NUREG-0737 II.K.3.30, SB LOCA OUTLINE		1	12/28/83
44072	BROWNS FERRY 1 - NUREG-0737 I.A.1.3.1, SHIFT MANNING OVERTIME LIMITS			02/28/84
<u>ACTIVE ACTIONS</u>				
46286	BROWNS FERRY 1 - NUREG-0737 III.A.2.2, METEOROLOGICAL DATA UPGRADE		1	<u>TARGET</u>
51143	BROWNS FERRY 1 - I.D.1 CONTROL ROOM DESIGN REVIEW			
51223	BROWNS FERRY 1 - I.D.2. SAFETY PARAMETER DISPLAY SYSTEM			
45680	BROWNS FERRY 1 - NUREG-0737 II.K.3.18, ADS ACTUATION STUDY		1	09/30/83
45584	BROWNS FERRY 1 - NUREG-0737 II.K.3.16, CHALLENGES AND FAILURES OF RELIEF VALVES		1	11/15/83
44832	BROWNS FERRY 1 - NUREG-0737 II.E.4.2, CONTAINMENT ISOLATION DEPENDABILITY		1	11/30/83
44214	BROWNS FERRY 1 - NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES		1	11/30/83

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(CONTINUATION)

BROWNS FERRY 1

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>TMI ACTIONS</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>ACTIVE ACTIONS</u> (CONTINUATION)				
44284	BROWNS FERRY 1 - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES		1	<u>TARGET</u> 11/30/83
44423	BROWNS FERRY 1 - NUREG-0737 II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS		1	12/31/83
48260	BROWNS FERRY 1 - NUREG 0737 II.K.3.28, VERIFY QUALIFICATION OF ACCUMULATORS		1	04/30/84
45116	BROWNS FERRY 1 - NUREG-0737 II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION		1	07/30/84
47849	BROWNS FERRY 1 - NUREG - 0737 II. 3.22B, RCIC SUCTION MODIFICATIONS		3	08/31/84
45927	BROWNS FERRY 1 - NUREG-0737 III.A.1.2, TECHNICAL SUPPORT CENTER		1	09/30/84
45998	BROWNS FERRY 1 - NUREG-0737 III.A.1.2, OPERATIONAL SUPPORT CENTER		1	09/30/84
46070	BROWNS FERRY 1 - NUREG-0737 III.A.1.2, EMERGENCY OPERATIONS FACILITY		1	09/30/84

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
52193	BROWNS FERRY 1 - CORE RELOAD (AMEND 91)			<u>COMPLETE</u> 12/01/83
52187	BROWNS FERRY 1 - TORUS MODIFICATIONS (AMEND 92)			12/12/83
52188	BROWNS FERRY 1 - MODIFICATION TO DRYWELL AIR SUPPLY			12/12/83
52189	BROWNS FERRY 1 - MODIFICATION TO PERSONNEL AIRLOCK DOORS			12/12/83
52190	BROWNS FERRY 1 - SCRAM DISCHARGE INSTRUMENT VOLUME MODIFICATION (AMEND 92)			12/12/83
52191	BROWNS FERRY 1 - ANALOG TRIP SYSTEM (AMEND 93)			12/16/83
51749	BROWN FERRY 1 - AUGUMENTED LEAK DETECTION REQUIREMENTS (AMEND 94).			12/27/83

<u>ACTIVE ACTIONS</u>				<u>TARGET</u>
53238	BROWNS FERRY 1 - GAP CONDUCTANCE CALCULATIONS			
53353	BROWNS FERRY 1 - REPORTING REQUIREMENTS OF 10 CFR 50.73			
53496	BROWNS FERRY 1 - RESPONSE ON II.K.3.22			
52699	BROWNS FERRY 2 - CORE RELOADS USING SRMS			
52341	BROWNS FERRY 1 - DEPTH OF WATER OVER FUEL			
52344	BROWNS FERRY 1 - SRO FOR FUEL HANDLING			
54629	BROWNS FERRY 1 - LOWERING MSIV SETPOINT			
54687	BROWNS FERRY 1 - REVISED PHYSICAL SECURITY PLAN			
54805	BROWNS FERRY UNIT 1, APRIL 9, 1984, TECH. SPEC. AMDT REQ. #TS-197			
54167	BROWNS FERRY 1 - ANALOG TRANSMITTER FAILURES			
52401	BROWNS FERRY 1 - AUDIT OF SAFEGUARDS PLAN			
52441	BROWNS FERRY 1 - NDT PVS T CURVES THRU 6 EFPY			
52444	BROWNS FERRY 1 - LIFE OF PLANT NDT CURVES			
52455	BROWNS FERRY 1 - AUDIT OF SECURITY PLAN/QA REVIEWS			
52505	BROWNS FERRY 1 - REPORTING ACTUATIONS OF SRVS			
52508	BROWNS FERRY 1 - SUPPRESSION CHAMBER DRAIN VALVES			
52576	BROWNS FERRY 1 - ROD BLOCK INSTRUMENTATION			
52579	BROWNS FERRY 1 - TURBINE CONTROL VALVE FAST CLOSURE			
52582	BROWNS FERRY 1 - ISOLATION DAMPERS, CONTROL ROOM			
49018	BROWNS FERRY 1 - UPDATE PRESSURE VS TEMPERATURE CURVE		1	08/15/83
48479	BROWNS FERRY 1 - ORGANIZATIONAL CHANGES			08/30/83
11569	BROWNS FERRY 1 - BOTTLED UP OPERATION		2	09/12/83
49984	BROWNS FERRY 1 - BYPASSING MAIN STEAM LINE TUNNEL TEMPERATURE SWITCHES			09/20/83
43698	BROWNS FERRY 1 - CAD SYSTEM VALVES		1	09/22/83
42805	BROWNS FERRY 1 - MAIN STEAM LINE SPACE HIGH TEMP ISOLATIONS		2	10/01/83
12611	BROWNS FERRY 1 - ISOLATION OF MAIN STEAM LINES ON HIGH TEMPERATURE		2	10/21/83
49011	BROWNS FERRY 1 - CUMULATIVE OUT OF SERVICE TIME FOR RBM (TS-176)		2	10/30/83
49014	BROWNS FERRY 1 - PERTURBATION OF REACTOR WATER LEVEL			10/30/83
49227	BROWNS FEERRY 1 - REVISED INSERVICE PUMP AND VALVE TESTING PROGRAM		1	11/11/83

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CONDENSED MANAGEMENT REPORT

(CONTINUATION)

BROWNS FERRY 1

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>ACTIVE ACTIONS</u> (CONTINUATION)				<u>TARGET</u>
11938	BROWNS FERRY 1,2,3- ISI AND T IMPLAEMENTATION DATES.		1	11/11/83
51752	BROWNS FERRY 1 - GROUP 7 ISOLATION VALVES			11/18/83
49224	BROWNS FERRY 1 - FREQUENCY OF MSIV TESTING		2	11/28/83
47455	BROWNS FERRY 1 - REVISED ISI PROGRAM		3	11/30/83
48476	BROWNS FERRY 1 - CRD MAINTENANCE REQUIREMENTS AND OTHER CHANGES TO T.S.			12/17/83
53352	BROWNS FERRY 1 - HYDRO TEST PRESSURE RELIEF			01/06/84
51859	BROWNS FERRY 1 - VERIFICATION TEST OF STATION DISTRIBUTION VOLTAGES			03/23/84
41011	BROWNS FERRY 1 - IREP		1	07/12/84
54518	BROWNS FERRY 1 - TECHNICAL SPECIFICATIONS COVERED BY GENERIC LETTERS 83-36, 83-37 FOR NUREG-0737			09/30/84
<u>ANTICIPATED ACTIONS</u>				<u>INITIATION</u>
53654	BROWNS FERRY 1 - ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS			11/01/84
53737	BROWNS FERRY 1- ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS			11/01/84
53820	BROWNS FERRY 1 - ITEM 3.2.3 - POST-MAINTENANCE TESTING -CHANGES TO TECH. SPEC. - ALL SR COMPONENTS			11/01/84
53964	BROWNS FERRY 1 - ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS			11/01/84
54047	BROWNS FERRY 1 - ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL TESTING - DIVERSE TRIP FEATURES			11/01/84
53571	BROWNS FERRY 1 - ITEM 1.2 - POST TRIP REVIEW-DATA AND INFORMATION CAPABILITY			11/01/84

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CONDENSED MANAGEMENT REPORT

FACILITY: BROWNS FERRY 2

PLANT LOCATION: 10 MI NW OF DECATUR, ALA
 SOCKET NUMBER: 050-00260
 ARCH/ENGINEER: TVA
 FIELD INSPECTOR: J. PAULK

LICENSED POWER: 3293 MWT
 DESIGN POWER: 1065 MWE
 NSSS VENDOR: GE

PROJECT MANAGER: R. CLARK
 BRANCH CHIEF: D. VASSALLO
 LIC. ASSISTANT: S. NORRIS

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>MULTI-PLANT</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
12296	BROWN'S FERRY 2 - HELB AND CONSEQUENTIAL SYSTEM FAILURE BROWN'S FERRY 2		3	<u>COMPLETE</u> 02/01/84
<u>ACTIVE ACTIONS</u>				
51074	BROWNS FERRY 2 - INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)			<u>TARGET</u>
52211	BROWNS FERRY 2 - CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPA#C-10)			
52393	BROWNS FERRY 2 - APPENDIX R EXEMPTION REQUESTS			
52740	BROWNS FERRY 2 - ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION AND PROCEDURES			
52821	BROWNS FERRY 2 - ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RTS COMPONENTS			
52902	BROWNS FERRY 2 - ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING PROCEDURES/VENDOR RECOMMENDATIONS - RTS COMPONENTS			
52983	BROWNS FERRY 2 - ITEM 3.1.3 - POST-MAINTENANCE TESTING-CHNGS TO TECH SPECS-RTS COMPONENTS		1	04/28/83
45653	BROWNS FERRY 2 - IMPLEMENTATION OF NUREG-0313, REV. 1		2	09/01/83
08716	BROWNS FERRY 2 - CONTAINMENT LEAK TESTING - APP. J (GENERIC)		1	11/11/83
11325	BROWNS FERRY 2 - INSERVICE TESTING		3	11/23/83
48132	BROWNS FERRY 2 - SINGLE LOOP OPERATION		2	12/30/83
07975	BROWNS FERRY 2 - CONTROL OF HEAVY LOADS OVER SPENT FUEL POOL (NUREG 0612)		1	02/17/84
08436	BROWNS FERRY 2 - BWR FEEDWATER & CONTROL ROD DRIVE NOZZLE CRACKLING		1	02/17/84
42569	BROWNS FERRY 2 - LONG TERM REVIEW CONTAINMENT PURGE AND VENT (B-24)		1	03/01/84
51670	BROWNS FERRY 2 - NUREG 0737 TECH SPECS		3	03/16/84
08134	BROWNS FERRY 2 - APPENDIX I T.S. IMPLEMENTATION REVIEW REVIEW		2	03/30/84
42874	BROWNS FERRY 2 - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11		1	03/30/84
43731	BROWNS FERRY 2 - SAFETY CONCERNS ASSOCIATED WITH PIPE BREAKS IN THE BWR SCRAM SYSTEM		1	06/30/84
07930	BROWNS FERRY 2 - MARK 1 CONTAINMENT LONG TERM PROGRAM IMPLEMENTATION		1	08/10/84
08930	BROWNS FERRY 2 - RPS POWER SUPPLY		1	09/07/84
42482	BROWNS FERRY 2 - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21)		1	09/30/84
48135	BROWNS FERRY 2 - FIRE PROTECTION APPENDIX R			
<u>TAC # TAC DESCRIPTION MULTI-PLANT PRIORITY CRITICAL DATE</u>				
<u>COMPLETED ACTIONS</u>				
44144	BROWNS FERRY 2 - NUREG-0737 I.A.2.1.4, UPGRADING OF RO AND SRO TRAINING		1	<u>COMPLETE</u> 10/12/83
44494	BROWNS FERRY 2 - NUREG-0737 II.B.4.1, TRAINING FOR MITIGATING CORE		1	10/12/83
48149	BROWNS FERRY 2 - NUREG 0737 II.K.3.31 COMPLIANCE WITH 10 CFR 50.46		1	12/28/83
45808	BROWNS FERRY 2 - NUREG-0737 II.K.3.30, SB LOCA OUTLINE		1	12/28/83
44073	BROWNS FERRY 2 - NUREG-0737 I.A.1.3.1, SHIFT MANNING OVERTIME LIMITS			02/28/84
<u>ACTIVE ACTIONS</u>				
51144	BROWNS FERRY 2 - I.D.1 CONTROL ROOM DESIGN REVIEW			<u>TARGET</u>
51224	BROWNS FERRY 2 - I.D.2. SAFETY PARAMETER DISPLAY SYSTEM		1	06/30/83
45681	BROWNS FERRY 2 - NUREG-0737 II.K.3.18, ADS ACTUATION STUDY		1	11/15/83
45585	BROWNS FERRY 2 - NUREG-0737 II.K.3.16, CHALLENGES AND FAILURES OF RELIEF VALVES		1	11/30/83
44833	BROWNS FERRY 2 - NUREG-0737 II.E.4.2, CONTAINMENT ISOLATION DEPENDABILITY		1	11/30/83
44215	BROWNS FERRY 2 - NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES		1	11/30/83
44285	BROWNS FERRY 2 - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES		1	12/31/83
44424	BROWNS FERRY 2 - NUREG-0737 II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS		1	

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(CONTINUATION)

BROWNS FERRY 2

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>IMI ACTIONS</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>ACTIVE ACTIONS</u> (CONTINUATION)				
48261	BROWNS FERRY 2	- NUREG 0737 II.K.3.28, VERIFY QUALIFICATION OF ACCUMULATORS ON ADS VALVES	1	<u>TARGET</u> 04/30/84
46287	BROWNS FERRY 2	- NUREG-0737 III.A.2.2, METEOROLOGICAL DATA UPGRADE	1	06/01/84
45117	BROWNS FERRY 2	- NUREG-0737 II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION	1	07/30/84
47850	BROWNS FERRY 2	- NUREG - 0737 II. 3.22B, RCIC SUCTION MODIFICATIONS	1	08/31/84
45928	BROWNS FERRY 2	- NUREG-0737 III.A.1.2, TECHNICAL SUPPORT CENTER	1	09/30/84
45999	BROWNS FERRY 2	- NUREG-0737 III.A.1.2, OPERATIONAL SUPPORT CENTER	1	09/30/84
46071	BROWNS FERRY 2	- NUREG-0737 III.A.1.2, EMERGENCY OPERATIONS FACILITY	1	09/30/84
<u>TAC #</u> <u>TA DESCRIPTION</u> <u>PLANT SPECIFIC</u> <u>PRIORITY</u> <u>CRITICAL DATE</u>				
<u>COMPLETED ACTIONS</u>				
51750	BROWNS FERRY 2	- AUGMENTED LEAK DETECTION REQUIREMENTS (AMEND 87)		<u>COMPLETE</u> 12/27/83
52348	BROWNS FERRY 2	- INCREASED CORE FLOW (AMEND 88)		03/06/84
52402	BROWNS FERRY 2	- AUDIT SAFEGUARDS PLAN (AMEND 89)		04/23/84
51753	BROWNS FERRY 2	- GROUP 7 ISOLATION VALVES (AMEND 90)		04/24/84
52580	BROWNS FERRY 2	- TURBINE CNTRL VALVE FAST CLOSURE (AMEND 91)		04/25/84
47456	BROWNS FERRY 2	- REVISED ISI PROGRAM (AMEND 92)	3	04/30/84
<u>ACTIVE ACTIONS</u> <u>TARGET</u>				
51860	BROWNS FERRY 2	- VERIFICATION TEST OF STATION DISTRIBUTION VOLTAGES		
52342	BROWNS FERRY 2	- DEPTH OF WATER OVER FUEL		
52345	BROWNS FERRY 2	- SRO FOR FUEL HANDLING		
52442	BROWNS FERRY 2	- NDT PVS T CURVES THRU 6 EPFY		
52445	BROWNS FERRY 2	- LIFE OF PLANT NDT CURVES		
52456	BROWNS FERRY 2	- AUDIT OF SECURITY PLANT/QA REVIEWS		
52583	BROWNS FERRY 2	- ISOLATION DAMPERS, CONTROL ROOM		
52700	BROWNS FERRY 2	- CORE RELOADS USING SRMS		
52506	BROWNS FERRY 2	- REPORTING ACTUATIONS OF SRVS		
54806	BROWNS FERRY UNIT 2,	APRIL 9, 1984, TECH. SPEC. AMDT REQ. #TS-197		
52509	BROWNS FERRY 2	- SUPPRESSION CHAMBER DRAIN VALVES		
52577	BROWNS FERRY 2	- ROD BLOCK INSTRUMENTATION		
53239	BROWNS FERRY 2	- GAP CONDUCTANCE CALCULATIONS		
53354	BROWNS FERRY 2	- REPORTING REQUIREMENTS OF 10 CFR 50.73		
53497	BROWNS FERRY 2	- RESPONSE ON II.K.3.22		
54630	BROWNS FERRY 2	- LOWERING MSIV SETPOINT		
54688	BROWNS FERRY 2	- REVISED PHYSICAL SECURITY PLAN		
54690	BROWNS FERRY 2	- ALLOWABLE RESPONSE TIME ON SDIV		
54791	BROWNS FERRY 2	- APPENDIX J TEST EXEMPTION		
51503	BROWNS FERRY 2	- INTEGRATED LEAK RATE TEST FAILURE		08/19/83
49321	BROWNS FERRY 2	- RECIRCULATION SYSTEM PIPE CRACKS	1	08/29/83
48480	BROWNS FERRY 2	- ORGANIZATIONAL CHANGES		08/30/83
11570	BROWNS FERRY 2	- BOTTLED UP OPERATION	2	09/12/83
42806	BROWNS FERRY 2	- MAIN STEAM LINE SPACE HIGH TEMP ISOLATIONS	2	10/01/83
12612	BROWNS FERRY 2	- ISOLATION OF MAIN STEAM LINES ON HIGH TEMPERATURE	2	10/21/83
46998	BROWNS FERRY 2	- FLOW BIASED PEAKING FACTOR	3	10/28/83
49012	BROWNS FERRY 2	- CUMULATIVE OUT OF SERVICE TIME FOR RBM (TS-176)	2	10/30/83
49015	BROWNS FERRY 2	- PERTURBATION OF REACTOR WATER LEVEL	2	10/30/83
49228	BROWNS FERRY 2	- REVISED INSERVICE PUMP AND VALVE TESTING PROGRAM	1	11/11/83
49985	BROWNS FERRY 2	- BYPASSING MAIN STEAM LINE TUNNEL TEMPERATURE SWITCHES		11/17/83
48477	BROWNS FERRY 2	- CRD MAINTENANCE REQUIREMENTS AND OTHER CHANGES TO T. S.		12/17/83

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CONDENSED MANAGEMENT REPORT

(CONTINUATION)

BROWNS FERRY 2

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>ACTIVE ACTIONS</u> (CONTINUATION)				<u>TARGET</u>
54519	BROWNS FERRY 2 - TECHNICAL SPECIFICATIONS COVERED BY GENERIC LETTERS 83-36, 83-37 FOR NUREG-0737			09/30/84
<u>ANTICIPATED ACTIONS</u>				<u>INITIATION</u>
53572	BROWNS FERRY 2 - ITEM 1.2 - POST TRIP REVIEW-DATA AND INFORMATION CAPABILITY			11/01/84
53655	BROWNS FERRY 2 - ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS			11/01/84
53738	BROWNS FERRY 2 - ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS			11/01/84
53821	BROWNS FERRY 2 - ITEM 3.2.3 - POST-MAINTENANCE TESTING -CHANGES TO TECH. SPEC. - ALL SR COMPONENTS			11/01/84
53965	BROWNS FERRY 2 - ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS			11/01/84
54048	BROWNS FERRY 2 - ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL TESTING - DIVERSE TRIP FEATURES			11/01/84

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CONDENSED MANAGEMENT REPORT

FACILITY: BROWNS FERRY 3

PLANT LOCATION: 10 MI NW OF DECATUR, ALA
 DOCKET NUMBER: 050-00296
 ARCH/ENGINEER: TVA
 IE INSPECTOR: J. PAULK

LICENSED POWER: 3293 MWT
 DESIGN POWER: 1065 MWE
 NSSS VENDOR: GE

PROJECT MANAGER: R. CLARK
 BRANCH CHIEF: D. VASSALLO
 LIC. ASSISTANT: S. NORRIS

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>MULTI-PLANT</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
42232	BROWNS FERRY 3 - BWR SCRAM DISCHARGE VOLUME CAPABILITY		1	<u>COMPLETE</u> 11/15/83
12297	BROWN'S FERRY 3 - HELB AND CONSEQUENTIAL SYSTEM FAILURE BROWN'S FERRY 3		3	02/01/84
<u>ACTIVE ACTIONS</u>				
51075	BROWNS FERRY 3 - INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)			<u>TARGET</u>
52212	BROWNS FERRY 3 - CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPA#C-10)			
52394	BROWNS FERRY 3 - APPENDIX R EXEMPTION REQUESTS			
52741	BROWNS FERRY 3 - ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION AND PROCEDURES			
52822	BROWNS FERRY 3 - ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RTS COMPONENTS			
52903	BROWNS FERRY 3 - ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING PROCEDURES/VENDOR RECOMMENDATIONS - RTS COMPONENTS			
52984	BROWNS FERRY 3-ITEM 3.1.3-POST-MAINTENANCE TESTING-CHNGS TO TECH SPECS-RTS COMPONENTS			
08717	BROWNS FERRY 3 - CONTAINMENT LEAK TESTING - APP. J (GENERIC)		2	09/01/83
46654	BROWNS FERRY 3 - IMPLEMENTATION OF NUREG-0313, REV. 1		2	09/28/83
42570	BROWNS FERRY 3 - LONG TERM REVIEW CONTAINMENT PURGE AND VENT (B-24)		1	09/30/83
11326	BROWNS FERRY 3 - INSERVICE TESTING		1	11/11/83
08438	BROWNS FERRY 3-CONTROL OF HEAVY LOADS OVER SPENT FUEL POOL (NUREG 0612)		2	12/30/83
08437	BROWNS FERRY 3 - BWR FEEDWATER & CONTROL ROD DRIVE NOZZLE CRACKING		1	02/17/84
51671	B'OWNS FERRY 3 NUREG 0737 TECH SPECS			03/01/84
08135	BROWNS FERRY 3 - APPENDIX I T.S. IMPLEMENTATION REVIEW REVIEW		3	03/16/84
42875	BROWNS FERRY 3 - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11		2	03/30/84
43736	BROWNS FERRY 3 - SAFETY CONCERNS ASSOCIATED WITH PIPE BREAKS IN THE BWR SCRAM SYSTEM		1	03/30/84
07931	BROWNS FERRY 3 - MARK I CONTAINMENT LONG TERM PROG. IMPLEMENT.		1	06/30/84
48136	BROWNS FERRY 3 - FIRE PROTECTION APPENDIX R		3	07/30/84
08931	BROWNS FERRY 3 - RPS POWER SUPPLY		1	08/10/84
42483	BROWNS FERRY 3 - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21)		1	09/07/84
48133	BROWNS FERRY 3 - SINGLE LOOP OPERATION		3	11/21/84
<u>TAC # TAC DESCRIPTION IMI ACTIONS PRIORITY CRITICAL DATE</u>				
<u>COMPLETED ACTIONS</u>				
44145	BROWNS FERRY 3 - NUREG-0737 I.A.2.1.4, UPGRADING OF RO AND SRO TRAINING		1	<u>COMPLETE</u> 10/12/83
44495	BROWNS FERRY 3 - NUREG-0737 II.B.4.1, TRAINING FOR MITIGATING CORE		1	10/12/83
45809	BROWNS FERRY 3 - NUREG-0737 II.K.3.30, SB LOCA OUTLINE		1	12/28/83
48150	BROWNS FERRY 3 -NUREG 0737 II.K.3.31 COMPLIANCE WITH 10 CFR 50.46			12/28/83
44074	BROWNS FERRY 3 - NUREG-0737 I.A.1.3.1, SHIFT MANNING OVERTIME LIMITS			02/28/84
<u>ACTIVE ACTIONS</u>				
51145	BROWNS FERRY 3 - I.D.1 CONTROL ROOM DESIGN REVIEW			<u>TARGET</u>
51225	BROWNS FERRY 3 - I.D.2. SAFETY PARAMETER DISPLAY SYSTEM			
45682	BROWNS FERRY 3 - NUREG-0737 II.K.3.18, ADS ACTUATION STUDY		1	09/30/83
45586	BROWNS FERRY 3 - NUREG-0737 II.K.3.16, CHALLENGES AND FAILURES OF RELIEF VALVES		1	11/15/83
44834	BROWNS FERRY 3 - NUREG-0737 II.E.4.2, CONTAINMENT ISOLATION DEPENDABILITY		1	11/30/83
44216	BROWNS FERRY 3 - NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES		1	11/30/83
44286	BROWNS FERRY 3 - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES		1	11/30/83

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BROWNS FERRY 3

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>IMI ACTIONS</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>ACTIVE ACTIONS</u> (CONTINUATION)				
44425	BROWNS FERRY 3	- NUREG-0737 II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS	1	<u>TARGET</u> 12/31/83
48262	BROWNS FERRY 3	- NUREG 0737 II.K.3.28, VERIFY QUALIFICATION OF ACCUMULATORS ON ADS VALVES	1	04/30/84
46288	BROWNS FERRY 3	- NUREG-0737 III.A.2.2, METEOROLOGICAL DATA UPGRADE	1	06/01/84
45118	BROWNS FERRY 3	- NUREG-0737 II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION	1	07/30/84
47851	BROWNS FERRY 3	- NUREG - 0737 II. 3.22B, RCIC SUCTION MODIFICATIONS	1	08/31/84
45929	BROWNS FERRY 3	- NUREG-0737 III.A.1.2, TECHNICAL SUPPORT CENTER	1	09/30/84
46000	BROWNS FERRY 3	- NUREG-0737 III.A.1.2, OPERATIONAL SUPPORT CENTER	1	09/30/84
46072	BROWNS FERRY 3	- NUREG-0737 III.A.1.2, EMERGENCY OPERATIONS FACILITY	1	09/30/84

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
51751	BROWNS FERRY 3	- AUGMENTED LEAK DETECTION REQUIREMENTS (AMEND 60)		<u>COMPLETE</u> 12/27/83
52349	BROWNS FERRY 3	- INCREASED CORE FLOW (AMEND 61)		03/06/84
52403	BROWNS FERRY 3	- AUDIT OF SAFEGUARDS PLAN (AMEND 62)		04/23/84
51754	BROWNS FERRY 3	- GROUP 7 ISOLATION VALVES		04/24/84
52581	BROWNS FERRY 3	-TURBINE CNTRL VALVE FAST CLOSURE (AMEND 64)		04/25/84

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>ACTIVE ACTIONS</u>				
52584	BROWNS FERRY 3	- ISOLATION DAMPERS, CONTROL ROOM		<u>TARGET</u>
52701	BROWNS FERRY 3	- CORE RELOADS USING SRMS		
52443	BROWNS FERRY 3	- NDT PVS T CURVES THRU 6 EFPY		
52446	BROWNS FERRY 3	- LIFE OF PLANT NDT CURVES		
52457	BROWNS FERRY 3	- AUDIT OF SECURITY PLAN/QA REVIEWS		
53240	BROWNS FERRY 3	- GAP CONDUCTANCE CALCULATIONS		
53355	BROWNS FERRY 3	- REPORTING REQUIREMENTS OF 10 CFR 50.73		
53498	BROWNS FERRY 3	- RSPONSE ON II.K.3.22		
53506	BROWNS FERRY 3	- CORE RELOAD		
54131	BROWNS FERRY 3	- THERMAL POWER MONITOR		
54132	BROWNS FERRY 3	- SDIV MODIFICATIONS		
54133	BROWNS FERRY 3	- MARK I TOROS MODIFICATIONS		
54134	BROWNS FERRY 3	- LPCI SUPPRESSION POOL COOLING		
54135	BROWNS FERRY 3	- DRYWELL AND SUPPRESSION CHAMBER COATINGS		
54136	BROWNS FERRY 3	- RPS POWER SUPPLY MODIFICATIONS		
54137	BROWNS FERRY 3	- ANALOG TRIP SYSTEM MODIFICATIONS		
54138	BROWNS FERRY 3	- BOTTLED-UP OPERATION SCRAMS		
54139	BROWNS FERRY 3	- ITEM II.F.1 MODS AND T.S.		
54140	BROWNS FERRY 3	- HYDROGEN ANALYZER VALVES		
54141	BROWNS FERRY 3	- TESTABLE PENETRATIONS		
54142	BROWNS FERRY 3	- DRYWELL AIR SUPPLY		
54143	BROWNS FERRY 3	- TORUS MAKEUP WATER SUPPLY		
52507	BROWNS FERRY 3	- REPORTING ACTUATIONS OF SRVS		
52510	BROWNS FERRY 3	- SUPPRESSION CHAMBER DRAIN VALVES		
52578	BROWNS FERRY 3	- ROD BLOCK INSTRUMENTATION		
52343	BROWNS FERRY 3	- DEPTH OF WATER OVER FUEL		
52346	BROWNS FERRY 3	- SRO FOR FUEL HANDLING		
54631	BROWNS FERRY 3	- LOWERING MSIV SETPOINT		
54689	BROWNS FERRY 3	- REVISED PHYSICAL SECURITY PLAN		
54807	BROWNS FERRY UNIT 2,	APRIL 9, 1984, TECH. SPEC. AMDT REQ. #TS-197		
54853	BROWNS FERRY 3	- EXEMPTION TO APPENDIX J TEST INTERNAL		

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(CONTINUATION)

BROWNS FERRY 3

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>ACTIVE ACTIONS</u> (CONTINUATION)				<u>TARGET</u>
49226	BROWNS FERRY 3 - FREQUENCY OF MSIV TESTING		2	07/30/83
48481	BROWNS FERRY 3 - ORGANIZATIONAL CHANGES			08/30/83
11571	BROWNS FERRY 3 - BOTTLED UP OPERATION		2	09/12/83
49986	BROWNS FERRY 3 - BYPASSING MAIN STEAM LINE TUNNEL TEMPERATURE SWITCHES			09/20/83
42807	BROWNS FERRY 3 - MAIN STEAM LINE SPACE HIGH TEMP ISOLATIONS		2	10/01/83
12613	BROWNS FERRY 3 - ISOLATION OF MAIN STEAM LINES ON HIGH TEMPERATURE		2	10/21/83
46999	BROWNS FERRY 3 - FLOW BIASED PEAKING FACTOR		3	10/28/83
49013	BROWNS FERRY 3 - CUMULATIVE OUT OF SERVICE TIME FOR RBM		2	10/30/83
49016	BROWNS FERRY 3 - PERTURBATION OF REACTOR WATER LEVEL		2	10/30/83
49229	BROWNS FERRY 3 - REVISED INSERVICE PUMP AND VALVE TESTING PROGRAM		1	11/30/83
48478	BROWNS FERRY 3 - CRD MAINTENANCE REQUIREMENTS AND OTHER CHANGE TO T.S.			12/17/83
47869	BROWNS FERRY - MSIV LEAKAGE			09/28/84
54520	BROWNS FERRY 3 - TECHNICAL SPECIFICATIONS COVERED BY GENERIC LETTERS 83-36, 83-37 FOR NUREG-0737			09/30/84
<u>ANTICIPATED ACTIONS</u>				<u>INITIATION</u>
53573	BROWNS FERRY 3 - ITEM 1.2 - POST TRIP REVIEW-DATA AND INFORMATION CAPABILITY			11/01/84
53656	BROWNS FERRY 3 - ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS			11/01/84
53739	BROWNS FERRY 3 - ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS			11/01/84
53822	BROWNS FERRY 3 - ITEM 3.2.3 - POST-MAINTENANCE TESTING -CHANGES TO TECH. SPEC. - ALL SR COMPONENTS			11/01/84
53966	BROWNS FERRY 3 - ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS			11/01/84
54049	BROWNS FERRY 3 - ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL TESTING - DIVERSE TRIP FEATURES			11/01/84

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CONDENSED MANAGEMENT REPORT

FACILITY: BRUNSWICK 1

PLANT LOCATION: 3 MI N OF SOUTHPORT, NC
 DOCKET NUMBER: 050-00325
 ARCH/ENGINEER: UEC
 IE INSPECTOR: D. JOHNSON

LICENSED POWER: 2436 MWT
 DESIGN POWER: 0821 MWE
 NSSS VENDOR: GE

PROJECT MANAGER: M. GROTENHUIS
 BRANCH CHIEF: D. VASSALLO
 LIC. ASSISTANT: S. NORRIS

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>MULTI-PLANT</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
				<u>COMPLETE</u>
<u>COMPLETED ACTIONS</u>				
42571	BRUNSWICK 1 - LONG TERM REVIEW CONTAINMENT PURGE AND VENT (B-24)		1	12/01/83
08106	BRUNSWICK 1 - APPENDIX I TECH SPEC IMPLEMENTATION REVIEW		2	12/27/83
12186	BRUNSWICK 1 - HIGH ENERGY LINE BREAK & CONSEQUENTIAL SYSTEM FAILURE		3	01/31/84
07932	BRUNSWICK 1 - MARK CONTAIN. LONG TERM PROGRAM IMPLEMENTATION		1	03/19/84
51673	BRUNSWICK 1 NUREG 0737 TECH SPECS		2	03/28/84
46655	BRUNSWICK 1 - IMPLEMENTATION OF NUREG-0313, REV. 1		1	03/31/84
				<u>TARGET</u>
<u>ACTIVE ACTIONS</u>				
48489	BRUNSWICK 1 - SINGLE LOOP OPERATION			
52742	BRUNSWICK 1 - ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION AND PROCEDURES			
52823	BRUNSWICK 1 - ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RTS COMPONENTS			
52904	BRUNSWICK 1 - ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING PROCEDURES/VENDOR RECOMMENDATIONS - RTS COMPONENTS			
52985	BRUNSWICK 1-ITEM 3.1.3 -POST-MAINTENANCE TESTING-CHANGES TO TECH SPECS-RTS COMPONENTS			
11264	BRUNSWICK 1 - 10 CFR 50.55A(G) IST PUMPS/VALVES		1	05/15/84
42484	BRUNSWICK 1 - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21)		1	05/22/84
07976	BRUNSWICK 1-CONTROL OF HEAVY LOADS OVER SPENT FUEL POOL (NUREG 0612)		3	05/31/84
42876	BRUNSWICK 1 - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11		2	06/30/84
51076	BRUNSWICK 1 - INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)			06/30/84
43418	BRUNSWICK 1 - NEW APPENDIX R TO 10 CFR 50 REGARDING FIRE PROTECTION FEATURES		3	07/01/84
43737	BRUNSWICK 1 - SAFETY CONCERNS ASSOCIATED WITH PIPE BREAKS IN THE BWR SCRAM SYSTEM		1	10/31/84
12829	BRUNSWICK 1 - ADEQUACY OF STATION ELECTRIC DISTRIBUTION SYSTEMS VOLTAGES		2	10/31/84
52213	BRUNSWICK 1 - CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPA#C-10)		3	09/30/85
				<u>TARGET</u>
<u>COMPLETED ACTIONS</u>				
45614	BRUNSWICK 1 - NUREG-0737 II.K.3.17, ECC SYSTEM OUTAGES		1	10/17/83
46431	BRUNSWICK 1 - NUREG-0737 III.D.3.4, CONTROL ROOM HABITABILITY		1	10/18/83
44527	BRUNSWICK 1 - II.D.1 RV AND SV TESTING		1	10/31/83
48151	BRUNSWICK 1 -NUREG 0737 II.K.3.31 COMPLIANCE WITH 10 CFR 50.46		2	12/15/83
45810	BRUNSWICK 1 - NUREG-0737 II.K.3.30, SB LOCA OUTLINE		1	01/04/84
				<u>TARGET</u>
<u>ACTIVE ACTIONS</u>				
45930	BRUNSWICK 1 - NUREG-0737 III.A.1.2, TECHNICAL SUPPORT CENTER		3	
46001	BRUNSWICK 1 - NUREG-0737 III.A.1.2, OPERATIONAL SUPPORT CENTER		3	
46073	BRUNSWICK 1 - NUREG-0737 III.A.1.2, EMERGENCY OPERATIONS FACILITY		3	
46289	BRUNSWICK 1 - NUREG-0737 III.A.2.2, METEOROLOGICAL DATA UPGRADE		1	
45119	BRUNSWICK 1 - NUREG-0737 II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION		2	
51226	BRUNSWICK 1 - I.D.2. SAFETY PARAMETER DISPLAY SYSTEM		3	
44217	BRUNSWICK 1 - NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES		1	
44287	BRUNSWICK 1 - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES		1	
47656	BRUNSWICK 1 - NUREG 0737 ITEM II F1.5 CONTAINMENT WATER LEVEL MONITOR		2	04/30/84
47727	BRUNSWICK 1 - NUREG 0737 ITEM II F1.6 CONTAINMENT HYDROGEN MONITOR		2	04/30/84
47585	BRUNSWICK 1 - NUREG 0737 ITEM II F1.4 - CONTAINMENT PRESSURE INSTRUMENT		2	05/30/84

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CONDENSED MANAGEMENT REPORT

(CONTINUATION)

BRUNSWICK 1

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>TMI ACTIONS</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>ACTIVE ACTIONS (CONTINUATION)</u>				
44426	BRUNSWICK 1 - NUREG-0737 II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS		1	<u>TARGET</u> 06/30/84
45683	BRUNSWICK 1 - NUREG-0737 II.K.3.18, ADS ACTUATION STUDY		1	06/30/84
45587	BRUNSWICK 1 - NUREG-0737 II.K.3.16, CHALLENGES AND FAILURES OF RELIEF VALVES		2	07/17/84
48263	BRUNSWICK 1 - NUREG 0737 II.K.3.28, VERIFY QUALIFICATION OF ACCUMULATORS ON ADS VALVES		2	10/31/84
51146	BRUNSWICK 1 - I.D.1 CONTROL ROOM DESIGN REVIEW		3	10/31/84
<u>COMPLETED ACTIONS</u>				
49273	BRUNSWICK 1 - ISOLATION ACTUATION INSTRUMENTATION			<u>COMPLETE</u> 11/28/83
51498	BRUNSWICK 1 - SALP REVIEW			12/01/83
49489	BRUNSWICK 1 - FIVE MICELLANEOUS TECH. SPECS.			12/02/83
52628	BRUNSWICK 1 - INTEGRATED LEAK RATE TEST - SHORT DURATION		2	12/09/83
49585	BRUNSWICK 1 - SECOND DIGITAL ANALOG TS CHANGE			12/12/83
43796	BRUNSWICK 1 - SPENT FUEL POOL EXPANSION		1	12/15/83
52610	BRUNSWICK 1 - REVISED RETS		2	12/27/83
48754	BRUNSWICK 1 - RPS MONITORING SYSTEM TS		1	12/27/83
48635	BRUNSWICK 1 - SDV INSTRUMENT MODIFICATIONS		1	12/28/83
48637	BRUNSWICK 1 - CRD RETURN LINE SNUBBERS		2	12/28/83
52486	BRUNSWICK 1 - INTEGRATED LEAK RATE TEST		1	01/11/84
52536	BRUNSWICK 1 - FEEDWATER & TURBINE BYPASS TS		3	01/18/84
47038	BRUNSWICK 1 - RADIATION AREA INTERPRETATION REQUEST		2	01/27/84
48812	BRUNSWICK 1 - DG LCO EXTENSION REQUEST		3	02/14/84
48822	BRUNSWICK 1 - MISCELLANEOUS TECHNICAL SPECIFICATIONS		2	02/27/84
53504	BRUNSWICK 1 OPERATOR REQUAL PROGRAM REV 8			02/28/84
49331	BRUNSWICK 1 - REACTIVITY ANOMALIES		2	04/17/84
<u>ACTIVE ACTIONS</u>				
49775	BRUNSWICK 1: ADMINISTRATIVE CONTROLS TSC2		2	<u>TARGET</u> 03/31/84
46768	BRUNSWICK 1 - ADMINISTRATIVE CONTROLS TECHNICAL SPECIFICATION CHANGE		2	03/31/84
52537	BRUNSWICK 1 - RESPONSE TO 7/22/83 CONF. ORDER			03/31/84
54190	BRUNSWICK 1 - SEALING FACTORS - RADWATER			04/30/84
54170	BRUNSWICK UNIT 1 - FUEL ENRICHMENT			05/01/84
54254	BRUNSWICK 1 - EXEMPTION - ANNUAL EMERGENCY EXERCISE			05/06/84
51816	BRUNSWICK 1 THERMAL - HYDRAULIC CODE		3	05/20/84
54270	BRUNSWICK 1 - 1983 INSERVICE INSPECTION REPORT			06/15/84
53233	BRUNSWICK 1 - DISPOSAL OF WASTE OIL			06/21/84
54840	BRUNSWICK 1 - CONTAINMENT INTEGRITY - MSIV CLOSURE			06/30/84
54961	BRUNSWICK 1 - ROYCHAN CABLE TEST			06/30/84
54354	BRUNSWICK 1 PURGE AND VENT OPERABILITY			09/30/84
54521	BRUNSWICK 1 - TECHNICAL SPECIFICATIONS COVERED BY GENERIC LETTERS 83-36, 83-37 FOR NUREG-0737			09/30/84
54193	BRUNSWICK 1 - DEGRADED VOLTAGE RITAG T/S			10/31/84
49276	BRUNSWICK 1 - ELECTRICAL DISTRIBUTION VOLTAGE		3	10/31/84
<u>ANTICIPATED ACTIONS</u>				
54964	BRUNSWICK 1 - REQUEST FOR SCHEDULAR EXEMPTION FROM H CONTROL			<u>INITIATION</u> 05/07/84
54999	BRUNSWICK 1 - DELETE SNUBBER TABLE			05/07/84
53574	BRUNSWICK 1 - ITEM 1.2 - POST TRIP REVIEW-DATA AND INFORMATION CAPABILITY			11/01/84
53657	BRUNSWICK 1 - ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS			11/01/84
53740	BRUNSWICK 1- ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS			11/01/84

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CONDENSED MANAGEMENT REPORT

(CONTINUATION)

BRUNSWICK 1

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>ANTICIPATED ACTIONS (CONTINUATION)</u>				
53823	BRUNSWICK 1 - ITEM 3.2.3 - POST-MAINTENANCE TESTING - CHANGES TO TECH. SPEC. - ALL SR COMPONENTS			<u>INITIATION</u> 11/01/84
53967	BRUNSWICK 1 - ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS			11/01/84
54050	BRUNSWICK 1 - ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL TESTING - DIVERSE TRIP FEATURES			11/01/84

DATA AS OF - 04/30/84

CONDENSED MANAGEMENT REPORT

FACILITY: BRUNSWICK 2

PLANT LOCATION: 3 MI N OF SOUTHPORT, NC
 DOCKET NUMBER: 050-00324
 ARCH/ENGINEER: UEC
 IE INSPECTOR: D. JOHNSON

LICENSED POWER: 2436 MWT
 DESIGN POWER: 0821 MWE
 NSSS VENDOR: GE

PROJECT MANAGER: M. GROTENHUIS
 BRANCH CHIEF: D. VASSALLO
 LIC. ASSISTANT: S. NORRIS

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>MULTI-PLANT</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
42572	BRUNSWICK 2 - LONG TERM REVIEW CONTAINMENT PURGE AND VENT (B-24)		1	<u>COMPLETE</u> 12/01/83
10906	BRUNSWICK 2 - APPENDIX I TECH SPEC IMPLEMENTATION		2	12/27/83
12187	BRUNSWICK 2 - HIGH ENERGY LINE BREAK & CONSEQUENTIAL SYSTEM FAILURE		3	01/31/84
46656	BRUNSWICK 2 - IMPLEMENTATION OF NUREG-0313, REV. 1		1	03/13/84
07933	BRUNSWICK 2 - MARK I CONTAINMENT LONG TERM PROGRAM IMPLEMEN.		1	03/19/84
51672	BRUNSWICK 1 NUREG 0737 TECH SPECS		2	03/28/84
<u>ACTIVE ACTIONS</u>				
52743	BRUNSWICK 2 - ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION AND PROCEDURES			<u>TARGET</u>
52824	BRUNSWICK 2 - ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RTS COMPONENTS			
52905	BRUNSWICK 2 - ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING PROCEDURES/VENDOR RECOMMENDATIONS - RTS COMPONENTS			
52986	BRUNSWICK 2-ITEM 3.1.3 -POST-MAINTENANCE TESTING-CHANGES TO TECH SPECS-RTS COMPONENTS			
46779	BRUNSWICK 2 - SINGLE LOOP OPERATION		2	04/15/84
07977	BRUNSWICK 2 -CONTROL OF HEAVY LOADS OVER SPENT FUEL POOL (NUREG 0612)		3	05/03/84
11265	BRUNSWICK 2 - INSERVICE TESTING AND INSPECTION (IST) PUMPS/VALVES		1	05/15/84
42485	BRUNSWICK 2 - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21)		1	05/22/84
12830	BRUNSWICK 2 - ADEQUACY OF STATION ELECTRIC DISTRIBUTION SYSTEMS VOLTAGES		2	05/31/84
42877	BRUNSWICK 2 - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11		2	06/03/84
51077	BRUNSWICK 2 - INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)			06/30/84
43417	BRUNSWICK 2 - NEW APPENDIX R TO 10 CFR 50 REGARDING FIRE PROTECTION FEATURES		3	07/01/84
43735	BRUNSWICK 2 - SAFETY CONCERNS ASSOCIATED WITH PIPE BREAKS IN THE BWR SCRAM SYSTEM		1	10/31/84
52214	BRUNSWICK 2 - CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPA#C-10)		3	09/30/85
<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>TMI ACTIONS</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
45615	BRUNSWICK 2 - NUREG-0737 II.K.3.17, ECC SYSTEM OUTAGES		1	<u>COMPLETE</u> 10/17/83
46432	BRUNSWICK 2 - NUREG-0737 III.D.3.4, CONTROL ROOM HABITABILITY		1	10/18/83
44568	BRUNSWICK 2 - RV AND SV TESTING		1	10/31/83
48152	BRUNSWICK 2 -NUREG 0737 II.K.3.31 COMPLIANCE WITH 10 CFR 50.46		2	12/15/83
45811	BRUNSWICK 2 - NUREG-0737 II.K.3.30, SB LOCA OUTLINE		1	01/04/84
<u>ACTIVE ACTIONS</u>				
45931	BRUNSWICK 2 - NUREG-0737 III.A.1.2, TECHNICAL SUPPORT CENTER		3	
46002	BRUNSWICK 2 - NUREG-0737 III.A.1.2, OPERATIONAL SUPPORT CENTER		3	
46074	BRUNSWICK 2 - NUREG-0737 III.A.1.2, EMERGENCY OPERATIONS FACILITY		3	
46290	BRUNSWICK 2 - NUREG-0737 III.A.2.2, METEOROLOGICAL DATA UPGRADE		1	
45120	BRUNSWICK 2 - NUREG-0737 II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION		2	
44218	BRUNSWICK 2 - NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES		1	
44288	BRUNSWICK 2 - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES		1	
51227	BRUNSWICK 2 - I.D.2. SAFETY PARAMETER DISPLAY SYSTEM		3	03/15/84
47586	BRUNSWICK 2 - NUREG 0737 ITEM II F1.4 - CONTAINMENT PRESSURE INSTRUMENT		2	04/30/84
47657	BRUNSWICK 2 - NUREG 0737 ITEM II F1.5 CONTAINMENT WATER LEVEL MONITOR		2	04/30/84
47728	BRUNSWICK 2 - NUREG 0737 ITEM II F1.6 CONTAINMENT HYDROGEN MONITOR		2	04/30/84

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CONDENSED MANAGEMENT REPORT

(CONTINUATION)

BRUNSWICK 2

TAC #	TAC DESCRIPTION	TMI ACTIONS	PRIORITY	CRITICAL DATE
<u>ACTIVE ACTIONS</u>		(CONTINUATION)		<u>TARGET</u>
45684	BRUNSWICK 2 - NUREG-0737 II.K.3.18, ADS ACTUATION STUDY		1	06/30/84
44427	BRUNSWICK 2 - NUREG-0737 II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS		1	06/30/84
45588	BRUNSWICK 2 - NUREG-0737 II.K.3.16, CHALLENGES AND FAILURES OF RELIEF VALVES		2	07/17/84
48264	BRUNSWICK 2 - NUREG 0737 II.K.3.28, VERIFY QUALIFICATION OF ACCUMULATORS ON ADS VALVES		2	10/31/84
51147	BRUNSWICK 2 - I.D.1 CONTROL ROOM DESIGN REVIEW		3	10/31/84
TAC #	TAC DESCRIPTION	PLANT SPECIFIC	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				<u>COMPLETE</u>
52587	BRUNSWICK 2 - ORDER MODIFICATION REQUEST PIPE CRACK			10/28/83
43695	BRUNSWICK 2 - INSERVICE SURVEILLANCE REQUIREMENTS FOR SNUBBERS		1	11/08/83
52601	BRUNSWICK 2 - PIPE CREEK ORDER MODIFICATION REQUEST 2 - SHORT RANGE			11/22/83
52602	BRUNSWICK 2 - PIPE CRACK ORDER MODIFICATIONS REQUEST 3 - LONG RANGE			11/22/83
52078	BRUNSWICK 2 - REVISED MCPR			11/28/83
49274	BRUNSWICK 2 - ISOLATION ACTUATION INSTRUMENTATION			11/28/83
51313	BRUNSWICK 2 - TMI ORDER AMENDMENT		1	12/01/83
49490	BRUNSWICK 2 - FIVE MICELLANEOUS TECH. SPECS.		2	12/02/83
52629	BRUNSWICK 2 INTEGRATED LEAK RATE TEST SHORT DURATION		2	12/09/83
52470	BRUNSWICK 2 - AUGMENTED OFF-GAS SCHEDULE		1	12/12/83
52538	BRUNSWICK 2 - REPSONSE TO AUG. 16, 1983 ORDER		1	12/13/83
43797	BRUNSWICK 2 - SPENT FUEL POOL EXPANSION		1	12/15/83
52611	BRUNSWICK 2 - REVISED RETS		2	12/27/83
48636	BRUNSWICK 2 - SDV INSTRUMENT MODIFICATIONS		1	12/28/83
48638	BRUNSWICK 2 - CRD RETURN LINE SNUBBERS		2	12/28/83
48755	BRUNSWICK 2 -RPS MONITORING SYSTEM TS		1	12/28/83
53292	BRUNSWICK 2 - BATTERY SURVEILLANCE REQUIREMENTS			12/31/83
52487	BRUNSWICK 2 - INTEGRATED LEAK RATE TEST		1	01/11/84
52535	BRUNSWICK 2 - FEEDWATER & TURBINE BYPASS TS		3	01/18/84
53293	BRUNSWICK 2 - RELIEF FROM HYDRO TEST			01/27/84
47039	BRUNSWICK 2 - RADIATION AREA INTERPRETATION REQUEST		2	01/27/84
48813	BRUNSWICK 2 - DG LCD EXTENSION REQUEST		3	02/14/84
48823	BRUNSWICK 2 - MISCELLANEOUS TECH SPEC REVISIONS		2	02/27/84
53505	BRUNSWICK 2 OPERATOR REQUAL PROGRAM REV 8			02/28/84
<u>ACTIVE ACT</u>				<u>TARGET</u>
06171	BRUNSWICK 2 - SINGLE LOOP OPERATION		2	
46767	BRUNSWICK 2 - ADMINISTRATIVE CONTROLS TECHNICAL SPECIFICATION CHANGE		2	03/31/84
49774	BRUNSWICK 2 : ADMINISTRATIVE CONTROLS TSC2		2	03/31/84
49332	BRUNSWICK 2 - REACTIVITY ANOMALIES		2	04/28/84
54191	BRUNSWICK 2 - SEALING FACTORS - RADWASTE			04/30/84
54147	BRUNSWICK 2 - BATTERY CAPACITY			04/31/84
53470	BRUNSWICK 2 CORE SPRAY SYSTEM SURVEILLANCE			05/01/84
54255	BRUNSWICK 2 - EXEMPTION - ANNUAL EMERGENCY EXERCISE			05/06/84
49586	BRUNSWICK 2 - SECOND DIGITAL ANALOG TS CHANGE		1	05/20/84
49922	BRUNSWICK 2 - FUEL BURNUP CODE		2	05/20/84
49923	BRUNSWICK 2 - CORE SIMULATOR CODE		2	05/20/84
49924	BRUNSWICK 2 - ANALYSIS METHODS		2	05/20/84
51817	BRUNSWICK 2 THERMAL - HYDRAULIC CODE		2	05/20/84
53234	BRUNSWICK 2 - DISPOSAL OF WASTE OIL			06/21/84
54839	BRUNSWICK 2 - CONTAINMENT INTEGRITY - MSIV CLOSURE			06/30/84

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CONDENSED MANAGEMENT REPORT

(CONTINUATION)

BRUNSWICK 2

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>ACTIVE ACTIONS</u> (CONTINUATION)				<u>TARGET</u>
54962	BRUNSWICK 2 - ROYCHAN CABLE TEST			06/30/84
54966	BRUNSWICK 2 - EQ EXTENSION REQUEST			06/30/84
54355	BRUNSWICK 2 PURGE AND VENT OPERABILITY			09/30/84
54522	BRUNSWICK 2 - TECHNICAL SPECIFICATIONS COVERED BY GENERIC LETTERS 83-36, 83-37 FOR NUREG-0737			09/30/84
54194	BRUNSWICK 2 - DEGRADED VOLTAGE RITAG T/S			10/31/84
49277	BRUNSWICK 2 - ELECTRICAL DISTRIBUTION VOLTAGE		3	10/31/84
<u>ANTICIPATED ACTIONS</u>				<u>INITIATION</u>
54965	BRUNSWICK 2 - REQUEST FOR SCHEDULAR EXEMPTION FROM H CONTROL			05/07/84
55000	BRUNSWICK 2 - DELET SHUBBER TABLE			05/07/84
55001	BRUNSWICK 1 - CHECK VALVE SURVEILLANCE EXTENSION			05/10/84
53575	BRUNSWICK 2 - ITEM 1.2 - POST TRIP REVIEW-DATA AND INFORMATION CAPABILITY			11/01/84
53658	BRUNSWICK 2 - ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS			11/01/84
53741	BRUNSWICK 2- ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS			11/01/84
53824	BRUNSWICK 2 - ITEM 3.2.3 - POST-MAINTENANCE TESTING -CHANGES TO TECH. SPEC. - ALL SR COMPONENTS			11/01/84
53968	BRUNSWICK 2 - ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS			11/01/84
54051	BRUNSWICK 2 - ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL TESTING - DIVERSE TRIP FEATURES			11/01/84

DATA AS OF - 04/30/84

CONDENSED MANAGEMENT REPORT

FACILITY: CALVERT CLIFFS 1

PLANT LOCATION: 40 MI S OF ANNAPOLIS, MD
 DOCKET NUMBER: 050-00317
 ARCH/ENGINEER: BECH
 IE INSPECTOR: R. ARCHITZEL

LICENSED POWER: 2700 MWT
 DESIGN POWER: 0845 MWE
 NSSS VENDOR: COMB

PROJECT MANAGER: D. JAFFE
 BRANCH CHIEF: J. MILLER
 LIC. ASSISTANT: P. KREUTZER

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>MULTI-PLANT</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
12314	CALVERT CLIFFS 1 - HELB AND CONSEQUENTIAL SYSTEM FAILURE CALVERT CLIFFS 1		3	<u>COMPLETE</u> 01/31/84
43634	CALVERT CLIFFS 1 - SEISMIC QUALIFICATION OF THE AUXILIARY FEEDWATER SYSTEM		1	03/21/84
<u>ACTIVE ACTIONS</u>				
51078	CALVERT CLIFFS 1 - INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)			<u>TARGET</u>
52215	CALVERT CLIFFS 1 - CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPA#C-10)			
52744	CALVERT CLIFFS 1 - ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION AND PROCEDURES			
52825	CALVERT CLIFFS 1 - ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RTS COMPONENTS			
52906	CALVERT CLIFFS 1 - ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING PROCEDURES/VENDOR RECOMMENDATIONS - RTS COMPONENTS			
52987	CALVERT CLIFFS 1-ITEM 3.1.3-POST-MAINT TESTING-CHANGES TO TECH SPECS-RTS COMPONENTS			
53062	CALVERT CLIFFS 1 - REACTOR TRIP SYSTEM RELIABILITY- VENDOR RELATED MODIFICATIONS			
53115	CALVERT CLIFFS 1 - ITEMS 4.2.1 & 4.2.2- PREVENTATIVE MAINT PROGRAM FOR REACTOR TRIP BREAKERS - MAINTENANCE & TRENDING			
42494	CALVERT CLIFFS 1 - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21)		1	06/30/84
42890	CALVERT CLIFFS 1 - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11		2	06/30/84
49674	CALVERT CLIFFS 1 - REACTOR COOLANT PUMP TRIP		1	07/01/84
08136	CALVERT CLIFFS 1 - APPENDIX I TECH. SPEC IMPLEMENTATION REVIEW		3	12/01/84
08951	CALVERT CLIFFS 1 - REVIEW OF ASYMMETRIC LOCA LOADS		2	12/30/84
<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>TMI ACTIONS</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
45260	CALVERT CLIFFS 1 - NUREG-0737 II.K.3.1, AUTO PORV ISOLATION		1	<u>COMPLETE</u> 10/06/83
45306	CALVERT CLIFFS 1 - NUREG-0737 II.K.3.2, REPORT ON PORV FAILURES		1	10/06/83
47793	CALVERT CLIFFS 1 - NUREG 0737 ITEM II K2.17 POTENTIAL FOR VOIDING IN RCS		1	10/24/83
<u>ACTIVE ACTIONS</u>				
51228	CALVERT CLIFFS 1 - I.D.2. SAFETY PARAMETER DISPLAY SYSTEM			<u>TARGET</u>
44569	CALVERT CLIFFS 1 - RV AND SV TESTING		1	06/15/84
45121	CALVERT CLIFFS 1 - NUREG-0737 II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION		1	06/30/84
44428	CALVERT CLIFFS 1 - NUREG-0737 II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS		1	06/30/84
47926	CALVERT CLIFFS 1-PLANT SHIELDING MODIFICATIONS (II.B.2.2) REGIONAL REVIEW		1	06/30/84
48153	CALVERT CLIFFS 1 -NUREG 0737 II.K.3.31 COMPLIANCE WITH 10 CFR 50.46		1	06/30/84
45812	CALVERT CLIFFS 1 - NUREG-0737 II.K.3.30, SB LOCA OUTLINE		1	06/30/84
45932	CALVERT CLIFFS 1 - NUREG-0737 III.A.1.2, TECHNICAL SUPPORT CENTER		1	09/30/84
46003	CALVERT CLIFFS 1 - NUREG-0737 III.A.1.2, OPERATIONAL SUPPORT CENTER		1	09/30/84
46075	CALVERT CLIFFS 1 - NUREG-0737 III.A.1.2, EMERGENCY OPERATIONS FACILITY		1	09/30/84
46291	CALVERT CLIFFS 1 - NUREG-0737 III.A.2.2, METEOROLOGICAL DATA UPGRADE		1	09/30/84
46879	CALVERT CLIFFS 1 - II K2.13 THERMAL MECHANICAL REPORT		1	09/30/84
44219	CALVERT CLIFFS 1 - NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES		1	11/30/84
44289	CALVERT CLIFFS 1 - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES		1	11/30/84
51148	CALVERT CLIFFS 1 - I.D.1 CONTROL ROOM DESIGN REVIEW			12/01/84
48443	CALVERT CLIFFS 1 - NUREG 0737, II.K.3.25 POWER ON PUMP SEALS			12/30/84

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CONDENSED MANAGEMENT REPORT

(CONTINUATION)

CALVERT CLIFFS 1

TAC #	TAC DESCRIPTION	PLANT SPECIFIC	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				<u>COMPLETE</u>
52157	CALVERT CLIFFS 1 - CYCLE 7 RELOAD		1	11/17/83
52422	CALVERT CLIFFS UNIT 1 - CHANGE TO T.S. FOR CSAS A-3 AND B-3		1	11/17/83
52431	CALVERT CLIFFS UNIT 1 - TS FOR CONTAINMENT VENT (MOV-6900 & 6901)			11/17/83
52432	CALVERT CLIFFS UNIT 1 - CHANGE TO TS FOR REMOTE SHUTDOWN INSTR.			11/17/83
52433	CALVERT CLIFFS UNIT 1 - CHANGE TO TS FOR POST ACCIDENT MONITORING			11/17/83
52434	CALVERT CLIFFS UNIT 1 - FINAL AFW MODIFICATIONS			11/17/83
52568	CALVERT CLIFFS UNIT 1 - CONTROL ROOM HVAC COOLER TS		1	12/30/83
52570	CALVERT CLIFFS UNIT 1 - CORRECT CONTAINMENT CAM TS		1	12/30/83
52430	CALVERT CLIFFS UNIT 1 - DELETION OF SNUBBER FROM TS		1	12/30/83
52429	CALVERT CLIFFS UNIT 1 - CHANGE TS SNUBBER NUMBER			12/30/83
54179	CALVERT CLIFFS UNIT 1 - MODIFICATION OF ORDER			03/02/84
53275	CALVERT CLIFFS UNIT 1 - APPENDIX R EXEMPTION - (OIL)		1	03/15/84
53276	CALVERT CLIFFS UNIT 1 - APPENDIX R EXEMPTION - (DOORS)		1	03/15/84
53277	CALVERT CLIFFS UNIT 1 - APPENDIX R EXEMPTION (SPRINKLER)		1	03/15/84
52424	CALVERT CLIFFS UNIT 1 - REED SWITCH TS		1	03/16/84
54622	CALVERT CLIFFS UNIT 1 - EMERGENCY TS CHANGE			04/16/84
53368	CALVERT CLIFFS UNIT 1 - CHANGE TO TS FOR 69KV SMECO LINE			04/19/84
53369	CALVERT CLIFFS UNIT 1 - CHANGE TO TS FOR 125VDC BATTERY			04/19/84
53370	CALVERT CLIFFS UNIT 1 - CHANGE TO TS FOR RCS VENT			04/19/84
53366	CALVERT CLIFFS UNIT 1 - TS FOR FUEL OIL TANK INSPECTION			04/19/84
<u>ACTIVE ACTIONS</u>				<u>TARGET</u>
52426	CALVERT CLIFFS UNIT 1 - TS FOR CONTAINMENT COOLING		1	04/15/84
53365	CALVERT CLIFFS UNIT 1 - TS FOR HYDROGEN PURGE SYSTEM			06/01/84
54625	CALVERT CLIFFS UNIT 1 - CHANGE TO ISI INTERVAL			06/01/84
54378	CALVERT CLIFFS UNIT 1 - EMERGENCY EXERCISE FREQUENCY			06/03/84
54774	CALVERT CLIFFS UNIT 1 - ORGANIZATION CHANGE			07/01/84
54776	CALVERT CLIFFS UNIT 1 - CHANGE TO ETS - REPORTING			07/01/84
54777	CALVERT CLIFFS UNIT 1 - FIRE PUMP SUREVILLANCE			07/01/84
54778	CALVERT CLIFFS UNIT 1 - FIRE DETECTION INSTRUMENTS			07/01/84
54779	CALVERT CLIFFS UNIT 1 - HIGH RAD ACCESS TS			07/01/84
54975	CALVERT CLIFFS UNIT 1 - CONTROL OF RAD AREAS			07/01/84
54976	CALVERT CLIFFS UNIT 1 - CHANGE TO ORG. TS			07/01/84
54977	CALVERT CLIFFS UNIT 1 - FIRE PUMP TESTING			07/01/84
54978	CALVERT CLIFFS UNIT 1 - TECHNICAL SPECIFICATION FOR FIRE PROTECTION INSTR.			07/01/84
54810	CALVERT CLIFFS UNIT 1 - IREP			09/01/84
54383	CALVERT CLIFFS 1- RCS VENTS TECHNICAL SPECIFICATIONS NUREG 073 ITEM II.B.1			09/30/84
54523	CALVERT CLIFFS 1 - TECHNICAL SPECIFICATIONS COVERED BY GENERIC LETTERS 83-36, 83-37 FOR NUREG-0737			09/30/84
<u>ANTICIPATED ACTIONS</u>				<u>INITIATION</u>
53576	CALVERT CLIFFS 1 - ITEM 1.2 - POST TRIP REVIEW-DATA AND INFORMATION CAPABILITY			11/01/84
53659	CALVERT CLIFFS 1 - ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS			11/01/84
53742	CALVERT CLIFFS 1- ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS			11/01/84
53825	CALVERT CLIFFS 1 - ITEM 3.2.3 - POST-MAINTENANCE TESTING -CHANGES TO TECH. SPEC. - ALL SR COMPONENTS			11/01/84
53902	-ITEMS 4.2.3 & 4.2.4-PREVENTATIVE MAINT. PROGRAM FOR REACTOR TRIP BREAKERS-LIFE TESTING & REPLACEMENT			11/01/84
53969	CALVERT CLIFFS 1 - ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS			11/01/84

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TECHNICAL ASSIGNMENT CONTROL SYSTEM

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CONDENSED MANAGEMENT REPORT

(CONTINUATION)

CALVERT CLIFFS 1

TAC # TAC DESCRIPTION

PLANT SPECIFIC

PRIORITY

CRITICAL DATE

ANTICIPATED ACTIONS (CONTINUATION)

54052 CALVERT CLIFFS 1 - ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL TESTING - DIVERSE TRIP FEATURES
53367 CALVERT CLIFFS UNIT 1 - REPORTING REQUIREMENTS PER GL 83-43

INITIATION

11/01/84
12/22/84

DATA AS OF - 04/30/84

CONDENSED MANAGEMENT REPORT

FACILITY: CALVERT CLIFFS 2

PLANT LOCATION: 40 MI S OF ANNAPOLIS, MD
 DOCKET NUMBER: 050-00318
 ARCH/ENGINEER: BECH
 IE INSPECTOR: R. ARCHITZEL

LICENSED POWER: 2700 MWT
 DESIGN POWER: 0845 MWE
 NSSS VENDOR: COMB

PROJECT MANAGER: D. JAFFE
 BRANCH CHIEF: J. MILLER
 LIC. ASSISTANT: P. KREUTZER

TAC #	TAC DESCRIPTION	MULTI-PLANT	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				
12315	CALVERT CLIFFS 2 - HELB AND CONSEQUENTIAL SYSTEM FAILURE CALVERT CLIFFS 2		3	COMPLETE 01/31/84
43635	CALVERT CLIFFS 2 - SEISMIC QUALIFICATION OF THE AUXILIARY FEEDWATER SYSTEM		1	03/21/84
<u>ACTIVE ACTIONS</u>				
51079	CALVERT CLIFFS 2 - INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)			TARGET
52216	CALVERT CLIFFS 2 - CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPA#C-10)			
52745	CALVERT CLIFFS 2 - ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION AND PROCEDURES			
52826	CALVERT CLIFFS 2 - ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RTS COMPONENTS			
52907	CALVERT CLIFFS - ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING PROCEDURES/VENDOR RECOMMENDATIONS - RTS COMPONENTS			
52988	CALVERT CLIFFS 2-ITEM 3.1.3-POST-MAINT TESTING-CHANGES TO TECH SPECS-RTS COMPONENTS			
53063	CALVERT CLIFFS 2 - REACTOR TRIP SYSTEM RELIABILITY- VENDOR RELATED MODIFICATIONS			
53116	CALVERT CLIFFS 2 - ITEMS 4.2.1 & 4.2.2- PREVENTATIVE MAINT PROGRAM FOR REACTOR TRIP BREAKERS - MAINTENANCE & TRENDING			
42495	CALVERT CLIFFS 2 - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21)		1	06/30/84
42891	CALVERT CLIFFS 2 - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11		2	06/31/84
49675	CALVERT CLIFFS 2 - REACTOR COOLANT PUMP TRIP		1	07/01/84
08137	CALVERT CLIFFS 2 - APPENDIX I TECH SPEC IMPLEMENTATION REVIEW		3	12/01/84
08950	CALVERT CLIFFS 2 - REVIEW OF ASYMMETRIC LOCA LOADS		2	12/30/84
<u>TAC # TAC DESCRIPTION TMI ACTIONS PRIORITY CRITICAL DATE</u>				
<u>COMPLETED ACTIONS</u>				
45261	CALVERT CLIFFS 2 - NUREG-0737 II.K.3.1, AUTO PORV ISOLATION		1	COMPLETE 10/06/83
45307	CALVERT CLIFFS 2 - NUREG-0737 II.K.3.2, REPORT ON PORV FAILURES		1	10/06/83
47794	CALVERT CLIFFS 2 - NUREG 0737 ITEM II K2.17 POTENTIAL FOR VOIDING IN RCS		1	10/24/83
<u>ACTIVE ACTIONS</u>				
51229	CALVERT CLIFFS 2 - I.D.2. SAFETY PARAMETER DISPLAY SYSTEM			TARGET
47927	CALVERT CLIFFS 2 - PLANT SHIELDING MODIFICATIONS (II.B.2.2)		1	12/30/83
44570	CALVERT CLIFFS 2 - RV AND SV TESTING		1	06/15/84
45122	CALVERT CLIFFS 2 - NUREG-0737 II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION		1	06/30/84
45813	CALVERT CLIFFS 2 - NUREG-0737 II.K.3.30, SB LOCA OUTLINE		1	06/30/84
44429	CALVERT CLIFFS 2 - NUREG-0737 II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS		1	06/30/84
48154	CALVERT CLIFFS 2 -NUREG 0737 II.K.3.31 COMPLIANCE WITH 10 CFR 50.46		1	06/30/84
45933	CALVERT CLIFFS 2 - NUREG-0737 III.A.1.2, TECHNICAL SUPPORT CENTER		1	09/30/84
46004	CALVERT CLIFFS 2 - NUREG-0737 III.A.1.2, OPERATIONAL SUPPORT CENTER		1	09/30/84
46076	CALVERT CLIFFS 2 - NUREG-0737 III.A.1.2, EMERGENCY OPERATIONS FACILITY		1	09/30/84
46292	CALVERT CLIFFS 2 - NUREG-0737 III.A.2.2. METEOROLOGICAL DATA UPGRADE		1	09/30/84
46880	CALVERT CLIFFS 2 - I? K2.13 THERMAL MECHANICAL REPORT		1	09/30/84
44220	CALVERT CLIFFS 2 - NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES		1	11/30/84
44290	CALVERT CLIFFS 2 - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES		1	11/30/84
51149	CALVERT CLIFFS 2 - I.D.1 CONTROL ROOM DESIGN REVIEW			12/01/84
48444	CALVERT CLIFFS 2 - NUREG 0737, II.K.3.25 POWER ON PUMP SEALS			12/30/84

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CONDENSED MANAGEMENT REPORT

(CONTINUATION)

CALVERT CLIFFS 2

TAC #	TAC DESCRIPTION	PLANT SPECIFIC	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				
48610	CALVERT CLIFFS UNIT 2 - HYDROGEN ANALYZER OPERABILITY EXTENSION			<u>COMPLETE</u> 12/29/83
52423	CALVERT CLIFFS UNIT 2 - CHANGE TO TS FOR CSAS A-3 AND B-3			12/30/83
52428	CALVERT CLIFFS UNIT 2 - ADD SNUBBER TO TS			12/30/83
52569	CALVERT CLIFFS UNIT 2 - CONTROL ROOM HVAC COOLER TS		1	12/30/83
52571	CALVERT CLIFFS UNIT 2 - CORRECT CONTAINMENT CAM TS			12/30/83
54180	CALVERT CLIFFS UNIT 2 - MODIFICATION OF ORDER			03/02/84
53278	CALVERT CLIFFS UNIT 2 - APPENDIX R EXEMPTION (OIL)		1	03/15/84
53279	CALVERT CLIFFS UNIT 2 - APPENDIX R EXEMPTION (DOORS)		1	03/15/84
53280	CALVERT CLIFFS UNIT 2 - APPENDIX R EXEMPTION (SPRIKLER)		1	03/15/84
52425	CALVERT CLIFFS UNIT 2 - REED SWITCH TS			03/16/84
53555	CALVERT CLIFFS UNIT 2 - GENERIC STEAM LINE BREAK			03/27/84
53371	CALVERT CLIFFS UNIT 2 - TS FOR FUEL OIL TANK INSPECTION			04/19/84
53373	CALVERT CLIFFS UNIT 2 - CHANGE TO TS FOR 69KV SMECO LINE			04/19/84
53374	CALVERT CLIFFS UNIT 2 - CHANGE TO TS FOR 125VDC BATTERY			04/19/84
53375	CALVERT CLIFFS UNIT 2 - CHANGE TO TS FOR RCS VENT			04/19/84
53376	CALVERT CLIFFS UNIT 2 - TS CHANGE FOR REPLACEMENT OF SNUBBERS WITH SWAY STRUTS			04/19/84
53377	CALVERT CLIFFS UNIT 2 - TS CHANGE FOR INSTALLATION OF DEDICATED SNUBBER RESERVOIRS			04/19/84
<u>ACTIVE ACTIONS</u>				
52427	CALVERT CLIFFS UNIT 2 - TS FOR CONTAINMENT COOLING			<u>TARGET</u> 04/15/84
53372	CALVERT CLIFFS UNIT 2 - REPORTING REQUIREMENTS PER GL 83-43			06/01/84
54379	CALVERT CLIFFS UNIT 2 - EMERGENCY EXERCISE FREQUENCY			06/03/84
54623	CALVERT CLIFFS UNIT 2 - WIDE RANGE NEUTRON FLUX			07/01/84
54624	CALVERT CLIFFS UNIT 2 - TS FOR CONTAINMENT PURGE VALVES			07/01/84
54775	CALVERT CLIFFS UNIT 2 - AUX. FDNTR TS			07/01/84
48946	CALVERT CLIFFS UNIT 2 - HIGH BURNUP REPORT			08/31/84
54384	CALVERT CLIFFS 2 - RCS VENTS TECHNICAL SPECIFICATIONS NUREG 073 ITEM II.B.1			09/30/84
54524	CALVERT CLIFFS 2 - TECHNICAL SPECIFICATIONS COVERED BY GENERIC LETTERS 83-36, 83-37 FOR NUREG-0737			09/30/84
<u>ANTICIPATED ACTIONS</u>				
53577	CALVERT CLIFFS 2 - ITEM 1.2 - POST TRIP REVIEW-DATA AND INFORMATION CAPABILITY			<u>INITIATION</u> 11/01/84
53660	CALVERT CLIFFS 2 - ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS			11/01/84
53743	CALVERT CLIFFS 2 - ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS			11/01/84
53826	CALVERT CLIFFS 2 - ITEM 3.2.3 - POST-MAINTENANCE TESTING -CHANGES TO TECH. SPEC. - ALL SR COMPONENTS			11/01/84
53903	-ITEMS 4.2.3 & 4.2.4-PREVENTATIVE MAINT. PROGRAM FOR REACTOR TRIP BREAKERS-LIFE TESTING & REPLACEMENT			11/01/84
53970	CALVERT CLIFFS 2 - ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS			11/01/84
54053	CALVERT CLIFFS 2 - ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL TESTING - DIVERSE TRIP FEATURES			11/01/84
53556	CALVERT CLIFFS UNIT 2 - TS FOR HYDROGEN PURGE SYSTEM			12/22/84

DATA AS OF - 04/30/84

CONDENSED MANAGEMENT REPORT

FACILITY: COOK 1

PLANT LOCATION: 11 MI S OF BENTON HARBOR, MI
 DOCKET NUMBER: 050-00315
 ARCH/ENGINEER: AEP
 IE INSPECTOR: E. SWANSON

LICENSED POWER: 3250 MWT
 DESIGN POWER: 1030 MWE
 NSSS VENDOR: WEST

PROJECT MANAGER: D. WIGGINTON
 BRANCH CHIEF: S. VARGA
 LIC. ASSISTANT: C. PARRISH

TAC #	TAC DESCRIPTION	MULTI-PLANT	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				
47164	COOK 1 NATURAL CIRCULATION COOLDOWN (GENERIC LTR 81-21)		2	<u>COMPLETE</u> 11/17/83
49727	- COOK 1 NUREG 0737 TECH SPECS (GENERIC LTR 82-16)		2	11/23/83
42093	COOK 1 - DECAY HEAT REMOVAL CAPABILITY TECH SPECS CAPABILITY TECH SPECS		2	11/23/83
42859	COOK 1 - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11		2	11/23/83
12264	COOK 1 - HELB AND CONSEQUENTIAL SYSTEM FAILURE COOK 1		3	01/31/84
08477	COOK 1 - REVIEW OF ASYMMETRIC LOCA LOADS SUBMITTAL		2	02/01/84
<u>ACTIVE ACTIONS</u>				
52746	COOK 1 - ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION AND PROCEDURES			<u>TARGET</u>
52827	COOK 1 - ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RTS COMPONENTS			
52908	COOK 1 - ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING PROCEDURES/VENDOR RECOMMENDATIONS - RTS COMPONENTS			
52989	COOK 1-ITEM 3.1.3 -POST-MAINTENANCE TESTING-CHANGES TO TECH SPECS-RTS COMPONENTS			
53064	COOK 1 - REACTOR TRIP SYSTEM RELIABILITY - VENDOR RELATED MODIFICATIONS			
53117	COOK 1 - ITEMS 4.2.1 & 4.2.2 - PREVENTATIVE MAINTENANCE PROGRAM FOR REACTOR TRIP BREAKERS - MAINTENANCE & TRENDING			
53167	COOK 1 - ITEM 4.3 - AUTOAMTC ACTUATION OF SHUNT TRIP ATTACHMENT FOR W AND B & W PLANTS		1	03/22/84
42460	COOK 1 - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21)			07/01/84
49701	COOK 1 - REACTOR COOLANT PUMP TRIP			11/15/84
11331	COOK 1 - INSERVICE TESTING - 1ST OF PUMPS & VALVES - 10 CFR 50.55A(G)		1	09/30/85
52217	COOK 1 - CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPA#C-10)		3	12/31/85
51080	COOK 1 - INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)			
TAC #	TAC DESCRIPTION	TMI ACTIONS	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				
47795	D. C. COOK 1 - NUREG 0737 ITEM II K2.17 POTENTIAL FOR VOIDING IN RCS		1	<u>COMPLETE</u> 12/16/83
<u>ACTIVE ACTIONS</u>				
51150	COOK 1 - I.D.1 CONTROL ROOM DESIGN REVIEW			<u>TARGET</u>
51230	COOK 1 - I.D.2. SAFETY PARAMETER DISPLAY SYSTEM			
44433	COOK 1 - NUREG-0737 II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS		1	06/30/83
45126	COOK 1 - NUREG-0737 II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION		1	06/30/83
46296	COOK 1 - NUREG-0737 III.A.2.2, METEOROLOGICAL DATA UPGRADE		1	11/01/83
45818	COOK 1 - NUREG-0737 II.K.3.30, SB LOCA OUTLINE		1	06/01/84
45737	COOK 1 - NUREG-0737 III.A.1.2, TECHNICAL SUPPORT CENTER		1	09/30/84
46008	COOK 1 - NUREG-0737 III.A.1.2, OPERATIONAL SUPPORT CENTER		1	09/30/84
46080	COOK 1 - NUREG-0737 III.A.1.2, EMERGENCY OPERATIONS FACILITY		1	09/30/84
46881	COOK 1 - II K2.13 THERMAL MECHANICAL REPORT		1	09/30/84
48158	D.C. COOK 1 -NUREG 0737 II.K.3.31 COMPLIANCE WITH 10 CFR 50.46		1	10/01/84
44573	COOK 1 - RV AND SV TESTING		1	10/30/84
44224	COOK 1 - NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES		1	11/30/84
44294	COOK 1 - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES		1	11/30/84

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CONDENSED MANAGEMENT REPORT

(CONTINUATION)

COOK 1

TAC #	TAC DESCRIPTION	PLANT SPECIFIC	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				
52439	COOK 1 - CHARGING PUMP FLOW			<u>COMPLETE</u> 10/07/83
48005	COOK-1 MISC T/S CHANGES 1/22/82		3	11/22/83
51321	COOK 1 - FIRE PROTECTION SAFE SHUTDOWN CAPABILITY		2	11/22/83
04958	D.C. COOK-LOTIC-3 ANALYSIS		1	11/28/83
48699	COOK 1 FIRE PROTECTION CHANGES RE STANDARD T.S.		1	03/16/84
49542	COOK 1 - LICENSE/TECH SPEC NRC UPDATE		3	04/27/84
54385	COOK 1- RCS VENTS TECHNICAL SPECIFICATIONS NUREG 073 ITEM II.B.1			04/30/84
<u>ACTIVE ACTIONS</u>				
48291	COOK 1 - MISCELLANEOUS T.S. CHANGES DATED 3.29/82 AEP659		3	<u>TARGET</u> 03/30/84
51323	COOK 1 - ISI (NRC) REQUESTED SUPPLEMENT			04/15/84
49520	COOK 1 - HYDROGEN CONTROL FINAL RESOLUTION		2	05/15/84
43072	COOK 1 - STRUCTURAL ANALYSIS OF THE CONTAINMENTS ULTIMATE STRENGTH		1	05/15/84
43523	COOK 1 - EQUIPMENT SURVIVABILITY: HYDROGEN BURN		1	05/15/84
53514	COOK 1 RCS VENTS TECH SPEC			05/30/84
54525	COOK 1 - TECHNICAL SPECIFICATIONS COVERED BY GENERIC LETTERS 83-36, 83-37 FOR NUREG-0737			09/30/84
51602	COOK 1 - SURVEILLANCE REFERENCE TO 4.0.5		2	11/15/84
<u>ANTICIPATED ACTIONS</u>				
53578	COOK 1 - ITEM 1.2 - POST TRIP REVIEW-DATA AND INFORMATION CAPABILITY			<u>INITIATION</u> 11/01/84
53661	COOK 1 - ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS			11/01/84
53744	COOK 1- ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS			11/01/84
53827	COOK 1 - ITEM 3.2.3 - POST-MAINTENANCE TESTING -CHANGES TO TECH. SPEC. - ALL SR COMPONENTS			11/01/84
53904	-ITEMS 4.2.3 & 4.2.4-PREVENTATIVE MAINT. PROGRAM FOR REACTOR TRIP BREAKERS-LIFE TESTING & REPLACEMENT			11/01/84
53971	COOK 1 - ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS			11/01/84
54054	COOK 1 - ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL CAPABILITY			11/01/84

DATA AS OF - 04/30/84

CONDENSED MANAGEMENT REPORT

FACILITY: COOK 2

PLANT LOCATION: 11 MI S OF BENTON HARBOR, MI
 DOCKET NUMBER: 050-00316
 ARCH/ENGINEER: AEP
 IE INSPECTOR: E. SWANSON

LICENSED POWER: 3411 MWT
 DESIGN POWER: 1100 MWE
 NSSS VENDOR: WEST

PROJECT MANAGER: D. WIGGINTON
 BRANCH CHIEF: S. VARGA
 LIC. ASSISTANT: C. PARRISH

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>MULTI-PLANT</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
47165	COOK 2 NATURAL CIRCULATION COOLDOWN (GENERIC LTR 81-21)		2	<u>COMPLETE</u> 11/17/83
42094	COOK 2 - DECAY HEAT REMOVAL CAPABILITY TECH SPECS CAPABILITY TECH SPECS		2	11/23/83
42860	COOK 2 - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11		2	11/23/83
49728	COOK 2 -"NUREG 0737 TECH SPECS (GENERIC LETTER 82-16 82-16 OR 83-02)"			11/23/83
11334	COOK 2 - INSERVICE TESTING OF PUMP AND VALVES- 10 CFR 50.55 A(G)		1	01/31/84
12265	COOK 2 - HELB AND CONSEQUENTIAL SYSTEM FAILURE DC COOK 2		3	01/31/84
08480	COOK 2 - REVIEW OF ASYMMETRIC LOCA LOADS SUBMITTAL		2	02/01/84
<u>ACTIVE ACTIONS</u>				
52747	COOK 2 - ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION AND PROCEDURES			<u>TARGET</u>
52828	COOK 2 - ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RTS COMPONENTS			
52909	COOK 2 - ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING PROCEDURES/VENDOR RECOMMENDATIONS - RTS COMPONENTS			
52990	COOK 2-ITEM 3.1.3 -POST-MAINTENANCE TESTING-CHANGES TO TECH SPECS-RTS COMPONENTS			
53065	COOK 2 - REACTOR TRIP SYSTEM RELIABILITY - VENDOR RELATED MODIFICATIONS			
53118	COOK 2 - ITEMS 4.2.1 & 4.2.2 - PREVENTATIVE MAINTENANCE PROGRAM FOR REACTOR TRIP BREAKERS - MAINTENANCE & TRENDING			
53168	COOK 2 - ITEM 4.3 - AUTOAMTIC ACTUATION OF SHUNT TRIP ATTACHMENT FOR W AND B & W PLANTS			
42461	COOK 2 - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21)		1	03/22/84
49702	COOK 2 - REACTOR COOLANT PUMP TRIP			07/01/84
52218	COOK 2 - CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPA#C-10)		3	09/30/85
51081	COOK 2 - INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)		2	12/31/85
<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>TMI ACTIONS</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
47796	D. C. COOK 2 - NUREG 0737 ITEM II K2.17 POTENTIAL FOR VOIDING IN RCS		1	<u>COMPLETE</u> 12/16/83
<u>ACTIVE ACTIONS</u>				
51151	COOK 2 - I.D.1 CONTROL ROOM DESIGN REVIEW			<u>TARGET</u>
51231	COOK 2 - I.D.2. SAFETY PARAMETER DISPLAY SYSTEM			
46297	COOK 2 - NUREG-0737 III.A.2.2, METEOROLOGICAL DATA UPGRADE		1	06/01/83
44434	COOK 2 - NUREG-0737 II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS		1	06/30/83
45127	COOK 2 - NUREG-0737 II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION		1	06/30/83
45819	COOK 2 - NUREG-0737 II.K.3.30, SB LOCA OUTLINE		1	06/30/83
48159	D.C. COOK 2 -NUREG 0737 II.K.3.31 COMPLIANCE WITH 10 CFR 50.46		1	09/30/83
46882	COOK 2 - II K2.13 THERMAL MECHANICAL REPORT		1	09/30/84
45938	COOK 2 - NUREG-0737 III.A.1.2, TECHNICAL SUPPORT CENTER		1	09/30/84
46009	COOK 2 - NUREG-0737 III.A.1.2, OPERATIONAL SUPPORT CENTER		1	09/30/84
46081	COOK 2 - NUREG-0737 III.A.1.2, EMERGENCY OPERATIONS FACILITY		1	09/30/84
44574	COOK 2 - RV AND SV TESTING		1	10/30/84
44225	COOK 2 - NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES		1	11/30/84
44295	COOK 2 - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES		1	11/30/84

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CONDENSED MANAGEMENT REPORT

(CONTINUATION)

COOK 2

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				<u>COMPLETE</u>
12463	COOK 2 - CHECK VALVE TESTING - LIC COND 3(C)		1	11/05/83
48004	COOK 2 - MISCEL. T/S CHANGES 1/22/82		2	11/22/83
51322	COOK 2 - FIRE PROTECTION SAFE SHUTDOWN CAPABILITY			11/22/83
48492	COOK 2 - STUDY FOR NOT CHANGING TECH SPEC LIMITS			01/10/84
48700	COOK 2 FIRE PROTECTION CHANGES RE STANDARD T.S.			03/16/84
49543	COOK 2 - LICENSE/TECH SPEC NRC UPDATE		3	04/27/84
54386	COOK 2- RCS VENTS TECHNICAL SPECIFICATIONS NUREG 073 ITEM II.B.1			04/30/84
<u>ACTIVE ACTIONS</u>				<u>TARGET</u>
43524	COOK 2 - EQUIPMENT SURVIVABILITY: HYDROGEN BURN		1	02/15/84
12619	COOK 2 - PANEL & ELEC SWITCHGEAR QUAL-LIC. COND. 3(R) COND. 3(R)		1	03/22/84
04997	COOK 2 - ELEC SYS (BARTONS) QUAL FOR STEAM BREAK/LOCA LOCA		1	03/22/84
48292	COOK 2 -MISCELLANEOUS T.S. CHANGES DATED 3/29/82 AEP659		2	03/30/84
51324	COOK 2 - ISI (NRC) REQUESTED SUPPLEMENT			04/15/84
52497	COOK 2 - CYCLE 4 CONDITIONS ON RELOAD		1	05/15/84
49519	COOK 2 - HYDROGEN CONTROL FINAL RESOLUTION		2	05/15/84
43071	COOK 2 - STRUCTURAL ANALYSIS OF THE CONTAINMENTS ULTIMATE STRENGTH		1	05/15/84
52498	COOK 2 - CYCLE 5 RELOAD		1	05/19/84
53515	COOK 2 - RCS VENTS TECH SPEC			05/30/84
54526	COOK 2 - TECHNICAL SPECIFICATIONS COVERED BY GENERIC LETTERS 83-36, 83-37 FOR NUREG-0737			09/30/84
51603	COOK 2 - SURVEILLANCE REFERENCE TO 4.0.5			11/15/84
<u>ANTICIPATED ACTIONS</u>				<u>INITIATION</u>
53579	COOK 2 - ITEM 1.2 - PCST TRIP REVIEW-DATA AND INFORMATION CAPABILITY			11/01/84
53662	COOK 2 - ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS			11/01/84
53745	COOK 2- ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS			11/01/84
53828	COOK 2 - ITEM 3.2.3 - POST-MAINTENANCE TESTING -CHANGES TO TECH. SPEC. - ALL SR COMPONENTS			11/01/84
53905	-ITEMS 4.2.3 & 4.2.4-PREVENTATIVE MAINT. PROGRAM FOR REACTOR TRIP BREAKERS-LIFE TESTING & REPLACEMENT			11/01/84
53972	COOK 2 - ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS			11/01/84
54055	COOK 2 - ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL TESTING - DIVERSE TRIP FEATURES			11/01/84

DATA AS OF - 04/30/84

CONDENSED MANAGEMENT REPORT

FACILITY: COOPER STATION

PLANT LOCATION: 23 MI S OF NEBRASKA CITY, NEB
 DOCKET NUMBER: 050-00298
 ARCH/ENGINEER: B&R
 IE INSPECTOR: D. DUBOIS

LICENSED POWER: 2381 MW
 DESIGN POWER: 0778 MWE
 NSSS VENDOR: GE

PROJECT MANAGER: B. SIEGEL
 BRANCH CHIEF: D. VASSALLO
 LIC. ASSISTANT: S. NORRIS

TAC #	TAC DESCRIPTION	MULTI-PLANT	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				
07982	COOPER STATION-CONTROL OF HEAVY LOADS OVER SPENT FUEL POOL (NUREG 0612)		2	COMPLETE 10/31/83
07934	COOPER - MARK I CONTAINMENT LONG TERM PROGRAM IMPLEMENTATION		1	01/20/84
12300	COOPER - HELB AND CONSEQUENTIAL SYSTEM FAILURE COOPER		3	02/02/84

TAC #	TAC DESCRIPTION	MULTI-PLANT	PRIORITY	CRITICAL DATE
<u>ACTIVE ACTIONS</u>				
51082	COOPER STATION - INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)			TARGET
52219	COOPER STATION - CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPA#C-10)			
52748	COOPER STATION 1 - ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION AND PROCEDURES			
52829	COOPER STATION - ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RTS COMPONENTS			
52910	COOPER STATION - ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING PROCEDURES/VENDOR RECOMMENDATIONS - RTS COMPONENTS			
52991	COOPER STATION-ITEM 3.1.3 -POST-MAINT TESTING-CHANGES TO TECH SPECS-RTS COMPONENTS		1	01/30/84
11251	COOPER - INSERVICE TESTING (IST)			03/01/84
51674	COOPER NUREG 0737 TECH SPECS		1	03/30/84
42486	COOPER - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21)		2	03/31/84
43729	COOPER - SAFETY CONCERNS ASSOCIATED WITH PIPE BREAKS IN THE BWR SCRAM SYSTEM		3	05/30/84
08140	COOPER - APPENDIX I TECH SPEC IMPLEMENTATION REVIEW		1	06/30/84
42573	COOPER - LONG TERM REVIEW CONTAINMENT PURGE AND VENT (B-24)		1	06/30/84
42418	COOPER - SINGLE LOOP OPERATION		2	06/30/84
46657	COOPER - IMPLEMENTATION OF NUREG-0313, REV. 1			

TAC #	TAC DESCRIPTION	TMI ACTIONS	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				
44430	COOPER - NUREG-0737 II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS		1	COMPLETE 12/07/83
48155	COOPER -NUREG 0737 II.K.3.31 COMPLIANCE WITH 10 CFR 50.46		1	12/28/83
45814	COOPER - NUREG-0737 II.K.3.30, SB LOCA OUTLINE		1	12/28/83

TAC #	TAC DESCRIPTION	TMI ACTIONS	PRIORITY	CRITICAL DATE
<u>ACTIVE ACTIONS</u>				
51152	COOPER STATION - I.D.1 CONTROL ROOM DESIGN REVIEW			TARGET
51232	COOPER STATION - I.D.2. SAFETY PARAMETER DISPLAY SYSTEM			
45589	COOPER - NUREG-0737 II.K.3.16, CHALLENGES AND FAILURES OF RELIEF VALVES		1	12/30/83
48265	COOPER - NUREG 0737 II.K.3.28, VERIFY QUALIFICATION OF ACCUMULATORS ON ADS VALVES		1	03/30/84
46293	COOPER - NUREG-0737 III.A.2.2, METEOROLOGICAL DATA UPGRADE		1	06/01/84
45123	COOPER - NUREG-0737 II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION		1	06/30/84
44221	COOPER - NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES		1	06/30/84
44291	COOPER - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES		1	06/30/84
45934	COOPER - NUREG-0737 III.A.1.2, TECHNICAL SUPPORT CENTER		1	09/30/84
46005	COOPER - NUREG-0737 III.A.1.2, OPERATIONAL SUPPORT CENTER		1	09/30/84
46077	COOPER - NUREG-0737 III.A.1.2, EMERGENCY OPERATIONS FACILITY		1	09/30/84

TAC #	TAC DESCRIPTION	PLANT SPECIFIC	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				
52009	COOPER - REQUEST FOR SCHEDULAR EXTENSION OF FINAL RULE ON ENVIRONMENTAL QUALIFICATION OF ELECTRICAL EQUIPMENT			COMPLETE 10/03/83

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CONDENSED MANAGEMENT REPORT

(CONTINUATION)

COOPER STATION

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u> (CONTINUATION)				<u>COMPLETE</u>
52609	COOPER - LICENSED OPERATOR STAFFING, 50.54(M)(2)			11/28/83
52608	COOPER - LICENSED OPERATOR STAFFING, 50.54(M)(2)(I)			01/20/84
52013	COOPER - PROPOSED CHANGE NO 5 TO TECHNICAL SPECIFICATIONS INSERVICE INSPECTION			01/20/84
52642	PROPOSED CHANGE NO 8 TO TECHNICAL SPECIFICATIONS ORGANIZATIONAL CHANGES			03/20/84
53322	COOPER - RESPONSE TO 10 CFR 50 APPENDIX R, FIRE PROTECTION OF SAFE SHUTDOWN CAPABILITY			04/16/84
<u>ACTIVE ACTIONS</u>				<u>TARGET</u>
54682	COOPER - 1ST PROGRAM - SECOND TEN YEAR INTERVAL			
54247	COOPER - PROPOSED CHANGE #12 - REVISED LICENSEE EVENT REPORT			05/24/84
53554	COOPER - PROPOSED CHANGE NO 10 TO TECHNICAL SPECIFICATIONS MIS CHANGES			05/30/84
54430	COOPER NUCLEAR STATION - NUREG-0737 TS IN RESPONSE TO GENERIC LETTER 83-36 - CNS CHANGE NO. 11			06/29/84
<u>ANTICIPATED ACTIONS</u>				<u>INITIATION</u>
55005	COOPER - PROPOSED CHANGE NO. 13 - MARK/CONTAINMENT MODIFICATION			05/14/84
53973	COOPER STATION - ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS			11/01/84
54056	COOPER STATION - ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL TESTING - DIVERSE TRIP FEATURES			11/01/84
53580	COOPER STATION - ITEM 1.2 - POST TRIP REVIEW-DATA AND INFORMATION CAPABILITY			11/01/84
53663	COOPER STATION - ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS			11/01/84
53746	COOPER STATION- ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS			11/01/84
53829	COOPER STATION - ITEM 3.2.3 - POST-MAINTENANCE TESTING -CHANGES TO TECH. SPEC. - ALL SR COMPONENTS			11/01/84

DATA AS OF - 04/30/84

CONDENSED MANAGEMENT REPORT

FACILITY: CRYSTAL RIVER 3

PLANT LOCATION: 7 MI NW OF CRYSTAL RIVER, FLA
 DOCKET NUMBER: 050-00302
 ARCH/ENGINEER: GIL
 IE INSPECTOR: T. STETKA

LICENSED POWER: 2544 MWT
 DESIGN POWER: 0825 MWE
 NSSS VENDOR: B&W

PROJECT MANAGER: R. HERNAN
 BRANCH CHIEF: J. STOLZ
 LIC. ASSISTANT: R. INGRAM

TAC #	TAC DESCRIPTION	MULTI-PLANT	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				
08843	CRYSTAL RIVER 3 - REVIEW OF ASYMMCTRIC LOCA LOADS SUBMITTAL		3	<u>COMPLETE</u> 01/11/84
12743	CRYSTAL RIVER 3 - ADEQUACY OF STATION ELECTRIC DISTRIBUTION SYSTEM VOLTAGE		2	04/19/84
<u>ACTIVE ACTIONS</u>				
42902	CRYSTAL RIVER 3 - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11		2	<u>TARGET</u> 05/01/84
42512	CRYSTAL RIVER 3 - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21)		2	06/01/84
08985	CRYSTAL RIVER 3 - APPENDIX I REVIEW		1	07/01/84
49729	-CRYSTAL RIVER 3 NUREG 0737 TECH SPECS (GENERIC LTR 82-16)		1	07/01/84
42121	CRYSTAL RIVER 3 - DECAY HEAT REMOVAL CAPABILITY TECH SPECS		2	07/30/84
04549	CRYSTAL RIVER 3 - REVIEW REQUEST FOR RELIEF FROM ASME SEC. XI PUMP & VALVE TESTING REQUIREMENTS		2	08/01/84
07983	CRYSTAL RIVER 3-CHTRL OF HEAVY LOADS OVER SPENT FUEL POOL (NUREG 0612)		2	10/01/84
12321	CRYSTAL RIVER 3 - HELB AND CONSEQUENTIAL SYSTEM FAILURE CRYSTAL RIVER 3		3	10/01/84
52749	CRYSTAL RIVER 3 - ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION AND PROCEDURES		1	11/05/84
52030	CRYSTAL RIVER 3 - ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RTS COMPONENTS		1	11/05/84
52911	CRYSTAL RIVER 3 - ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING PROCEDURES/VENDOR RECOMMENDATIONS - RTS COMPONENTS		1	11/05/84
52992	CRYSTAL RIVER 3-ITEM 3.1.3 -POST-MAINT TESTING-CHANGES TO TECH SPECS-RTS COMPONENTS		1	11/05/84
53066	CRYSTAL RIVER 3-ITEM 4.1 RX TRIP SYSTEM RELIABILITY-VENDOR RELATED MODIFICATIONS		1	11/05/84
53119	CRYSTAL RIVER 3 - ITEMS 4.2.1 & 4.2.2 - PREVENTATIVE MAINT PROGRAM FOR REACTOR TRIP BREAKERS - MAINTENANCE & TRENDING		1	11/05/84
53169	CRYSTAL RIVER 3 - ITEM 4.3 - AUTOAMTIC ACTUATION OF SHUNT TRIP ATTACHMENT FOR W AND B & W PLANTS			11/05/84
52220	CRYSTAL RIVER 3 - CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPA#C-10)			12/01/84
47166	CRYSTAL RIVER 3 NATURAL CIRCULATION COOLDOWN (GENERIC LTR 81-21)		2	05/01/85
49668	CRYSTAL RIVER 3 - REACTOR COOLANT PUMP TRIP		3	06/01/85
51083	CRYSTAL RIVER 3 - INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)			02/28/86
TAC #	TAC DESCRIPTION	TMI ACTIONS	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				
45619	CRYSTAL RIVER 3 - NUREG-0737 II.K.3.17, ECC SYSTEM OUTAGES		1	<u>COMPLETE</u> 10/06/83
47929	CRYSTAL RIVER 3 - PLANT SHIELDING MODIFICATIONS (II.B.2.2)		1	11/01/83
44668	CRYSTAL RIVER 3 - NUREG-0737 II.E.1.1, AFW SYSTEM EVALUATION		1	04/30/84
<u>ACTIVE ACTIONS</u>				
44431	CRYSTAL RIVER 3 - NUREG-0737 II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS		1	<u>TARGET</u> 07/01/84
44292	CRYSTAL RIVER 3 - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES		1	08/01/84
45268	CRYSTAL RIVER 3 - NUREG-0737 II.K.3.1, AUTO PORV ISOLATION		1	08/10/84
45333	CRYSTAL RIVER 3 - NUREG-0737 II.K.3.3, REPORT ON RV/SV FAILURES		1	08/30/84
45197	CRYSTAL RIVER 3 - NUREG-0737 II.K.2.13, THERMAL-MECHANICAL REPORT		1	08/30/84
44571	CRYSTAL RIVER 3 - RV AND SV TESTING		1	08/30/84
45935	CRYSTAL RIVER 3 - NUREG-0737 III.A.1.2, TECHNICAL SUPPORT CENTER		1	08/30/84
46006	CRYSTAL RIVER 3 - NUREG-0737 III.A.1.2, OPEKATIONAL SUPPORT CENTER		1	08/30/84
46078	CRYSTAL RIVER 3 - NUREG-0737 III.A.1.2, EMERGENCY OPERATIONS FACILITY		1	08/30/84

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CONDENSED MANAGEMENT REPORT

(CONTINUATION)

CRYSTAL RIVER 3

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>TMI ACTIONS</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>ACTIVE ACTIONS</u> (CONTINUATION)				<u>TARGET</u>
46294	CRYSTAL RIVER 3 - NUREG-0737 III.A.2.2, METEOROLOGICAL DATA UPGRADE		1	09/30/84
45815	CRYSTAL RIVER 3 - NUREG-0737 II.K.3.30, SB LOCA OUTLINE		1	12/31/84
44222	CRYSTAL RIVER 3 - NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES		1	01/27/85
51153	CRYSTAL RIVER 3 - I.D.1 CONTROL ROOM DESIGN REVIEW			04/03/85
48156	CRYSTAL RIVER 3 - NUREG 0737 II.K.3.31 COMPLIANCE WITH 10 CFR 50.46		1	06/01/85
45124	CRYSTAL RIVER 3 - NUREG-0737 II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION		1	06/30/85
51233	CRYSTAL RIVER 3 - I.D.2. SAFETY PARAMETER DISPLAY SYSTEM			07/01/85
<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				<u>COMPLETE</u>
52655	CRYSTAL RIVER, UNIT 3 - EVAL OFF-SITE FIRE PROTECTION ASSISTANCE			12/01/83
49637	CRYSTAL RIVER 3 - EMERG. F. W. FLOW INSTRUMENT ULTRASONIC INDICATION TECH. SPEC. CHANGE		1	01/17/84
11946	CRYSTAL RIVER 3 - SYSTEM CYCLS		3	03/29/84
49498	CRYSTAL RIVER 3 - STATION BATTERY ELECTROLYTE SPECIFIC GRAVITY - LCO CHANGE IN T.S.		3	04/13/84
54372	CRYSTAL RIVER, UNIT 3/REVIEW FLORIDA POWER PROPOSED ALTERNATE OFF-SITE POWER SUPPLY			04/19/84
<u>ACTIVE ACTIONS</u>				<u>TARGET</u>
53248	CRYSTAL RIVER, UNIT 3 REVIEW SUPPLEMENTS 1 AND 2 TO TS CHANGE REQUEST 36 (RETS)			05/15/84
54798	CRYSTAL RIVER, UNIT 3 - CHANGE PUMP INSERVICE TESTING TO QUARTERLY			05/31/84
54175	CRYSTAL RIVER UNIT 3 - REVISE STEAM GEN OPERATING LEVEL LIMITS			06/01/84
53405	CRYSTAL RIVER, UNIT 3/REVIEW RADWASTE PROCESS CONTROL PROGRAM		1	06/01/84
48298	CRYSTAL RIVER 3 - REACTOR COOLANT PUMP SEAL DAMAGE		3	06/01/84
48472	CRYSTAL RIVER 3 - INTAKE CANAL ACCEPTANCE CRITERIA		3	06/01/84
48921	CRYSTAL RIVER 3 - IREP REVIEW		2	06/01/84
49988	CRYSTAL RIVER 3 - CHANGE TO HIGH RADIATION AREA TECH SPEC 6.12		1	06/01/84
48450	CRYSTAL RIVER 3 - PHYSICAL SECURITY PLAN REVISIONS		1	07/01/84
49440	CRYSTAL RIVER 3 - GENERIC LTR 82-17 EMERG. PLAN, AND 82-23 SECURITY PLAN - 12 MONTH AUDIT INTO T. S.		2	07/01/84
53330	CRYSTAL RIVER, UNIT 3/TECHNICAL SPECIFICATION CHANGE - FUEL POOL ENRICHMENT LIMIT			07/01/84
12526	CRYSTAL RIVER 3 - CALIBRATION OF NUCLEAR INSTRUMENTATION CHANGE 53		3	07/01/84
54445	CRYSTAL RIVER, UNIT 3 - REVIEW TECH SEC AMENDMENT FOR DECAY HEAT REMOVAL SYSTEM			07/30/84
54892	CRYSTAL RIVER, UNIT 3 - TSCR #114 DELETE LER ADMIN CONT. VALVES			08/01/84
55004	CRYSTAL RIVER, UNIT 3/TSCR #114 DELETE LER ADMIN REQMS			09/01/84
54527	CRYSTAL RIVER 3 - TECHNICAL SPECIFICATIONS COVERED BY GENERIC LETTERS 83-36, 83-37 FOR NUREG-0737			09/30/84
54387	CRYSTAL RIVER 3- RCS VENTS TECHNICAL SPECIFICATIONS NUREG 073 ITEM II.B.1			09/30/84
52669	CRYSTAL RIVER, UNIT 3 - REVIEW FPC PROPOSAL FOR ON-LINE ES LOGIC TESTING		2	12/01/84
42027	CRYSTAL RIVER 3 - STEAM GEN. RUPTURE MATRIX DISCONNECT FROM AFW VALVES		3	04/01/85
49823	CRYSTAL RIVER 3- SMALL BREAK LOCA PROCEDURES & MAINTAINING PROPER SG LEVEL		1	05/01/85
49059	CRYSTAL RIVER 3 - UPGRADE OF POWER AND INSTRUMENTATION FOR A FW SYSTEM BEYOND NUREG 0737		3	06/01/85
52039	CRYSTAL RIVER, UNIT 3 - DEGRADED GRID VOLTAGE PROT TS & DESIGN			08/01/85
<u>ANTICIPATED ACTIONS</u>				<u>INITIATION</u>
53581	CRYSTAL RIVER 3 - ITEM 1.2 - POST TRIP REVIEW-DATA AND INFORMATION CAPABILITY			11/01/84
53664	CRYSTAL RIVER 3 - ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS			11/01/84
53747	CRYSTAL RIVER- ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS			11/01/84
53830	CRYSTAL RIVER 3 - ITEM 3.2.3 - POST-MAINTENANCE TESTING -CHANGES TO TECH. SPEC. - ALL SR COMPONENTS			11/01/84

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CONDENSED MANAGEMENT REPORT

(CONTINUATION)

CRYSTAL RIVER 3

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>	<u>INITIATION</u>
<u>ANTICIPATED ACTIONS (CONTINUATION)</u>					
53906	-ITEMS 4.2.3 & 4.2.4-PREVENTATIVE MAINT. PROGRAM FOR REACTOR TRIP BREAKERS-LIFE TESTING & REPLACEMENT				11/01/84
53953	CRYSTAL RIVER 3 - ITEM 4.4 - IMPROVEMENTS IN MAINTENANCE AND TEST PROCEDURES FOR B & W PLANTS				11/01/84
53974	CRYSTAL RIVER 3 - ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS				11/01/84
54057	CRYSTAL RIVER 3 - ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL TESTING - DIVERSE TRIP FEATURES				11/01/84

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CONDENSED MANAGEMENT REPORT

FACILITY: DAVIS-BESSE 1

PLANT LOCATION: 21 MI E OF TOLEDO, OH
 DOCKET NUMBER: 050-00346
 ARCH/ENGINEER: BECH
 IE INSPECTOR: W. ROGERS

LICENSED POWER: 2772 MWT
 DESIGN POWER: 0906 MWE
 NSSS VENDOR: B&W

PROJECT MANAGER: A. DEAGAZIO
 BRANCH CHIEF: J. STOLZ
 LIC. ASSISTANT: R. INGRAM

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>MULTI-PLANT</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
12322	DAVIS BESSE 1 - HELB AND CONSEQUENTIAL SYSTEM FAILURE DAVIS BESSE 1		3	<u>COMPLETE</u> 02/02/84
10986	DAVIS BESSE 1 - RPV SUPPORTS (ASYMETRIC LOCA LOAD)		3	04/09/84
<u>ACTIVE ACTIONS</u>				
52750	DAVIS BESSE 1 - ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION AND PROCEDURES			<u>TARGET</u>
52831	DAVIS BESSE 1 - ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RTS COMPONENTS			
52912	DAVIS BESSE 1 - ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING PROCEDURES/VENDOR RECOMMENDATIONS - RTS COMPONENTS			
52993	DAVIS BESSE 1-ITEM 3.1.3 -POST-MAINT TESTING-CHANGES TO TECH SPECS-RTS COMPONENTS			
53067	DAVIS BESSE 1 - REACTOR TRIP SYSTEM RELIABILITY - VENDOR RELATED MODIFICATIONS			
53120	DAVIS BESSE 1 - ITEMS 4.2.1 & 4.2.2 - PREVENTATIVE MAINT PROGRAM FOR REACTOR TRIP BREAKERS - MAINTENANCE & TRENDING			
53170	DAVIS BESSE 1 - ITEM 4.3 - AUTOAMTIC ACTUATION OF SHUNT TRIP ATTACHMENT FOR W AND B & W PLANTS		1	04/30/84
11316	DAVIS BESSE 1 - INSERVICE TESTING PROGRAM			
43038	DAVIS BESSE 1 - TECHNICAL SPECIFICATIONS DEFINING OPERABILITY FOR SAFETY SYSTEMS		2	05/15/84
49730	DAVIS BESSE 1 -"NUREG 0737 TECH SPECS (GENERIC LETTER 82-16 OR 83-02)"		2	05/30/84
42508	DAVIS BESSE 1 - ENVIRONMENTAL QUALIFICATIONS OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21)		1	06/30/84
08141	DAVIS BESSE 1 - APPENDIX I TECH SPEC IMPLEMENTATION REVIEW		3	06/30/84
11311	DAVIS BESSE 1 - MECHANICAL SNUBBERS		2	09/30/84
11312	DAVIS BESSE 1 - HYDRAULIC SNUBBERS		3	09/30/84
10993	DAVIS-BESSE 1-CONTROL OF HEAVY LOADS OVER SPENT FUEL POOL (NUREG 0612)		2	12/31/84
47182	DAVIS BESSE 1 NATURAL CIRCULATION COOLDOWN (GENERIC LTR 81-21)		2	12/31/84
49662	DAVIS BESSE 1 - REACTOR COOLANT PUMP TRIP		2	12/31/84
42903	DAVIS BESSE 1 - MASONRY WAL DESIGN. RESPONSE TO IE BULLETIN 80-11		2	09/30/85
52221	DAVIS BESSE 1 - CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPA#C-10)			09/30/85
51084	DAVIS BESSE 1 - INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)		2	12/31/85
<u>TAC # TAC DESCRIPTION TMI ACTIONS PRIORITY CRITICAL DATE</u>				
<u>COMPLETED ACTIONS</u>				
44364	DAVIS BESSE - NUREG-0737 II.B.1, RCS HIGH POINT VENTS		1	<u>COMPLETE</u> 10/05/83
44669	DAVIS BESSE - NUREG-0737 II.E.1.1, AFW SYSTEM EVALUATION		1	02/21/84
<u>ACTIVE ACTIONS</u>				
44751	DAVIS BESSE - NUREG-0737 II.E.1.2.2, AFW SAFETY GRADE FLOW INDICATION		1	<u>TARGET</u> 05/30/84
42964	DAVIS-BESSE - NUREG-0737, II.E.1.2.1, AFW SYSTEM SAFETY GRADE INITIATION		1	05/30/84
45198	DAVIS BESSE - NUREG-0737 II.K.2.13, THERMAL-MECHANICAL REPORT		1	06/30/84
45315	DAVIS BESSE - NUREG-0737 II.K.3.2, REPORT ON PORV FAILURES		1	09/30/84
45936	DAVIS BESSE - NUREG-0737 III.A.1.2, TECHNICAL SUPPORT CENTER		1	09/30/84
46007	DAVIS BESSE - NUREG-0737 III.A.1.2, OPERATIONAL SUPPORT CENTER		1	09/30/84
46079	DAVIS BESSE - NUREG-0737 III.A.1.2, EMERGENCY OPERATIONS FACILITY		1	09/30/84
46295	DAVIS BESSE - NUREG-0737 III.A.2.2, METEOROLOGICAL DATA UPGRADE		1	09/30/84
44432	DAVIS BESSE - NUREG-0737 II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS		1	09/30/84
44572	DAVIS BESSE 1 - RV AND SV TESTING		1	09/30/84
45817	DAVIS BESSE - NUREG-0737 II.K.3.30, SB LOCA OUTLINE		1	12/31/84

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CONDENSED MANAGEMENT REPORT

(CONTINUATION)

DAVIS-BESSE 1

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>TMI ACTIONS</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>ACTIVE ACTIONS</u> (CONTINUATION)				<u>TARGET</u>
51234	DAVIS BESSE 1 - I.D.2. SAFETY PARAMETER DISPLAY SYSTEM		1	05/30/85
45125	DAVIS BESSE - NUREG-0737 II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION		1	06/30/85
44223	DAVIS BESSE - NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES		1	09/30/85
44293	DAVIS BESSE - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES		1	09/30/85
51154	DAVIS BESSE 1 - I.D.1 CONTROL ROOM DESIGN REVIEW		1	09/30/85
48157	DAVIS-BESSE -NUREG 0737 II.K.3.31 COMPLIANCE WITH 10 CFR 50.46		1	09/30/85
<u>TAC #</u> <u>TAC DESCRIPTION</u> <u>PLANT SPECIFIC</u>				<u>PRIORITY</u> <u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				<u>COMPLETE</u>
46676	DAVIS-BESSE 1 - DELETION OF AFW PUMP TURBINE SPEED SWITCHES		3	10/26/83
48831	DAVIS-BESSE DNB MARGIN ANALYSIS		3	11/03/83
51387	DAVIS BESSE 1 - FIRE PROTECTION TECH SPECS		2	01/03/84
53427	DB-1, TECH SPEC CHANGE ON CONTAINMT RECIRC FANS			01/20/84
51964	DAVIS DEBBE 1 - AFW FLOW PATH VERIFICATION			02/21/84
53383	DAVIS BESSE 1 - REQUEST FOR SCHEDULE EXTENSION OF II.K.1.1 & II.F.1.2			02/27/84
47043	DAVIS-BESSE 1 - DOSE EQUIVALENT I-131		3	04/19/84
43513	DAVIS BESSE 1 - DIVERSE POWERED AFW PUMP		1	04/23/84
54353	DAVIS BESSE 1 - TECH SPECS FOR CONT SUMP LEVEL, CONT PRESSURE, HIGH PT VENTS			04/24/84
<u>ACTIVE ACTIONS</u>				<u>TARGET</u>
54796	DAVIS BESSE 1 - EMERGENCY EXERCISE EXEMPTION			
51388	DAVIS BESSE 1 - BAAT HEAT TRACE		3	04/30/84
51792	DAVIS BESSE 1 - MSIV CYCLING REQUIREMENTS			04/30/84
52417	DAVIS BESSE 1 - CONT. PURGE/EXHAUST VALVE LEAK RATE			05/15/84
52484	DAVIS BESSE 1 - ON SITE DISPOSAL, 20.302(A) REQUEST			05/30/84
51791	DAVIS BESSE 1 - AFW LOGIC & MANUAL INITIATION CIRCUIT SURVEILLANCE			05/30/84
53298	DAVIS BESSE 1 - NON-SAFETY GRADE AFW FLOW INDICATOR			05/30/84
53300	DAVIS BESSE 1 - EXEMPTION REQUEST PART 50.54 (M)(LLL)			05/30/84
54261	DAVIS BESSE 1 - CYCLE 4 COASTDOWN			05/30/84
49481	DAVIS BESSE 1 - INCONSISTENCY BETWEEN TSS AND 50.54(T)		3	05/30/84
49482	DAVIS BESSE 1 - INCONSISTENCY BETWEEN TSS AND PART 73.40 (D)		3	05/30/84
46677	DAVIS-BESSE 1 - EMERGENCY DIESEL GENERATOR SURVEILLANCE REQUIREMENTS CHANGE		3	05/30/84
52462	DAVIS BESSE 1 - EXEMPTION REQUEST - FIRE PROTECTION			05/30/84
54497	DAVIS BESSE 1 - REQUEST FOR REVISED SCHEDULE: EMERGENCY RESPONSE CAPABILITY			06/01/84
47040	DAVIS BESSE 1 - STEAM GENERATOR CONTAINMENT ISOLATION VALVES		3	06/30/84
43253	DAVIS BESSE 1 - REVIEW OF SFAS LEVEL 3 SINGLE FAILURE CRITERIA		3	06/30/84
54260	DAVIS BESSE 1 - DELETE SFAS TO CONTAINMENT SAMPLE VALVES			06/30/84
51738	DAVIS BESSE: STATION DIST. VOLTAGE VERIF. TEST			06/30/84
53296	DAVIS BESSE 1 - ACTION FOR MISSED SURVEILLANCE TESTS			06/30/84
53297	DAVIS BESSE 1 - DEENERGIZE DHR SYSTEM VALVES			06/30/84
52418	DAVIS BESSE 1 - ENVIRONMENTAL PROTECTION PLAN			06/30/84
52419	DAVIS BESSE 1 - CONTAINMENT ISOLATION VALVE			06/30/84
52420	DAVIS BESSE 1 - REVIEW ENVIRONMENTAL SURVEILLANCE RESULTS			06/30/84
52415	DAVIS BESSE 1 - INCORE DETECTOR OPERABILITY			06/30/84
52416	DAVIS BESSE 1 - DHR OPERABILITY			06/30/84
43333	DAVIS BESSE 1 - ORGANIZATIONAL CHANGE NO. 2		2	07/30/84
54259	DAVIS BESSE 1 - FIRE PROTECTION T.S.S.			09/30/84
54528	DAVIS BESSE 1 - TECHNICAL SPECIFICATIONS COVERED BY GENERIC LETTERS 83-36, 83-37 FOR NUREG-0737			09/30/84
54681	DAVIS BESSE 1 - APPEAL RELATIVE TO II.E.1.1.			09/30/84

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CONDENSED MANAGEMENT REPORT

FACILITY: DIABLO CANYON 1

PLANT LOCATION: 12 MI WSW OF SAN LUIS OBISPO
 DOCKET NUMBER: 050-00275
 ARCH/ENGINEER: PG&E
 IE INSPECTOR: T. YOUNG

LICENSED POWER: 000 MWT
 DESIGN POWER: 1084 MWE
 NSSS VENDOR: WEST

PROJECT MANAGER: H. SCHIERLING
 BRANCH CHIEF: G. KNIGHTON
 LIC. ASSISTANT: J. LEE

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>MULTI-PLANT</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>ACTIVE ACTIONS</u>				
12131	DIABLO CANYON 1, EMERGENCY PLAN REVIEW		1	<u>TARGET</u>
52222	DIABLO CANYON 1 - CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPA#C-10)			
52375	DIABLO CANYON - CONTROL OF HEAVY LOADS (PHASE I)			
52751	DIABLO CANYON 1 - ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION AND PROCEDURES			
52832	DIABLO CANYON 1 - ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RTS COMPONENTS			
52913	DIABLO CANYON 1 - ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING PROCEDURES/VENDOR RECOMMENDATIONS - RTS COMPONENTS			
52994	DIABLO CANYON 1-ITEM 3.1.3 -POST-MAINT TESTING-CHANGES TO TECH SPECS-RTS COMPONENTS			
53068	DIABLO CANYON 1 - REACTOR TRIP SYSTEM RELIABILITY - VENDOR RELATED MODIFICATIONS			
53121	DIABLO CANYON 1 - ITEMS 4.2.1 & 4.2.2 - PREVENTATIVE MAINT PROGRAM FOR REACTOR TRIP BREAKERS - MAINTENANCE & TRENDING			
53171	DIABLO CANYON 1 - ITEM 4.3 - AUTOAMTIC ACTUATION OF SHUNT TRIP ATTACHMENT FOR W AND B & W PLANTS			
49424	DIABLO CANYON 1 - BLOCKING SI SIGNAL DURING COOLDOWN			03/31/83
51994	DIABLO CANYON NATURAL CIRCULATION COOLDOWN			07/30/83
49654	DIABLO CANYON 1 - REACTOR COOLANT PUMP TRIP			01/01/84
<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>TMI ACTIONS</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>ACTIVE ACTIONS</u>				
49111	DIABLO CANYON 1 - NUREG-0737 II.K.3.5, AUTO TRIP OF RCPS			<u>TARGET</u>
52154	DIABLO CANYON UNIT 1 - CONTROL ROOM DESIGN REVIEW (API.D.1)			
52155	DIABLO CANYON UNIT 1 - SAFETY PARAMETER DISPLAY SYSTEM (API.D.2)			
51590	DIABLO CANYON RV AND SV TESTING (II.D.1)			
54986	DIABLO CANYON 1, II.K.2.13, THERMAL MECHANICAL REPORT			06/30/84
<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>ACTIVE ACTIONS</u>				
48045	DIABLO CANYON 1 - DESIGN VERIFICATION PROGRAM			<u>TARGET</u>
51633	DIABLO CANYON UNIT 1 PROPOSED MODS TO SECTIONS 3.7.9.1 & 4.7.9.4 OF TECH SPECS DEALING WITH FIRE PROTECTION			
51634	DIABLO CANYON UNIT 1 PROPOSED MODS TO TABLE 3.3-11 OF TECH SPECS			
52310	DIABLO CANYON UNIT 1 - TECH SPEC CHG RE INSTALLING NEW PERSUATION PUMP & THE NEED TO REPLACE HALON INIT EVERY 5 YRS			
53210	DIABLO CANYON UNIT 1 ALLEGATIONS			
52309	DIABLO CANYON UNIT 1 - CONTAINMENT SPRAY TIMING			
51632	DIABLO CANYON UNIT 1 EMERGENCY SUPPORT FACILITY IMPLEMENTATION DATE GL-82-33			
51635	DIABLO CANYON UNIT 1 INSERVICE INSPECTION & TESTING PROGRAM			
51636	DIABLO CANYON UNIT 1 REPLACEMENT OF INVERTER TECH SPEC CHANGE			
51637	DIABLO CANYON UNIT 1 EMERGENCY PREPAREDNESS CHANGE FROM 24 MOS TO 12 MOS B) SAFEGUARD AUDTI 24MOS TO 12 MOS			
48467	DIABLO CANYON UNIT 1 - PROPOSED ORGANIZATIONAL CHANGES TO SECTIONS 6.2 & 6.3 OF TECH. SPECS.			06/18/82
51008	DIABLO CANYON UNITS 1 - REQUESTED VARIANCE FROM STANDARD W COLD LICENSE PROGRAM			04/15/83
49935	DIABLO CANYON UNIT 1 - SER ON COMPONENT COOLING WATER SYSTEM			05/15/83

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CONDENSED MANAGEMENT REPORT

(CONTINUATION)

DIABLO CANYON 1

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>ACTIVE ACTIONS</u> (CONTINUATION)				<u>TARGET</u>
51583	DIABLO CANYON UNIT 1 - PROPOSED MODS TO TABLE 3.6-1 OF TECH. SPECS. DEALING WITH CONTAINMENT ISOLATION VALVES			05/31/83
51638	DIABLO CANYON UNIT 1 NATURAL CIRCULATION BORAN MIXING TEST			09/30/83
52060	DIABLO CANYON UNIT 1 - APPENDIX R REVIEW & PREPARATION OF SSER			09/30/83
52622	DIABLO CANYON 1 - STRUCTURAL ALLEGATIONS			11/30/83
52623	DIABLO CANYON 1 - RHR ALEGATIONS			11/30/83
53246	DIABLO CANYON UNIT 1 - SYSTEMS INTERACTION REPORT			12/09/83
54492	DIABLO CANYON 1 - POST FUEL - LOADING INITIAL TEST PROGRAM			03/28/84
54435	DIABLO CANYON - TECH SPEC UPDATE			03/30/84
54434	DIABLO CANYON - RESOLUTION OF ALLEGATIONS INCLUDING SSER PREPARATION			06/30/84
<u>ANTICIPATED ACTIONS</u>				<u>INITIATION</u>
53583	DIABLO CANYON 1 - ITEM 1.2 - POST TRIP REVIEW-DATA AND INFORMATION CAPABILITY			11/01/84
53666	DIABLO CANYON 1 - ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS			11/01/84
53749	DIABLO CANYON 1- ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS			11/01/84
53832	DIABLO CANYON 1 - ITEM 3.2.3 - POST-MAINTENANCE TESTING -CHANGES TO TECH. SPEC. - ALL SR COMPONENTS			11/01/84
53908	-ITEMS 4.2.3 & 4.2.4-PREVENTATIVE MAINT. PROGRAM FOR REACTOR TRIP BREAKERS-LIFE TESTING & REPLACEMENT			11/01/84
53976	DIABLO CANYON 1 - ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS			11/01/84
54059	DIABLO CANYON 1 - ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL TESTING - DIVERSE TRIP FEATURES			11/01/84

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CONDENSED MANAGEMENT REPORT

FACILITY: DRESDEN 1

PLANT LOCATION: 9 MI E OF MORRIS, ILL
 DOCKET NUMBER: 050-00010
 ARCH/ENGINEER: BECH
 IE INSPECTOR: T. TONGUE

LICENSED POWER: 0700 MWT
 DESIGN POWER: 0200 MWE
 NSSS VENDOR: GE

PROJECT MANAGER: P. OCONNOR
 BRANCH CHIEF: D. CRUTCHFIELD
 LIC. ASSISTANT: H. SMITH

IAC # IAC DESCRIPTIONMULTI-PLANTPRIORITYCRITICAL DATEACTIVE ACTIONS

07985 DRESDEN 1 - CONTROL OF HEAVY LOADS OVER SPENT FUEL POOL (NUREG 0612)

2

TARGET
06/11/79

IAC # IAC DESCRIPTIONPLANT SPECIFICPRIORITYCRITICAL DATECOMPLETED ACTIONS

54301 DRESDEN 1 - CONTROL OF RAD. CONT. MATERIALS - DE MINIMUS LEVELS

COMPLETE
04/13/84

ACTIVE ACTIONS

54771 DRESDEN 1 - EMERGENCY EXERCISE EXEMPTION REVIEW
 54768 DRESDEN 1 - REVIEW OF MOBILE VOLUME REDUCTION SYSTEM

TARGET
06/05/84
04/30/85

DATA AS OF - 04/30/84

CONDENSED MANAGEMENT REPORT

FACILITY: DRESDEN 2

PLANT LOCATION: 9 MI E OF MORRIS, ILL
 DOCKET NUMBER: 050-00237
 ARCH/ENGINEER: S&L
 IE INSPECTOR: T. TONGUE

LICENSED POWER: 2527 MWT
 DESIGN POWER: 0794 MWE
 NSSS VENDOR: GE

PROJECT MANAGER: R. GILBERT
 BRANCH CHIEF: D. CRUTCHFIELD
 LIC. ASSISTANT: H. SMITH

TAC #	TAC DESCRIPTION	MULTI-PLANT	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				
08552	DRESDEN 2 - BWR FEEDWATER & CONTROL ROD DRIVE NOZZLE CRACKING		1	COMPLETE 11/01/83
51675	DRESDEN 2 NUREG 0737 TECH SPECS			03/19/84
<u>ACTIVE ACTIONS</u>				
51085	DRESDEN 2 - INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)			TARGET
52752	DRESDEN 2 - ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION AND PROCEDURES			
52833	DRESDEN 2 - ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RTS COMPONENTS			
52914	DRESDEN 2 - ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING PROCEDURES/VENDOR RECOMMENDATIONS - RTS COMPONENTS			
52995	DRESDEN 2-ITEM 3.1.3 -POST-MAINTENANCE TESTING-CHANGES TO TECH SPECS-RTS COMPONENTS			
42519	DRESDEN 2 - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21)		1	04/30/84
08110	DRESDEN 2 - APPENDIX I		3	06/30/84
42608	DRESDEN 2 - LONG TERM REV CONTAINMENT PURGE AND VENT (B-24)		1	07/30/84
43719	DRESDEN 2 - SAFETY CONCERNS ASSOCIATED WITH PIPE BREAKS IN THE BWR SCRAM SYSTEM		2	07/31/84
42909	DRESDEN 2 - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11		2	08/31/84
46658	DRESDEN 2 - IMPLEMENTATION OF NUREG-0313, REV. 1		2	12/31/84
07935	DRESDEN 2 - MARK I CONTAINMENT LONG TERM PROGRAM IMPLEMENTATION		1	12/31/84
52223	DRESDEN 2 - CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPA#C-10)			09/30/85
<u>TMI ACTIONS</u>				
<u>COMPLETED ACTIONS</u>				
44368	DRESDEN 2 - NUREG-0737 II.B.1, RCS HIGH POINT VENTS		1	COMPLETE 10/03/83
45624	DRESDEN 2 - NUREG-0737 II.K.3.17, ECC SYSTEM OUTAGES		1	12/14/83
45821	DRESDEN 2 - NUREG-0737 II.K.3.30, SB LOCA OUTLINE		1	01/23/84
45907	DRESDEN 2 - NUREG-0737 II.K.3.45, MANUAL DEPRESSURIZATION		1	01/23/84
48160	DRESDEN 2 -NUREG 0737 II.K.3.31 COMPLIANCE WITH 10 CFR 50.46		1	01/23/84
<u>ACTIVE ACTIONS</u>				
51155	DRESDEN 2 - I.D.1 CONTROL ROOM DESIGN REVIEW			TARGET
51235	DRESDEN 2 - I.D.2. SAFETY PARAMETER DISPLAY SYSTEM			
44227	DRESDEN 2 - NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES		1	04/30/83
44297	DRESDEN 2 - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES		1	04/30/83
48266	DRESDEN 2 - NUREG 0737 II.K.3.28, VERIFY QUALIFICATION OF ACCUMULATORS ON ADS VALVES		1	09/30/83
44436	DRESDEN 2 - NUREG-0737 II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS		1	02/28/84
44506	DRESDEN 2 - NUREG-0737 II.B.4.1, TRAINING FOR MITIGATING CORE		1	02/28/84
44156	DRESDEN 2 - NUREG-0737 I.A.2.1.4, UPGRADING OF RO AND SRO TRAINING		1	02/28/84
45129	DRESDEN 2 - NUREG-0737 II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION		1	04/30/84
45591	DRESDEN 2 - NUREG-0737 II.K.3.16, CHALLENGES AND FAILURES OF RELIEF VALVES		1	04/30/84
45687	DRESDEN 2 - NUREG-0737 II.K.3.18, ADS ACTUATION STUDY		1	06/30/84
45940	DRESDEN 2 - NUREG-0737 III.A.1.2, TECHNICAL SUPPORT CENTER		1	09/30/84
46011	DRESDEN 2 - NUREG-0737 III.A.1.2, OPERATIONAL SUPPORT CENTER		1	09/30/84
46083	DRESDEN 2 - NUREG-0737 III.A.1.2, EMERGENCY OPERATIONS FACILITY		1	09/30/84
46299	DRESDEN 2 - NUREG-0737 III.A.2.2, METEOROLOGICAL DATA UPGRADE		1	09/30/84

DATA AS OF - 04/30/84

CONDENSED MANAGEMENT REPORT

(CONTINUATION)

DRESDEN 2

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
51803	DRESDEN 2 - TECH SPEC CHANGES FOR MECHANICAL HYDRAULIC SNUBBERS			<u>COMPLETE</u>
49364	DRESDEN UNIT 2 - HIGH ENERGY PIPE BREAK			10/21/83
51518	DRESDEN 2 - LIMIT REQUIREMENT TO VERIFY NUCLEAR RESPONSE TO CRD MOTION			10/27/83
07644	DRESDEN 2 - CONTROL ROD COUPLING VERIFICATION		3	11/03/83
52678	DRESDEN 2 - MID CYCLE PIPE CRACK INSPECTION FOR IGSCC CHANGES			11/03/83
51523	DRESDEN 2 - NUREG 0737 - ITEM II.K.3.15 - ISOLATION OF HPCI AND RCIC			11/29/83
51545	DRESDEN 2 - CONTROL ROD DRIVE SCRAM TIME TESTING TECHNICAL SPECIFICATIONS			12/12/83
52360	DRESDEN 2 - PROPOSED T.S. RE RPS POWER MONITORING SYSTEM			12/12/83
52677	DRESDEN 2 - ASSESSMENT OF CECO PROGRAM TO CORRECT DEFICIENCIES WHICH LED TO APPENDIX J EXEMPTIONS			02/14/84
52316	DRESDEN 2 - T.S. AMENDMENT RE S/R VALVE POSITION INDICATION			02/24/84
54302	DRESDEN 2 - CONTROL RAD. CONT. MATERIALS - DEMINIMUS LEVELS			04/13/84
47033	DRESDEN 2 - ISI PROGRAM, SECOND 10 YEARS		1	04/26/84
<u>ACTIVE ACTIONS</u>				
52326	DRESDEN 2 - REVIEW OF REVISED LICENSED OPERATOR REQUALIFICATION TOPICAL REPORT			<u>TARGET</u>
49367	DRESDEN UNIT 2 - LEAK DETECTION			10/28/83
49370	DRESDEN UNIT 2 - REACTOR COOLANT IODOINE ACTIVITY LIMITS			02/28/84
48258	DRESDEN 2 - PROPOSED TS CHANGE RE SAFETY/RELIEF VALVE POSITION INDICATION		3	02/28/84
49361	DRESDEN UNIT 2 - DESIGN CODES, LOAD COMBINATION			02/28/84
49363	DRESDEN UNIT 2 - CAPABILITY WITHSTAND WIND AND TORNADOES (INCLUDING MISSILES)			02/28/84
51516	DRESDEN 2 - ECONOMIC GENERATION CONTROL			03/15/84
49366	DRESDEN UNIT 2 - THERMAL OVERLOADS			03/16/84
51066	DRESDEN UNITS 2 - MULTIPLE CHANGES TO ADMINISTRATIVE CONTROLS			03/31/84
52141	DRESDEN 2 - POL -> FTOL CONVERSION - ENVIRONMENTAL EVALUATION			03/31/84
49368	DRESDEN UNIT 2 - SHORT CIRCUIT CALCULATIONS & BREAKER COORDINATION			04/30/84
49369	DRESDEN UNIT 2 - ELECTRICAL ISOLATION OF RPS			04/30/84
52436	DRESDEN 2 - INTEGRATED ASSESSMENT SUPPLEMENT			05/31/84
54469	DRESDEN 2 - CHANGES TO TABLE 3.7.1, CONTAINMENT ISOLATION VALVES			05/31/84
54772	DRESDEN 2 - EMERGENCY EXERCISE EXEMPTION REVIEW			06/05/84
49365	DRESDEN UNIT 2 - SEISMIC QUALIFICATION			06/15/84
54664	DRESDEN 2 - REVISION OF ENTIRE TECHNICAL SPECIFICATIONS			06/29/84
54662	DRESDEN 2 - UPDATE OF SNUBBER TABLES			06/29/84
48299	DRESDEN 2 - PROPOSED TS CHANGES RE APPENDICES H AND G, METAL SURVEILLANCE & MIN. TEMP. REQ		3	06/30/84
49083	DRESDEN 2 - IST PROGRAM, SECOND 10 YRS			06/30/84
54529	DRESDEN 2 - TECHNICAL SPECIFICATIONS COVERED BY GENERIC LETTERS 83-36, 83-37 FOR NUREG-0737			09/30/84
11222	DRESDEN 2 - CONVERSION POL TO FTOL		3	11/30/84
48858	DRESDEN 2 - REVIEW CECO POSITION RE 10CFR50.44			12/31/84
54769	DRESDEN 2 - REVIEW OF MOBILE VOLUME REDUCTION SYSTEM			04/30/85
<u>ANTICIPATED ACTIONS</u>				
53584	DRESDEN 2 - ITEM 1.2 - POST TRIP REVIEW-DATA AND INFORMATION CAPABILITY			<u>INITIATION</u>
53667	DRESDEN 2 - ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS			11/01/84
53750	DRESDEN 2- ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS			11/01/84
53833	DRESDEN 2 - ITEM 3.2.3 - POST-MAINTENANCE TESTING -CHANGES TO TECH. SPEC. - ALL SR COMPONENTS			11/01/84
53977	DRESDEN 2 - ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS			11/01/84
54060	DRESDEN 2 - ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL TESTING - DIVERSE TRIP FEATURES			11/01/84

DATA AS OF - 04/30/84

CONDENSED MANAGEMENT REPORT

FACILITY: DRESDEN 3

PLANT LOCATION: 9 MI E OF MORRIS, ILL
 DOCKET NUMBER: 050-00249
 ARCH/ENGINEER: S&L
 IE INSPECTOR: T. TONGUE

LICENSED POWER: 2527 MWT
 DESIGN POWER: 0794 MWE
 NSSS VENDOR: GE

PROJECT MANAGER: R. GILBERT
 BRANCH CHIEF: D. CRUTCHFIELD
 LIC. ASSISTANT: H. SMITH

TAC #	TAC DESCRIPTION	MULTI-PLANT	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				
08483	DRESDEN 3 - BWR FEEDWATER & CONTROL ROD DRIVE NOZZLE CRACKING		1	<u>COMPLETE</u> 11/01/83
12301	DRESDEN 3 - HELB AND CONSEQUENTIAL SYSTEM FAILURE DRESDEN 3		3	01/31/84
51676	DRESDEN 3 NUREG 0737 TECH SPECS			03/19/84
<u>ACTIVE ACTIONS</u>				
51086	DRESDEN 3 - INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)			<u>TARGET</u>
52753	DRESDEN 3 - ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION AND PROCEDURES			
52834	DRESDEN 3 - ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RTS COMPONENTS			
52915	DRESDEN 3 - ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING PROCEDURES/VENDOR RECOMMENDATIONS - RTS COMPONENTS			
52996	DRESDEN 3-ITEM 3.1.3 -POST-MAINTENANCE TESTING-CHANGES TO TECH SPECS-RTS COMPONENTS			
42487	DRESDEN 3 - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21)		1	01/30/84
08111	DRESDEN 3 - APPENDIX I TECH SPEC IMPLEMENTATION REVIEW		3	06/30/84
42574	DRESDEN 3 - LONG TERM REVIEW CONTAINMENT PURGE AND VENT (B-24)		1	07/30/84
07936	DRESDEN 3 - MARK I CONTAINMENT LONG TERM PROGRAM IMPLEMENTATION		1	07/31/84
43722	DRESDEN 3 - SAFETY CONCERNS ASSOCIATED WITH PIPE BREAKS IN THE BWR SCRAM SYSTEM		1	07/31/84
42879	DRESDEN 3 - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11		2	08/31/84
46659	DRESDEN 3 - IMPLEMENTATION OF NUREG-0313, REV. 1		2	12/31/84
52224	DRESDEN 3 - CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MP#C-10)			09/30/85
TAC #	TAC DESCRIPTION	TMI ACTIONS	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				
44369	DRESDEN 3 - NUREG-0737 II.B.1, RCS HIGH POINT VENTS		1	<u>COMPLETE</u> 10/03/83
45625	DRESDEN 3 - NUREG-0737 II.K.3.17, ECC SYSTEM OUTAGES		1	12/14/83
45822	DRESDEN 3 - NUREG-0737 II.K.3.30, SB LOCA OUTLINE		1	01/23/84
45908	DRESDEN 3 - NUREG-0737 II.K.3.45, MANUAL DEPRESSURIZATION		1	01/23/84
48161	DRESDEN 3 -NUREG 0737 II.K.3.31 COMPLIANCE WITH 10 CFR 50.46		1	01/23/84
<u>ACTIVE ACTIONS</u>				
51156	DRESDEN 3 - I.D.1 CONTROL ROOM DESIGN REVIEW			<u>TARGET</u>
51236	DRESDEN 3 - I.D.2. SAFETY PARAMETER DISPLAY SYSTEM			
44228	DRESDEN 3 - NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES		1	04/30/83
44298	DRESDEN 3 - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES		1	04/30/83
48267	DRESDEN 3 - NUREG 0737 II.K.3.28, VERIFY QUALIFICATION OF ACCUMULATORS ON ADS VALVES		1	09/30/83
44157	DRESDEN 3 - NUREG-0737 I.A.2.1.4, UPGRADING OF RO AND SRO TRAINING		1	02/28/84
44437	DRESDEN 3 - NUREG-0737 II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS		1	02/28/84
45130	DRESDEN 3 - NUREG-0737 II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION		1	04/30/84
45592	DRESDEN 3 - NUREG-0737 II.K.3.16, CHALLENGES AND FAILURES OF RELIEF VALVES		1	04/30/84
45688	DRESDEN 3 - NUREG-0737 II.K.3.18, ADS ACTUATION STUDY		1	06/30/84
45941	DRESDEN 3 - NUREG-0737 III.A.1.2, TECHNICAL SUPPORT CENTER		1	09/30/84
46012	DRESDEN 3 - NUREG-0737 III.A.1.2, OPERATIONAL SUPPORT CENTER		1	09/30/84
46084	DRESDEN 3 - NUREG-0737 III.A.1.2, EMERGENCY OPERATIONS FACILITY		1	09/30/84
46300	DRESDEN 3 - NUREG-0737 III.A.2.2, METEOROLOGICAL DATA UPGRADE		1	09/30/84

DATA AS OF - 04/30/84

CONDENSED MANAGEMENT REPORT

(CONTINUATION)

DRESDEN 3

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
51781	DRESDEN 3 - TECH SPEC CHANGES FOR MECHANICAL AND HYDRAULIC SNUBBERS			<u>COMPLETE</u> 10/21/83
51519	DRESDEN 3 - LIMIT REQUIREMENT TO VERIFY NUCLEAR RESPONSE TO CRD MOTION			11/03/83
08536	DRESDEN 3 - CONTROL ROD COUPLING VERIFICATION		1	11/03/83
51524	DRESDEN 3 - NUREG 0737 - ITEM III.K.3.15 - ISOLATION OF HPCI AND RCIC			12/12/83
51546	DRESDEN 3 - CONTROL ROD DRIVE SCRAM TESTING TECHNICAL SPECIFICATIONS			12/12/83
52177	DRESDEN 3 - TECHNICAL SPECIFICATION CHANGE CONCERNING SAFETY/RELIEF VALVE SETPOINTS			01/04/84
52361	DRESDEN 3 - PROPOSED T.S. RE RPS POWER MONITORING SYSTEM			02/14/84
52317	DRESDEN 3 - T.S. AMENDMENT RE S/R VALVE POSITION INDICATION			02/24/84
52034	DRESDEN 3 - TS AMENDMENT TO ALLOW USE OF ASEA-ATOM CRD BLADES FOR CYCLE 9 AND BEYOND			03/09/84
52176	DRESDEN 3 - CYCLE 9 RELOAD REVIEW			03/09/84
48321	DRESDEN 3 - ANALYSIS FOR REDUCED FLOW OPERATION		2	03/09/84
52679	DRESDEN 3 - CYCLE 9 RELOAD IGSCC PIPE CRACK INSPECTION			03/15/84
54303	DRESDEN 3 - CONTROL RAD. CONT. MATERIALS - DEMINIMUS LEVELS			04/13/84
47095	DRESDEN 3-ISI PROGRAM, SECOND 10 YRS		1	04/26/84
<u>ACTIVE ACTIONS</u>				
54493	DRESDEN 3 - T/S CHANGE RE EXTENSION OF CERTAIN MAPLHGR CURVES			<u>TARGET</u>
51517	DRESDEN 3 - ECONOMIC GENERATION CONTROL			
54773	DRESDEN 3 - EMERGENCY EXERCISE EXEMPTION REVIEW			
48259	DRESDEN 3 - PROPOSED TS CHANGE RE SAFETY/RELIEF VALVE POSITION INDICATION		3	02/28/84
54151	DRESDEN 3 - REVIEW OF REVISED LICENSED OPERATOR REQUALIFICATION TOPICAL REPORT			03/30/84
51067	DRESDEN UNIT 3- MULTIPLE CHANGES TO ADMINISTRATIVE CONTROLS			03/31/84
54770	DRESDEN 3 - REVIEW OF MOBILE VOLUME REDUCTION SYSTEM			04/30/84
54239	DRESDEN 3 - T.S. CHANGE FOR SEP TOPIC VI-7.C.1-125/250 V BATTERIES			05/31/84
54240	DRESDEN 3 - T.S. CHANGE FOR SEP TOPIC XV-16-PRIMARY COOLANT ACTIVITY			05/31/84
54470	DRESDEN 3 - CHANGES TO TABLE 3.7.1, CONTAINMENT ISOLATION VALVES			05/31/84
54661	DRESDEN 3 - INSTALLATION OF BYPASS VALVE IN RWCU SYSTEM			06/29/84
54663	DRESDEN 3 - UPDATE OF SNUBBER TABLES			06/29/84
54665	DRESDEN 3 - REVISION OF ENTIRE TECHNICAL SPECIFICATIONS			06/29/84
48300	DRESDEN 3 - PROPOSED TS CHANGES RE APPENDICES H AND G, METAL SURVEILLANCE AND MIN TEMP REQ		3	06/30/84
47096	DRESDEN 3 - IST PROGRAM, SECOND 10 YRS		1	06/30/84
54530	DRESDEN 3 - TECHNICAL SPECIFICATIONS COVERED BY GENERIC LETTERS 83-36, 83-37 FOR NUREG-0737			09/30/84
48857	DRESDEN 3 - REVIEW CECO POSITION RE 10CFR50.44			12/31/84
<u>ANTICIPATED ACTIONS</u>				
53585	DRESDEN 3 - ITEM 1.2 - POST TRIP REVIEW-DATA AND INFORMATION CAPABILITY			<u>INITIATION</u> 11/01/84
53668	DRESDEN 3 - ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS			11/01/84
53751	DRESDEN 3- ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS			11/01/84
53834	DRESDEN 3 - ITEM 3.2.3 - POST-MAINTENANCE TESTING -CHANGES TO TECH. SPEC. - ALL SR COMPONENTS			11/01/84
53978	DRESDEN 3 - ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS			11/01/84
54061	DRESDEN 3 - ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL TESTING - DIVERSE TRIP FEATURES			11/01/84

DATA AS OF - 04/30/84

CONDENSED MANAGEMENT REPORT

FACILITY: DUANE ARNOLD

PLANT LOCATION: 8 MI NW OF CEDAR RAPIDS, IA
 DOCKET NUMBER: 050-00331
 ARCH/ENGINEER: BECH
 IE INSPECTOR: G. WRIGHT

LICENSED POWER: 1658 MWT
 DESIGN POWER: 0538 MWE
 NSSS VENDOR: GE

PROJECT MANAGER: M. THADANI
 BRANCH CHIEF: D. VASSALLO
 LIC. ASSISTANT: S. NORRIS

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>MULTI-PLANT</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
11035	DUANE ARNOLD - LEAK RATE TESTING		2	<u>COMPLETE</u> 01/17/84
12302	DUANE ARNOLD - HELB AND CONSEQUENTIAL SYSTEM FAILURE DUANE ARNOLD		3	01/31/84
51677	DUANE ARNOLD NUREG 0737 TECH SPECS			02/27/84
42575	DUANE ARNOLD - LONG TERM REVIEW CONTAINMENT PURGE AND VENT (B-24)		1	03/29/84
<u>ACTIVE ACTIONS</u>				
52225	DUANE ARNOLD - CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPA#C-10)			<u>TARGET</u>
52754	DUANE ARNOLD - ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION AND PROCEDURES			
52835	DUANE ARNOLD - ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RTS COMPONENTS			
52916	DUANE ARNOLD - ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING PROCEDURES/VENDOR RECOMMENDATIONS - RTS COMPONENTS			
52997	DUANE ARNOLD - ITEM 3.1.3 - POST-MAINT TESTING - CHANGES TO TECH SPECS - RTS COMPONENTS			
51087	DUANE ARNOLD - INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)			03/01/84
07937	DUANE ARNOLD - MARK I CONTAINMENT LONG TERM PROGRAM IMPLEMENTATION		1	05/31/84
07988	DUANE ARNOLD CONTROL OF HEAVY LOADS OVER SPENT FUEL POOL (NUREG 0612)		2	07/31/84
42880	DUANE ARNOLD - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11		2	08/08/84
42488	DUANE ARNOLD - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21)		1	08/10/84
43730	DUANE ARNOLD - SAFETY CONCERNS ASSOCIATED WITH PIPE BREAKS IN THE BWR SCRAM SYSTEM		1	08/31/84
08142	DUANE ARNOLD - APPENDIX I TECH SPEC IMPLEMENTATION REVIEW		3	09/15/84
46660	DUANE ARNOLD - IMPLEMENTATION OF NUREG-0313, REV. 1		2	10/31/84
<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>TMI ACTIONS</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
48162	DUANE ARNOLD - NUREG 0737 II.K.3.31 COMPLIANCE WITH 10 CFR 50.46		1	<u>COMPLETE</u> 12/29/83
45823	DUANE ARNOLD - NUREG-0737 II.K.3.30, SB LOCA OUTLINE		1	12/29/83
47855	DUANE ARNOLD - NUREG - 0737 II. 3.22B, RCIC SUCTION MODIFICATIONS		1	01/13/84
48268	DUANE ARNOLD - NUREG 0737 II.K.3.28, VERIFY QUALIFICATION OF ACCUMULATORS ON ADS VALVES		1	04/11/84
<u>ACTIVE ACTIONS</u>				
51157	DUANE ARNOLD - I.D.1 CONTROL ROOM DESIGN REVIEW			<u>TARGET</u>
51237	DUANE ARNOLD - I.D.2. SAFETY PARAMETER DISPLAY SYSTEM			
44299	DUANE ARNOLD - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES		1	11/30/83
45593	DUANE ARNOLD - NUREG-0737 II.K.3.16, CHALLENGES AND FAILURES OF RELIEF VALVES		1	03/31/84
45689	DUANE ARNOLD - NUREG-0737 II.K.3.18, ADS ACTUATION STUDY		1	03/31/84
44847	DUANE ARNOLD - NUREG-0737 II.E.4.2, CONTAINMENT ISOLATION DEPENDABILITY		1	03/31/84
44578	DUANE ARNOLD - RV AND SV TESTING		1	04/30/84
45131	DUANE ARNOLD - NUREG-0737 II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION		1	05/14/84
44229	DUANE ARNOLD - NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES		1	05/14/84
44438	DUANE ARNOLD - NUREG-0737 II.C.3.2, POST ACCIDENT SAMPLING MODIFICATIONS		1	05/31/84
47935	DUANE ARNOLD - PLANT SHIELDING MODIFICATIONS (II.B.2.2)		1	06/15/84
46301	DUANE ARNOLD - NUREG-0737 III.A.2.2, METEOROLOGICAL DATA UPGRADE		1	08/01/84
45942	DUANE ARNOLD - NUREG-0737 III.A.1.2, TECHNICAL SUPPORT CENTER		1	09/30/84
46013	DUANE ARNOLD - NUREG-0737 III.A.1.2, OPERATIONAL SUPPORT CENTER		1	09/30/84
46085	DUANE ARNOLD - NUREG-0737 III.A.1.2, EMERGENCY OPERATIONS FACILITY		1	09/30/84

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CONDENSED MANAGEMENT REPORT

(CONTINUATION)

DUANE ARNOLD

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>IMI ACTIONS</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>ACTIVE ACTIONS (CONTINUATION)</u>				
46229	DUANE ARNOLD - III.A.2.1 EMERGENCY PLAN UPGRADE TO MEET RULE		1	<u>TARGET</u> 09/30/84
<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
41062	DUANE ARNOLD - DC POWER SUPPLY FAILURE EFFECT ECCS		1	<u>COMPLETE</u> 10/24/83
51883	DAEC - ORDER ON LONG TERM SDV MODS			12/05/83
49452	DAEC - ADDITIONAL APP R EXEMPTION REQUEST			12/19/83
51764	DAEC-MOD OF TURBINE CONTROL VLV OPENING (RTS-151)			01/23/84
54172	DAEC - TECH. SPEC. AMENDMENTS FOR NUREG-0737 RTS-157			02/09/84
47427	DAEC - ADMINISTRATIVE T.S. CHANGES (RTS-130)		3	02/14/84
53254	DAEC - REVIEW OF 10/28/83 REACTOR SCRAM			02/28/84
06395	DUANE ARNOLD - VERIFICATION OF SENSOR RESPONSE TIME		1	03/08/84
49334	DAEC - ADMIN CHANGES CONCERNING SAFETY COMM, SECURITY PLAN, ETC., RTS - 145		3	03/13/84
52082	DAEC - NUREG 0737 II.K.3.3 SRV CHALLENGE & FAILURE OR MALFUNCTION REPORTS (RTS-153)			03/20/84
52083	DAEC - NUREG-0737-II.K.3.13B & II.K.3.22 RCIC RESET & RCIC SUCTION (RTS-154)			04/13/84
<u>ACTIVE ACTIONS</u>				
51861	DAEC - CONTAINMENT PURGE & VENT VALVE OPERABILITY			<u>TARGET</u>
53474	DAEC REVIEW OF TS AMENDMENT RTS 143			
54173	DAEC - REVISION TO ENVIRONMENTAL TECH. SPECS. - ETS-30			
54227	DAEC - RTS-157 FOR II.B.3			
54228	DAEC RTS-157 CHANGES FOR II.F.1-1 AND II.F.1-2			
54229	DAEC RTS-157 CHANGES FOR II.F.1.3			
54230	DAEC RTS-157 CHANGES FOR II.F.1-4 THROUGH 6			
54231	DAEC RTS-157 CHANGES FOR III.D.3.4			
54232	DAEC - TECH. SPEC. CHANGES - MINIMUM STAFFING REQUIREMENTS			
54312	DAEC - RTS-163 TO INCORPORATE ADDITIONAL MAPLHGR LIMITS			
54679	DUANE ARNOLD - MATERIALS OF INDETERMINATE QUALITY - TIA			
54855	DAEC - RTS 160 REPORTING REQUIREMENTS			
52084	DAEC - ANALYSIS OF REDUCED RHR SERVICE WTR FLOW (RTS-152)			03/31/84
52205	DAEC SURVEILLANCE REQUIREMENTS FOR THE LOW-LOW SET LOGIC FUNCTION (RTS-155)			04/30/84
47837	DUANE ARNOLD ENERGY CENTER - APP FOR 50% POWER OPERATION WITH SINGLE RECIRC LOOP		3	06/30/84
48521	DAEC - REVIEW & EVAL OF PROPOSED CHGS CONCERNING CONTAINMENTVENT & PURGE SYS		1	07/29/84
54429	DAEC (DUANE ARNOLD) - INSERVICE INSPECTION/TESTING			08/12/84
48230	DUANE ARNOLD - SCHEDULAR EXEMPTION RE H2 RECOMBINERS		1	11/05/84
<u>ANTICIPATED ACTIONS</u>				
53586	DUANE ARNOLD - ITEM 1.2 - POST TRIP REVIEW-DATA AND INFORMATION CAPABILITY			<u>INITIATION</u> 11/01/84
53669	DUANE ARNOLD - ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS			11/01/84
53752	DUANE ARNOLD- ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS			11/01/84
53835	DUANE ARNOLD - ITEM 3.2.3 - POST-MAINTENANCE TESTING -CHANGES TO TECH. SPEC. - ALL SR COMPONENTS			11/01/84
53979	DUANE ARNOLD - ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS			11/01/84
54062	DUANE ARNOLD - ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL TESTING - DIVERSE TRIP FEATURES			11/01/84
54171	DAEC - SDV VALVE TEST/RTS-158			11/01/84

DATA AS OF - 04/30/84

CONDENSED MANAGEMENT REPORT

FACILITY: FARLEY 1

PLANT LOCATION: 28 MI SE OF DOTHAN, ALA
 DOCKET NUMBER: 050-00348
 ARCH/ENGINEER: SSI
 IE INSPECTOR: W. BRADFORD

LICENSED POWER: 2652 MWT
 DESIGN POWER: 0829 MWE
 NSSS VENDOR: WEST

PROJECT MANAGER: E. REEVES
 BRANCH CHIEF: S. VARGA
 LIC. ASSISTANT: C. PARRISH

TAC #	TAC DESCRIPTION	MULTI-PLANT	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				
47172	FARLEY 1 NATURAL CIRCULATION COOLDOWN (GENERIC LTR 81-21)		2	<u>COMPLETE</u> 11/15/83
49418	FARLEY 1 - BLOCKING SI SIGNAL DURING COOLDOWN		2	01/09/84
12266	FARLEY 1 - HELB AND CONSEQUENTIAL SYSTEM FAILURE JOSEPH M.		3	01/31/84
<u>ACTIVE ACTIONS</u>				
52755	FARLEY 1 - ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION AND PROCEDURES			<u>TARGET</u>
52836	FARLEY 1 - ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RTS COMPONENTS			
52917	FARLEY 1 - ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING PROCEDURES/VENDOR RECOMMENDATIONS - RTS COMPONENTS			
52998	FARLEY 1-ITEM 3.1.3 -POST-MAINTENANCE TESTING-CHANGES TO TECH SPECS-RTS COMPONENTS			
53069	FARLEY 1 - REACTOR TRIP SYSTEM RELIABILITY- VENDOR RELATED MODIFICATIONS			
53122	FARLEY 1 - ITEMS 4.2.1 & 4.2.2 - PREVENTATIVE MAINTENANCE PROGRAM FOR REACTOR TRIP BREAKERS - MAINTENANCE & TRENDING			
53172	FARLEY 1 - ITEM 4.3 - AUTOAMTIC ACTUATION OF SHUNT TRIP ATTACHMENT FOR W AND B & W PLANTS		2	06/05/84
42861	FARLEY 1 - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11		2	06/30/84
49731	FARLEY 1 -"NUREG 0737 TECH SPECS (GENERIC LETTER 82-16 82-16 OR 83-02)"		2	06/30/84
48569	FARLEY 1 - REQUEST FOR 5 EXEMPTIONS FOR IMPLEMENTATION DATES FOR FIRE PROTECTION MODIFICATIONS		2	06/30/84
42462	FARLEY 1 - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21)		1	06/30/84
49688	FARLEY 1 - REACTOR COOLANT PUMP TRIP		2	07/01/84
51088	FARLEY 1 - INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)		2	07/30/84
07989	FARLEY 1 -CONTROL OF HEAVY LOADS OVER SPENT FUEL POOL (NUREG 0612)		2	09/30/84
52226	FARLEY 1 - CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPA#C-10)		3	09/30/85
TAC #	TAC DESCRIPTION	TMI ACTIONS	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				
44371	FARLEY 1 - NUREG-0737 II.B.1, RCS HIGH POINT VENTS		1	<u>COMPLETE</u> 11/07/83
47936	FARLEY 1 - PLANT SHIELDING MODIFICATIONS (II.B.2.2)		1	12/15/83
47797	FARLEY 1 - NUREG 0737 ITEM II K2.17 POTENTIAL FOR VOIDING IN RCS		1	01/10/84
<u>ACTIVE ACTIONS</u>				
46302	FARLEY 1 - NUREG-0737 III.A.2.2, METEOROLOGICAL DATA UPGRADE		1	<u>TARGET</u>
45824	FARLEY 1 - NUREG-0737 II.K.3.30, SB LOCA OUTLINE		1	06/30/84
44439	FARLEY 1 - NUREG-0737 II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS		1	06/30/84
44579	FARLEY 1 - RV AND SV TESTING		1	07/30/84
51158	FARLEY 1 - I.D.1 CONTROL ROOM DESIGN REVIEW		2	08/30/84
51238	FARLEY 1 - I.D.2. SAFETY PARAMETER DISPLAY SYSTEM		2	08/30/84
45132	FARLEY 1 - NUREG-0737 II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION		1	09/30/84
45943	FARLEY 1 - NUREG-0737 III.A.1.2, TECHNICAL SUPPORT CENTER		1	09/30/84
46014	FARLEY 1 - NUREG-0737 III.A.1.2, OPERATIONAL SUPPORT CENTER		1	09/30/84
46086	FARLEY 1 - NUREG-0737 III.A.1.2, EMERGENCY OPERATIONS FACILITY		1	09/30/84
46886	FARLEY 1 - II K2.13 THERMAL MECHANICAL REPORT		1	09/30/84
48163	FARLEY 1 -NUREG 0737 II.K.3.31 COMPLIANCE WITH 10 CFR 50.46		1	09/30/84

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CONDENSED MANAGEMENT REPORT

(CONTINUATION)

FARLEY 1

TAC #	TAC DESCRIPTION	PLANT SPECIFIC	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				<u>COMPLETE</u>
49184	FARLEY 1 - CORPORATE ORGANIZATIONAL CHANGE AND OTHER ADMIN CHANGES		2	10/14/83
51999	FARLEY 1 - ADMINISTRATIVE TECHNICAL SPECIFICATION CHANGES		2	10/14/83
52499	FARLEY 1 - SUMP MONITOR INOPERABLE		1	10/14/83
51437	FARLEY 1 - ADD BACKUP MET TOWER TO TECH SPEC TABLES 3.3-8 AND 4.3.5		2	10/21/83
52105	FARLEY 1 - REQUEST FOR EXTENSION OF EQ FOR RG1.97 ITEMS		1	10/21/83
51891	FARLEY 1 - ONE TIME EXTENSION OF VISUAL INSPECTION OF TWO HYDRAULIC SNUBBERS		1	10/31/83
52562	FARLEY 1 - ONE-TIME ACCEPTANCE OF CORE FLUXMAP OF 10/13/83 (TS 3.3.3.2A)		1	11/01/83
51436	FARLEY 1 - COMPLETION OF GENERIC LETTER 83-07 (WASTE POLICY ACT 1982) ACTION		2	12/15/83
52042	FARLEY 1 - T.S. CHANGE FOR SHIFT MANNING (GEN LTR 82-02 AND 82-12)		2	12/30/83
52565	FARLEY 1 - TECH SPEC CHANGES FOR POS MTC OPERATION		1	12/30/83
53268	FARLEY 1 - FIRE PROTECTION EXEMPTION FOR SEC III.G.2 IN CONTAINMENT		1	12/30/83
49068	FARLEY 1 - CHANGE TS 6.5.2.8.E PER GENERIC LETTER 82-17		2	01/06/84
52449	FARLEY 1 - REDESIGNATE PRIMARY PRESSURE ISOLATION VALVES IN 1ST PROGRAM		2	01/26/84
52015	FARLEY 1 - 1ST SER CHANGES PLUS ONE RELIEF REEVALUATION		2	01/26/84
52539	FARLEY 1 - ADD TURBINE OVERSPEED PROTECTIONS TS 3/4.3.4		1	01/27/84
52181	FARLEY 1 - FIRE SYSTEM TECH SPEC CHANGE (3 ADMINS)		2	02/09/84
49547	FARLEY 1 - REQUEST FOR ADDITIONAL RELIEF FROM ISI REQUIREMENTS		2	02/10/84
52618	FARLEY 1 - CHANGE ACTION STATEMENTS FOR (LCO 3.1.3.1) CONTROL RODS		2	02/17/84
52298	FARLEY 1 - CORRECT MECHANICAL AND HYDRAULIC SNUBBER LISTS		2	02/28/84
51537	FARLEY 1 - REACTOR TRIP BREAKERS (DS-416)		2	03/19/84
54471	FARLEY 1 - RELIEF FROM ASME CODE HYDRO ON SG REDUCER WELDS		1	03/30/84
54233	FARLEY 1 - EXTENSION REQUEST POST ACCIDENT EQUIPMENT PER 50.49		1	04/16/84
<u>ACTIVE ACTIONS</u>				<u>TARGET</u>
54237	FARLEY 1 - CHANGE INOPERABLE CONTROL ROD TIME TO 36 HOURS		2	05/25/84
49374	FARLEY 1 - REVISE DG OVERLOAD TEST AND DELETE RIVER WATER SYSTEM TECH SPECS		2	05/31/84
54350	FARLEY 1-TECH SPECS FOR PURGE/VENT SYSTEM		2	06/15/84
49953	FARLEY 1 - CHANGES TO CHARCOAL FILTER TECH SPECS (3/4.7.7, 3/4.7.8 AND 3/4.9.14)		2	06/22/84
52020	FARLEY 1 - UV CONFIRMATORY LIFE TEST PROGRAM (DS-416 BREAKER)		2	06/30/84
51629	FARLEY 1 - REVISE TECH SPECS FOR BATTERIES PER IEEE 450-1980		2	07/02/84
54450	FARLEY 1 - REVISE MET TOWER TECH SPECS		2	07/08/84
54235	FARLEY 1 - REPORTING REQUIREMENTS TS PER GEN LTR 83-43		2	08/06/84
54452	FARLEY 1 - ADMIN TECH SPEC CHANGES		2	09/01/84
53468	FARLEY 1 - EXEMPTION FOR USE OF RADIOIODINE CANISTER		2	09/09/84
48714	FARLEY 1 - PRESSURE ISOLATION VALVE TS CHANGE IN CRITERIA		2	09/28/84
54531	FARLEY 1 - TECHNICAL SPECIFICATIONS COVERED BY GENERIC LETTERS 83-36, 83-37 FOR NUREG-0737		2	09/30/84
54389	FARLEY 1- RCS VENTS TECHNICAL SPECIFICATIONS NUREG 073 ITEM II.B.1			09/30/84
54672	FARLEY 1 - TECH. SPEC. CHANGES FOR INCREASED FUEL ENRICHMENT			10/01/84
54449	FARLEY 1 - REACTOR VESSEL CAPSULE REPORT 3:02 EFPY			10/14/84
54894	FARLEY 1 - TECH. SPECS. FOR 7.0 EFPY			10/22/84
<u>ANTICIPATED ACTIONS</u>				<u>INITIATION</u>
53587	FARLEY 1 - ITEM 1.2 - POST TRIP REVIEW-DATA AND INFORMATION CAPABILITY			11/01/84
53670	FARLEY 1 - ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS			11/01/84
53753	FARLEY 1- ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS			11/01/84
53836	FARLEY 1 - ITEM 3.2.3 - POST-MAINTENANCE TESTING -CHANGES TO TECH. SPEC. - ALL SR COMPONENTS			11/01/84
53909	-ITEMS 4.2.3 & 4.2.4-PREVENTATIVE MAINT. PROGRAM FOR REACTOR TRIP BREAKERS-LIFE TESTING & REPLACEMENT			11/01/84
53980	FARLEY 1 - ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS			11/01/84

TECHNICAL ASSIGNMENT CONTROL SYSTEM
CONDENSED MANAGEMENT REPORT

R-1208620-001

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(CONTINUATION)

IAC # IAC DESCRIPTION

54063 ANTICIPATED ACTIONS (CONTINUATION)
FARLEY 1 - ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL TESTING - DIVERSE TRIP FEATURES

FARLEY 1

PLANT SPECIFIC

PRIORITY

CRITICAL DATE

INITIATION
11/01/84

DATA AS OF - 04/30/84

CONDENSED MANAGEMENT REPORT

FACILITY: FARLEY 2

PLANT LOCATION: 28 MI SE OF DOTHAN, ALA
 DOCKET NUMBER: 050-00364
 ARCH/ENGINEER: SSI
 IE INSPECTOR: W. BRADFORD

LICENSED POWER: 2652 MWT
 DESIGN POWER: 0829 MWE
 NSSS VENDOR: WEST

PROJECT MANAGER: E. REEVES
 BRANCH CHIEF: S. VARGA
 LIC. ASSISTANT: C. PAARISH

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>MULTI-PLANT</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
47173	FARLEY 2 NATURAL CIRCULATION COOLDOWN (GENERIC LTR 81-21)		2	<u>COMPLETE</u> 11/14/83
49419	FARLEY 2 - BLOCKING SI SIGNAL DURING COOLDOWN		2	01/09/84
<u>ACTIVE ACTIONS</u>				
52756	FARLEY 2 - ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION AND PROCEDURES		2	<u>TARGET</u>
52837	FARLEY 2 - ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RTS COMPONENTS		2	
52918	FARLEY 2 - ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING PROCEDURES/VENDOR RECOMMENDATIONS - RTS COMPONENTS		2	
52999	FARLEY 2-ITEM 3.1.3 -POST-MAINTENANCE TESTING-CHANGES TO TECH SPECS-RTS COMPONENTS		2	
53070	FARLEY 2 - REACTOR TRIP SYSTEM RELIABILITY- VENDOR RELATED MODIFICATIONS		2	
53123	FARLEY 2 - ITEMS 4.2.1 & 4.2.2 - PREVENTATIVE MAINTENANCE PROGRAM FOR REACTOR TRIP BREAKERS - MAINTENANCE & TRENDING		2	
53173	FARLEY 2 - ITEM 4.3 - AUTOAMTC ACTUATION OF SHUNT TRIP ATTACHMENT FOR W AND B & W PLANTS		2	
48016	FARLEY 2 - REGUEST TO DELETE LIC CONDITION 2 C (19) (D) TURBINE DISC INSPECT OR REPLACMNT AT 1ST REFUELING OUTAGE		2	05/30/84
49732	FARLEY 2 -"NUREG 0737 TECH SPECS (GENERIC LETTER 82-16 82-16 OR 83-02)"		2	06/30/84
48570	FARLEY 2 - REQUEST FOR 5 EXEMPTIONS FOR IMPLEMENTATION DATES FOR FIRE PROTECTION MODIFICATIONS		2	06/30/84
46590	FARLEY 2 - EQUIPMENT ENVIRONMENTAL QUALIFICATION (NUREG 0588)		2	06/30/84
49689	FARLEY 2 - REACTOR COOLANT PUMP TRIP		2	07/01/84
51089	FARLEY 2 - INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)		2	07/30/84
43965	FARLEY 2 -CONTROL OF HEAVY LOADS OVER SPENT FUEL POOL (NUREG 0612)		3	09/30/84
52227	FARLEY 2 - CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPA#C-10)			09/30/85
<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>TMI ACTIONS</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
44372	FARLEY 2 - NUREG-0737 II.B.1, RCS HIGH POINT VENTS		1	<u>COMPLETE</u> 11/07/83
47937	FARLEY 2 - PLANT SHIELDING MODIFICATIONS (II.B.2.2)		1	12/15/83
47798	FARLEY 2 - NUREG 0737 ITEM II K2.17 POTENTIAL FOR VOIDING IN RCS		1	01/10/84
<u>ACTIVE ACTIONS</u>				
46303	FARLEY 2 - NUREG-0737 III.A.2.2, METEOROLOGICAL DATA UPGRADE		1	<u>TARGET</u>
44440	FARLEY 2 - NUREG-0737 II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS		1	06/30/84
45133	FARLEY 2 - NUREG-0737 II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION		1	06/30/84
45825	FARLEY 2 - NUREG-0737 II.K.3.30, SB LOCA OUTLINE		1	06/30/84
44580	FARLEY 2 - RV AND SV TESTING		1	07/30/84
51159	FARLEY 2 - I.D.1 CONTROL ROOM DESIGN REVIEW		2	08/30/84
51239	FARLEY 2 - I.D.2. SAFETY PARAMETER DISPLAY SYSTEM		2	08/30/84
45944	FARLEY 2 - NUREG-0737 III.A.1.2, TECHNICAL SUPPORT CENTER		1	09/30/84
46015	FARLEY 2 - NUREG-0737 III.A.1.2, OPERATIONAL SUPPORT CENTER		1	09/30/84
46087	FARLEY 2 - NUREG-0737 III.A.1.2, EMERGENCY OPERATIONS FACILITY		1	09/30/84
46887	FARLEY 2 - II K2.13 THERMAL MECHANICAL REPORT		1	09/30/84
48164	FARLEY 2 -NUREG 0737 II.K.3.31 COMPLIANCE WITH 10 CFR 50.46		1	09/30/84

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CONDENSED MANAGEMENT REPORT

(CONTINUATION)

FARLEY 2

TAC #	TAC DESCRIPTION	PLANT SPECIFIC	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				<u>COMPLETE</u>
49185	FARLEY 2 - CORPORATE ORGANIZATIONAL CHANGE AND OTHER ADMIN CHANGES		2	10/14/83
52000	FARLEY 2 - ADMINISTRATIVE TECHNICAL SPECIFICATION CHANGES		2	10/14/83
51438	FARLEY 2 - ADD BACKUP MET TOWER TO TECH SPEC TABLES 3.3-8 AND 4.3.5		2	10/21/83
52106	FARLEY 2 - REQUEST FOR EXTENSION OF EQ FOR RG1.97 ITEMS		1	10/21/83
53308	FARLEY 2 - COMPLETION OF GENERIC LETTER 83-07 (WASTE POLICY ACT 1982) ACTION		2	12/15/83
52566	FARLEY 2 - TECH SPEC CHANGES FOR POS MTC OPERATION		2	12/30/83
52043	FARLEY 2 - T.S. CHANGE FOR SHIFT MANNING (GEN LTR 82-02 AND 82-12)		2	12/30/83
53269	FARLEY 2 - FIRE PROTECTION EXEMPTION FOR SEC III.G.2 IM CONTAINMENT		1	12/30/83
49069	FARLEY 2 - CHANGE TS 6.5.2.8.E PER GENERIC LETTER 82-17		2	01/06/84
49287	FARLEY 2 - DELETE EIGHT NON-SAFETY RELATED HYD SNUBBERS TS TABLES 3.7-4A		2	01/10/84
52450	FARLEY 2 - REDESIGNATE PRIMARY PRESSURE ISOLATION VALVES IN 1ST PROGRAM		2	01/26/84
52182	FARLEY 2 - FIRE SYSTEM TECH SPEC CHANGES (3 ADMINS)		2	02/09/84
52619	FARLEY 2 - CHANGE ACTION STATEMENTS FOR (LCO 3.1.3.1) CONTROL RODS		2	02/17/84
51536	FARLEY 2 - REACTOR TRIP BREAKERS (DS-416)		2	03/19/84
<u>ACTIVE ACTIONS</u>				<u>TARGET</u>
54238	FARLEY 2 - CHANGE INOPERABLE CONTROL ROD TIME TO 36 HOURS		2	05/25/84
49220	FARLEY 2 - DELETE LICENSE COND. 2.C.(21) (I) (4) SMALL BREAK LOCA MODEL		2	05/30/84
53266	FARLEY 2 - REACTOR VESSEL CAPSULE REPORT 1.1. EFPY		3	05/30/84
49182	FARLEY 2 - UPDATE TS FOR FIRST OUTAGE DESIGN CHANGES		2	05/30/84
49212	FARLEY 2 - DELETE LICENSE COND. 2.C.(18) ENVIRONMENTAL QUALIFICATION (NUREG-0588)		2	05/30/84
49213	FARLEY 2 - DELETE LICENSE COND. 2.C.(20) REGULATORY GUIDE 1.97		2	05/30/84
49214	FARLEY 2 - DELETE LICENSE COND. 2.C.(21) EMERGENCY OPERATING PROCEDURES		2	05/30/84
49215	FARLEY 2 - DELETE LICENSE COND. 2.C.(21) (B) REACTOR COOLANT SYS. VENTS		2	05/30/84
49216	FARLEY 2 - DELETE LICENSE COND. 2.C.(21) (G) RV LEVEL INSTR		2	05/30/84
49217	FARLEY 2 - DELETE LICENSE COND. 2.C.(21) (H) (1) ANALYSIS OF THERMAL MECH COND		2	05/30/84
49218	FARLEY 2 - DELETE LICENSE COND. 2.C.(21) (H) (2) VOIDING IN RCS		2	05/30/84
49021	FARLEY 2 - EVALUATE SAFETY GRADE BACKUP MEANS OF RCS DEPRESSURIZATION (LICENSE CONDITION 2.C.(12)(B))		3	05/30/84
49067	FARLEY 2 - PAD 3.3 AS APPLICABLE TO SUCCESSIVE FUEL CYCLES (LIC COND 2.D.(19)(A))		3	05/30/84
49375	FARLEY 2 - REVISE DG OVERLOAD TEST AND DELETE RIVER WATER SYSTEM TECH SPECS		2	05/31/84
49954	FARLEY 2 - CHANGES TO CHARCOAL FILTER TECH SPECS (3/4.7.7, 3/4.7.8 AND 3/4.9.14)		2	06/22/84
47479	FARLEY 2 - MODIFIED CONTAINMENT VENT AND PURGE SYSTEM (LIC COND 2.C.(17))		3	06/28/84
52021	FARLEY 2 - UV CONFIRMATORY LIFE TEST PROGRAM (DS-416 BREAKERS)		2	06/30/84
51631	FARLEY 2 - REVISE TECH SPECS FOR BATTERIES PER IEEE 450-1980		2	07/02/84
53402	FARLEY 2 - SAFETY RELATED SNUBBER TS TABLE 3.7-4A		2	07/06/84
54451	FARLEY 2 - REVISE MET TOWER TECH SPECS		2	07/08/84
54236	FARLEY 2 - REPORTING REQUIREMENTS TS PER GEN LTR 83-43		2	08/06/84
54453	FARLEY 2 - ADMIN TECH SPEC CHANGES		2	09/01/84
53469	FARLEY 2 - EXEMPTION FOR USE OF RADIOIODINE CANISTER		2	09/09/84
48715	FARLEY 2 - PRESSURE ISOLATION VALVE TS CHANGE IN CRITERIA		2	09/28/84
54532	FARLEY 2 - TECHNICAL SPECIFICATIONS COVERED BY GENERIC LETTERS 83-36, 83-37 FOR NUREG-0737		2	09/30/84
54390	FARLEY 2 - RCS VENTS TECHNICAL SPECIFICATIONS NUREG 073 ITEM II.B.1		2	09/30/84
54673	FARLEY 2 - TECH. SPEC. CHANGES FOR INCREASED FUEL ENRICHMENT			10/01/84
54234	FARLEY 2 - REVISE TECH SPECS HEATUP/COOLDOWN CURVES		3	01/10/85
<u>ANTICIPATED ACTIONS</u>				<u>INITIATION</u>
53910	-ITEMS 4.2.3 & 4.2.4-PREVENTATIVE MAINT. PROGRAM FOR REACTOR TRIP BREAKERS-LIFE TESTING & REPLACEMENT		2	11/01/84
53981	FARLEY 2 - ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS		2	11/01/84

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CONDENSED MANAGEMENT REPORT

(CONTINUATION)

FARLEY 2

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>ANTICIPATED ACTIONS (CONTINUATION)</u>				
54064	FARLEY 2 - ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL TESTING - DIVERSE TRIP FEATURES		2	<u>INITIATION</u> 11/01/84
53588	FARLEY 2 - ITEM 1.2 - POST TRIP REVIEW-DATA AND INFORMATION CAPABILITY		2	11/01/84
53671	FARLEY 2 - ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS		2	11/01/84
53754	FARLEY 2- ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS		2	11/01/84
53837	FARLEY 2 - ITEM 3.2.3 - POST-MAINTENANCE TESTING -CHANGES TO TECH. SPEC. - ALL SR COMPONENTS		2	11/01/84

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CONDENSED MANAGEMENT REPORT

FACILITY: FITZPATRICK

PLANT LOCATION: 8 MI NE OF OSWEGO, NY
 DOCKET NUMBER: 050-00333
 ARCH/ENGINEER: S&W
 IE INSPECTOR: J. LINVILLE

LICENSED POWER: 2436 MWT
 DESIGN POWER: 0821 MWE
 NSSS VENDOR: GE

PROJECT MANAGER: H. ABELSON
 BRANCH CHIEF: D. VASSALLO
 LIC. ASSISTANT: S. NORRIS

TAC #	TAC DESCRIPTION	MULTI-PLANT	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				
08932	FITZPATRICK - RPS POWER SUPPLY		1	COMPLETE 11/07/83
10780	FITZPATRICK IST (A-14) REVIEW		1	12/05/83
12303	FITZPATRICK - HELB AND CONSEQUENTIAL SYSTEM FAILURE FITZPATRICK		3	01/31/84
J8315	FITZPATRICK - 10 CFR 50.55A(G) - INSERVICE INSPECTION - GENERIC		1	01/31/84
<u>ACTIVE ACTIONS</u>				
51090	FITZPATRICK - INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)			
52228	FITZPATRICK - CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPA#C-10)			
52757	FITZPATRICK - ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION AND PROCEDURES			
52838	FITZPATRICK - ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RTS COMPONENTS			
52919	FITZPATRICK 1 - ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING PROCEDURES/VENDOR RECOMMENDATIONS - RTS COMPONENTS			
53000	FITZPATRICK-ITEM 3.1.3 -POST-MAINTENANCE TESTING-CHANGES TO TECH SPECS-RTS COMPONENTS		2	05/15/84
43019	FITZPATRICK - TECHNICAL SPECIFICATIONS DEFINING OPERABILITY FOR SAFETY SYSTEMS		2	07/15/84
42881	FITZPATRICK - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11		2	07/15/84
80598	FITZPATRICK - HYDRAULIC SNUBBERS		2	07/15/84
42489	FITZPATRICK - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21)		1	07/15/84
10756	FITZPATRICK - MECHANICAL SNUBBERS		2	07/15/84
46661	FITZPATRICK - IMPLEMENTATION OF NUREG-0313, REV. 1		2	08/15/84
49371	FITZPATRICK - SINGLE LOOP OPERATION		2	08/15/84
08045	FITZPATRICK - APPENDIX I TECH SPECS (EVALUATE RESPONSE TO NRC LETTER OF 7/11/78.)		3	08/15/84
07938	FITZPATRICK - MARK I CONTAINMENT LONG TERM PROGRAM IMPLEMENTATION		1	08/15/84
42576	FITZPATRICK - LONG TERM REVIEW CONTAINMENT PURGE AND VENT (B-24)		1	09/15/84
51678	FITZPATRICK - NUREG 0737 TECH SPECS			09/15/84
07990	FITZPATRICK -CONTROL OF HEAVY LOADS OVER SPENT FUEL POOL (NUREG 0612)		2	10/15/84
<u>TMI ACTIONS</u>				
<u>COMPLETED ACTIONS</u>				
48165	FITZPATRICK -NUREG 0737 II.K.3.31 COMPLIANCE WITH 10 CFR 50.46		1	COMPLETE 12/29/83
45826	FITZPATRICK - NUREG-0737 II.K.3.30, SB LOCA OUTLINE			12/29/83
<u>ACTIVE ACTIONS</u>				
51160	FITZPATRICK - I.D.1 CONTROL ROOM DESIGN REVIEW			
51240	FITZPATRICK - I.D.2. SAFETY PARAMETER DISPLAY SYSTEM			
44779	FITZPATRICK - NUREG-0737 II.E.4.1.2, DEDICATED HYDROGEN PENETRATIONS		1	06/15/84
44441	FITZPATRICK - NUREG-0737 II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS		1	07/15/84
47599	FITZPATRICK - NUREG 0737 ITEM II F1.4 - CONTAINMENT PRESSURE INSTRUMENT		1	07/15/84
47670	FITZPATRICK - NUREG 0737 ITEM II F1.5 CONTAINMENT WATER LEVEL MONITOR		1	07/15/84
47741	FITZPATRICK - NUREG 0737 ITEM II F1.6 CONTAINMENT HYDROGEN MONITOR		1	07/15/84
48269	FITZPATRICK - NUREG 0737 II.K.3.28. VERIFY QUALIFICATION OF ACCUMULATORS ON ADS VALVES		1	08/15/84
45594	FITZPATRICK - NUREG-0737 II.K.3.16, CHALLENGES AND FAILURES OF RELIEF VALVES		1	08/15/84
46304	FITZPATRICK - NUREG-0737 III.A.2.2, METEOROLOGICAL DATA UPGRADE		1	09/15/84
45945	FITZPATRICK - NUREG-0737 III.A.1.2, TECHNICAL SUPPORT CENTER		1	09/30/84
46016	FITZPATRICK - NUREG-0737 III.A.1.2, OPERATIONAL SUPPORT CENTER		1	09/30/84

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CONDENSED MANAGEMENT REPORT

(CONTINUATION)

FITZPATRICK

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>TMI ACTIONS</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>ACTIVE ACTIONS</u> (CONTINUATION)				
46088	FITZPATRICK - NUREG-0737 III.A.1.2, EMERGENCY OPERATIONS FACILITY		1	TARGET 09/30/84
45134	FITZPATRICK - NUREG-0737 II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION		1	09/30/84
44232	FITZPATRICK - NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES		1	09/30/84
44302	FITZPATRICK - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES		1	09/30/84
<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
51390	FITZPATRICK - REVISE TS MCPR TABLE ODN CODE INPUT ERROR		1	COMPLETE 12/13/83
48350	FITZPATRICK - APPENDIX R EXEMPTIONS		3	02/01/84
46709	FITZPATRICK - SNUBBER SURVEILLANCE TECH. SPECS.			02/28/84
51725	FITZPATRICK - TS CHANGE RE PROCESS PIPING PENETRATING PRIMARY CONTAINMENT			03/20/84
54374	FITZPATRICK - EVALUATION OF CRACK IN RECIRC. SUCTION LINE			04/13/84
49232	FITZPATRICK - PROPOSED TS CHANGES RE MANAGEMENT ORGANIZATION STRUCTURE			04/24/84
<u>ACTIVE ACTIONS</u>				
53439	FITZPATRICK - TS CHANGE RE TURBINE CONTROL VALVE MODS			TARGET 05/15/84
52379	FITZPATRICK - EVALUATION OF PIPE SUPPORT ALLEGATIONS IN VCS 2.206 PETITION			05/15/84
49103	FITZPATRICK - PROPOSED TS CHANGE TO FIRE PROTECTION SYSTEM SURVEILLANCE REQUIREMENTS			05/15/84
54821	FITZPATRICK - SCHEDULED EXEMPTION FOR EMERGENCY PREPAREDNESS DRILL			06/15/84
53486	FITZPATRICK - SRO STAFFING EXEMPTION REQUEST		1	06/15/84
53560	FITZPATRICK - TS CHANGE CONTAINMENT ATM MONITOR SYSTEM ISOLATION		2	06/15/84
47394	FITZPATRICK - CONTAINMENT AIRLOCK TESTING		1	06/15/84
47525	FITZPATRICK - CONTROL ROD DRIVE SURVEILLANCE FREQUENCY		3	06/15/84
49915	FITZPATRICK - PROPOSED TS CHANGE RE SURVEILLANCE, AUDIT FREQUENCY, AND ADMINISTRATIVE ITEMS		1	07/15/84
06864	FITZPATRICK - MSIV LEAK DETECTION SYSTEM			07/15/84
54294	FITZPATRICK - DEDICATED HYDROGEN PENETRATIONS			07/15/84
53438	FITZPATRICK - TS CHANGE RE RBCLCWS ISOLATION VALVES			07/15/84
52147	FITZPATRICK - DISPOSAL OF CONTAMINATED FUEL OIL PER 10 CFR 20.302		3	07/15/84
47018	FITZPATRICK - NUREG-0313 TECHNICAL SPECIFICATIONS			08/15/84
53473	FITZPATRICK - TS CHANGES RE CONTAINMENT PURGE/VENT			09/15/84
54533	FITZPATRICK - TECHNICAL SPECIFICATIONS COVERED BY GENERIC LETTERS 83-36, 83-37 FOR NUREG-0737			09/30/84
<u>ANTICIPATED ACTIONS</u>				
53589	FITZPATRICK - ITEM 1.2 - POST TRIP REVIEW-DATA AND INFORMATION CAPABILITY			INITIATION 11/01/84
53672	FITZPATRICK - ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS			11/01/84
53755	FITZPATRICK - ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS			11/01/84
53838	FITZPATRICK - ITEM 3.2.3 - POST-MAINTENANCE TESTING -CHANGES TO TECH. SPEC. - ALL SR COMPONENTS			11/01/84
53982	FITZPATRICK - ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS			11/01/84
54065	FITZPATRICK - ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL TESTING - DIVERSE TRIP FEATURES			11/01/84

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CONDENSED MANAGEMENT REPORT

FACILITY: FORT CALHOUN 1

PLANT LOCATION: 19 MI N OF OMAHA, NEB
 DOCKET NUMBER: 050-00285
 ARCH/ENGINEER: GHDR
 IE INSPECTOR: L. YANDELL

LICENSED POWER: 1500 MWT
 DESIGN POWER: 0478 MWE
 NSSS VENDOR: COMB

PROJECT MANAGER: E. TOURIGNY
 BRANCH CHIEF: J. MILLER
 LIC. ASSISTANT: P. KREUTZER

TAC #	TAC DESCRIPTION	MULTI-PLANT	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				
42586	FT. CALHOUN 1 - LONG TERM REVIEW CONTAINMENT PURGE AND VENT (B-24)		1	COMPLETE 10/19/83
47180	FORT CALHOUN 1 - NATURAL CIRCULATION COOLDOWN (GENERIC LTR 81-21)		2	10/28/83
49614	FORT CALHOUN - PLANS FOR PREVENTING EXCEEDING PTS SCREENING CRITERION			11/23/83
12316	FT. CALHOUN 1 - HELB AND CONSEQUENTIAL SYSTEM FAILURE FT. CALHOUN		3	01/31/84
08922	FT. CALHOUN 1 - REVIEW OF ASYMETRIC LOCA LOADS SUBMITTAL		1	02/01/84
<u>ACTIVE ACTIONS</u>				
42892	FT. CALHOUN - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11		1	TARGET 06/30/84
43319	FORT CALHOUN 1 - CONTROL OF HEAVY LOADS OVER SPENT FUEL POOL (NUREG 0612)		1	06/30/84
42491	FT. CALHOUN 1 - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21)		1	06/30/84
49733	FORT CALHOUN - "NUREG 0737 TECH SPECS (GENERIC LETTER 82-16 82-16 OR 83-02)"			06/30/84
51755	FORT CALHOUN - THERMAL SHIELD FOLLOWUP ANALYSIS			07/15/84
49696	FORT CALHOUN 1 - REACTOR COOLANT PUMP TRIP			09/01/84
51091	FORT CALHOUN 1 - INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)			09/30/84
12738	FT. CALHOUN - ADEQUACY OF STATION ELECTRIC DISTRIBUTION SYSTEMS VOLTAGES		1	09/30/84
08143	FT. CALHOUN 1 - APPENDIX I TECH SPEC IMPLEMENTATION REVIEW		1	09/30/84
52758	FORT CALHOUN 1 - ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION AND PROCEDURES			12/30/84
52839	FORT CALHOUN 1 - ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RTS COMPONENTS			12/30/84
52920	FORT CALHOUN 1 - ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING PROCEDURES/VENDOR RECOMMENDATIONS - RTS COMPONENTS			12/30/84
53001	FT CALHOUN 1 - ITEM 3.1.3 - POST-MAINTENANCE TESTING - CHANGES TO TECH SPECS - RTS COMPONENTS			12/30/84
53071	FORT CALHOUN 1 - REACTOR TRIP SYSTEM RELIABILITY - VENDOR RELATED MODIFICATIONS			12/30/84
53124	FORT CALHOUN 1 - ITEMS 4.2.1 & 4.2.2 - PREVENTATIVE MAINT PROGRAM FOR REACTOR TRIP BREAKERS - MAINTENANCE & TRENDING			12/30/84
52229	FORT CALHOUN 1 - CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPA#C-10)			09/30/85
TAC #	TAC DESCRIPTION	TMI ACTIONS	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				
47799	FORT CALHOUN - NUREG 0737 ITEM II K2.17 POTENTIAL FOR VOIDING IN RCS		1	COMPLETE 10/17/83
<u>ACTIVE ACTIONS</u>				
45827	FT. CALHOUN 1 - NUREG-0737 II.K.3.30, SB LOCA OUTLINE		1	TARGET 06/30/84
44233	FT. CALHOUN 1 - NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES		1	06/30/84
44303	FT. CALHOUN 1 - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES			06/30/84
45946	FT. CALHOUN 1 - NUREG-0737 III.A.1.2, TECHNICAL SUPPORT CENTER		1	09/30/84
46017	FT. CALHOUN 1 - NUREG-0737 III.A.1.2, OPERATIONAL SUPPORT CENTER		1	09/30/84
46089	FT. CALHOUN 1 - NUREG-0737 III.A.1.2, EMERGENCY OPERATIONS FACILITY		1	09/30/84
46305	FT. CALHOUN 1 - NUREG-0737 III.A.2.2, METEOROLOGICAL DATA UPGRADE		1	09/30/84
46883	FT. CALHOUN - II K2.13 THERMAL MECHANICAL REPORT		1	09/30/84
44582	FORT CALHOUN 1 - RV AND S ⁰ TESTING		1	09/30/84
45135	FT. CALHOUN 1 - NUREG-0737 II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION		1	09/30/84
51161	FORT CALHOUN 1 - I.D.1 CONTROL ROOM DESIGN REVIEW		1	09/30/84
51241	FORT CALHOUN 1 - I.D.2. SAFETY PARAMETER DISPLAY SYSTEM			09/30/84
48166	FORT CALHOUN - NUREG 0737 II.K.3.31 COMPLIANCE WITH 10 CFR 50.46		1	12/30/84

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CONDENSED MANAGEMENT REPORT

(CONTINUATION)

FORT CALHOUN 1

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
52194	FT. CALHOUN STATION - SALP REVIEW - SEPTEMBER 1, 1982 - AUGUST 31, 1983			<u>COMPLETE</u> 12/20/83
52512	FT. CALHOUN STATION - ADMINISTRATIVE TS CHANGES			01/26/84
49491	FT. CALHOUN STATION - EVALUATION OF LICENSE RESPONSE TO GENERIC LETTERS 82-17/23			01/26/84
52030	FT. CALHOUN STATION - RELIEF REQUESTS FROM ASME-BXPV CODE, SECTION XI			04/06/84
54181	FT. CALHOUN STATION - TS'S FOR CYCLE 9 OPERATION			04/26/84
<u>ACTIVE ACTIONS</u>				
54792	FT. CALHOUN STATION - EQ DEADLINE TIME EXTENSION			<u>TARGET</u>
54804	FT. CALHOUN STATION - PUMPS AND VALVES RELIEF REQUEST FROM ASME BXPV CODE			
54857	FT. CALHOUN STATION - EVALUATION OF LICENSEE'S POSITION REGARDING ADEQUACY OF CURRENT AFWS TECH SPECS			
53219	FT. CALHOUN STATION - ADMINISTRATIVE TS CHANGES AS A RESULT OF RULE CHANGES			05/15/84
53337	FT. CALHOUN STATION - REVIEW OF S.C.U. METHODOLOGY FOR RELOADS			05/30/84
52513	FT. CALHOUN STATION - SNUMMBER AND SURVEILLANCE CAPSULES TS CHANGES			05/30/84
52552	FT. CALHOUN STATION - REVIEW OF RELOAD METHODOLOGY REPORTS			05/30/84
52353	FT. CALHOUN STATION - APPENDIX R EXEMPTION REQUESTS			05/30/84
48807	FT. CALHOUN STATION - AMEND TS'S REGARDING APPENDIX J			06/30/84
48534	FT. CALHOUN STATION - AMENDED TS'S INOPERABILITY OF RPS/ESFAS CHANNELS			09/30/84
54391	FORT CALHOUN 1 - RCS VENTS TECHNICAL SPECIFICATIONS NUREG 073 ITEM II.B.1			09/30/84
54534	FORT CALHOUN 1 - TECHNICAL SPECIFICATIONS COVERED BY GENERIC LETTERS 83-36, 83-37 FOR NUREG-0737			09/30/84
<u>ANTICIPATED ACTIONS</u>				
53673	FORT CALHOUN 1 - ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS			<u>INITIATION</u> 11/01/84
53756	FORT CALHOUN 1 - ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS			11/01/84
53839	FORT CALHOUN 1 - ITEM 3.2.3 - POST-MAINTENANCE TESTING -CHANGES TO TECH. SPEC. - ALL SR COMPONENTS			11/01/84
53911	-ITEMS 4.2.3 & 4.2.4-PREVENTATIVE MAINT. PROGRAM FOR REACTOR TRIP BREAKERS-LIFE TESTING & REPLACEMENT			11/01/84
53983	FORT CALHOUN 1 - ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS			11/01/84
54066	FORT CALHOUN 1 - ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL TESTING - DIVERSE TRIP FEATURES			11/01/84
53590	FORT CALHOUN 1 - ITEM 1.2 - POST TRIP REVIEW-DATA AND INFORMATION CAPABILITY			11/01/84

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CONDENSED MANAGEMENT REPORT

FACILITY: FORT ST VRAIN

PLANT LOCATION: 35 MI N OF DENVER, COL
 DOCKET NUMBER: 050-00267
 ARCH/ENGINEER: S&L
 IE INSPECTOR: G. PLUMLEE

LICENSED POWER: 0842 MWT
 DESIGN POWER: 0330 MWE
 NSSS VENDOR: GAC

PROJECT MANAGER: P. WAGNER
 BRANCH CHIEF: E. JOHNSON
 LIC. ASSISTANT: E. HAYCRAFT

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>MULTI-PLANT</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
48341	FORT ST. VRAIN - RETS		3	<u>COMPLETE</u> 11/23/83
47417	FORT ST VRAIN - SNUBBERS		2	01/25/84
11025	FORT ST. VRAIN-CONTROL OF HEAVY LOADS OVER SPENT FUEL POOL (NUREG 0612)		2	03/06/84
54196	FORT ST. VRAIN - NUREG 0737 TECH SPECS (G.L. 83-36 AND 37)			04/23/84
<u>ACTIVE ACTIONS</u>				
42925	FORT ST VRAIN - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11		2	<u>TARGET</u> 07/15/84
54535	FORT ST. VRAIN - TECHNICAL SPECIFICATIONS COVERED BY GENERIC LETTERS 83-36, 83-37 FOR NUREG-0737			09/30/84
52840	FORT ST. VRAIN - ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RTS COMPONENTS			12/30/84
52921	FORT ST. VRAIN - ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING PROCEDURES/VENDOR RECOMMENDATIONS - RTS COMPONENTS			12/30/84
53002	FT ST VRAIN-ITEM 3.1.3 -POST-MAINTENANCE TESTING-CHANGES TO TECH SPECS-RTS COMPONENTS			12/30/84
51092	FORT ST. VRAIN - INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)			12/30/84
52759	FORT ST. VRAIN - ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION AND PROCEDURES			12/30/84
42527	FORT ST VRAIN - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21)		1	12/30/84
52230	FORT ST. VRAIN - CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPA#C-10)		3	09/30/85
<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>IMI ACTIONS</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
44091	FORT ST VRAIN - NUREG-0737 I.A.1.3.1, SHIFT MANNING OVERTIME LIMITS		1	<u>COMPLETE</u> 10/13/83
<u>ACTIVE ACTIONS</u>				
44234	FORT ST VRAIN - NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES		1	<u>TARGET</u> 05/30/84
44304	FORT ST VRAIN - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES		1	05/30/84
44443	FORT ST VRAIN - NUREG-0737 II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS		1	06/30/84
44923	FORT ST VRAIN - NUREG-0737 II.F.1.1, NOBLE GAS MONITOR		1	06/30/84
45065	FORT ST VRAIN - NUREG-0737 II.F.1.3, CONTAINMENT HIGH RANGE MONITOR		1	06/30/84
45947	FORT ST VRAIN - NUREG-0737 III.A.1.2, TECHNICAL SUPPORT CENTER		1	10/30/84
46018	FORT ST VRAIN - NUREG-0737 III.A.1.2, OPERATIONAL SUPPORT CENTER		1	10/30/84
46090	FORT ST VRAIN - NUREG-0737 III.A.1.2, EMERGENCY OPERATIONS FACILITY		1	10/30/84
46306	FORT ST VRAIN - NUREG-0737 III.A.2.2, METEOROLOGICAL DATA UPGRADE		1	10/30/84
51162	FORT ST. VRAIN - I.D.1 CONTROL ROOM DESIGN REVIEW			12/30/84
51242	FORT ST. VRAIN - I.D.2. SAFETY PARAMETER DISPLAY SYSTEM			12/30/84
<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
49023	FORT ST. VRAIN - STAFFING TECH SPECS		2	<u>COMPLETE</u> 10/13/83
47412	FORT ST. VRAIN - CONTROL ROD DRIVE HOT SPOTS		3	10/13/83
47409	FORT ST. VRAIN - PLATEOUT PROBE DATA EVALUATION		3	11/10/83
47413	FORT ST. VRAIN - PCRV HOT SPOTS		3	11/18/83
52647	FORT ST. VRAIN - RESPOND TO 2 QUESTIONS FROM R4 RE: APPLICABILITY OF APP.R TO FSV		2	12/19/83
52459	FORT ST. VRAIN - PLATEOUT PROBE REMOVAL SCHEDULE CHANGE		1	01/03/84

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CONDENSED MANAGEMENT REPORT

(CONTINUATION)

FORT ST VRAIN

TAC #	TAC DESCRIPTION	PLANT SPECIFIC	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u> (CONTINUATION)				<u>COMPLETE</u>
49623	FORT ST. VRAIN - STEAM GENERATOR TUBE LEAK INVESTIGATION		1	02/01/84
53461	FORT ST. VRAIN - DISPOSAL REQUEST FOR SULPHUR 35		2	02/14/84
53226	FORT ST. VRAIN - MOISTURE MONITOR DETECTOR RELOCATION		2	02/24/84
47422	FORT ST. VRAIN - GRAPHITE CORROSION & SURVEILLANCE		3	03/01/84
47423	FORT ST. VRAIN - CORE SUPPORT BLOCK STRESSES		3	03/01/84
49624	FORT ST. VRAIN - CYCLE 4, SEGMENT 10 RELOAD (H-451 GRAPHITE)		1	03/02/84
52458	FORT ST. VRAIN - CALIBRATION SOURCES LICENSE CONDITION		2	03/08/84
<u>ACTIVE ACTIONS</u>				<u>TARGET</u>
51954	FORT ST. VRAIN - EXEMPTION TO 10CFR 50.54(W)		1	04/15/84
52354	FORT ST. VRAIN - STARTUP TEST PROGRAM CHANGES		2	05/30/84
54639	FORT ST. VRAIN - PCRV TENDON CORROSION PROBLEM			06/06/84
48863	FORT ST. VRAIN - DETECTOR DECALIBRATION		2	06/30/84
49055	FORT ST. VRAIN - FUEL BLOCK CRACK MODELING		3	07/01/84
54760	FORT ST. VRAIN - EMERGENCY EXERCISE EXEMPTION			07/01/84
51955	FORT ST. VRAIN - ELECTRICAL MODIFICATIONS INCLUSION INTO TECH SPECS		2	07/10/84
54183	FORT ST. VRAIN - STEAM GENERATOR TUBE ISI		1	07/14/84
52634	FORT ST. VRAIN - LCO 4.1.9 NON-CONSERVATIVE ANALYSIS ASSUMPTIONS		1	07/15/84
54598	FORT ST VRAIN-REG IV TIA, STEAM GENERATOR TUBE LEAK			07/30/84
52144	FORT ST. VRAIN - FIRE PROTECTION APPRAISAL APPENDIX R		1	08/30/84
54265	FORT ST. VRAIN - SECURITY PLAN REVISION 14		1	09/01/84
51535	FORT ST. VRAIN - TS CHANGES TO 4.4		2	09/01/84
54373	FORT ST. VRAIN - APPENDIX R EXEMPTION REQUESTS		1	09/09/84
53417	FORT ST. VRAIN - ISI/IST TECH SPECS		2	09/30/84
52035	FORT ST. VRAIN - CORE SEGMENT 2 REPORT REVIEW (CRACKED FUEL ASSEMBLIES)		3	12/30/84
47416	FORT ST. VRAIN - INSTRUMENT SETPOINTS		3	06/30/85
<u>ANTICIPATED ACTIONS</u>				<u>INITIATION</u>
53591	FORT ST. VRAIN - ITEM 1.2 - POST TRIP REVIEW-DATA AND INFORMATION CAPABILITY		2	11/01/84
53674	FORT ST. VRAIN - ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS		2	11/01/84
53757	FT ST VRAIN- ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS		3	11/01/84
53840	FORT ST. VRAIN - ITEM 3.2.3 - POST-MAINTENANCE TESTING -CHANGES TO TECH. SPEC. - ALL SR COMPONENTS		2	11/01/84
53984	FORT ST. VRAIN - ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS		2	11/01/84
54067	FORT ST. VRAIN - ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL TESTING - DIVERSE TRIP FEATURES		2	11/01/84

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CONDENSED MANAGEMENT REPORT

FACILITY: GINNA

PLANT LOCATION: 15 MI NE OF ROCHESTER, NY
 DOCKET NUMBER: 050-00244
 ARCH/ENGINEER: GIL
 IE INSPECTOR: R. ZIMMERMAN

LICENSED POWER: 1520 MWT
 DESIGN POWER: 0470 MWE
 NSSS VENDOR: WEST

PROJECT MANAGER: G. DICK
 BRANCH CHIEF: D. CRUTCHFIELD
 LIC. ASSISTANT: H. SMITH

TAC #	TAC DESCRIPTION	MULTI-PLANT	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				
07992	GINNA - CONTROL OF HEAVY LOADS OVER SPENT FUEL POOL (NUREG 0612)		2	<u>COMPLETE</u> 01/18/84
49546	GINNA BLOCKING SI SIGNAL DURING COOLDOWN			01/30/84
06137	GINNA - REACTOR VESSEL SUPPORT (NRR GENERIC A-2)		2	02/09/84
<u>ACTIVE ACTIONS</u>				
52760	GINNA - ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION AND PROCEDURES			<u>TARGET</u>
52841	GINNA 1 - ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RTS COMPONENTS			
52922	GINNA - ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING PROCEDURES/VENDOR RECOMMENDATIONS - RTS COMPONENTS			
53003	GINNA-ITEM 3.1.3 -POST-MAINTENANCE TESTING-CHANGES TO TECH SPECS-RTS COMPONENTS			
53072	GINNA - REACTOR TRIP SYSTEM RELIABILITY - VENDOR RELATED MODIFICATIONS			
53125	GINNA - ITEMS 4.2.1 & 4.2.2 - PREVENTATIVE MAINTENANCE PROGRAM FOR REACTOR TRIP BREAKERS - MAINTENANCE & TRENDING			
53174	GINNA - ITEM 4.3 - AUTOAMTIC ACTUATION OF SHUNT TRIP ATTACHMENT FOR W AND B & W PLANTS			
43047	GINNA - TECHNICAL SPECIFICATIONS DEFINING OPERABILITY FOR SAFETY SYSTEMS		2	05/30/84
49734	GINNA - "NUREG 0737 TECH SPECS (GENERIC LETTER 82-16 82-16 OR 83-02)"			06/15/84
42520	GINNA - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21)		1	06/30/84
42609	GINNA - LONG TERM REVIEW CONTAINMENT PURGE AND VENT (B-24)		1	06/30/84
08414	GINNA - TS SURVEILLANCE - HYDRAULIC SNUBBERS		3	07/15/84
08416	GINNA - TS SURVEILLANCE FOR MECHANICAL SNUBBERS TEST RESULTS		2	07/15/84
07739	GINNA - ECCS EVAL THAT ALLOWS FOR UPPER PLENUM INJECTION(UPI)		3	09/30/84
42910	GINNA - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11		2	10/30/84
49677	GINNA - REACTOR COOLANT PUMP TRIP			10/30/84
52231	GINNA - CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPA#C-10)			12/31/85
51093	GINNA - INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)			06/30/86
<u>TAC # TAC DESCRIPTION MULTI-PLANT PRIORITY CRITICAL DATE</u>				
<u>COMPLETED ACTIONS</u>				
47800	GINNA - NUREG 0737 ITEM II.K.2.17 POTENTIAL FOR VOIDING IN RCS			<u>COMPLETE</u> 12/27/83
44444	GINNA - NUREG-0737 II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS		1	04/24/84
<u>ACTIVE ACTIONS</u>				
45829	GINNA - NUREG-0737 II.K.3.30, SB LOCA OUTLINE		1	<u>TARGET</u> 04/30/84
47602	GINNA - NUREG 0737 ITEM II F1.4 - CONTAINMENT PRESSURE INSTRUMENT		1	06/30/84
47673	GINNA - NUREG 0737 ITEM II F1.5 CONTAINMENT WATER LEVEL MONITOR		1	06/30/84
47744	GINNA - NUREG 0737 ITEM II F1.6 CONTAINMENT HYDROGEN MONITOR		1	06/30/84
47941	GINNA - PLANT SHIELDING MODIFICATIONS (II.B.2.2)		1	06/30/84
44584	GINNA - RV AND SV TESTING		1	09/01/84
44235	GINNA - NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES		1	11/30/84
44305	GINNA - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES		1	11/30/84
45137	GINNA - NUREG-0737 II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION		1	12/31/84
48168	GINNA -NUREG 0737 II.K.3.31 COMPLIANCE WITH 10 CFR 50.46		1	12/31/84
51163	GINNA - I.D.1 CONTROL ROOM DESIGN REVIEW			02/28/86
51243	GINNA - I.D.2. SAFETY PARAMETER DISPLAY SYSTEM			06/30/86

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CONDENSED MANAGEMENT REPORT

(CONTINUATION)

GINNA

TAC #	TAC DESCRIPTION	TMI ACTIONS	PRIORITY	CRITICAL DATE
<u>ACTIVE ACTIONS</u> (CONTINUATION)				
45948	GINNA - NUREG-0737 III.A.1.2, TECHNICAL SUPPORT CENTER		1	<u>TARGET</u> 06/30/86
46019	GINNA - NUREG-0737 III.A.1.2, OPERATIONAL SUPPORT CENTER		1	06/30/86
46091	GINNA - NUREG-0737 III.A.1.2, EMERGENCY OPERATIONS FACILITY		1	06/30/86
46307	GINNA - NUREG-0737 III.A.2.2, METEOROLOGICAL DATA UPGRADE		1	06/30/86
46884	GINNA - II K2.13 THERMAL MECHANICAL REPORT		1	12/31/86
TAC #	TAC DESCRIPTION	PLANT SPECIFIC	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				
49344	GINNA - SGTR - THERMOHYDRAULIC ANALYSIS		2	<u>COMPLETE</u> 10/05/83
49345	GINNA - RCP TRIP & RESTART CRITERIA		2	10/14/83
49349	GINNA - FIBER OPTIC INSPECTION OF S/G TUBES		2	10/25/83
49351	GINNA - STAGNANT FLOW/PTS		1	10/31/83
48914	GINNA - REORGANIZATION OF PLANT STAFF		2	11/02/83
49348	GINNA - LETDOWN SYSTEM & PER PRESSURE INST MODS		2	11/09/83
49347	GINNA - SIMULATOR RESPONSE TIME - SGTR		2	12/14/83
51053	GINNA INSERTION INTO THE SPENT FUEL RACKS A FUEL ASSEMBLY OF PREVIOUSLY UNANALYZED DESIGN & ENRICHMENT			02/08/84
54174	GINNA - EXEMPTION FROM SHIFT STAFFING REQUIREMENTS			02/22/84
48884	GINNA - EFFECTS OF GROUNDWATER LEVEL ON STRUCTURES SEP III-3.A			04/18/84
48885	GINNA - INSERVICE INSPECTION OF WATER CONTROL STRUCTURES SEP III-3.C			04/18/84
48886	GINNA - STEAM GENERATOR BLOWDOWN VALVE RESTRAINTS SEP III-4.C			04/18/84
48891	GINNA - RHR TUBE FAILURES SEP V-10.A			04/18/84
48892	GINNA - RHR PROCEDURES SEP V-10.B			04/18/84
48893	GINNA - CONTAINMENT ISOLATION SEP TOPIC VI-4			04/18/84
48894	GINNA - ESF SWITCHOVER PROCEDURES SEP VI-7.B			04/18/84
48895	GINNA - BATTERY MONITORING SYSTEM SEP VIII-3.B			04/18/84
<u>ACTIVE ACTIONS</u>				
54364	GINNA - SEP STRUCTURAL UPGRADE PROGRAM FOLLOWUP			
54659	GINNA - TECH. SPEC. CHANGE - REPORTING REQUIREMENTS - CONF. WITH 50.72, 50.73, & 50.49			
54762	GINNA - INCREASE CAPACITY OF SPENT FUEL POOL STORAGE RACKS			
54786	GINNA - SEP TIA ITEMS			
54849	GINNA - LICENSE CONVERSION SER SUPPLEMENT			
49803	GINNA - INTEGRATED PLANT SAFETY ASSESSMENT REPT SUPPLEMENT			
53430	GINNA - WESTINGHOUSE OPTIMIZED FUEL ASSEMBLY			05/05/84
54503	GINNA-STEAM GENERATOR TUBESHEET SLEEVES			05/09/84
54632	GINNA - EXEMPTION FROM SHIFT STAFFING REQUIREMENTS			05/09/84
49178	GINNA - TECH SPEC CHANGE FOR RWST OPERABILITY AND OTHER CLARIFICATIONS			05/28/84
49177	GINNA - TECH SPEC CHANGE TO ALLOW LOW POWER PHYSICS TESTING PRIOR TO OR DURING CERTAIN VALVE TESTING			05/30/84
53311	GINNA - TECHNICAL SPECIFICATION CHANGE - USE OF TSC BATTERY IN PLACE OF INOPERABLE 1E BATTERY BANK			05/30/84
54291	GINNA - EXEMPTION REQUEST - EMERGENCY EXERCISE			06/15/84
52404	GINNA - TS CHANGE - REVISE LIST OF HYDRAULIC SNUBBERS			06/30/84
52099	GINNA - CONSOLIDATION OF MANAGEMENT RESPONSIBILITIES			06/30/84
49346	GINNA - PROCEDURES - SGTR FOLLOW-UP		2	06/30/84
54357	GINNA - T/S CHANGE - MANAGEMENT PERSONNEL AND CORRECTION IN ORGANIZATION CHART			07/15/84
54363	GINNA - T/S CHANGE - DELETE RESTRICTION ON MOVEMENT OF SHIPPING CASKS WITH AUX BLDG CRANE			07/15/84
54371	GINNA - SEISMIC STRUCTURAL EVAL. MAIN CONTROL BOARD SEP III-6			07/30/84

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CONDENSED MANAGEMENT REPORT

(CONTINUATION)

GINNA

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>ACTIVE ACTIONS</u> (CONTINUATION)				<u>TARGET</u>
54509	GINNA - DISPOSAL OF LOW LEVEL RADIOACTIVE WASTE			07/30/84
53431	GINNA - ALTERNATIVE SHUTDOWN SYSTEM - APP R			07/30/84
48474	GINNA - REVIEW OF ADDITIONAL SUBMITTALS ON SGTR			07/30/84
12461	GINNA - TENDON SURVEILLANCE PROGRAM		1	07/30/84
53432	GINNA - PROCESS CONTROL PROGRAM			07/30/84
54369	GINNA - T/S CHANGE - DELETE SPACING RESTRICTIONS OF RECENTLY DISCHARGED FUEL IN SPENT FUEL RACKS			08/15/84
53433	GINNA - TS CHANGE ADJUSTMENT OF ISI PGM 10 YR INTERVAL START DATES			08/30/84
52399	GINNA - SEP TS CHANGES			08/30/84
49292	GINNA - HEATUP & COOLDOWN - REACTOR SURVEILLANCE CAPSULE T		2	08/30/84
49135	GINNA - PUMP & VALVE INSERVICE TESTING			08/30/84
49175	GINNA - TECH SPEC CHANGE VALVE EXERCISING AND SURVEILLANCE REQUIREMENTS		2	08/30/84
54370	GINNA - T/S CHANGE - REDUCE REQUIREMENT FOR LOW PZR WATER LEVEL AND P-10 PERMISSIVE			09/01/84
54392	GINNA - RCS VENTS TECHNICAL SPECIFICATIONS NUREG 073 ITEM II.B.1			09/30/84
54536	GINNA - TECHNICAL SPECIFICATIONS COVERED BY GENERIC LETTERS 83-36, 83-37 FOR NUREG-0737			09/30/84
52396	GINNA - T.S. CHANGE - DELETE RESPONSE TIME TESTING OF AUX FD SYS AND CONT. 150 VALVE CIRCUITS			09/30/84
52395	GINNA - COMPUTER ANALYSIS OF JAN. 1982 STEAM GEN TUBE RUPTURE			10/30/84
49341	GINNA - RADIOLOGICAL CONSEQUENCES OF SGTR		2	12/30/84
49899	GINNA - TECH SPEC TO INCREASE THE MAXIMUM ALLOWABLE ACTIVITY IN THE REACTOR COOLANT			12/30/84
11220	GINNA - CONVERSION PDL TO FTOL		1	12/30/84
43307	GINNA - NON-TECH SPEC TMI RELATED ITEMS		1	12/31/84
48880	GINNA - TECH SPEC CHANGES FOR SEP TOPICS II-A AND III-7.A			01/02/85
<u>ANTICIPATED ACTIONS</u>				<u>INITIATION</u>
53675	GINNA - ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS			11/01/84
53758	GINNA - ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS			11/01/84
53841	GINNA - ITEM 3.2.3 - POST-MAINTENANCE TESTING - CHANGES TO TECH. SPEC. - ALL SR COMPONENTS			11/01/84
53912	-ITEMS 4.2.3 & 4.2.4 - PREVENTATIVE MAINT. PROGRAM FOR REACTOR TRIP BREAKERS - LIFE TESTING & REPLACEMENT			11/01/84
53985	GINNA - ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS			11/01/84
54068	GINNA - ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL CAPABILITY			11/01/84
53592	GINNA - ITEM 1.2 - POST TRIP REVIEW - DATA AND INFORMATION CAPABILITY			11/01/84

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CONDENSED MANAGEMENT REPORT

FACILITY: GRAND GULF 1

PLANT LOCATION: 25 MI S OF VICKSBURG, MISS
 DOCKET NUMBER: 050-00416
 ARCH/ENGINEER: RECH
 IE INSPECTOR: A. WAGNER

LICENSED POWER: 3833 MWT
 DESIGN POWER: 1250 MWE
 NSSS VENDOR: GE

PROJECT MANAGER: D. HOUSTON
 BRANCH CHIEF: A. SCHWENCER
 LIC. ASSISTANT: E. HYLTON

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>MULTI-PLANT</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>ACTIVE ACTIONS</u>				
51094	GRAND GULF - INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)			<u>TARGET</u>
52232	GRAND GULF - CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPA#C-10)			
52761	GRAND GULF - ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION AND PROCEDURES			
52842	GRAND GULF - ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RTS COMPONENTS			
52923	GRAND GULF - ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING PROCEDURES/VENDOR RECOMMENDATIONS - RTS COMPONENTS			
53004	GRAND GULF-ITEM 3.1.3 -POST-MAINTENANCE TESTING-CHANGES TO TECH SPECS-RTS COMPONENTS			
51679	GRAND GULF 1 NUREG 0737 TECH SPECS			06/15/84
49994	GRAND GULF UNIT 1 - CONTROL OF HEAVY LOADS OVER SPENT FUEL POOL			06/30/84
49996	GRAND GULF UNIT 1 - MASONRY WALL DESIGN IE BULLETIN 80-11 IE BULLETIN 80-11			06/30/85
<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>TMI ACTIONS</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>ACTIVE ACTIONS</u>				
49459	GRAND GULF 1 - INSTRUMENTS FOR DETECTION ON INADEQUATE CORE COOLING (II.F.2)			
51164	GRAND GULF - I.D.1 CONTROL ROOM DESIGN REVIEW			
51244	GRAND GULF - I.D.2. SAFETY PARAMETER DISPLAY SYSTEM			
54587	GRAND GULF 1 - II.D.1 - RV & SV TESTING			
49992	GRAND GULF UNIT 1 - QUALIFICATION OF ADS ACCUMULATORS NUREG-0737, II.K.3.28			06/30/84
49993	GRAND GULF UNIT 1 - POST ACCIDENT SAMPLING NUREG-0737, II.B.3			09/30/84
<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
51337	GRAND GULF UNIT 1 - FIRE PROTECTION PLAN			COMPLETE 11/10/83
51479	GRAND GULF UNIT 1 - FIRE PROTECTION TS CHANGES			11/10/83
51862	GRAND GULF 1 - CHLORINE ACCIDENT ANALYSIS			11/30/83
53434	GRAND GULF UNIT 1 - STARTUP TESTS			02/03/84
51771	GRAND GULF UNIT 1 - TECH SPEC PKG #6			02/21/84
51478	GRAND GULF UNIT 1 - TECH SPEC CHANGE PACKAGE #3			02/21/84
52367	GRAND GULF UNIT 1 - SRM TECH. SPEC. CHANGE			02/21/84
51957	GRAND GULF UNIT 1 - TECH SPEC PACKAGE #8			02/21/84
52523	GRAND GULF UNIT 1 - TECH SPEC CHANGE PKG. 14			04/18/84
52524	GRAND GULF UNIT 1 - TECH SPEC CHANGE PKG. 15			04/18/84
54318	GRAND GULF - 1, TECH. SPEC. CHANGE, DTD 9/9/83, ITEM 6 - ECCS RESPONSE TIMES			04/18/84
54602	GRAND GULF 1-T.S. CHANGE, DTD 3/29/84, ITEM 2 MINIMUM OPERABLE CHANNELS-RADIATION MONITOR			04/18/84
54603	GRAND GULF 1-T.S. CHANGE, DTD 3/29/84, ITEM 1 SNUBBER ADDITION			04/18/84
54604	GRAND GULF 1-T.S. CHANGE, DTD 3/29/84, ITEM 3 CONTAINMENT SPRAY INSTRUMENTATION			04/18/84
54605	GRAND GULF - 1, TECH. SPEC. CHANGE, DTD 03/29/84, ITEM 4, ADS INSTRUMENTATION			04/18/84
54606	GRAND GULF 1-T.S. CHANGE, DTD 3/29/84, ITEM 5 CHLORINE DETECTION INSTRU. SURVEILLANCE			04/18/84
<u>ACTIVE ACTIONS</u>				
54457	GRAND GULF - 1, TECH. SPEC. CHANGE DRAFTS FOR 32 PROBLEM SHUTS			<u>TARGET</u>
54458	GRAND GULF - 1, TECH SPEC CHANGE PROBLEM SHEETS (001-382)			
54478	GRAND GULF - 1, T/S CHANGE, DTD 03/20/84, ITEM 1, ADS VALUE OPERABILITY REQUIREMENTS (PROBLEM SHEET 001)			

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CONDENSED MANAGEMENT REPORT

(CONTINUATION)

GRAND GULF 1

TAC #	TAC DESCRIPTION	PLANT SPECIFIC	PRIORITY	CRITICAL DATE
<u>ACTIVE ACTIONS</u> (CONTINUATION)				
54479	GRAND GULF - 1, T/S CHANGE, DTD 03/20/84, ITEM 2, REACTOR WATER CLEANUP SYSTEM ISOLATION (PROBLEM SHEET 005)			<u>TARGET</u>
53419	GRAND GULF 1 - AGASTAT RELAYS			
52412	GRAND GULF UNIT 1 - REVIEW OF REVISED TECH SPECS			12/15/83
54313	GRAND GULF - 1, TECH. SPEC. CHANGE, DTD 9/9/83, ITEM 1 - SNUBBER IDENTIFICATION			03/30/84
54683	GRAND GULF - 1, TECH. SPEC. CHANGE, DTD 4/10/84, ITEM NO. 1 ROOM AIR TEMPERATURE TRIPS			04/20/84
54684	GRAND GULF - 1, TECH. SPEC. CHANGE, DTD 4/10/84, ITEM 2 CONTAINMENT ISOLATION VALVES			04/20/84
54685	GRAND GULF - 1, TECH. SPEC. CHANGE, DTD 4/10/84, ITEM 3 ACCIDENT MONITORING INSTRUMENTS			04/20/84
54686	GRAND GULF - 1, TECH. SPEC. CHANGE, DTD 04/11/84, ITEM NO. 1 ECCS SURVEILLANCE			04/20/84
54808	GRAND GULF - 1, TECH. SPEC. CHANGE PROBLEM SHEETS			05/02/84
54498	GRAND GULF UNIT 1 - HIGH GROUNDWATER LEVEL			05/15/84
54314	GRAND GULF - 1, TECH. SPEC. CHANGE, DTD 9/9/83, ITEM 2 OFFGAS RADIATION MONITOR			05/30/84
54222	GRAND GULF UNIT 1 - ONSITE/OFFSITE POWER RELIABILITY			05/30/84
52631	GRAND GULF UNIT 1 - 50.55 INSPECTION RELIEF			05/30/84
52139	GRAND GULF UNIT 1 - TECH SPEC CHANGE PKG. 11			05/30/84
52175	GRAND GULF UNIT 1 - DWR PIPE CRACK			05/30/84
52368	GRAND GULF UNIT 1 - TECH. SPEC. CHANGE PKG. 13			05/30/84
51641	GRAND GULF UNIT 1 - EMERGENCY RESPONSE CAPABILITY (82-33)			06/01/84
52603	GRAND GULF UNIT 1 - STAFFING REVIEW			06/01/84
53395	GRAND GULF UNIT 1 - ISOLATION INSTR. TECH SPECS			06/01/84
53490	GRAND GULF 1 - CONTROL ROOM LEAKAGE			06/01/84
53491	GRAND GULF 1 - TS CHANGES, PLANT ORG. STAFF			06/15/84
54185	GRAND GULF 1 - TECH SPEC AUDIT			06/15/84
54323	GRAND GULF - 1, TECH. SPEC. CHANGE, DTD 9/9/83, ITEM 11 - A. C. POWER OPERABILITY			06/15/84
52049	GRAND GULF 1 - EXTENSION OF DATES IN LICENSE CONDITIONS			06/15/84
54336	GRAND GULF - 1, TECH. SPEC. CHANGE, DTD 9/9/83, ITEM 25 - CONTAINMENT VALVE LIS V			06/15/84
54337	GRAND GULF - 1, TECH. SPEC. CHANGE, DTD 9/9/83, ITEM 26 - A.C. POWER SURVEILLANCE			06/15/84
54338	GRAND GULF - 1, TECH. SPEC. CHANGE, DTD 9/9/83, ITEM 27 - RPS INSTRUMENTATION			06/30/84
54339	GRAND GULF - 1, TECH. SPEC. CHANGE, DTD 9/9/83, ITEM 28 - ECCS INSTRUMENTS			06/30/84
54340	GRAND GULF - 1, TECH. SPEC. CHANGE, DTD 9/9/83, ITEM 29 - DEFINITION OF UNRESTRICTED AREA			06/30/84
54341	GRAND GULF - 1, TECH. SPEC. CHANGE, DTD 9/9/83, ITEM 30 - RECORDS REQUIREMENT			06/30/84
54342	GRAND GULF - 1, TECH. SPEC. CHANGE, DTD 9/9/83, ITEM 32 - CONTROL ROD INSERTION LINES			06/30/84
54343	GRAND GULF - 1, TECH. SPEC. CHANGE, DTD 9/9/83, ITEM 33 - CONTROL ROOM TEMPERATURE			06/30/84
54344	GRAND GULF - 1, TECH. SPEC. CHANGE, DTD 1/18/84, ITEM 1			06/30/84
54345	GRAND GULF - 1, TECH. SPEC. CHANGE, DTD 2/27/84, ITEM 1 - APRM SCRAM SETPOINT			06/30/84
54324	GRAND GULF - 1, TECH. SPEC. CHANGE, DTD 9/9/83, ITEM 12 - CONTAINMENT H2 CONTROL			06/30/84
54334	GRAND GULF - 1, TECH. SPEC. CHANGE, DTD 9/9/83, ITEM 23 - FIRE HOSE STATIONS			06/30/84
54335	GRAND GULF - 1, TECH. SPEC. CHANGE, DTD 9/9/83, ITEM 24 - LPCI PUMP ACTUATION			06/30/84
52370	GRAND GULF UNIT 1 - ISOLATION INSTRUMENTATION			06/30/84
51866	GRAND GULF 1: STRUCTURAL INTEGRITY TEST			06/30/84
54319	GRAND GULF - 1, TECH. SPEC. CHANGE, DTD 9/9/83, ITEM 7 - ECCS ACTUATION INSTRUMENTS			06/30/84
54320	GRAND GULF - 1, TECH. SPEC. CHANGE, DTD 9/9/83, ITEM 8 - RCS ISOLATION VALVE LEAKAGE			06/30/84
54321	GRAND GULF - 1, TECH. SPEC. CHANGE, DTD 9/9/83, ITEM 9 - SERVICE WATER FOR ESF			06/30/84
54322	GRAND GULF - 1, TECH. SPEC. CHANGE, DTD 9/9/83, ITEM 10 - ISOLATION VALVE RESPONSE TIME			06/30/84
54315	GRAND GULF - 1, TECH. SPEC. CHANGE, DTD 9/9/83, ITEM 3 - RE EMBANKMENT SLOPE STABILITY AND BASES			06/30/84
54316	GRAND GULF - 1, TECH. SPEC. CHANGE, DTD 9/9/83, ITEM 4 - REACTOR VESSEL TEMPERATURE			06/30/84
54317	GRAND GULF - 1, TECH. SPEC. CHANGE, DTD 9/9/83, ITEM 5 - RADIATION INSTRUMENTS			06/30/84
54325	GRAND GULF - 1, TECH. SPEC. CHANGE, DTD 9/9/83, ITEM 13 - CONTROL ROD BLOCK INTERLOCK			06/30/84
54326	GRAND GULF - 1, TECH. SPEC. CHANGE, DTD 9/9/83, ITEM 14 - INITIAL CONTAINMENT PRESSURE			06/30/84

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(CONTINUATION)

GRAND GULF 1

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>ACTIVE ACTIONS</u> (CONTINUATION)				<u>TARGET</u>
54327	GRAND GULF - 1, TECH. SPEC. CHANGE, DTD 9/9/83, ITEM 15 - SUPPRESSION POOL LEVEL			06/30/84
54328	GRAND GULF - 1, TECH. SPEC. CHANGE, DTD 9/9/83, ITEM 16 - VENTILATION RADIATION MONITORS			06/30/84
54329	GRAND GULF - 1, TECH. SPEC. CHANGE, DTD 9/9/83, ITEM 17 - REPORTING OF SRV CHALLENGES			06/30/84
54330	GRAND GULF - 1, TECH. SPEC. CHANGE, DTD 9/9/83, ITEM 18 - RPS & ESFAS INSTRUMENTATION			06/30/84
54331	GRAND GULF - 1, TECH. SPEC. CHANGE, DTD 9/9/83, ITEM 19 - VENTILATION FLOW MONITORS			06/30/84
54332	GRAND GULF - 1, TECH. SPEC. CHANGE, DTD 9/9/83, ITEM 21 - ECCS ACTUATION			06/30/84
54333	GRAND GULF - 1, TECH. SPEC. CHANGE, DTD 9/9/83, ITEM 22 - DRYWALL PRESSURE			06/30/84
53418	GRAND GULF 1 - REMOTE PANEL TRANSFER SWITCHES			08/01/84
52411	GRAND GULF UNIT 1 - DELAVAL DIESEL GENERATOR			08/01/84
49997	GRAND GULF UNIT 1 - INSERVICE INSPECTION			08/30/84
53548	GRAND GULF 1 - CONTAINMENT/DRYWELL PURGE			09/01/84
51477	GRAND GULF UNIT 1 - INSERVICE TESTING AND INSPECTION (IST) PUMPS/VALVES			09/30/84
54537	GRAND GULF - TECHNICAL SPECIFICATIONS COVERED BY GENERIC LETTERS 83-36, 83-37 FOR NUREG-0737			09/30/84
49995	GRAND GULF UNIT 1 - SOIL STRUCTURE INTERACTION			06/30/85
48736	GRAND GULF UNIT 1 - HYDROGEN CONTROL MEASURES			06/30/85
<u>ANTICIPATED ACTIONS</u>				<u>INITIATION</u>
53593	GRAND GULF - ITEM 1.2 - POST TRIP REVIEW-DATA AND INFORMATION CAPABILITY			11/01/84
53676	GRAND GULF - ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS			11/01/84
53759	GRAND GULF - ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS			11/01/84
53842	GRAND GULF - ITEM 3.2.3 - POST-MAINTENANCE TESTING -CHANGES TO TECH. SPEC. - ALL SR COMPONENTS			11/01/84
53986	GRAND GULF - ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS			11/01/84
54069	GRAND GULF - ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL TESTING - DIVERSE TRIP FEATURES			11/01/84

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CONDENSED MANAGEMENT REPORT

FACILITY: HADDAM NECK

PLANT LOCATION: 13 MI E OF MERIDEN, CONN
 DOCKET NUMBER: 050-00213
 ARCH/ENGINEER: S&W
 IE INSPECTOR: T. REBELOWSKI

LICENSED POWER: 1825 MWT
 DESIGN POWER: 0582 MWE
 NSSS VENDOR: WEST

PROJECT MANAGER: J. LYONS
 BRANCH CHIEF: D. CRUTCHFIELD
 LIC. ASSISTANT: H. SMITH

TAC #	TAC DESCRIPTION	MULTI-PLANT	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				
11234	HADDAM NECK - INSERVICE TESTING (IST)		1	<u>COMPLETE</u> 12/29/83
49735	HADDAM NECK - "NUREG 0737 TECH SPECS (GENERIC LETTER 82-16 82-16 OR 83-02)"		2	01/30/84
08463	HADDAM NECK - REVIEW OF ASYMMETRIC LOCA LOADS SUBMITTAL		2	02/01/84
11976	HADDAM NECK - REACTOR CAVITY SEAL RING GENERIC ISSUE			
<u>ACTIVE ACTIONS</u>				
51095	HADDAM NECK - INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)			
52233	HADDAM NECK - CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPA#C-10)			
52762	HADDAM NECK - ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION AND PROCEDURES			
52843	HADDAM NECK - ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RTS COMPONENTS			
52924	HADDAM NECK - ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING PROCEDURES/VENDOR RECOMMENDATIONS			
53005	HADDAM NECK - ITEMS 3.1.3 - POST-MAINTENANCE TESTING-CHANGES TO TECH SPECS-RTS COMPONENTS			
53073	HADDAM NECK - REACTOR TRIP SYSTEM RELIABILITY - VENDOR RELATED MODIFICATIONS			
53126	HADDAM NECK - ITEMS 4.2.1 & 4.2.2 - PREVENTATIVE MAINT PROGRAM FOR REACTOR TRIP BREAKERS - MAINTENANCE & TRENDING			
53175	HADDAM NECK - ITEM 4.3 - AUTOMATIC ACTUATION OF SHUNT TRIP ATTACHMENT FOR W AND B & W PLANTS		1	06/30/84
43625	HADDAM NECK - APPENDIX R- EXEMPTION REQUEST			07/01/84
49667	HADDAM NECK - REACTOR COOLANT PUMP TRIP		2	07/01/84
07993	HADDAM NECK - CONTROL OF HEAVY LOADS OVER SPENT FUEL POOL PLANTS (NUREG 0612)		1	07/30/84
42521	HADDAM NECK - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21)		2	07/31/84
42911	HADDAM NECK - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11		3	11/30/84
08098	HADDAM NECK - APPENDIX I TECH SPEC IMPLEMENTATION REVIEW		3	12/01/84
08465	HADDAM NECK - TECH SPEC SURVEILLANCE FOR HYD. SNUBBERS		2	12/01/84
08469	HADDAM NECK - TECH SPEC SURVEILLANCE FOR MECH. SNUBBERS			
<u>TMI ACTIONS</u>				
<u>COMPLETED ACTIONS</u>				
47801	HADDAM NECK - NUREG 0737 ITEM II K2.17 POTENTIAL FOR VOIDING IN RCS		1	<u>COMPLETE</u> 12/22/83
<u>ACTIVE ACTIONS</u>				
51165	HADDAM NECK - I.D.1 CONTROL ROOM DESIGN REVIEW			
51245	HADDAM NECK - I.D.2. SAFETY PARAMETER DISPLAY SYSTEM			
47674	HADDAM NECK - NUREG 0737 ITEM II F1.5 CONTAINMENT WATER LEVEL MONITOR		1	05/14/84
47745	HADDAM NECK - NUREG 0737 ITEM II F1.6 CONTAINMENT HYDROGEN MONITOR		1	05/15/84
47603	HADDAM NECK - NUREG 0737 ITEM II F1.4 - CONTAINMENT PRESSURE INSTRUMENT		1	05/15/84
41066	HADDAM NECK - NUREG-0737 - II.B.3.2, PCST ACCIDENT SAMPLING MODIFICATIONS		1	06/01/84
44585	HADDAM NECK - RV AND SV TESTING		1	06/01/84
46308	HADDAM NECK - NUREG-0737 III.A.2.2, METEOROLOGICAL DATA UPGRADE		1	06/30/84
45138	HADDAM NECK - NUREG-0737 II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION		1	07/31/84
48420	HADDAM NECK - NUREG 0737, II.K.3.25 POWER ON PUMP SEALS		1	08/30/84
45830	HADDAM NECK - NUREG-0737 II.K.3.30, SB LOCA OUTLINE		1	09/30/84
45949	HADDAM NECK - NUREG-0737 III.A.1.2, TECHNICAL SUPPORT CENTER		1	09/30/84
46020	HADDAM NECK - NUREG-0737 III.A.1.2, OPERATIONAL SUPPORT CENTER		1	09/30/84

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CONDENSED MANAGEMENT REPORT

(CONTINUATION)

HADDAM NECK

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>TMI ACTIONS</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>ACTIVE ACTIONS</u> (CONTINUATION)				
46092	HADDAM NECK - NUREG-0737 III.A.1.2, EMERGENCY OPERATIONS FACILITY		1	<u>TARGET</u> 09/30/84
46450	HADDAM NECK - NUREG-0737 III.D.3.4, CONTROL ROOM HABITABILITY		1	09/30/84
46885	HADDAM NECK - II K2.13 THERMAL MECHANICAL REPORT		1	09/30/84
44236	HADDAM NECK - NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES		1	11/30/84
44306	HADDAM NECK - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES		1	11/30/84
48169	HADDAM NECK -NUREG 0737 II.K.3.31 COMPLIANCE WITH 10 CFR 50.46		1	12/31/84
<u>TAC # TAC DESCRIPTION PLANT SPECIFIC PRIORITY CRITICAL DATE</u>				
<u>COMPLETED ACTIONS</u>				
51931	HADDAM NECK - VENTILATION SYSTEMS			<u>COMPLETE</u> 11/01/83
51475	HADDAM NECK - DELETION OF TS ON TRANSMISSION LINE RIGHTS-OF-WAY MGMT			01/31/84
51995	HADDAM NECK - CONTAINMENT AND RHR LEAK ROM AND CAR TS			02/21/84
54249	HADDAM NECK - ULTIMATE HEAT SINK			02/28/84
53247	HADDAM NECK - INTEGRATED SAFETY ASSESSMENT			04/05/84
<u>ACTIVE ACTIONS</u>				
54295	HADDAM NECK - STA TRAINING			<u>TARGET</u>
54299	HADDAM NECK - TECH. SPEC. TO LIMIT OVERTIME			
54300	HADDAM NECK - REPORTING SAFETY AND RELIEF VALVE FAILURES			
54444	HADDAM NECK - RCP ISI EXEMPTION			
54797	HADDAM NECK - EQUIPMENT QUALIFICATION EXTENSION			
54837	HADDAM NECK - DC POWER SYSTEM BUS VOLTAGE MONITORING			
51599	HADAAM NECK - MISCELLANEOUS ADMINISTRATIVE TS			04/01/84
54615	HADDAM NECK - TECHNICAL SPECIFICATIONS FOR COASTDOWN			05/14/84
49852	HADDAM NECK - COMBUSTIBLE GAS CONTROL IN CONTAINMENT			05/30/84
48825	HADDAM NECK - TECH SPEC DEFINITION OF CONT INTEGRITY REMOVAL PLAN FOR REACTOR VESSEL SURVEILLANCE CAPSULES			05/31/84
53301	HADDAM NECK - SHIFT STAFFING TECH SPEC CHANGE			05/31/84
49314	HADDAM NECK - FIRE PROTECTION ALTERNATE SAFE SHUTDOWN EXEMPTION REQUEST			06/30/84
51639	HADDAM NECK - ADM CONTROLS TS CHANGE			06/30/84
51932	HADDAM NECK - SERVICE WATER SYSTEM			06/30/84
51933	HADDAM NECK - RPS ISOLATION			06/30/84
51934	HADDAM NECK - ONSITE STANDBY AC POWER SYSTEM			06/30/84
51935	HADDAM NECK - CONTAINMENT ISOLATION SYSTEM			06/30/84
51938	HADDAM NECK - MISSILES			06/30/84
51939	HADDAM NECK - INTEGRATED STRUCTURAL ANALYSIS			06/30/84
47367	HADDAM NECK - SURVEILLANCE AND OPERABILITY REQUIREMENTS FOR DIESEL GENERATORS		3	06/31/84
52074	HADDAM NECK - DELETION OF TS FOR EQ OF ELECT EQUIP.			09/30/84
54393	HADDAM NECK - RCS VENTS TECHNICAL SPECIFICATIONS NUREG 073 ITEM II.B.1			09/30/84
54538	HADDAM NECK - TECHNICAL SPECIFICATIONS COVERED BY GENERIC LETTERS 83-36, 83-37 FOR NUREG-0737			09/30/84
51936	HADDAM NECK - PIPE BREAKS			10/15/84
51937	HADDAM NECK - SEISMIC			10/15/84
51930	HADDAM NECK - FLOODING EVALUATION			10/15/84
52010	HADDAM NECK INTEGRATED ASSESSMENT SUPPLEMENT			10/30/84
48019	HADDAM NECK - STANDARD TECH SPEC REVIEW		3	12/15/84
51704	HADDAM NECK - DEGRADED GRID VOLTAGE PROCEDURES			12/31/84
<u>ANTICIPATED ACTIONS</u>				
54900	HADDAM NECK - SHUTDOWN MARGIN DURING THREE-LOOP OPS.			<u>INITIATION</u> 05/02/84

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CONDENSED MANAGEMENT REPORT

(CONTINUATION)

HADDAM NECK

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>	<u>INITIATION</u>
<u>ANTICIPATED ACTIONS (CONTINUATION)</u>					
53594	HADDAM NECK - ITEM 1.2 - POST TRIP REVIEW-DATA AND INFORMATION CAPABILITY				11/01/84
53677	HADDAM NECK - ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS				11/01/84
53760	HADDAM NECK- ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS				11/01/84
53843	HADDAM NECK - ITEM 3.2.3 - POST-MAINTENANCE TESTING -CHANGES TO TECH. SPEC. - ALL SR COMPONENTS				11/01/84
53913	-ITEMS 4.2.3 & 4.2.4-PREVENTATIVE MAINT. PROGRAM FOR REACTOR TRIP BREAKERS-LIFE TESTING & REPLACEMENT				11/01/84
53987	HADDAM NECK - ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS				11/01/84
54070	HADDAM NECK - ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL TESTING - DIVERSE TRIP FEATURES				11/01/84

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CONDENSED MANAGEMENT REPORT

FACILITY: HATCH 1

PLANT LOCATION: 11 MI N OF BAXLEY, GA
 DOCKET NUMBER: 050-00321
 ARCH/ENGINEER: BECH
 IE INSPECTOR: R. ROGERS

LICENSED POWER: 2436 MW
 DESIGN POWER: 0777 MWE
 NSSS VENDOR: GE

PROJECT MANAGER: G. RIVENBARK
 BRANCH CHIEF: J. STOLZ
 LIC. ASSISTANT: R. INGRAM

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>MULTI-PLANT</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
42219	HATCH 1 - BWR SCRAM DISCHARGE VOLUME CAPABILITY		1	<u>COMPLETE</u> 01/04/84
42603	HATCH 1 - LONG TERM REVIEW CONTAINMENT PURGE AND VENT (B-24)		1	01/16/84
07939	HATCH 1 - MARK I CONTAINMENT LONG TERM PROGRAM IMPLEMENTATION		1	01/25/84
47007	HATCH 1 - CONTROL OF HEAVY LOADS OVER SPENT FUEL POOL		2	04/19/84
<u>ACTIVE ACTIONS</u>				
52234	HATCH 1 - CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPA#C-10)			<u>TARGET</u>
52763	HATCH 1 - ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION AND PROCEDURES			
52844	HATCH 1 - ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RTS COMPONENTS			
52925	HATCH 1 - ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING PROCEDURES/VENDOR RECOMMENDATIONS - RTS COMPONENTS			
53006	HATCH 1 - ITEM 3.1.3 - POST-MAINTENANCE TESTING-CHANGES TO TECH SPECS-RTS COMPONENTS			
10710	HATCH 1 - CONTAINMENT LEAK TESTING APP J(GENERIC)		2	05/15/84
11260	HATCH 1 - IST		1	05/30/84
13277	HATCH 1 - DEFN OF OPERABLE		1	06/15/84
62505	HATCH 1 - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21)		1	06/15/84
08310	HATCH 1 - SURV. OF MECHANICAL SNUBBERS		2	06/30/84
08043	HATCH 1 - APPENDIX I TECH SPECS		2	08/30/84
46662	HATCH 1 - IMPLEMENTATION OF NUREG-0313, REV. 1		2	08/30/84
51680	HATCH 1 NUREG 0737 TECH SPECS			09/30/84
51096	HATCH 1 - INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)		1	12/30/84
43733	HATCH 1 - SAFETY CONCERNS ASSOCIATED WITH PIPE BREAKS IN THE BWR SCRAM SYSTEM		1	06/30/85
<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>TMI ACTIONS</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
44586	HATCH 1 - II.D.1 RV AND SV TESTING		1	<u>COMPLETE</u> 12/12/83
45831	HATCH 1 - NUREG-0737 II.K.3.30, SB LOCA OUTLINE		1	01/04/84
48170	HATCH 1 - NUREG 0737 II.K.3.31 COMPLIANCE WITH 10 CFR 50.46		1	01/04/84
44855	HATCH 1 - NUREG-0737 II.E.4.2, CONTAINMENT ISOLATION DEPENDABILITY		1	01/16/84
<u>ACTIVE ACTIONS</u>				
51166	HATCH 1 - I.D.1 CONTROL ROOM DESIGN REVIEW		1	<u>TARGET</u>
51246	HATCH 1 - I.D.2. SAFETY PARAMETER DISPLAY SYSTEM			
44094	HATCH 1 - NUREG-0737 I.A.1.3.1, SHIFT MANNING OVERTIME LIMITS			06/30/84
44445	HATCH 1 - NUREG-0737 II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS		1	08/30/84
45595	HATCH 1 - NUREG-0737 II.K.3.16, CHALLENGES AND FAILURES OF RELIEF VALVES		1	08/30/84
45691	HATCH 1 - NUREG-0737 II.K.3.18, ADS ACTUATION STUDY		1	09/30/84
45139	HATCH 1 - NUREG-0737 II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION		1	09/30/84
45950	HATCH 1 - NUREG-0737 III.A.1.2, TECHNICAL SUPPORT CENTER		1	09/30/84
46021	HATCH 1 - NUREG-0737 III.A.1.2, OPERATIONAL SUPPORT CENTER		1	09/30/84
46093	HATCH 1 - NUREG-0737 III.A.1.2, EMERGENCY OPERATIONS FACILITY		1	09/30/84
46309	HATCH 1 - NUREG-0737 III.A.2.2, METEOROLOGICAL DATA UPGRADE		1	09/30/84
44237	HATCH 1 - NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES		1	09/30/84
44307	HATCH 1 - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES		1	09/30/84

DATA AS OF - 04/30/84

CONDENSED MANAGEMENT REPORT

(CONTINUATION)

HATCH 1

TAC # TAC DESCRIPTION

PLANT SPECIFIC

PRIORITY

CRITICAL DATE

COMPLETED ACTIONS

TAC #	TAC DESCRIPTION	PRIORITY	CRITICAL DATE
43684	HATCH 1 - ADDS 3 VALVES IN SCRAM DISCH. VOL. SYST. TO T.S LIST OF ISOLATION VALVES	2	10/03/83
48294	HATCH 1 - HPCI AND RCIC SYSTEM OPERABILITY PRESSURE LCO AT 150 PSIG	2	10/03/83
52320	HATCH 1 - REVISE CONFIRMATORY ORDER ON NUREG 0737 ITEM II.B.3		10/14/83
52448	HATCH 1 - MAPHGRT MCPR CHANGES FOR REPLACEMENT OF LEAKING FUEL ASSEMBLIES		12/06/83
53306	HATCH UNIT 1 - REVISE TS TO OPERATE WITHOUT RECIRCULATION FLOW		02/06/84
51392	HATCH UNIT 1 - AUDIT FREQUENCY CHANGE FOR EMERGENCY AND SECURITY PLANS	3	02/22/84
49071	HATCH 1 - MINIMUM SHIFT CREW INCREASE NUREG-0737 RELATED T.S. CHANGE	1	02/22/84
51998	HATCH UNIT #1 TECH SPEC DELETION OF DRYWELL TO TORUS D/P SYSTEM		03/06/84
53478	HATCH 1 - EXTEND COMPLETION SCHEDULE FOR NUREG-0737-II.B.3		03/08/84
48748	HATCH 1 - FIRE PROTECTION PROGRAM - APP.R EXEMPTION REQUESTS & SAFE SHUTDOWN REPORT	1	04/18/84

COMPLETEACTIVE ACTIONS

TAC #	TAC DESCRIPTION	PRIORITY	CRITICAL DATE
48120	HATCH 1 - ADMIN. TECH. SPEC. CHANGE RELATED TO TYPE B AND TYPE C TEST REPORTING	2	05/15/84
48520	HATCH 1 - PROVIDES TECH SPEC LCO FOR SECONDARY CONTAINMENT	2	05/30/84
54168	HATCH 1 - ANALOG TRANSMITTER TRIP SYSTEM TS		06/15/84
54432	HATCH 1 - APRM/RBM TECH SPEC CHANGES		06/15/84
51984	HATCH #1 PURGE & VENT VALVE SYST DESIGN AND TS CHANGES		06/15/84
49076	HATCH 1 - TITLE CHANGES OF PLANT REVIEW BOARD MEMBERS IM APP.A & B T.S.		06/30/84
53327	HATCH #1 - CLOSURE TIME FOR SDV VENT AND DRAIN VALVES		06/30/84
53476	HATCH - 1 TS IODINE LIMITS		06/30/84
54459	HATCH #1 - EMERGENCY EXERCISE EXEMPTION		06/30/84
54461	HATCH #1 - TECH SPECS FOR TMI ACTION PLAN ITEM II.F.1.1, .2, .3		07/15/84
54495	HATCH #1 - HPCI AND RCIC LCO CHANGE TO 150 PSI		07/15/84
53454	HATCH UNIT 1 - H2 AND O2 MONITORING TS		07/31/84
49090	HATCH 1 - PROPOSED T.S. CHANGE FOR FLOW MONITORING OF GASEOUS RELEASES FROM VENT	2	08/30/84
54802	HATCH #1 APPENDIX R SCHEDULAR EXEMPTIONS		08/30/84
48771	HATCH 1 - TWO STAGE TARGET ROCK SAFETY/RELIEF VALVE PROBLEMS (TIA)	1	09/30/84

TARGETANTICIPATED ACTIONS

TAC #	TAC DESCRIPTION	PRIORITY	CRITICAL DATE
53595	HATCH 1 - ITEM 1.2 - POST TRIP REVIEW-DATA AND INFORMATION CAPABILITY		11/01/84
53678	HATCH 1 - ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS		11/01/84
53761	HATCH 1 - ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS		11/01/84
53844	HATCH 1 - ITEM 3.2.3 - POST-MAINTENANCE TESTING -CHANGES TO TECH. SPEC. - ALL SR COMPONENTS		11/01/84
53988	HATCH 1 - ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS		11/01/84
54071	HATCH 1 - ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL CAPABILITY		11/01/84

INITIATION

DATA AS OF - 04/30/84

CONDENSED MANAGEMENT REPORT

FACILITY: HATCH 2

PLANT LOCATION: 11 MI N OF BAXLEY, GA
 DOCKET NUMBER: 050-00366
 ARCH/ENGINEER: BECH
 IE INSPECTOR: R. ROGERS

LICENSED POWER: 2436 MWT
 DESIGN POWER: 0784 MWE
 NSSS VENDOR: GE

PROJECT MANAGER: G. RIVENBARK
 BRANCH CHIEF: J. STOLZ
 LIC. ASSISTANT: R. INGRAM

TAC #	TAC DESCRIPTION	MULTI-PLANT	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				
42218	HATCH 2 - BWR SCRAM DISCHARGE VOLUME CAPABILITY		1	<u>COMPLETE</u> 01/04/84
42604	HATCH 2 - LONG TERM REVIEW CONTAINMENT PURGE AND VENT (B-24)		1	01/16/84
07940	HATCH 2 - MARK I CONTAINMENT LONG TERM PROGRAM IMPLEMENTATION		1	01/25/84
47008	HATCH 2 - CONTROL OF HEAVY LOADS OVER SPENT FUEL POOL		2	04/19/84
<u>ACTIVE ACTIONS</u>				
52235	HATCH 2 - CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPA#C-10)			<u>TARGET</u>
52764	HATCH 2 - ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION AND PROCEDURES			
52845	HATCH 2 - ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RTS COMPONENTS			
52926	HATCH 2 - ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING PROCEDURES/VENDOR RECOMMENDATIONS - RTS COMPONENTS			
53007	HATCH 2 - ITEM 3.1.3 - POST-MAINTENANCE TESTING - CHANGES TO TECH SPECS - RTS COMPONENTS			
11261	HATCH 2 - IST		1	05/30/84
13278	HATCH 2 - DEFN OF OPERABLE		2	06/30/84
08298	HATCH 2 - SURV. OF MECHANICAL SNUBBERS		2	06/30/84
42506	HATCH 2 - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21)		1	06/30/84
51681	HATCH 2 NUREG 0737 TECH SPECS			06/30/84
46663	HATCH 2 - IMPLEMENTATION OF NUREG-0313, REV. 1		2	08/30/84
08095	HATCH 2 - APPENDIX I TECH SPEC IMPLEMENTATION REVIEW		2	08/30/84
51097	HATCH 2 - INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)		1	12/30/84
43738	HATCH 2 - SAFETY CONCERNS ASSOCIATED WITH PIPE BREAKS IN THE BWR SCRAM SYSTEM		2	06/30/85
<u>TAC # TAC DESCRIPTION MULTI-PLANT PRIORITY CRITICAL DATE</u>				
<u>COMPLETED ACTIONS</u>				
44587	HATCH 2 - RV AND SV TESTING		1	<u>COMPLETE</u> 12/12/83
45832	HATCH 2 - NUREG-0737 II.K.3.30, SB LOCA OUTLINE		1	01/04/84
48171	HATCH 2 - NUREG 0737 II.K.3.31 COMPLIANCE WITH 10 CFR 50.46		1	01/04/84
44856	HATCH 2 - NUREG-0737 II.E.4.2, CONTAINMENT ISOLATION DEPENDABILITY		1	01/16/84
<u>ACTIVE ACTIONS</u>				
51167	HATCH 2 - I.D.1 CONTROL ROOM DESIGN REVIEW		1	<u>TARGET</u>
51247	HATCH 2 - I.D.2. SAFETY PARAMETER DISPLAY SYSTEM		1	
44095	HATCH 2 - NUREG-0737 I.A.1.3.1, SHIFT MANNING OVERTIME LIMITS		1	06/30/84
44446	HATCH 2 - NUREG-0737 II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS		1	08/30/84
45596	HATCH 2 - NUREG-0737 II.K.3.16, CHALLENGES AND FAILURES OF RELIEF VALVES		1	08/30/84
45692	HATCH 2 - NUREG-0737 II.K.3.18, ADS ACTUATION STUDY		1	09/30/84
45140	HATCH 2 - NUREG-0737 II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION		1	09/30/84
44238	HATCH 2 - NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES		1	09/30/84
44308	HATCH 2 - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES		1	09/30/84
45951	HATCH 2 - NUREG-0737 III.A.1.2, TECHNICAL SUPPORT CENTER		1	09/30/84
46022	HATCH 2 - NUREG-0737 III.A.1.2, OPERATIONAL SUPPORT CENTER		1	09/30/84
46094	HATCH 2 - NUREG-0737 III.A.1.2, EMERGENCY OPERATIONS FACILITY		1	09/30/84
46310	HATCH 2 - NUREG-0737 III.A.2.2, METEOROLOGICAL DATA UPGRADE		1	09/30/84

DATA AS OF - 04/30/84

CONDENSED MANAGEMENT REPORT

(CONTINUATION)

HATCH 2

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
43685	HATCH 2 - ADDS 3 VALVES IN SCRAM DISCH. VOL. SYST. TO T.S. LIST OF ISOLATION VALVES		1	<u>COMPLETE</u> 10/03/83
52115	HATCH 2 - SPECIAL IGSCC INSPECITON PROGRAM			10/06/83
52321	HATCH 2 - REVISE CONFIRMATORY ORDER ON NUREG 0737 ITEM II.B.3			10/14/83
52103	HATCH 2 - TURBINE BYPASS AND LEVEL 8 TRIP SYSTEM TECH SPEC			10/28/83
53307	HATCH UNIT 2 - REVISE TS TO OPERATE WITHOUT RECIRCULATION FLOW			02/06/84
48296	HATCH 2 - ROD PATTERN ADJUSTMENTS - EXTENDS LCO TO SIX HOURS FROM TWO HOURS		2	02/22/84
51393	HATCH UNIT 2 - AUDIT FREQUENCY CHANGE FOR EMERGENCY AND SECURITY PLANS		3	02/22/84
49072	HATCH 2 - MINIMUM SHIFT CREW INCREASE NUREG-0737 RELATED T.S. CHANGE			02/22/84
49263	HATCH 2 - CORRECT ERROR IN T.S. RELATED TO SECTIONAL REFERENCES		2	02/22/84
52483	HATCH 2 - PIPE REPLACEMENT PLAN - RECIRC SYSTEM			02/28/84
53479	HATCH 2 - EXTEND COMPLETION SCHEDULE FOR NUREG 0737 ITEM II.B.3			03/08/84
48297	HATCH 2 - INOPERABLE LPC1 INVERTER - EXTENDS LCO TO SEVEN DAYS FROM 44 HOURS		2	03/27/84
48749	HATCH 2 - FIRE PROTECTION PROGRAM - APP.R EXEMPTION REQUESTS & SAFE SHUTDOWN REPORT		1	04/18/84
<u>ACTIVE ACTIONS</u>				
54607	HATCH 2 - TENDMENT - (CYCLE 5) RELOAD 4			<u>TARGET</u> 06/15/84
54169	HATCH 2 - ANALOGUE TRANSMITTER TRIP SYSTEM - TS			06/15/84
54433	HATCH 2 - APRM/RBM TECH SPEC CHANGES			06/15/84
51985	HATCH #2 PURGE & VENT VALVE - SYST-DESIGN & TS CHANGES			06/15/84
49077	HATCH 2 - TITLE CHANGES OF PLANT REVIEW BOARD MEMBERS IN APP.A & B T.S.		2	06/30/84
48121	HATCH 2 - ADMIN. TECH SPEC CHNG RELATED TO REPORTING RESULTS OF TYPE B&C LEAK TESTS		2	06/30/84
53477	HATCH 2 - IODINE LIMITS			06/30/84
52640	HATCH UNIT 2 - ADD TECH SPECS FOR ITEM II.K.3.22 RCIC SUCTION AND TRANSFER			06/30/84
53328	HATCH #2 - CLOSURE TIME FOR SDV VENT AND DRAIN VALVES			06/30/84
54460	HATCH #2 EMERGENCY EXERCISE EXEMPTION			06/30/84
54462	HATCH #2 TECH SPECS FOR TMI ACTION ITEM IIF.1.1, 2 AND 3			07/15/84
11158	HATCH 2 - SETTLEMENT OF CLASS I STRUCTURES		1	07/30/84
53398	HATCH 2 - RELAXATION OF TIP LCO			07/31/84
53455	HATCH #2 H2 AND O2 MONITORING TS			07/31/84
49091	HATCH 2 - PROPOSED T.S. CHANGE FOR FLOW MONITORING OF GASEOUS RELEASES FROM VENT		2	08/30/84
<u>ANTICIPATED ACTIONS</u>				
53596	HATCH 2 - ITEM 1.2 - POST TRIP REVIEW-DATA AND INFORMATION CAPABILITY			<u>INITIATION</u> 11/01/84
53679	HATCH 2 - ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS			11/01/84
53762	HATCH 2- ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS			11/01/84
53845	HATCH 2 - ITEM 3.2.3 - POST-MAINTENANCE TESTING -CHANGES TO TECH. SPEC. - ALL SR COMPONENTS			11/01/84
53989	HATCH 2 - ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS			11/01/84
54072	HATCH 2 - ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL TESTING - DIVERSE TRIP FEATURES			11/01/84

DATA AS OF - 04/30/84

CONDENSED MANAGEMENT REPORT

FACILITY: HUMBOLDT BAY 3

PLANT LOCATION: 4 MI SW OF EUREKA, CA
 DOCKET NUMBER: 050-00133
 ARCH/ENGINEER: BECH
 IE INSPECTOR: P. MORRILL

LICENSED POWER: 0220 MWT
 DESIGN POWER: 0065 MWE
 NSSS VENDOR: GE

PROJECT MANAGER: P. ERICKSON
 BRANCH CHIEF: D. CRUTCHFIELD
 LIC. ASSISTANT: H. SMITH

<u>IAC #</u>	<u>IAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
	<u>ACTIVE ACTIONS</u>			<u>TARGET</u>
01863	HUMBOLDT BAY - SEISMIC CONCERNS		1	05/01/84
52637	HUMBOLT BAY UNIT 2 - DISPOSAL OF VERY LOW-LEVEL RADIOACTIVE WASTE			06/01/84
53378	HUMBOLT BAY UNIT 3 - MICROSEISMIC MONITORING NETWORK			07/01/84

DATA AS OF - 04/30/84

CONDENSED MANAGEMENT REPORT

FACILITY: INDIAN POINT 2

PLANT LOCATION: 25 MI N OF NEW YORK CITY, NY
 DOCKET NUMBER: 050-00247
 ARCH/ENGINEER: UEC
 IE INSPECTOR: T. REBELOWSKI

LICENSED POWER: 2758 MWT
 DESIGN POWER: 0873 MWE
 NSSS VENDOR: WEST

PROJECT MANAGER: R. PEDERSEN
 BRANCH CHIEF: S. VARGA
 LIC. ASSISTANT: C. PARRISH

TAC # TAC DESCRIPTIONMULTI-PLANTPRIORITYCRITICAL DATECOMPLETED ACTIONS

42862 INDIAN POINT 2 - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11
 08296 INDIAN POINT 2 - 10 CFR 50.55A(G) - INSERVICE - IST
 08700 INDIAN POINT 2 - FUEL HANDLING ACCIDENT INSIDE CONTAINMENT

2 COMPLETE
 10/19/83
 1 11/28/83
 2 02/21/84

ACTIVE ACTIONS

51098 INDIAN POINT 2 - INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)
 52236 INDIAN POINT 2 - CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPA#C-10)
 52765 INDIAN POINT 2 - ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION AND PROCEDURES
 52846 INDIAN POINT 2 - ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RTS COMPONENTS
 52927 INDIAN POINT 2 - ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING PROCEDURES/VENDOR
 RECOMMENDATIONS - RTS COMPONENTS
 53008 INDIAN PT 2-ITEM 3.1.3 -POST-MAINTENANCE TESTING-CHANGES TO TECH SPECS-RTS COMPONENTS
 53074 INDIAN POINT 2 - REACTOR TRIP SYSTEM RELIABILITY - VENDOR RELATED MODIFICATIONS
 53127 INDIAN POINT 2 - ITEMS 4.2.1 & 4.2.2 - PREVENTATIVE MAINT PROGRAM FOR REACTOR TRIP BREAKERS -
 MAINTENANCE & TRENDING
 53176 INDIAN POINT 2 - ITEM 4.3 - AUTOAMTIC ACTUATION OF SHUNT TRIP ATTACHMENT FOR W AND B & W PLANTS
 42096 INDIAN POINT 2 - DECAY HEAT REMOVAL CAPABILITY TECH SPECS
 06603 INDIAN POINT 2 - REACTOR VESSEL OVERPRESSURE PROTECTION SYSTEM
 06901 INDIAN POINT 2 - ASYMETRIC LOCA LOADS
 07996 INDIAN PT. 2 -CONTROL OF HEAVY LOADS OVER SPENT FUEL POOL (NUREG 0612)
 08113 INDIAN POINT 2 - APPENDIX I TECH SPEC IMPLEMENTATION
 43006 INDIAN POINT 2 - TECHNICAL SPECIFICATIONS DEFINING OPERABILITY FOR SAFETY SYSTEMS
 43963 INDIAN POINT 2 - SAFE SHUTDOWN CAPABILITY
 49736 INDIAN POINT 2 -"NUREG 0737 TECH SPECS (GENERIC LETTER 82-16 OR 83-02)"
 08763 INDIAN POINT 2 - HYDRAULIC SNUBBERS
 46839 INDIAN POINT 2 - PWR MSLB WITH CONTINUED FEEDWATER ADDITION
 49672 INDIAN POINT 2 - REACTOR COOLANT PUMP TRIP
 42464 INDIAN POINT 2 - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-20)

2 03/30/84
 2 03/30/84
 2 03/30/84
 2 03/30/84
 2 03/31/84
 3 03/31/84
 2 03/31/84
 1 04/30/84
 2 06/30/84
 3 06/30/84
 2 06/30/84
 1 07/01/84
 1 12/30/84

TARGETTAC # TAC DESCRIPTIONTMI ACTIONSPRIORITYCRITICAL DATECOMPLETED ACTIONS

45636 INDIAN POINT 2 - NUREG-0737 II.K.3.17, ECC SYSTEM OUTAGES
 47945 INDIAN POINT 2 - PLANT SHIELDING MODIFICATIONS (II.B.2.2)
 47802 INDIAN POINT 2 - NUREG 0737 ITEM II K2.17 POTENTIAL FOR VOIDING IN RCS
 45141 INDIAN POINT 2 - NUREG-0737 II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION

1 COMPLETE
 10/19/83
 1 10/26/83
 1 01/03/84
 03/20/84

ACTIVE ACTIONS

51168 INDIAN POINT 2 - I.D.1 CONTROL ROOM DESIGN REVIEW
 51248 INDIAN POINT 2 - I.D.2. SAFETY PARAMETER DISPLAY SYSTEM
 44447 INDIAN POINT 2 - NUREG-0737 II.B.3.2, POST ACCIDENT SAMPLINGMODIFICATIONS
 48172 INDIAN POINT 2 -NUREG 0737 II.K.3.31 COMPLIANCE WITH 10 CFR 50.46
 44588 INDIAN POINT 2 - RV AND SV TESTING
 51912 INDIAN POINT 2 - NUREG 0737 TECH. SPEC. CHANGE PER 10/7/81 AND 11/19/81 II.F.1.1 & II.F.1.2
 45833 INDIAN POINT 2 - NUREG-0737 II.K.3.30, SB LOCA OUTLINE

1 03/30/84
 03/30/84
 1 03/31/84
 03/31/84
 06/30/84

TARGET

DATA AS OF - 04/30/84

CONDENSED MANAGEMENT REPORT

(CONTINUATION)

INDIAN POINT 2

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>IMI ACTIONS</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>ACTIVE ACTIONS</u> (CONTINUATION)				
45952	INDIAN POINT 2 - NUREG-0737	III.A.1.2, TECHNICAL SUPPORT CENTER		<u>TARGET</u> 09/30/84
46023	INDIAN POINT 2 - NUREG-0737	OPERATIONAL SUPPORT CENTER		09/30/84
46095	INDIAN POINT 2 - NUREG-0737	III.A.1.2, EMERGENCY OPERATIONS FACILITY		09/30/84
46239	INDIAN POINT 2 - NUREG-0737	III.A.1.2, EMERGENCY PLAN UPGRADE TO MEET RULE		09/30/84
46311	INDIAN POINT 2 - NUREG-0737	III.A.2.2, METEOROLOGICAL DATA UPGRADE		09/30/84
46888	INDIAN PT. 2 - II K2.13	THERMAL MECHANICAL REPORT	1	09/30/84
44309	INDIAN POINT 2 - NUREG-0737	I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES		09/30/84
44239	INDIAN POINT 2 - NUREG-0737	I.C.1.2.A INADEQUATE CORE COOLING GUIDELINES		11/30/84

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
52521	INDIAN POINT 2 - APPENDIX H	SCHEDULED EXEMPTION		<u>COMPLETE</u> 11/07/83
51923	INDIAN POINT 2 - OPERABILITY	TECHNICAL SPECIFICATION CHANGE		11/15/83
08776	INDIAN POINT 2 - PWR PUMP & S/G	SUPPORTS - LAMELLAR TEARING & FRACTURE TOUGHNESS	2	11/23/83

<u>ACTIVE ACTIONS</u>				<u>TARGET</u>
53429	INDIAN POINT 2 - CONTAINMENT	RESPONSE WITH SPRAY ISOLATED		
54221	INDIAN POINT 2 - ORGANIZATION	STRUCTURE TS CHANGE		
54680	INDIAN POINT 2 - EMERGENCY	PREPAREDNESS EXERCISE EXTENSION		
51916	INDIAN POINT 2 - POST ACCIDENT	CONTAINMENT VENTING TECH. SPEC.		02/28/84
49799	INDIAN POINT 2 - CONTAINMENT	SEISMIC CAPABILITY		03/30/84
48223	INDIAN POINT 2 BORON INJECTION	TANK OPERATION AND SURVEILLANCE REQUIREMENTS	3	03/30/84
51913	INDIAN POINT 2 - INSERVICE	INSPECTION PROGRAM TECH. SPEC. CHANGE		03/31/84
51917	INDIAN POINT 2 - REACTOR VESSEL	SURVEILLANCE CAPSULE PROGRAM TECH. SPEC.		03/31/84
51919	INDIAN POINT 2 - WATER ABOVE	REACTOR TECHNICAL SPECIFICATION		03/31/84
51921	INDIAN POINT 2 - EXTERNAL	FLOODING CONDITIONS TECH. SPEC.		03/31/84
51922	INDIAN POINT 2 - CONTROL OF	HEAVY LOADS TECHNICAL SPECIFICATION		03/31/84
12842	INDIAN POINT 2 - GAS TURBINE	OPERATION	1	03/31/84
51918	INDIAN POINT 2 - BORIC ACID	ADDITION TECH. SPEC. CHANGE		05/31/84
51925	INDIAN POINT 2 - STS	UPGRADE		05/31/84
54963	INDIAN POINT 2 - REVIEW AND	APPROVE STEAM GENERATOR EXAM PROGRAM		06/08/84
51914	INDIAN POINT 2 - ALTERNATE	SHUTDOWN CAPABILITY TECH. SPEC. CHANGE		06/30/84
48055	INDIAN POINT 2 - PRA		3	06/30/84
54394	INDIAN POINT 2 - RCS VENTS	TECHNICAL SPECIFICATIONS NUREG 073 ITEM II.B.1		09/30/84
54539	INDIAN POINT 2 - TECHNICAL	SPECIFICATIONS COVERED BY GENERIC LETTERS 83-36, 83-37 FOR NUREG-0737		09/30/84
54226	INDIAN POINT 2 - IST	PROGRAM II		02/28/85

<u>ANTICIPATED ACTIONS</u>				<u>INITIATION</u>
53597	INDIAN POINT 2 - ITEM 1.2 -	POST TRIP REVIEW-DATA AND INFORMATION CAPABILITY		11/01/84
53680	INDIAN POINT 2 - ITEM 2.2 -	EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS		11/01/84
53763	INDIAN PT 2- ITEMS 3.2.1 & 3.2.2 -	POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS		11/01/84
53846	INDIAN POINT 2 - ITEM 3.2.3 -	POST-MAINTENANCE TESTING -CHANGES TO TECH. SPEC. - ALL SR COMPONENTS		11/01/84
53914	-ITEMS 4.2.3 & 4.2.4-	PREVENTATIVE MAINT. PROGRAM FOR REACTOR TRIP BREAKERS-LIFE TESTING & REPLACEMENT		11/01/84
53990	INDIAN POINT 2 - ITEMS 4.5.2 & 4.5.3 -	REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS		11/01/84

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TECHNICAL ASSIGNMENT CONTROL SYSTEM

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CONDENSED MANAGEMENT REPORT

(CONTINUATION)

INDIAN POINT 2

TAC # TAC DESCRIPTION

PLANT SPECIFIC

PRIORITY

CRITICAL DATE

ANTICIPATED ACTIONS (CONTINUATION)

INITIATION

54073 INDIAN POINT 2 - ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL TESTING - DIVERSE TRIP FEATURES

11/01/84

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CONDENSED MANAGEMENT REPORT

FACILITY: INDIAN POINT 3

PLANT LOCATION: 25 MI N OF NEW YORK CITY, NY
 DOCKET NUMBER: 050-00286
 ARCH/ENGINEER: UEC
 IE INSPECTOR: T. KENNY

LICENSED POWER: 3025 MWT
 DESIGN POWER: 0965 MWE
 NSSS VENDOR: WEST

PROJECT MANAGER: P. POLK
 BRANCH CHIEF: S. VARGA
 LIC. ASSISTANT: C. PARRISH

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>MULTI-PLANT</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
49421	INDIAN POINT 3 - BLOCKING SI SIGNAL DURING COOLDOWN			<u>COMPLETE</u> 04/26/84
<u>ACTIVE ACTIONS</u>				
51099	INDIAN POINT 3 - INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)			<u>TARGET</u>
52237	INDIAN POINT 3 - CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPA#C-10)			
52766	INDIAN POINT 3 - ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION AND PROCEDURES			
52847	INDIAN POINT 3 - ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RTS COMPONENTS			
52928	INDIAN POINT 3 - ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING PROCEDURES/VENDOR RECOMMENDATIONS - RTS COMPONENTS			
53009	INDIAN PT 3-ITEM 3.1.3 -POST-MAINTENANCE TESTING-CHANGES TO TECH SPECS-RTS COMPONENTS			
53075	INDIAN POINT 3 - REACTOR TRIP SYSTEM RELIABILITY - VENDOR RELATED MODIFICATIONS			
53128	INDIAN POINT 3 - ITEMS 4.2.1 & 4.2.2 - PREVENTATIVE MAINT PROGRAM FOR REACTOR TRIP BREAKERS - MAINTENANCE & TRENDING			
53177	INDIAN POINT 3 - ITEM 4.3 - AUTOAMTIC ACTUATION OF SHUNT TRIP ATTACHMENT FOR W AND B & W PLANTS			
10029	INDIAN POINT 3 - POTENTIAL EQUIPMENT FAILURES ASSOCIATED WITH GRID VOLTAGE		2	04/30/83
12993	INDIAN POINT 3 - ADEQUACY OF STATION ELECTRIC DISTRIBUTION SYSTEMS VOLTAGES		2	04/31/83
08114	INDIAN POINT 3 - APPENDIX I TECH SPEC IMPLEMENTATION REVIEW		3	06/30/83
08337	INDIAN POINT 3 - REACTOR VESSEL OVERPRESSURE PROTECTION		2	07/01/83
08765	INDIAN POINT 3 - HYDRAULIC SNUBBERS		3	07/01/83
42097	INDIAN POINT 3 - DECAY HEAT REMOVAL CAPABILITY TECH SPECS		2	07/01/83
42558	INDIAN POINT 3 - LONG TERM CONTAINMENT PURGE AND VENT (B-24)		1	07/01/83
07997	INDIAN PT 3 -CONTROL OF HEAVY LOADS OVER SPENT FUEL POOL (NUREG 0612)		2	08/31/83
49737	INDIAN POINT 3 -"NUREG 0737 TECH SPECS (GENERIC LETTER 82-16 OR 83-02)"			09/30/83
08773	INDIAN POINT 3 - REVIEW OF ASYMMETRIC LOCA LOADS SUBMITTAL		2	12/30/83
42474	INDIAN POINT 3 - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21)		1	12/31/83
11209	INDIAN POINT 3 - IST PUMPS AND VALVES		1	12/31/83
49687	INDIAN POINT 3 - REACTOR COOLANT PUMP TRIP			07/01/84
<u>TAC # TAC DESCRIPTION MULTI-PLANT PRIORITY CRITICAL DATE</u>				
<u>COMPLETED ACTIONS</u>				
46240	INDIAN POINT 3 - III.A.2.1 EMERGENCY PLAN UPGRADE TO MEET RULE		1	<u>COMPLETE</u> 10/03/83
47803	INDIAN POINT 3 - NUREG 0737 ITEM II K2.17 POTENTIAL FOR VOIDING IN RCS		1	01/10/84
<u>ACTIVE ACTIONS</u>				
51169	INDIAN POINT 3 - I.D.1 CONTROL ROOM DESIGN REVIEW			<u>TARGET</u>
51249	INDIAN POINT 3 - I.D.2. SAFETY PARAMETER DISPLAY SYSTEM			
46312	INDIAN POINT 3 - NUREG-0737 III.A.2.2, METEOROLOGICAL DATA UPGRADE		1	06/01/83
48173	INDIAN POINT 3 -NUREG 0737 II.K.3.31 COMPLIANCE WITH 10 CFR 50.46		1	06/30/83
45142	INDIAN POINT 3 - NUREG-0737 II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION		1	06/30/83
45834	INDIAN POINT 3 - NUREG-0737 II.K.3.30, SB LOCA OUTLINE		1	06/30/83
45283	INDIAN POINT 3 - NUREG-0737 II.K.3.2, REPORT ON PORV FAILURES		1	07/01/83
48422	INDIAN POINT 3 - NUREG 0737, II.K.3.25 POWER ON PUMP SEALS			07/01/83
47946	INDIAN POINT 3 - PLANT SHIELDING MODIFICATIONS (II.B.2.2)		1	10/30/83
45237	INDIAN POINT 3 - NUREG-0737 II.K.3.1, AUTO PORV ISOLATION		1	11/25/83

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(CONTINUATION)

INDIAN POINT 3

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>TMI ACTIONS</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>ACTIVE ACTIONS (CONTINUATION)</u>				
44448	INDIAN POINT 3 - NUREG-0737 II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS		1	TARGET 06/30/84
45953	INDIAN POINT 3 - NUREG-0737 III.A.1.2, TECHNICAL SUPPORT CENTER		1	09/30/84
46024	INDIAN POINT 3 - NUREG-0737 III.A.1.2, OPERATIONAL SUPPORT CENTER		1	09/30/84
46889	INDIAN PT. 3 - II K2.13 THERMAL MECHANICAL REPORT		1	09/30/84
46096	INDIAN POINT 3 - NUREG-0737 III.A.1.2, EMERGENCY OPERATIONS FACILITY		1	11/30/84
44240	INDIAN POINT 3 - NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES		1	11/30/84
44310	INDIAN POINT 3 - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES		1	11/30/84
<u>PLANT SPECIFIC</u>				
<u>TAC #</u>	<u>TAC DESCRIPTION</u>		<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
08777	INDIAN POINT 3 - PWR PUMP & S/G SUPPORTS - LAMELLAR TEARING & FRACTURE TOUGHNESS		2	COMPLETE 12/28/83
53227	INDIAN POINT 3 - SHIFT MANNING - PLANT SPECIFIC			12/31/83
49591	INDIAN POINT 3 - SHUTDOWN REACTIVITY			01/13/84
48735	INDIAN POINT 3 - SAFE SHUTDOWN CAPABILITY (APPENDIX R REV)			04/16/84
54650	INDIAN POINT 3 - CHARCOAL DISPOSAL			04/24/84
<u>ACTIVE ACTIONS</u>				
49873	INDIAN POINT 3 - REVIEW OF CONTAINMENT COOLING TECHNICAL SPECIFICATIONS			TARGET 06/30/83
51944	INDIAN POINT 3 - CONTAINMENT ISOLATION VALVES TECH SPEC CHANGE			12/31/83
51946	INDIAN POINT 3 - RETS TECH SPEC CHANGE			12/31/83
51947	INDIAN POINT 3 - BOP CHEMISTRY TECH SPEC CHANGE			12/31/83
51949	INDIAN POINT 3 - SNUBBER SURVEILLANCE TECH SPEC CHANGE			12/31/83
51950	INDIAN POINT 3 - MANAGEMENT REORGANIZATION TECH SPEC CHANGE (6/82)			12/31/83
51951	INDIAN POINT 3 - MANAGEMENT REORGANIZATION TECH SPEC CHANGE (3/83)			12/31/83
51952	INDIAN POINT 3 - FIRE PROTECTION MODIFICATIONS TECH SPEC CHANGE		1	12/31/83
12817	INDIAN POINT 3 - STANDARD TECH SPECS			01/30/84
51943	INDIAN POINT 3 - SITE MANAGEMENT REORGANIZATION TECH SPEC CHANGE			02/20/84
51945	INDIAN POINT 3 - CONTAINMENT ISOLATION VALVE TECH SPEC CHANGE			09/30/84
54395	INDIAN POINT 3 - RCS VENTS TECHNICAL SPECIFICATIONS NUREG 073 ITEM II.B.1			09/30/84
54540	INDIAN POINT 3 - TECHNICAL SPECIFICATIONS COVERED BY GENERIC LETTERS 83-36, 83-37 FOR NUREG-0737		3	12/30/84
48056	INDIAN POINT 3 - PRA			12/31/84
54850	INDIAN POINT 3 - LER ADMINISTRATIVE CONTROLS TECHNICAL SPECIFICATION		1	09/30/85
43373	INDIAN POINT 3 - SI REVIEW AND EVALUATION			
<u>ANTICIPATED ACTIONS</u>				
53764	INDIAN PT 3 - ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS			INITIATION 11/01/84
53847	INDIAN POINT 3 - ITEM 3.2.3 - POST-MAINTENANCE TESTING - CHANGES TO TECH. SPEC. - ALL SR COMPONENTS			11/01/84
53915	-ITEMS 4.2.3 & 4.2.4 - PREVENTATIVE MAINT. PROGRAM FOR REACTOR TRIP BREAKERS - LIFE TESTING & REPLACEMENT			11/01/84
53991	INDIAN POINT 3 - ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS			11/01/84
54074	INDIAN POINT 3 - ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL TESTING - DIVERSE TRIP FEATURES			11/01/84
53598	INDIAN POINT 3 - ITEM 1.2 - POST TRIP REVIEW - DATA AND INFORMATION CAPABILITY			11/01/84
53681	INDIAN POINT 3 - ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS			11/01/84

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CONDENSED MANAGEMENT REPORT

FACILITY: KEWAUNEE

PLANT LOCATION: 27 MI E OF GREEN BAY, WI.
 DOCKET NUMBER: 050-00305
 ARCH/ENGINEER: PSE
 IE INSPECTOR: R. NELSON

LICENSED POWER: 1650 MWT
 DESIGN POWER: 0535 MWE
 NSSS VENDOR: WEST

PROJECT MANAGER: M. GROTENHUIS
 BRANCH CHIEF: S. VARGA
 LIC. ASSISTANT: C. PARRISH

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>MULTI-PLANT</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
08239	KEWAUNEE - PWR SECONDARY CHEMISTRY MONITORING REQUIREMENTS - GENERIC		2	<u>COMPLETE</u> 10/06/83
12269	KEWAUNEE - HELB AND CONSEQUENTIAL SYSTEM FAILURE KEWAUNEE		3	02/02/84
49738	- KEWAUNEE NUREG 0737 TECH SPECS (GENERIC LTR 82-16)			02/10/84
46747	KEWAUNEE - CONTROL OF HEAVY LOADS OVER SPENT FUEL POOL NUREG-0612, MPA MC-10.		2	03/16/84

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>MULTI-PLANT</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>ACTIVE ACTIONS</u>				
51100	KEWAUNEE - INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)			<u>TARGET</u>
52767	KEWAUNEE - ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION DESCRIPTION AND PROCEDURES			
52848	KEWAUNEE - ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RTS COMPONENTS			
52929	KEWAUNEE - ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING TESTING PROCEDURES/VENDOR RECOMMENDATIONS - RTS COMPONENTS			
53010	KEWAUNEE-ITEM 3.1.3 -POST-MAINTENANCE TESTING-CHANGES TO TECH SPECS-RTS COMPONENTS			
53076	KEWAUNEE - REACTOR TRIP SYSTEM RELIABILITY- VENDOR RELATED MODIFICATIONS			
53129	KEWAUNEE - ITEMS 4.2.1 & 4.2.2 - PREVENTATIVE MAINTENANCE PROGRAM FOR REACTOR TRIP BREAKERS - MAINTENANCE & TRENDING			
53178	KEWAUNEE - ITEM 4.3 - AUTOAMTIC ACTUATION OF SHUNT TRIP ATTACHMENT FOR W AND B & W PLANTS			
42465	KEWAUNEE - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21)		1	
42864	KEWAUNEE - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11		2	12/30/83
08525	KEWAUNEE - TECH SPEC SURVEILLANCE OF HYDRAULIC SNUBBERS		3	12/30/83
107-4	KEWAUNEE - ASYMMETRIC LOCA LOADS		2	12/31/83
08145	KEWAUNEE - APPENDIX I TECH SPEC IMPLEMENTATION REVIEW		3	12/31/83
43008	KEWAUNEE - TECHNICAL SPECIFICATIONS DEFINING OPERABILITY FOR SAFETY SYSTEMS		2	12/31/83
06886	KEWAUNEE - REACTOR VESSEL OVERPRESSURE PROTECTION		2	01/31/84
08350	KEWAUNEE - WESTINGHOUSE UPPER PLENUM ECCS INJECTION - LONG RANGE		3	06/30/84
49671	KEWAUNEE - REACTOR COOLANT PUMP TRIP			07/01/84
52238	KEWAUNEE - CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPA#C-10)			09/30/85

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>IMI ACTIONS</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
47804	KEWAUNEE - NUREG 0737 ITEM II K2.17 POTENTIAL FOR VOIDING IN RCS		1	<u>COMPLETE</u> 01/09/84
44098	KEWAUNEE - NUREG-0737 I.A.1.3.1, SHIFT MANNING OVERTIME LIMITS			04/18/84

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>IMI ACTIONS</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>ACTIVE ACTIONS</u>				
44449	KEWAUNEE - NUREG-0737 II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS		1	<u>TARGET</u>
44590	KEWAUNEE - RV AND SV TESTING		1	
45835	KEWAUNEE - NUREG-0737 II.K.3.30, SB LOCA OUTLINE		1	
46313	KEWAUNEE - NUREG-0737 III.A.2.2, METEOROLOGICAL DATA UPGRADE		1	
48174	KEWAUNEE -NUREG 0737 II.K.3.31 COMPLIANCE WITH 10 CFR 50.46		1	
51170	KEWAUNEE - I.D.1 CONTROL ROOM DESIGN REVIEW			
51250	KEWAUNEE - I.D.2. SAFETY PARAMETER DISPLAY SYSTEM			
45143	KEWAUNEE - NUREG-0737 II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION		1	12/31/83
45954	KEWAUNEE - NUREG-0737 III.A.1.2, TECHNICAL SUPPORT CENTER		1	09/30/84
46025	KEWAUNEE - NUREG-0737 III.A.1.2, OPERATIONAL SUPPORT CENTER		1	09/30/84
46097	KEWAUNEE - NUREG-0737 III.A.1.2, EMERGENCY OPERATIONS FACILITY		1	09/30/84

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CONDENSED MANAGEMENT REPORT

(CONTINUATION)

KEWAUNEE

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>TMI ACTIONS</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>ACTIVE ACTIONS</u> (CONTINUATION)				
46890	KEWAUNEE - II K2.13 THERMAL MECHANICAL REPORT		1	<u>TARGET</u> 09/30/84
44241	KEWAUNEE - NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES		1	11/30/84
44311	KEWAUNEE - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES		1	11/30/84
<u>PLANT SPECIFIC</u>				
<u>COMPLETED ACTIONS</u>				
49860	KEWAUNEE - OPERABILITY OF PRATT PURGE VALVES			<u>COMPLETE</u> 11/21/83
52148	KEWAUNEE - PURGE & VENT			11/21/83
51500	KEWAUNEE - PROP. AMEND 54 - MISCEL 4/29/83			12/28/83
49360	KEWAUNEE			02/09/84
52389	KEWAUNEE - APPENDIX R SCHEDULE EXEMPTION 2			02/28/84
54476	KEWAUNEE - EXEMPTION FROM APPENDIX J			04/24/84
54209	KEWAUNEE - UNDERVOLTAGE TRIP SETPOINTS			04/30/84
<u>ACTIVE ACTIONS</u>				
49609	KEWAUNEE RESPONSE TO GL 82-17			<u>TARGET</u>
49131	KEWAUNEE - APPENDIX J TECH. SPECS.		1	12/31/83
49859	KEWAUNEE - ANALOG ROD POSITION INDICATION			12/31/83
46817	KEWAUNEE: PRESSURE ISOLATION VALVES		3	02/01/84
53488	KEWAUNEE - EVALUATE REMOVAL OF RHR AUTOCLOSURE FEATURE			02/24/84
53358	KEWAUNEE - TOTAL PEAKING FACTOR AND EXPOSURE RANGE			03/01/84
54195	KEWAUNEE - RELIEF REQUESTS (HYDRO TESTS)			03/15/84
53493	KEWAUNEE - ISI TECHNICAL SPECIFICATIONS			06/30/84
54396	KEWAUNEE- RCS VENTS TECHNICAL SPECIFICATIONS NUREG 073 ITEM II.B.1			09/30/84
54541	KEWAUNEE - TECHNICAL SPECIFICATIONS COVERED BY GENERIC LETTERS 83-36, 83-37 FOR NUREG-0737			09/30/84
54739	KEWAUNEE 1ST - 2ND TEN YEAR PROGRAM			04/30/85
54740	KEWAUNEE - ISI AND 1ST 2ND YEAR PROGRAM			04/30/85
<u>ANTICIPATED ACTIONS</u>				
53599	KEWAUNEE - ITEM 1.2 - POST TRIP REVIEW-DATA AND INFORMATION CAPABILITY			<u>INITIATION</u> 11/01/84
53682	KEWAUNEE - ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS			11/01/84
53765	KEWAUNEE- ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS			11/01/84
53848	KEWAUNEE - ITEM 3.2.3 - POST-MAINTENANCE TESTING -CHANGES TO TECH. SPEC. - ALL SR COMPONENTS			11/01/84
53916	-ITEMS 4.2.3 & 4.2.4-PREVENTATIVE MAINT. PROGRAM FOR REACTOR TRIP BREAKERS-LIFE TESTING & REPLACEMENT			11/01/84
53992	KEWAUNEE 1 - ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS			11/01/84
54075	KEWAUNEE - ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL TESTING - DIVERSE TRIP FEATURES			11/01/84

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CONDENSED MANAGEMENT REPORT

FACILITY: LA CROSSE

PLANT LOCATION: 19 MI S OF LACROSSE, WISC
 DOCKET NUMBER: 050-00409
 ARCH/ENGINEER: S&L
 IE INSPECTOR: M. BRANCH

LICENSED POWER: 0165 MWT
 DESIGN POWER: 0050 MWE
 NSSS VENDOR: AC

PROJECT MANAGER: R. DUDLEY
 BRANCH CHIEF: D. CRUTCHFIELD
 LIC. ASSISTANT: H. SMITH

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>MULTI-PLANT</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
46664	LACROSSE - IMPLEMENTATION OF NUREG-0313, REV. 1		2	<u>COMPLETE</u> 10/14/83
43980	LACROSSE - APPENDIX R REVIEW		1	04/04/84
42611	LACROSSE - LONG TERM REVIEW CONTAINMENT PURGE AND VENT (B-24)		1	04/30/84

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>MULTI-PLANT</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>ACTIVE ACTIONS</u>				
52768	LACROSSE - ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION DESCRIPTION AND PROCEDURES			<u>TARGET</u>
52849	LACROSSE - ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR VENDOR INTERFACE - RTS COMPONENTS			
52930	LACROSSE - ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING PROCEDURES/VENDOR RECOMMENDATIONS - RTS COMPONENTS			
53011	LACROSSE-ITEM 3.1.3 -POST-MAINTENANCE TESTING-CHANGES TO TECH SPECS-RTS COMPONENTS			
08099	LACROSSE - APPENDIX I TECH SPEC IMPLEMENTATION REVIEW		3	06/15/84
51682	LA CROSSE NUREG 0737 TECH SPECS			08/15/84
52239	LACROSSE - CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPA#C-10)			09/30/84
42522	LACROSSE - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21)		1	09/30/84
42912	LACROSSE - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11		2	10/15/84
51101	LACROSSE - INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)			01/01/86

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>TMI ACTIONS</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
48175	LACROSSE -NUREG 0737 II.K.3.31 COMPLIANCE WITH 10 CFR 50.46		1	<u>COMPLETE</u> 02/02/84
45836	LACROSSE - NUREG-0737 II.K.3.30, SB LOCA OUTLINE		1	02/02/84
45889	LACROSSE - NUREG-0737 II.K.3.44, TRANSIENTS WITH SINGLE FAILURES		1	03/26/84

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>TMI ACTIONS</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>ACTIVE ACTIONS</u>				
47680	LACROSSE - NUREG 0737 ITEM II F1.5 CONTAINMENT WATER LEVEL MONITOR		1	<u>TARGET</u> 04/01/84
47751	LACROSSE - NUREG 0737 ITEM II F1.6 CONTAINMENT HYDROGEN MONITOR		1	04/01/84
47609	LACROSSE - NUREG 0737 ITEM II F1.4 CONTAINMENT PRESSURE INSTRUMENT		1	07/01/84
51171	LACROSSE - I.D.1 CONTROL ROOM DESIGN REVIEW		1	09/30/84
45955	LACROSSE - NUREG-0737 III.A.1.2, TECHNICAL SUPPORT CENTER		1	09/30/84
46026	LACROSSE - NUREG-0737 III.A.1.2, OPERATIONAL SUPPORT CENTER		1	09/30/84
46098	LACROSSE - NUREG-0737 III.A.1.2, EMERGENCY OPERATIONS FACILITY		1	09/30/84
46314	LACROSSE - NUREG-0737 III.A.2.2, METEOROLOGICAL DATA UPGRADE		1	09/30/84
44450	LACROSSE - NUREG-0737 II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS		1	11/30/84
44591	LA CROSSE - RV AND SV TESTING		1	11/30/84
44242	LACROSSE - NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES		1	07/15/85
44312	LACROSSE - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES		1	07/15/85
45144	LACROSSE - NUREG-0737 II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION		1	09/01/85
51251	LACROSSE - I.D.2. SAFETY PARAMETER DISPLAY SYSTEM		1	09/30/85

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
51876	LACROSSE - SEP TOPIC VI-7.C.1, APPENDIX K - ELECTRICAL REVIEWS			<u>COMPLETE</u> 10/07/83
49529	LA CROSSE - DEGRADED GRID VOLTAGE TECH SPECS		3	10/14/83

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CONDENSED MANAGEMENT REPORT

(CONTINUATION)

LA CROSSE

TAC #	TAC DESCRIPTION	PLANT SPECIFIC	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u> (CONTINUATION)				<u>COMPLETE</u>
49536	LACROSSE: REVIEW FIRE PROTECTION TECH SPECS		2	10/14/83
51039	LACROSSE - REVIEW ESWSS SURVEILLANCE REQUIREMENTS		3	10/14/83
47533	LACROSSE TECH SPEC CHANGES (MISC)		3	10/14/83
49971	LACROSSE - EQUIPMENT QUALIFICATION SCHEDULE EXEMPTION REQUEST		2	10/19/83
51877	LACROSSE - SEP TOPIC VII-1.A, ISOLATION OF RPS			10/27/83
51872	LACROSSE - SEP TOPIC III-5.B, PIPE BREAK OUTSIDE CONTAINMENT			10/27/83
53224	LA CROSSE - TEMPORARY CHANGE TO OPERATION REQUALIF PROGRAM			12/02/83
49040	LA CROSSE: VENTILATION VALVE SEAL TESTING		2	12/12/83
52306	LA CROSSE - OPERATOR STAFFING RULE EXEMPTION			12/30/83
51875	LACROSSE - SEP TOPIC VI-4, CONTAINMENT ISOLATION			04/26/84
<u>ACTIVE ACTIONS</u>				<u>TARGET</u>
53264	LA CROSSE - BYPASS LINWELD FAILURE			04/15/84
52171	LA CROSSE - RT-NDT SHIFT TECH SPEC			04/30/84
52172	LA CROSSE - NUCLEAR INST. TECH SPEC			04/30/84
51878	LACROSSE - SEP TOPIC VIII-3.B, DC BUS VOLTAGE MONITORING			04/30/84
51879	LACROSSE - SEP TOPIC IX-5, VENTILATION SYSTEMS			04/30/84
52169	LA CROSSE - ECCS TECH SPEC REVISION			05/01/84
52170	LA CROSSE - SAFETY VALVES			05/01/84
53413	LACROSSE - SEP TECH SPECS			05/01/84
53414	LACROSSE - WATER QUALITY & FLOODING TECH SPECS			06/01/84
51874	LACROSSE - SEP TOPIC III-7.B, DESIGN CODES, DESIGN CRITERIA, LOAD COMBINATIONS			06/01/84
49088	LA CROSSE - SECURITY PLAN AMENDMENT		2	06/01/84
51871	LACROSSE - SEP TOPIC III-5.A, PIPE BREAK INSIDE CONTAINMENT			06/15/84
51868	LACROSSE - SEP TOPIC III-3.A, EFFECTS ON HIGH WATER LEVEL ON STRUCTURES			07/01/84
51870	LACROSSE - SEP TOPIC III-4.B, TURBINE MISSILES			07/01/84
54197	LA CROSSE - POST ACCIDENT TECH SPECS			07/01/84
51318	LACROSSE - VENT VALVE OPERABILITY TEST		1	07/15/84
51880	LACROSSE INTEGRATED ASSESSMENT SUPPLEMENT			07/31/84
54621	LA CROSSE - AIRBORNE DOSE CALCULATIONS			08/01/84
51867	LACROSSE - SEP TOPIC III-2, WIND & TORNADO LOADINGS			08/30/84
51869	LACROSSE - SEP TOPIC III-4.1, TORNADO MISSILES			08/30/84
54660	LA CROSSE - SEP TOPIC II-3.C: LOSS OF ULTIMATE HEAT SINK			09/30/84
54666	LA CROSSE - LER RULE REPORTING REQUIREMENTS			09/30/84
54542	LACROSSE - TECHNICAL SPECIFICATIONS COVERED BY GENERIC LETTERS 83-36, 83-37 FOR NUREG-0737			09/30/84
49136	LACROSSE - IN-SERVICE TESTING		1	09/30/84
11233	LACROSSE - FTL CONVERSION		1	09/30/84
51873	LACROSSE - SEP TOPIC III-6, SEISMIC DESIGN CONSIDERATIONS			10/30/84
48681	LA CROSSE - CONTAINMENT LEAK TESTING		2	11/01/84
49895	LA CROSSE - LOCAL LEAK DETECTION CALCULATIONS		2	11/01/84
52514	LA CROSSE - CONFIRMATION OF SEP ACTION ITEMS			01/31/85
<u>ANTICIPATED ACTIONS</u>				<u>INITIATION</u>
54895	LA CROSSE - APPENDIX J: AILOCK EXEMPTION			05/03/84
54076	LACROSSE - ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL TESTING - DIVERSE TRIP FEATURES			11/01/84
53600	LACROSSE - ITEM 1.2 - POST TRIP REVIEW-DATA AND INFORMATION CAPABILITY			11/01/84
53683	LACROSSE - ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS			11/01/84
53766	LACROSSE - ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS			11/01/84
53849	LACROSSE - ITEM 3.2.3 - POST-MAINTENANCE TESTING -CHANGES TO TECH. SPEC. - ALL SR COMPONENTS			11/01/84

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(CONTINUATION)

LA CROSSE

TAC # TAC DESCRIPTION

PLANT SPECIFIC

PRIORITY

CRITICAL DATE

ANTICIPATED ACTIONS (CONTINUATION)

INITIATION

53993 LACROSSE - ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS

11/01/84

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CONDENSED MANAGEMENT REPORT

FACILITY: LASALLE 1

PLANT LOCATION: 11 MI SE OF OTTAWA, ILL
 DOCKET NUMBER: 050-00373
 ARCH/ENGINEER: S&L
 IE INSPECTOR: R. HEISHMAN

LICENSED POWER: 3323 MWT
 DESIGN POWER: 1078 MWE
 NSSS VENDOR: GE

PROJECT MANAGER: A. BOURNIA
 BRANCH CHIEF: A. SCHWENCER
 LIC. ASSISTANT: E. HYLTON

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>MULTI-PLANT</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
51683	LA SALLE 1 NUREG 0737 TECH SPECS			<u>COMPLETE</u> 02/09/84
<u>ACTIVE ACTIONS</u>				
52240	LASALLE 1 - CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPA#C-10)			<u>TARGET</u>
52769	LASALLE 1 - ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION AND PROCEDURES			
52850	LASALLE 1 - ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RTS COMPONENTS			
52931	LASALLE 1 - ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING PROCEDURES/VENDOR RECOMMENDATIONS - RTS COMPONENTS			
53012	LASALLE 1-ITEM 3.1.3 -POST-MAINTENANCE TESTING-CHANGES TO TECH SPECS-RTS COMPONENTS			
48482	LA SALLE 1 - IMPLEMENTATION OF NUREG-0313, REV. 1			03/30/84
51102	LASALLE 1 - INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)			12/21/84
42536	LASALLE 1 - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (SO-CLI-21)		1	03/31/85
<u>TAC # TAC DESCRIPTION MULTI-PLANT PRIORITY CRITICAL DATE</u>				
<u>ACTIVE ACTIONS</u>				
51172	LASALLE 1 - I.D.1 CONTROL ROOM DESIGN REVIEW			<u>TARGET</u>
51252	LASALLE 1 - I.D.2. SAFETY PARAMETER DISPLAY SYSTEM			
49460	LA SALLE 1 - INSTRUMENTS FOR DETECTION ON INADEQUATE CORE COOLING (II.F.2)			
54582	LASALLE 1 - II.D.1 - RV & SV TESTING			
<u>TAC # TAC DESCRIPTION PLANT SPECIFIC PRIORITY CRITICAL DATE</u>				
<u>COMPLETED ACTIONS</u>				
52073	LA SALLE UNIT 1 - REVIEW OF SEISMIC & LOCA LOADS ON FUELS			<u>COMPLETE</u> 11/01/83
52716	LASALLE 1 - REACTOR FEEDWATER INBOARD CHECK VALVE TYPE C TEST			12/06/83
53382	LA SALLE UNIT 1 - TECH SPEC CHANGES FOR SPECIFICATION 30.4 NOT BEING APPLICABLE WITH CONTROL ROD NOT OPERABLE			01/17/84
53466	LA SALLE UNIT 1 - 2.206 REQUEST FROM GOGAL ON INTEGRATED LEAK RATE TESTING OF CONTAINMENT			03/16/84
53303	LA SALLE UNIT 1 - EXIGENT TECH SPEC CHANGE REGARDING DIESEL GENERATORS FAST STARTS			03/19/84
<u>ACTIVE ACTIONS</u>				
54308	LASALLE 1 - CONTROL RAD. CONT. MATERIALS - DEMINIMUS LEVELS			<u>TARGET</u> 04/30/84
51494	LA SALLE 1 - TECH SPEC REQUIREMENTS RESPONSE TIME TESTING OF TURBINE STOP VALVE CLOSURE SCRAM & RECIRC PUMP TRIP			05/30/84
53480	LASALLE UNIT 1 UPGRADE THE TECH SPEC TO MAKE THEM CONSISTENT WITH THE UNIT 2 TECH SPECS			06/01/84
52297	LA SALLE UNIT 1- REVIEW OF EXTENDED BLOWDOWN TEST EVALUATION OF SUPPRESSION POOL TEMPERATURE MEASUREMENTS			09/28/84
<u>ANTICIPATED ACTIONS</u>				
54971	LA SALLE UNIT 1 - MEETING TO DISCUSS FINE MOTION CONTROL ROD DRIVE DEMONSTRATION			<u>INITIATION</u> 05/16/84
53601	LASALLE 1 - ITEM 1.2 - POST TRIP REVIEW-DATA AND INFORMATION CAPABILITY			11/01/84
53684	LASALLE 1 - ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS			11/01/84
53767	LASALLE 1- ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS			11/01/84

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CONDENSED MANAGEMENT REPORT

(CONTINUATION)

LASALLE 1

<u>IAC #</u>	<u>IAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>ANTICIPATED ACTIONS (CONTINUATION)</u>				<u>INITIATION</u>
53850	LASALLE 1 - ITEM 3.2.3 - POST-MAINTENANCE TESTING -CHANGES TO TECH. SPEC. - ALL SR COMPONENTS			11/01/84
53994	LASALLE 1 - ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS			11/01/84
54077	LASALLE 1 - ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL TESTING - DIVERSE TRIP FEATURES			11/01/84

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CONDENSED MANAGEMENT REPORT

FACILITY: LASALLE 2

PLANT LOCATION: 11 MI SE OF OTTAWA, ILL
 DOCKET NUMBER: 050-00374
 ARCH/ENGINEER: S&L
 IE INSPECTOR: R. HEISHMAN

LICENSED POWER: 000 MWT
 DESIGN POWER: 1078 MWE
 NSSS VENDOR: GE

PROJECT MANAGER: A. BOURNIA
 BRANCH CHIEF: A. SCHWENCER
 LIC. ASSISTANT: E. HYLTON

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>MULTI-PLANT</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>ACTIVE ACTIONS</u>				
42537	LASALLE 2 - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21)		1	<u>TARGET</u> 03/31/85
<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>TMI ACTIONS</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>ACTIVE ACTIONS</u>				
54583	LASALLE 2 - II.D.1 - RV & SV TESTING			<u>TARGET</u>
<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
53467	LASALLE UNIT 2 2.206 REQUEST FROM GOGOL ON INTEGRATED LEAK RATE TESTING OF CONTAINMENT			<u>COMPLETE</u> 03/16/84
54178	LA SALLE UNIT 2 - RESPOND TO APPEAL LETTER ON FIRE PROTECTION			03/22/84
54248	LA SALLE 2 - REVIEW OF FIRE PROTECTION SUBMITTALS			03/22/84
54184	LA SALLE 2 - TECH SPEC AUDIT			03/27/84
54309	LASALLE 2 - CONTROL RAD. CONT. MATERIALS - DEMINIMUS LEVELS			04/13/84
<u>ANTICIPATED ACTIONS</u>				
53602	LASALLE 2 - ITEM 1.2 - POST TRIP REVIEW-DATA AND INFORMATION CAPABILITY			<u>INITIATION</u> 11/01/84
53685	LASALLE 2 - ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS			11/01/84
53768	LASALLE 2- ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS			11/01/84
53851	LASALLE 2 - ITEM 3.2.3 - POST-MAINTENANCE TESTING -CHANGES TO TECH. SPEC. - ALL SR COMPONENTS			11/01/84
53995	LASALLE 2 - ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS			11/01/84
54078	LASALLE 2 - ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL TESTING - DIVERSE TRIP FEATURES			11/01/84

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CONDENSED MANAGEMENT REPORT

FACILITY: MAINE YANKEE

PLANT LOCATION: 10 MI N OF BATH, ME
 DOCKET NUMBER: 050-00309
 ARCH/ENGINEER: S&W
 IE INSPECTOR: R. SWETLAND

LICENSED POWER: 2630 MW
 DESIGN POWER: 0825 MWE
 NSSS VENDOR: COMB

PROJECT MANAGER: K. HEITNER
 BRANCH CHIEF: J. MILLER
 LIC. ASSISTANT: P. KREUTZER

TAC #	TAC DESCRIPTION	MULTI-PLANT	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				
49613	MAINE YANKEE - PLANS FOR PREVENTING EXCEEDING PTS SCREENING CRITERION		2	<u>COMPLETE</u> 12/02/83
43328	MAINE YANKEE - CONTROL OF HEAVY LOADS AT NUCLEAR POWER PLANT		3	12/30/83
12317	MAINE YANKEE - HELB AND CONSEQUENTIAL SYSTEM FAILURE MAINE YANKEE		2	01/31/84
08909	MAINE YANKEE - FUEL HANDLING ACCIDENT INSIDE CONTAINMENT		2	03/14/84
10032	MAINE YANKEE - POTENTIAL EQUIPMENT FAILURES ASSOCIATED WITH GRID VOLTAGE		2	03/27/84
<u>ACTIVE ACTIONS</u>				
52241	MAINE YANKEE - CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPA#C-10)			
52770	MAINE YANKEE ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION AND PROCEDURES			
52851	MAINE YANKEE - ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RTS COMPONENTS			
52932	MAINE YANKEE - ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING PROCEDURES/VENDOR RECOMMENDATIONS - RTS COMPONENTS			
53013	MAINE YANKEE-ITEM 3.1.3 -POST-MAINTENANCE TESTING-CHNGS TO TECH SPECS-RTS COMPONENTS			
53077	MAINE YANKEE - REACTOR TRIP SYSTEM RELIABILITY - VENDOR RELATED MODIFICATIONS			
53130	MAINE YANKEE - ITEMS 4.2.1 & 4.2.2 - PREVENTATIVE MAINT PROGRAM FOR REACTOR TRIP BREAKERS - MAINTENANCE & TRENDING			
51103	MAINE YANKEE - INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)			01/15/84
51756	MAINE YANKEE - THERMAL SHIELD FOLLOW UP ANALYSIS		2	06/01/84
08898	MAINE YANKEE - REVIEW OF ASYMMETRIC LOCA LOADS SUBMITTAL		3	06/01/84
08904	MAINE YANKEE - HYDRAULIC SNUBBERS		1	06/01/84
42490	MAINE YANKEE - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21)		2	06/30/84
42893	MAINE YANKEE - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11			07/01/84
49673	MAINE YANKEE - REACTOR COOLANT PUMP TRIP		1	08/30/84
43653	MAINE YANKEE - SEISMIC QUALIFICATION OF THE AUXILIARY FEEDWATER SYSTEM		3	09/30/84
07752	MAINE YANKEE - APPENDIX I TECH SPEC IMPLEMENTATION REVIEW			
<u>TMI ACTIONS</u>				
<u>COMPLETED ACTIONS</u>				
45263	MAINE YANKEE - NUREG-0737 II.K.3.1, AUTO PORV ISOLATION		1	<u>COMPLETE</u> 10/04/83
45309	MAINE YANKEE - NUREG-0737 II.K.3.2, REPORT ON PORV FAILURES		1	10/04/83
47805	MAINE YANKEE - NUREG 0737 ITEM II K2.17 POTENTIAL FOR VOIDING IN RCS		1	10/11/83
<u>ACTIVE ACTIONS</u>				
51173	MAINE YANKEE - I.D.1 CONTROL ROOM DESIGN REVIEW			
51253	MAINE YANKEE - I.D.2. SAFETY PARAMETER DISPLAY SYSTEM		1	12/30/83
44664	MAINE YANKEE - NUREG-0737 II.E.1.1, AFW SYSTEM EVALUATION		1	06/30/84
45837	MAINE YANKEE - NUREG-0737 II.K.3.30, SB LOCA OUTLINE		1	06/30/84
44451	MAINE YANKEE - NUREG-0737 II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS		1	09/30/84
45956	MAINE YANKEE - NUREG-0737 III.A.1.2, TECHNICAL SUPPORT CENTER		1	09/30/84
46027	MAINE YANKEE - NUREG-0737 III.A.1.2, OPERATIONAL SUPPORT CENTER		1	09/30/84
46099	MAINE YANKEE - NUREG-0737 III.A.1.2, EMERGENCY OPERATIONS FACILITY		1	09/30/84
48176	MAINE YANKEE -NUREG 0737 II.K.3.31 COMPLIANCE WITH 10 CFR 50.46		1	09/30/84
46891	MAINE YANKEE - II K2.13 THERMAL MECHANICAL REPORT		1	09/30/84
44592	MAINE YANKEE - RV AND SV TESTING		1	10/31/84

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CONDENSED MANAGEMENT REPORT

(CONTINUATION)

MAINE YANKEE

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>TMI ACTIONS</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>ACTIVE ACTIONS</u> (CONTINUATION)				
44243	MAINE YANKEE	- NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES	1	TARGET 11/30/84
44313	MAINE YANKEE	- NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES	1	11/30/84
46315	MAINE YANKEE	- NUREG-0737 III.A.2.2, METEOROLOGICAL DATA UPGRADE	1	12/01/84
45145	MAINE YANKEE	- NUREG-0737 II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION	1	12/30/84

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
52095	MAINE YANKEE	- FINANCIAL QUALS - SAFETY REVIEW		COMPLETE 10/04/83
49285	MAINE YANKEE	- FIRE EXEMPTION REQUESTS FOR SAFE SHUTDOWN		11/01/83
52386	MAINE YANKEE	- MFW TRIP DESIGN DEFICIENCY		11/14/83
52174	MAINE YANKEE	- MODIFICATIONS ON OPERABILITY SPECS		11/14/83
52301	MAINE YANKEE	- ISI TECH SPECS		12/21/83
52488	MAINE YANKEE	- SECOND EXEMPTION SHIFT MANNING RULE		01/12/84
53333	MAINE YANKEE	- APPEAL ON LIMIT OVERTIME		01/26/84
48674	MAINE YANKEE	- REVIEW OF FEEDWATER TRIP SYSTEM DESIGN		02/14/84
48962	MAINE YANKEE	- TECH SPEC FOR VENT & PURGE		02/14/84
49832	MAINE YANKEE	- YAEC-1 TECH SPECS		03/21/84
52300	MAINE YANKEE	- BOARD NOTIFICATION ON SECY PAPER		03/22/84

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>ACTIVE ACTIONS</u>				
51703	MAINE YANKEE	0 DEGRADED GRID VOLTAGE PROCEDURES		TARGET
52011	MAINE YANKEE	- SECOND ISI INTERVAL		12/30/83
51406	MAINE YANKEE	- SCHEDULE FOR NUREG-0737 SUPPLEMENT 1		12/31/83
48999	MAINE YANKEE	- ESFAS & RP5 SENSOR TECH SPEC		02/28/84
49711	MAINE YANKEE	- SECOND IST INTERVAL		02/28/84
52173	MAINE YANKEE	- SEISMIC AND OTHER QUALS ON 8IN. VENT/PURGE VALVE		03/01/84
48632	MAINE YANKEE	- APP J TECH SPEC CHANGE		03/30/84
53546	MAINE YANKEE	- TECH SPEC FOR AFW INITIATION		04/30/84
53460	MAINE YANKEE	- STEAM GENERATOR WATER LEVEL		05/15/84
53547	MAINE YANKEE	- CYCLE 8 RELOAD		06/01/84
48612	MAINE YANKEE	- SPENT FUEL POOL HEARING AND LICENSE AMENDMENT		06/30/84
54431	MAINE YANKEE	- EMERGENCY EXERCISE EXEMPTION REQUEST		06/30/84
53475	MAINE YANKEE	- APS VOLTAGE STUDY		06/30/84
53407	MAINE YANKEE	- REVIEW OF 10CFR 61		06/30/84
53449	MAINE YANKEE	- REVISED DEFINITION OF CONTAINMENT INTEGRITY		06/30/84
53450	MAINE YANKEE	- CONTAINMENT LEAK TESTING		06/30/84
49475	MAINE YANKEE	- BRIEFINGS ON SEISMIC REVIEW PLAN		06/31/84
51389	MAINE YANKEE	- ADMINISTRATIVE TECH. SPEC. REVIEW		07/30/84
48582	MAINE YANKEE	- FIRE PROTECTION EXEMPTION FOR		09/01/84
53513	MAINE YANKEE	- TECH SPECS FOR SAFE SHUTDOWN		09/30/84
54543	MAINE YANKEE	- TECHNICAL SPECIFICATIONS COVERED BY GENERIC LETTERS 83-36, 83-37 FOR NUREG-0737		09/30/84
54397	MAINE YANKEE	- RCS VENTS TECHNICAL SPECIFICATIONS NUREG 073 ITEM II.B.1		09/30/84
53408	MAINE YANKEE	- FIRE PROTECTION - EXTRA EXEMPTION REQUESTS		01/01/85

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>ANTICIPATED ACTIONS</u>				
54958	MAINE YANKEE	- THERMAL SHIELD INSPECTION AND REPAIR		INITIATION 05/07/84
54959	MAINE YANKEE	- LTOP TECHNICAL SPECIFICATION CHANGES		05/07/84
54960	MAINE YANKEE	- PROPOSED CHANGE 107		05/07/84
53603	MAINE YANKEE	- ITEM 1.2 - POST TRIP REVIEW-DATA AND INFORMATION CAPABILITY		11/01/84

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CONDENSED MANAGEMENT REPORT

(CONTINUATION)

MAINE YANKEE

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>ANTICIPATED ACTIONS (CONTINUATION)</u>				<u>INITIATION</u>
53686	MAINE YANKEE - ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS			11/01/84
53769	MAINE YANKEE- ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS			11/01/84
53852	MAINE YANKEE - ITEM 3.2.3 - POST-MAINTENANCE TESTING -CHANGES TO TECH. SPEC. - ALL SR COMPONENTS			11/01/84
53917	-ITEMS 4.2.3 & 4.2.4-PREVENTATIVE MAINT. PROGRAM FOR REACTOR TRIP BREAKERS-LIFE TESTING & REPLACEMENT			11/01/84
53996	MAINE YANKEE - ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS			11/01/84
54079	MAINE YANKEE - ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL TESTING - DIVERSE TRIP FEATURES			11/01/84

DATA AS OF - 04/30/84

CONDENSED MANAGEMENT REPORT

FACILITY: MCGUIRE 1

PLANT LOCATION: 17 MI N OF CHARLOTTE, NC
 DOCKET NUMBER: 050-00369
 ARCH/ENGINEER: DUKE
 IE INSPECTOR: W. ORDERS

LICENSED POWER: 3411 MWT
 DESIGN POWER: 1180 MWE
 NSSS VENDOR: WEST

PROJECT MANAGER: R. BIRKEL
 BRANCH CHIEF: E. ADENSAM
 LIC. ASSISTANT: M. DUNCAN

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>MULTI-PLANT</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>ACTIVE ACTIONS</u>				
52933	MCGUIRE 1 -- ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING PROCEDURES/VENDOR RECOMMENDATIONS - RTS COMPONENTS			<u>TARGET</u>
53014	MCGUIRE 1-ITEM 3.1.3 -POST-MAINTENANCE TESTING-CHANGES TO TECH SPECS-RTS COMPONENTS			
53078	MCGUIRE 1 - REACTOR TRIP SYSTEM RELIABILITY - VENDOR RELATED MODIFICATIONS			
53131	MCGUIRE 1 - ITEMS 4.2.1 & 4.2.2 - PREVENTATIVE MAINTENANCE PROGRAM FOR REACTOR TRIP BREAKERS - MAINTENANCE & TRENDING			
53179	MCGUIRE 1 - ITEM 4.3 - AUTOAMTIC ACTUATION OF SHUNT TRIP ATTACHMENT FOR W AND B & W PLANTS			
51104	MCGUIRE 1 - INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)			
52242	MCGUIRE 1 - CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPA#C-10)			
52771	MCGUIRE 1 - ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION AND PROCEDURES			
52852	MCGUIRE 1 - ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RTS COMPONENTS			
47127	MCGUIRE 1 - CONTROL OF HEAVY LOADS AT NUCLEAR POWER PLANTS (NUREG 0612)		2	01/02/84
49653	MCGUIRE 1 - REACTOR COOLANT PUMP TRIP			07/01/84
<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>IMI ACTIONS</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>ACTIVE ACTIONS</u>				
51174	MCGUIRE 1 - I.D.1 CONTROL ROOM DESIGN REVIEW			<u>TARGET</u>
51254	MCGUIRE 1 - I.D.2. SAFETY PARAMETER DISPLAY SYSTEM			03/01/84
45146	MCGUIRE - NUREG-0737 II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION		1	07/01/84
46316	MCGUIRE - NUREG-0737 III.A.2.2, METEOROLOGICAL DATA UPGRADE		1	07/01/84
<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
49804	MCGUIRE UNIT #1 - EVALUATE REQUEST (2/18/83) TO TRANSFER SPENT FUEL BETWEEN MCGUIRE UNITS 1 AND 2			<u>COMPLETE</u> 10/19/83
52111	MCGUIRE UNIT 1 - EVAL TECH SPEC CHANGES RELATING TO D/G & TURBINE VALVE SURVEILLANCE TEST'G (REF: DUKE LTR 8/1/83)			10/26/83
52390	MCGUIRE UNIT #1: EVALUTE REQUEST TO INCREASE CONTAINMENT LOWER CMPTMNT TMP TO 125 DEG F(T.S.3.6.1.5) REF: DUKE LTR)			12/02/83
53214	MCGUIRE UNIT #1: EVAL PROPOSED T.S. CHANGE RELATING TO MINIMUM RCS FLOW REQ'T AT 100% 3 REF: DUKE LTR NOV 18, 1983			02/04/84
52328	MCGUIRE UNIT 1 - EVAL T.S. CHANGE RELATING TO ADMIN & TYPO ERROR & DELETION OF 1 SNUBBER (TBL3.7-4B) REF: DUKE 8/2/83			02/28/84
52198	MCGUIRE UNIT 1: EVAL CHANGE IN STEAM LINE RESPONSE TIME (TS TBL 3.3.5) AND INOP S/G LEVEL INSTRU (TS TBL 3.3-1)			02/28/84
52164	MCGUIRE UNIT 1: EVAL PROPOSAL TO ALLOW CONTROL VENTILATION SYS TO BE INOP IN MODES 5/6;TS 3.7.6(REF: DUKE 8/2/83 LTR)			03/19/84
53257	MCGUIRE UNIT #1: EVAL REVISION TO EMERGENCY LIGHTING FOR FIRE PROTECTION; REF: DUKE LTR 11/18/83			04/16/84
53319	MCGUIRE UNIT 1 - EVALUATE MCGUIRE 1/CYCLE 2 OPTIMIZED FUEL ASSEMBLY RELOADS REF: DUKE LETTER 12/12/83			04/20/84

ACTIVE ACTIONSTARGET

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CONDENSED MANAGEMENT REPORT

(CONTINUATION)

MCGUIRE 1

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>ACTIVE ACTIONS</u> (CONTINUATION)				<u>TARGET</u>
52062	MCGUIRE UNIT #1 10CFR PART 21 INTERPRETATION REQUEST BY DUKE POWER CO (REF: LTR 7/20/83)			03/25/84
53503	MCGUIRE UNIT 1 - EVALUATE REVISED IMPLEMENTATION DATE FOR LICENSE CONDITION 2.C.(11)F.(3)-ITEM II.F.2; REF DUKE LTR			05/15/84
54882	MCGUIRE UNIT #1 - RELIEF REQUEST ASME CODE SECTION XI, SNUBBER TESTING; REF: DUKE LETTER 4/12/84			06/30/84
53531	MCGUIRE UNIT #1 EVALUATE MCGUIRE STATION SPENT FUEL POOLS RERACKING PROGRAM			07/01/84
53255	MCGUIRE UNIT #1: EVAL PROPOSED T.S. SURVEIL CHG RELATING TO DIESEL GENERATOR FUEL; T.S.4.B.1.1.2; REF: DUKE LTR 11/83			07/01/84
51610	MCGUIRE UNIT #1: EVALUATE LICENSEE JUSTIFICATION FOR LONGTERM OPERATION WITH RCS THERMAL SLEEVES REMOVED			07/01/84
52178	MCGUIRE UNIT 1 - EVALUATE PROPOSED TECH. SPEC. SURV CHANGE RELATING TO I.C. INLET DOORS (T.S. 4.6.5.3.1)			07/01/84
52160	MCGUIRE UNIT 1: EVAL PROPOSAL ALLOWING UNIT OP LESS THAN OR EQ TO 46% RATED THERMAL POWER WITH UHI INOP TS 3.5.1.2			07/01/84
52327	MCGUIRE UNIT 1 - EVAL T.S. CHANGES RELATED TO CONTAINMENT INTEGRITY (4.6.1.1.A) BYPASS LEAKAGE (TLB.3.6-1) & DISTRIB			07/01/84
52329	MCGUIRE UNIT 1 - EVAL T.S. CHANGE RELATING TO POSITION OF INACCESSIBLE SPRINKLER SYS VALV (4.7.10.2) REF: DUKE 8/2/83			09/30/84
54398	MCGUIRE 1- RCS VENTS TECHNICAL SPECIFICATIONS NUREG 073 ITEM II.B.1			09/30/84
54544	MCGUIRE 1 - TECHNICAL SPECIFICATIONS COVERED BY GENERIC LETTERS 83-36, 83-37 FOR NUREG-0737			
<u>ANTICIPATED ACTIONS</u>				<u>INITIATION</u>
53604	MCGUIRE 1 - ITEM 1.2 - POST TRIP REVIEW-DATA AND INFORMATION CAPABILITY			11/01/84
53687	MCGUIRE 1 - ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS			11/01/84
53770	MCGUIRE 1- ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS			11/01/84
53853	MCGUIRE 1 - ITEM 3.2.3 - POST-MAINTENANCE TESTING -CHANGES TO TECH. SPEC. - ALL SR COMPONENTS			11/01/84
53918	-ITEMS 4.2.3 & 4.2.4-PREVENTATIVE MAINT. PROGRAM FOR REACTOR TRIP BREAKERS-LIFE TESTING & REPLACEMENT			11/01/84
53997	MCGUIRE 1 - ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS			11/01/84
54080	MCGUIRE 1 - ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL TESTING - DIVERSE TRIP FEATURES			11/01/84

DATA AS OF - 04/30/84

CONDENSED MANAGEMENT REPORT

FACILITY: MCGUIRE 2

PLANT LOCATION: 17 MI N OF CHARLOTTE, NC
 DOCKET NUMBER: 050-00370
 ARCH/ENGINEER: DUKE
 IE INSPECTOR: W. COTTLE

LICENSED POWER: 3411 MWT
 DESIGN POWER: 1180 MWE
 NSSS VENDOR: WEST

PROJECT MANAGER: R. BIRKEL
 BRANCH CHIEF: E. ADENSAM
 LIC. ASSISTANT: M. DUNCAN

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>MULTI-PLANT</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>ACTIVE ACTIONS</u>				
52243	MCGUIRE 2 - CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPA#C-10)			<u>TARGET</u>
52772	MCGUIRE 2 - ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION AND PROCEDURES			
52853	MCGUIRE 2 - ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RTS COMPONENTS			
52934	MCGUIRE 2 - ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING PROCEDURES/VENDOR RECOMMENDATIONS - RTS COMPONENTS			
53015	MCGUIRE 2-ITEM 3.1.3 -POST-MAINTENANCE TESTING-CHANGES TO TECH SPECS-RTS COMPONENTS			
53079	MCGUIRE 2 - REACTOR TRIP SYSTEM RELIABILITY - VENDOR RELATED MODIFICATIONS			
53180	MCGUIRE 2 - ITEM 4.3 - AUTOMATIC ACTUATION OF SHUNT TRIP ATTACHMENT FOR W AND B & W PLANTS			
53132	MCGUIRE 2 - ITEMS 4.2.1 & 4.2.2 - PREVENTATIVE MAINTENANCE PROGRAM FOR REACTOR TRIP BREAKERS - MAINTENANCE & TRENDING			01/00/84
51105	MCGUIRE 2 - INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)			01/01/84
<u>TAC # TAC DESCRIPTION MULTI-PLANT PRIORITY CRITICAL DATE</u>				
<u>ACTIVE ACTIONS</u>				
54601	MCGUIRE 2 - II.D.1 - RV & SV TESTING			<u>TARGET</u>
51175	MCGUIRE 2 - I.D.1 CONTROL ROOM DESIGN REVIEW			03/01/84
54985	MCGUIRE 2, II.K.2.13, THERMAL MECHANICAL REPORT			06/30/84
51255	MCGUIRE 2 - I.D.2. SAFETY PARAMETER DISPLAY SYSTEM			07/01/84
<u>TAC # TAC DESCRIPTION PLANT SPECIFIC PRIORITY CRITICAL DATE</u>				
<u>COMPLETED ACTIONS</u>				
52112	MCGUIRE UNIT 2 - EVAL TECH SPEC CHANGES RELATING TO D/G & TURBINE VALVE SURVEILLANCE TEST'G (REF: DUKE LTR 8/1/83)			<u>COMPLETE</u> 10/26/83
52391	MCGUIRE UNIT 2: EVAL REQ TO INCREASE CONTAINMENT LOWER COMPARTMENT TMP TO 125 DEG F(T.S.3.6.1.5) REF:DUKE LTR			12/02/83
53289	MCGUIRE UNIT #2 - EXEMPTION TO MULTIPLE CRISIS MANAGEMENT CENTER STAFF DRILLS (REF: TAC# 51035)			01/06/84
53215	MCGUIRE UNIT #2: EVAL PROPOSED T.S. CHANGE RELATING TO MINIMUM RCS FLOW REQTS AT 100% REF: DUKE LTR 11/18/83			02/04/84
52331	MCGUIRE UNIT 2 - EVAL T.S. CHANGE RELATING TO ADMIN & TYPO ERROR & DELETION OF 1 SNUBBER (TBL3.7-4B) REF: DUKE 8/2/83			02/28/84
52199	MCGUIRE UNIT 2: EVAL CHANGE IN STEAM LINE RESPONSE TIME (TS TBL 3.3.5) AND INOP S/G LEVEL INSTRU (TS TBL 3.3-1)			02/28/84
52163	MCGUIRE UNIT 2: EVAL PROPOSAL TO ALLOW CONTROL VENTILATION SYS TO BE INOP IN MODES 5/6;TS 3.7.6(REF: DUKE 8/2/83 LTR)			03/19/84
49981	MCGUIRE UNIT #2: EVALUATE PROPOSED T.S. CHANGE 3/4.5.2 CONCERNING MAXIMUM CENTRIFUGAL CHARGING PUMP FLOW RATE			04/13/84
53258	MCGUIRE UNIT #2: EVALUATE REVISION TO EMERGENCY LIGHTING FOR FIRE PROTECTION; REF: DUKE LTR - NOV 18/1983			04/16/84
53320	MCGUIRE UNIT #2 - EVALUATE MCGUIRE 1/CYCLE 2 OPTIMIZED FUEL ASSEMBLY RELOAD; DUKE LTR. 12/12/83			04/20/84
<u>ACTIVE ACTIONS</u>				
52063	MCGUIRE UNIT #2: 10CFR21 INTERPRETATION REQUEST BY DUKE POWER CO. (REF: LTR 7/20/83)			<u>TARGET</u>
54883	MCGUIRE UNIT #1 - RELIEF REQUEST ASME CODE SECTION XI, SNUBBER TESTING; REF: DUKE LETTER 4/12/84			05/15/84

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CONDENSED MANAGEMENT REPORT

(CONTINUATION)

MCGUIRE 2

TAC #	TAC DESCRIPTION	PLANT SPECIFIC	PRIORITY	CRITICAL DATE
<u>ACTIVE ACTIONS</u> (CONTINUATION)				<u>TARGET</u>
53532	MCGUIRE UNIT #2: EVALUATE MCGUIRE STATION SPENT FUEL POOLS RERACKING PROGRAM			06/30/84
53256	MCGUIRE UNIT #2: EVAL PROPOSED T.S. SURVEIL CHG RELATNG TO DIESEL GENERATOR FUEL:T.S.4.B.1.1.2: REF: 11/18/83			07/01/84
53287	MCGUIRE UNIT #2 - NUREG-0737, III.A.2.2, METEOROLOGICAL DATA UPGRADE (REF: TAC# 46316)			07/01/84
53288	MCGUIRE UNIT #2 - NUREG-0737, II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION (REF: TAC# 45146)			07/01/84
53290	MCGUIRE UNIT #2 - REACTOR COOLANT PUMP TRIP (REF: TAC# 49653)			07/01/84
52159	MCGUIRE UNIT 2: EVAL PROPOSAL ALLOWING UNIT OP LESS THAN OR EQ TO 46% RATED THERMAL POWER WITH UHI INOP TS 5.3.5.2			07/01/84
51609	MCGUIRE UNIT #2: EVALUATE LICENSEE JUSTIFICATION FOR LONGTERM OPERATION WITH RCS THERMAL SLEEVES REMOVED			07/01/84
52179	MCGUIRE UNIT 2 - EVALUATE PROPOSED TECH. SPEC. SURV CHANGES RELATING TO I.C INLET DOORS (T.S.4.6.5.3.1)			07/01/84
52332	MCGUIRE UNIT 2 - EVAL T.S. CHANGE RELATING TO POSITION OF INACCESSIBLE SPRINKLER SYS. VALVES(4.7.10.2) REF: DUKE LTR			07/01/84
52330	MCGUIRE UNIT 2 - EVAL T.S. CHANGES RELATED TO CONTAINMENT INTEGRITY (4.6.1.1.A); BYPASS LEAKAGE(TBL 3.4-1) & DISTRIB			07/01/84
<u>ANTICIPATED ACTIONS</u>				<u>INITIATION</u>
53605	MCGUIRE 2 - ITEM 1.2 - POST TRIP REVIEW-DATA AND INFORMATION CAPABILITY			11/01/84
53688	MCGUIRE 2 - ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS			11/01/84
53771	MCGUIRE 2- ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS			11/01/84
53854	MCGUIRE 2 - ITEM 3.2.3 - POST-MAINTENANCE TESTING -CHANGES TO TECH. SPEC. - ALL SR COMPONENTS			11/01/84
53919	-ITEMS 4.2.3 & 4.3.4-PREVENTATIVE MAINT. PROGRAM FOR REACTOR TRIP BREAKERS-LIFE TESTING & REPLACEMENT			11/01/84
53998	MCGUIRE 2 - ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS			11/01/84
54081	MCGUIRE 2 - ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL TESTING - DIVERSE TRIP FEATURES			11/01/84

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CONDENSED MANAGEMENT REPORT

FACILITY: MILLSTONE 1

PLANT LOCATION: 5 MI SW OF NEW LONDON, CONN
 DOCKET NUMBER: 050-00245
 ARCH/ENGINEER: EBASCO
 IE INSPECTOR: T. SHEDLOSKY

LICENSED POWER: 2011 MWT
 DESIGN POWER: 0660 MWE
 NSSS VENDOR: GE

PROJECT MANAGER: J. SHEA
 BRANCH CHIEF: D. CRUTCHFIELD
 LIC. ASSISTANT: H. SMITH

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>MULTI-PLANT</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
08060	MILLSTONE 1 - CONTROL OF HEAVY LOADS OVER SPENT FUEL POOL (NUREG 0612)		2	<u>COMPLETE</u> 10/04/83
51684	MILLSTONE 1 NUREG 0737 TECH SPECS			03/19/84
<u>ACTIVE ACTIONS</u>				
52244	MILLSTONE 1 - CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPA#C-10)			<u>TARGET</u>
52773	MILLSTONE 1 - ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION AND PROCEDURES			
52854	MILLSTONE 1 - ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RTS COMPONENTS			
52935	MILLSTONE 1 - ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING PROCEDURES/VENDOR RECOMMENDATIONS - RTS COMPONENTS			
53016	MILLSTONE 1 - ITEM 3.1.3 - POST-MAINTENANCE TESTING-CHANGES TO TECH SPECS-RTS COMPONENTS		2	04/14/84
43050	MILLSTONE 1 - TECHNICAL SPECIFICATIONS DEFINING OPERABILITY FOR SAFETY SYSTEMS		1	06/31/84
10910	MILLSTONE 1 - RPS POWER SUPPLY		1	07/12/84
07941	MILLSTONE 1 - MARK I CONTAINMENT LONG TERM PROGRAM IMPLEMENTATION		1	07/30/84
42523	MILLSTONE 1 - ENVIRONMENTAL QUALIFICATIONS OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21)		1	07/31/84
46665	MILLSTONE 1 - IMPLEMENTATION OF NUREG-0313, REV. 1		2	07/31/84
5106	MILLSTONE 1 - INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)			12/12/84
72612	MILLSTONE 1 - LONG TERM REVIEW CONTAINMENT PURGE AND VENT (B-24)		1	12/15/84
10137	MILLSTONE 1 - CONTAINMENT LEAK TESTING - APPENDIX J (GENERIC)		2	12/30/84
42913	MILLSTONE 1 - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11		2	12/31/84
<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>TMI ACTIONS</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
48178	MILLSTONE 1 - NUREG 0737 II.K.3.31 COMPLIANCE WITH 10 CFR 50.46		1	<u>COMPLETE</u> 12/19/83
45839	MILLSTONE 1 - NUREG-0737 II.K.3.30, SB LOCA OUTLINE		1	12/19/83
<u>ACTIVE ACTIONS</u>				
51176	MILLSTONE 1 - I.D.1 CONTROL ROOM DESIGN REVIEW			<u>TARGET</u>
51256	MILLSTONE 1 - I.D.2. SAFETY PARAMETER DISPLAY SYSTEM			
44863	MILLSTONE 1 - NUREG-0737 II.E.4.2, CONTAINMENT ISOLATION DEPENDABILITY		1	10/30/82
46533	MILLSTONE 1 - NUREG-0737, II.K.3.16, CHALLENGES AND FAILURES OF RELIEF VALVES		1	01/28/83
46317	MILLSTONE 1 - NUREG-0737 III.A.2.2, METEOROLOGICAL DATA UPGRADE		1	06/01/83
45147	MILLSTONE 1 - NUREG-0737 II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION		1	06/30/83
48280	MILLSTONE 1 - NUREG 0737 II.K.3.28, VERIFY QUALIFICATION OF ACCUMULATORS ON ADS VALVES		1	09/30/83
44315	MILLSTONE 1 - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES		1	11/30/83
47754	MILLSTONE 1 - NUREG 0737 ITEM II F1.6 CONTAINMENT HYDROGEN MONITOR		1	05/14/84
46541	MILLSTONE 1 - NUREG-0737, II.K.3.45, MANUAL DEPRESSURIZATION		1	06/30/84
44453	MILLSTONE 1 - NUREG-0737 II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS		1	06/30/84
45958	MILLSTONE 1 - NUREG-0737 III.A.1.2, TECHNICAL SUPPORT CENTER		1	09/30/84
46029	MILLSTONE 1 - NUREG-0737 III.A.1.2, OPERATIONAL SUPPORT CENTER		1	09/30/84
46101	MILLSTONE 1 - NUREG-0737 III.A.1.2, EMERGENCY OPERATIONS FACILITY		1	09/30/84
44245	MILLSTONE 1 - NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES		1	11/30/84

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CONDENSED MANAGEMENT REPORT

(CONTINUATION)

MILLSTONE 1

TAC #	TAC DESCRIPTION	PLANT SPECIFIC	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				
49376	MILLSTONE UNIT 1 - INTEGRATED STRUCTURAL ANALYSIS			<u>COMPLETE</u> 10/03/83
49377	MILLSTONE UNIT 1 - MISSILES			10/03/83
49379	MILLSTONE UNIT 1 - SEISMIC DESIGN			10/03/83
49380	MILLSTONE UNIT 1 - MOV INTERLOCKS			10/03/83
49381	MILLSTONE UNIT 1 - LEAK DETECTION			10/03/83
49382	MILLSTONE UNIT 1 - WATER CHEMISTRY			10/03/83
52044	MILLSTONE - 1 CLARIFY T.S. RELATED TO FIRE PROTECTION DETECTION			11/02/83
51586	MILLSTONE 1 - REVIEW OF MODIFIED AMENDED SECURITY PLANS PER EISENHUT MEMORANDUM MAY 16, 1983			11/10/83
51987	MILLSTONE 1 - REQUIRED ATWS ACTIONS G.L.83-28			11/25/83
51978	MILLSTONE 1 - INTEGRATED LEAK RATE TEST IN LESS THAN 24 HOURS			12/19/83
49247	MILLSTONE 1 - SALP-1983			12/19/83
52529	MILLSTONE 1 - REV. TO ADM. TECH. SPEC. SECT. 6 LIC. OPERATOR STAFFING			02/16/84
53299	MILLSTONE 1 - PREPARE RESPONSE TO NNECO 11/18/83 STA TRAINING			02/28/84
52022	MILLSTONE 1 - T.S. CHANGES - ADMINISTRATIVE PROVIDE SER			03/19/84
49457	MILLSTONE 1 - AUDITS OF EMERGENCY & SECURITY PLANS RE: GEN LETTERS 82-17 AND 82-23			03/19/84
<u>ACTIVE ACTIONS</u>				
49786	MILLSTONE 1 IPSAR SUPPLEMENT			
51986	MILLSTONE 1 - REVIEW & PREPARE SER OF NNECO QUAL ASS. REPORT SUBMITTED BY NNECO LTR 6/9/83			
51961	MILLSTONE 1 - ALTERNATIVE APPENDIX I TECH. SPEC. REVIEW			
53349	MILLSTONE 1 - SALP 1984			
53294	MILLSTONE 1 - INTEGRATED SAFETY ASSESSMENT PROGRAM			12/30/83
49384	MILLSTONE 1 - FAULT TRANSFERS			12/30/83
49385	MILLSTONE UNIT 1 - ELECTRICAL ISOLATION			03/06/84
49780	MILLSTONE UNIT 1 - FLOODING (NON-STRUCTURAL)			03/31/84
51559	MILLSTONE 1 - REVIEW NEED FOR SCRAM DISCHARGE VOLUME DIVERSE LEVEL INSTRUMENTATION			04/13/84
54189	MILLSTONE 1 - NEW STEAM TUNNEL VENTILATION SYSTEM			04/21/84
49488	MILLSTONE 1 - REVIEW FINAL ENVIRONMENTAL STATEMENT UPDATE			04/27/84
54657	MILLSTONE-1 PROVIDE COMMENTS ON ATTACHED NNECO SUBMITTAL DATED 3/27/84 (ISO CONDENSER)			04/30/84
11210	MILLSTONE 1 - INSERVICE TESTING(IST)		1	05/01/84
53445	MILLSTONE 1 - RESPONSE TO GEN LTR 81-34 RE MPA 8-65			05/16/84
53446	MILLSTONE 1 - RECIRC LINE PIPE CRACK - CLAMPS			06/01/84
54491	MILLSTONE - 1: REVIEW RESPONSE TO GE SIL-402 DRYWELL VENT HEADER DISTRIBUTER IN TORUS CRACKS			06/01/84
49388	MILLSTONE UNIT 1 - BATTERY INSTRUMENTATION AND TS LIMITS		3	06/12/84
48229	MILLSTONE 1 - SUBCOOLED POST DBLOCA RADIOLYSIS			06/30/84
51698	MILLSTONE - 1 T.S. CHANGLS TO MEET FINAL RULE ON ENVIR QUAL 10 CFR 50.49			07/01/84
54656	MILLSTONE-1 - PREPARE SER FOR FUEL VUNDLE "BP8DRB 300"			07/13/84
54658	MILLSTONE-1 - RELOAD 9 (PREPARE SER)			07/30/84
51581	MILLSTONE-1. IREP - REPORT EVALUATION - NNECO FEEDBACK BY 7-1-83			07/30/84
51305	MILLSTONE 1 - PREPARE SER CONSIDERING LICNESEE COMMENTS & IREP-REPORT. NUREG/CR 3085			08/10/84
49290	MILLSTONE 1 - SAFETY FACTOR FOR PENETRATION X-10A			08/10/84
07426	MILLSTONE 1 - CONTAINMENT LEAK RATE CONTINUOUS MONITORING		2	09/29/84
48664	MILLSTONE - 1 EXEMPTION REQUESTS - FIRE PROT APP.R RULE			09/30/84
54545	MILLSTONE 1 - TECHNICAL SPECIFICATIONS COVERED BY GENERIC LETTERS 83-36, 83-37 FOR NUREG-0737			10/17/84
51702	MILLSTONE 1 - DEGRADED GRID VOLTAGE PROCEDURES			10/30/84
49386	MILLSTONE UNIT 1 - GRID SEPARATION PROCEDURES			12/08/84
51795	MILLSTONE - 1 RESPONSE TO HRC 6/8/83 LETTER OPTION ON II K.3.18			12/12/84
53435	MILLSTONE 1 - OCCUPATIONAL DOSE REDUCTION BNL			12/12/84
48293	MILLSTONE 1 - SUBCOOLED POST DBLOCA RADIOLYSIS		3	12/31/84
49387	MILLSTONE 1 - EMERGENCY POWER			

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CONDENSED MANAGEMENT REPORT

(CONTINUATION)

MILLSTONE 1

<u>IAC #</u>	<u>IAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>ACTIVE ACTIONS</u> (CONTINUATION)				
11221	MILLSTONE 1 - CONVERSION POL TO FTOL		1	<u>TARGET</u> 02/02/85
<u>ANTICIPATED ACTIONS</u>				
53606	MILLSTONE 1 - ITEM 1.2 - POST TRIP REVIEW-DATA AND INFORMATION CAPABILITY			<u>INITIATION</u> 11/01/84
53689	MILLSTONE 1 - ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS			11/01/84
53772	MILLSTONE 1- ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS			11/01/84
53855	MILLSTONE 1 - ITEM 3.2.3 - POST-MAINTENANCE TESTING -CHANGES TO TECH. SPEC. - ALL SR COMPONENTS			11/01/84
53999	MILLSTONE 1 - ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS			11/01/84
54082	MILLSTONE 1 - ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL TESTING - DIVERSE TRIP FEATURES			11/01/84
49390	MILLSTONE UNIT 1 - ACTIVITY LIMITS			12/01/84

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CONDENSED MANAGEMENT REPORT

FACILITY: MILLSTONE 2

PLANT LOCATION: 5 MI SW OF NEW LONDON, CONN
 DOCKET NUMBER: 050-00336
 ARCH/ENGINEER: BECH
 IE INSPECTOR: T. SHEDLOSKY

LICENSED POWER: 2700 MWT
 DESIGN POWER: 0870 MWE
 NSSS VENDOR: COMB

PROJECT MANAGER: D. OSBORNE
 BRANCH CHIEF: J. MILLER
 LIC. ASSISTANT: P. KREUTZER

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>MULTI-PLANT</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
12318	MILLSTONE 2 - HELB AND CONSEQUENTIAL SYSTEM FAILURE		3	<u>COMPLETE</u> 01/31/84
<u>ACTIVE ACTIONS</u>				
51107	MILLSTONE 2 - INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)			<u>TARGET</u>
52245	MILLSTONE 2 - CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPA#C-10)			
52774	MILLSTONE 2 - ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION AND PROCEDURES			
52855	MILLSTONE 2 - ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RTS COMPONENTS			
52936	MILLSTONE 2 - ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING PROCEDURES/VENDOR RECOMMENDATIONS - RTS COMPONENTS			
53017	MILLSTONE 2 - ITEM 3.1.3 - POST-MAINTENANCE TESTING-CHANGES TO TECH SPECS-RTS COMPONENTS			
53080	MILLSTONE 2 - REACTOR TRIP SYSTEM RELIABILITY - VENDOR RELATED MODIFICATIONS			
53133	MILLSTONE 2 - ITEMS 4.2.1 & 4.2.2 - PREVENTATIVE MAINT PROGRAM FOR REACTOR TRIP BREAKERS - MAINTENANCE & TRENDING			
49558	MILLSTONE 2 - REVIEW ALTERNATIVE APPENDIX I TECH SPECS		2	11/01/83
08949	MILLSTONE 2 - REVIEW OF ASYMMETRIC LOCA LOADS			11/30/83
51757	MILLSTONE 2 - THERMAL SHIELD FOLLOWUP ANALYSIS		2	01/15/84
08061	MILLSTONE 2 - CONTROL OF HEAVY LOADS (NUREG 0612)			03/30/84
49658	MILLSTONE 2 - REACTOR COOLANT PUMP TRIP			07/01/84
48642	MILLSTONE 2 - EXEMPTION FROM APP. R. IIIG TECHNICAL		1	10/01/84
42588	MILLSTONE 2 - LONG TERM REVIEW CONTAINMENT PURGE AND VENT (B-24)		1	11/30/84
42894	MILLSTONE 2 - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11		2	12/01/84
08952	MILLSTONE 2 - TS SURVEILLANCE FOR MECHANICAL SNUBBERS		2	12/01/84
08969	MILLSTONE 2 - TS SURVEILLANCE FOR HYDRAULIC SNUBBERS		3	12/01/84
42493	MILLSTONE 2 - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21)		1	12/01/84
<u>TAC # TAC DESCRIPTION MULTI-PLANT PRIORITY CRITICAL DATE</u>				
<u>COMPLETED ACTIONS</u>				
44807	MILLSTONE 2 - NUREG 0737 ITEM II K2.17 POTENTIAL FOR VOIDING IN RCS		1	<u>COMPLETE</u> 10/21/83
44104	MILLSTONE 2 - NUREG-0737 I.A.1.3., SHIFT MANNING			02/03/84
<u>ACTIVE ACTIONS</u>				
51177	MILLSTONE 2 - I.D.1 CONTROL ROOM DESIGN REVIEW			<u>TARGET</u>
51257	MILLSTONE 2 - I.D.2. SAFETY PARAMETER DISPLAY SYSTEM			
44864	MILLSTONE 2 - NUREG-0737 II.E.4.2, CONTAINMENT ISOLATION DEPENDABILITY		1	11/30/83
45148	MILLSTONE 2 - NUREG-0737 II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION		1	02/28/84
45840	MILLSTONE 2 - NUREG-0737 II.K.3.30, SB LOCA OUTLINE		1	06/15/84
44454	MILLSTONE 2 - NUREG-0737 II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS		1	06/30/84
46318	MILLSTONE 2 - NUREG-0737 III.A.2.2, METEOROLOGICAL DATA UPGRADE		1	09/01/84
46893	MILLSTONE 2 - II K2.13 THERMAL MECHANICAL REPORT		1	09/30/84
45959	MILLSTONE 2 - NUREG-0737 III.A.1.2, TECHNICAL SUPPORT CENTER		1	09/30/84
46030	MILLSTONE 2 - NUREG-0737 III.A.1.2, OPERATIONAL SUPPORT CENTER		1	09/30/84
46102	MILLSTONE 2 - NUREG-0737 III.A.1.2, EMERGENCY OPERATIONS FACILITY		1	09/30/84
48179	MILLSTONE 2 - NUREG 0737 II.K.3.31 COMPLIANCE WITH 10 CFR 50.46		1	09/30/84
44595	MILLSTONE 2 - RV AND SV TESTING		1	10/30/84

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CONDENSED MANAGEMENT REPORT

(CONTINUATION)

MILLSTONE 2

<u>IAC #</u>	<u>IAC DESCRIPTION</u>	<u>TMI ACTIONS</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>ACTIVE ACTIONS</u> (CONTINUATION)				
44246	MILLSTONE 2 - NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES		1	<u>TARGET</u> 11/30/84
44316	MILLSTONE 2 - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES		1	11/30/84
<u>IAC #</u>	<u>IAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
48919	MILLSTONE 2 - BORIC ACID FLOWPATH TECH. SPEC. REVISION			<u>COMPLETE</u> 10/07/83
51997	MILLSTONE 2 - EVALUATION OF WELD FLAWS IN S.G. #1 TGP HEAD STEAM DOME			12/01/83
51646	MILLSTONE 2 - NON-LOCA TRANSIENT ANALYSIS FOR OPERATION WITH 15.3% STEAM GENERATOR TUBEL PLUGGED			12/30/83
49062	MILLSTONE 2 - LOCA REANALYSES WITH 15.3% OF SG TUBES PLUGGED		3	12/30/83
51351	MILLSTONE 2 - SLEEVING STEAM GENERATOR TUBES			12/30/83
49798	MILLSTONE 2 - RELOAD CYCLE 6			12/30/83
48063	MILLSTONE 2 - MEASUREMENT UNCERTAINTIES		3	01/06/84
53462	MILLSTONE NO 2 - REVISED RELOAD CALCULATIONS			01/27/84
52023	MILLSTONE 2 - ADMINISTRATIVE T.S. CHANGES PROVIDE SER			02/19/84
48383	MILLSTONE 2 - LIQUID WASTE EFFLUENTS			03/28/84
54511	MILLSTONE UNIT NO. 2 - TEMPORARY RELOCATION OF TSC			04/02/84
53512	MILLSTONE NO 2 - PT CURVE CHANGES			04/10/84
51969	MILLSTONE 2 - THERMAL SHIELD DAMAGE RESOLUTION			04/12/84
<u>ACTIVE ACTIONS</u>				
51701	MILLSTONE 2 - DEGRADED GRID VOLTAGE PROCEDURES			<u>TARGET</u>
48644	MILLSTONE 2 : EXEMPTION FROM APP.R III G, SCHEDULAR			12/31/83
52573	MILLSTONE 2 - REVISED CONFIRMATORY ORDER			03/30/84
53384	MILLSTONE NO. 2 - PREPARE CONFIRMATORY ORDERS TO GENERIC LETTER 82-33			04/31/84
52504	MILLSTONE NO. 2 - PRESSURIZER LEVEL BAND			07/01/84
51574	MILLSTONE 2 - FOLLOWUP AMENDMENT OF IST PROGRAM RELATIVE TO PUMPS AND VALVES			09/15/84
49061	MILLSTONE 2 - CEA POSITION INDICATION, SIT LEVEL AND CONTAINMENT INTEGRITY		2	09/30/84
54399	MILLSTONE 2- RCS VENTS TECHNICAL SPECIFICATIONS NUREG 073 ITEM II.B.1			09/30/84
54546	MILLSTONE 2 - TECHNICAL SPECIFICATIONS COVERED BY GENERIC LETTERS 83-36, 83-37 FOR NUREG-0737			09/30/84
54741	MILLSTONE UNIT 2 - STEAM GENERATOR LEAK RATE DETECTION			10/01/84
48643	MILLSTONE 2 - EXEMPTION FROM APPENDIX R, ALTERNATE SAFE SHUTDOWN CAPABILITY			10/28/84
52339	MILLSTONE 2 - TECH. SPEC. CHANGE RE STEAM GENERATOR TUBE INSPECTION			12/31/84
54199	MILLSTONE UNIT 2 - CYCLE 6 CONFIRMATIONS SUBMITTALS			
<u>ANTICIPATED ACTIONS</u>				
55011	MILLSTONE UNIT NO 2 - RELIEF FROM INSERVICE INSPECTION REQUIREMENTS			<u>INITIATION</u> 05/16/84
53607	MILLSTONE 2 - ITEM 1.2 - POST TRIP REVIEW-DATA AND INFORMATION CAPABILITY			11/01/84
53690	MILLSTONE 2 - ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS			11/01/84
53773	MILLSTONE 2- ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS			11/01/84
53856	MILLSTONE 2 - ITEM 3.2.3 - POST-MAINTENANCE TESTING -CHANGES TO TECH. SPEC. - ALL SR COMPONENTS			11/01/84
53920	-ITEMS 4.2.3 & 4.2.4-PREVENTATIVE MAINT. PROGRAM FOR REACTOR TRIP BREAKERS-LIFE TESTING & REPLACEMENT			11/01/84
54000	MILLSTONE 2 - ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS			11/01/84
54083	MILLSTONE 2 - ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL TESTING - DIVERSE TRIP FEATURES			11/01/84

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CONDENSED MANAGEMENT REPORT

FACILITY: MONTICELLO

PLANT LOCATION: 30 MI NW OF MINNEAPOLIS, MINN
 DOCKET NUMBER: 050-00263
 ARCH/ENGINEER: BECH
 IE INSPECTOR: C. BROWN

LICENSED POWER: 1670 MW
 DESIGN POWER: 0545 MWE
 NSSS VENDOR: GE

PROJECT MANAGER: V. ROONEY
 BRANCH CHIEF: D. VASSALLO
 LIC. ASSISTANT: S. NORRIS

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>MULTI-PLANT</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
43497	MONTICELLO - 10 CFR 50.48 AND 10 CFR 50 APPENDIX R FIRE PROTECTION REQUIREMENTS		1	<u>COMPLETE</u> 01/03/84
42883	MONTICELLO - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11		2	01/06/84
43020	MONTICELLO - TECHNICAL SPECIFICATIONS DEFINING OPERABILITY FOR SAFETY SYSTEMS		2	01/23/84
12306	MONTICELLO - HELB AND CONSEQUENTIAL SYSTEM FAILURE MONTICELLO		3	01/31/84
08062	MONTICELLO - CONTROL OF HEAVY LOADS OVER SPENT FUEL POOL (NUREG 0612)		2	03/19/84
<u>ACTIVE ACTIONS</u>				
51108	MONTICELLO - INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)			
52246	MONTICELLO - CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPA#C-10)			
52775	MONTICELLO - ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION AND PROCEDURES			
52856	MONTICELLO - ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RTS COMPONENTS			
52937	MONTICELLO - ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING PROCEDURES/VENDOR RECOMMENDATIONS - RTS COMPONENTS			
53018	MONTICELLO - ITEM 3.1.3 - POST-MAINTENANCE TESTING-CHANGES TO TECH SPECS-RTS COMPONENTS			
51685	MONTICELLO NUREG 0737 TECH SPECS			
42475	MONTICELLO - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21)		1	03/01/84
11134	MONTICELLO - APP J-CONTAINMENT LEAK TESTING		2	04/30/84
42577	MONTICELLO - LONG TERM REVIEW CONTAINMENT PURGE AND VENT (B-24)		1	05/30/84
46666	MONTICELLO - IMPLEMENTATION OF NUREG-0313, REV. 1		2	09/30/84
43720	MONTICELLO - SAFETY CONCERNS ASSOCIATED WITH PIPE BREAKS IN THE BWR SCRAM SYSTEM		3	10/30/84
07942	MONTICELLO - MARK I CONTAINMENT LONG TERM PROGRAM IMPLEMENTATION		1	06/30/85
<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>TMI ACTIONS</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
44105	MONTICELLO - NUREG-0737 I.A.1.3.1, SHIFT MANNING OVERTIME LIMITS			<u>COMPLETE</u> 11/16/83
45841	MONTICELLO - NUREG-0737 II.K.3.30, SB LOCA OUTLINE		1	12/28/83
48180	MONTICELLO - NUREG 0737 II.K.3.31 COMPLIANCE WITH 10 CFR 50.46		1	12/28/83
44455	MONTICELLO - NUREG-0737 II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS		1	03/12/84
44865	MONTICELLO - NUREG-0737 II.E.4.2, CONTAINMENT ISOLATION DEPENDABILITY		1	03/14/84
<u>ACTIVE ACTIONS</u>				
44247	MONTICELLO - NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES		1	
44317	MONTICELLO - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES		1	
51178	MONTICELLO - I.D.1 CONTROL ROOM DESIGN REVIEW			
51258	MONTICELLO - I.D.2. SAFETY PARAMETER DISPLAY SYSTEM			
45694	MONTICELLO - NUREG-0737 II.K.3.18, ADS ACTUATION STUDY		1	01/29/84
45598	MONTICELLO - NUREG-0737 II.K.3.16, CHALLENGES AND FAILURES OF RELIEF VALVES		1	03/30/84
44596	MONTICELLO - RV AND SV TESTING		1	03/30/84
47857	MONTICELLO - NUREG - 0737 II. 3.22B, RCIC SUCTION MODIFICATIONS		1	03/30/84
45149	MONTICELLO - NUREG-0737 II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION		1	06/29/84
46319	MONTICELLO - NUREG-0737 III.A.2.2, METEOROLOGICAL DATA UPGRADE		1	09/29/84
45960	MONTICELLO - NUREG-0737 III.A.1.2, TECHNICAL SUPPORT CENTER		1	09/30/84
46031	MONTICELLO - NUREG-0737 III.A.1.2, OPERATIONAL SUPPORT CENTER		1	09/30/84
46103	MONTICELLO - NUREG-0737 III.A.1.2, EMERGENCY OPERATIONS FACILITY		1	09/30/84

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CONDENSED MANAGEMENT REPORT

(CONTINUATION)

MONTICELLO

TAC #	TAC DESCRIPTION	PLANT SPECIFIC	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				
48998	MONTICELLO - HIGH TEMP IN MAIN STEAM LINE TUNNEL			<u>COMPLETE</u> 11/18/83
51016	MONTICELLO - PLANT OPERATING PROCEDURES			12/30/83
52563	MONTICELLO - EXTENSION REQUEST PER 10CFR50.54(M)(2) ON LICENSED OPERATOR STAFFING REQUIREMENTS			12/30/83
48951	MONTICELLO - SOURCE RANGE MONITOR COUNT RATE		1	01/16/84
41061	MONTICELLO - DC POWER SUPPLY FAILURE EFFECT ECCS		2	01/31/84
49157	MONTICELLO - SHUTDOWN COOLING SUPPLY ISOLATION			02/02/84
49932	MONTICELLO - FOUR SECOND RPS TIME DELAY			04/03/84
<u>ACTIVE ACTIONS</u>				
46510	MONTICELLO - ISI/IST PROGRAM-SECOND TEN YEAR INTERVAL		3	<u>TARGET</u> 06/29/83
52435	MONTICELLO - REVIEW OF SURVEILLANCE TECH RPT ON REACTOR VESSEL RADIATION FROM RAM			01/31/84
52086	MONTICELLO - REVIEW OF MODIFICATIONS ON SRV LOW-LOW SETPOINT			03/30/84
52204	MONTICELLO - REPLACEMENT OF RECIRCULATION PIPING REVIEW			03/30/84
49934	MONTICELLO - MAXIMUM ALLOWABLE LEAKAGE RATE INCREASE LA			03/30/84
49093	MONTICELLO - VENT AND DRAIN VALVES OF THE SDV SYSTEM			03/30/84
53412	MONTICELLO - EXEMPTION FROM SCHEDULE FOR EMERGENCY PREPAREDNESS EXERCISE			03/30/84
53561	MONTICELLO - EXEMPTION ON SHIFT STAFFING RULE			03/30/84
51338	MONTICELLO - EXEMPTION REQUEST FROM FIXED SUPPRESSION IN THE CONTROL ROOM			04/30/84
49102	MONTICELLO - SURVEILLANCE REQUIREMENTS		1	05/31/84
49158	MONTICELLO - JET PUMP OPERABILITY		1	06/29/84
48457	MONTICELLO - REQUEST FOR SCHEDULAR RELIEF ON COMBUSTIBLE GAS CONTROL SYSTEMS		1	06/29/84
49101	MONTICELLO - ALTERNATE ROD INJECTION		1	06/30/84
49931	MONTICELLO - SCREEN WASH/FIRE PUMP			06/30/84
52089	MONTICELLO - REVIEW OF THE UNDERVOLTAGE RELAY TRIP EVENT			06/30/84
54748	MONTICELLO - CYCLE 11 RELOAD			08/01/84
54692	MONTICELLO - DEFER INTEGRATED LEAK TEST			08/30/84
48648	MONTICELLO - REVIEW OF LICENSEE'S RESPONSE TO UNRESOLVED FIRE PROTECTION SER ITEMS			09/28/84
51785	MONTICELLO - REVISED LICENSE AMENDMENT ON ENVIRONMENTAL TECHNICAL SPECIFICATION			09/29/84
51335	MONTICELLO - STATION BATTERY SYSTEM			09/29/84
49787	MONTICELLO - TURBINE BYPASS VALVES			09/29/84
49788	MONTICELLO - FEEDWATER/TURBINE TRIP INSTRUMENTATION			09/29/84
48646	MONTICELLO - REVISED LICENSE AMENDMENT ON SINGLE LOOP OPERATION			09/30/84
54547	MONTICELLO - TECHNICAL SPECIFICATIONS COVERED BY GENERIC LETTERS 83-36, 83-37 FOR NUREG-0737			09/30/84
<u>ANTICIPATED ACTIONS</u>				
53608	MONTICELLO - ITEM 1.2 - POST TRIP REVIEW-DATA AND INFORMATION CAPABILITY			<u>INITIATION</u> 11/01/84
53691	MONTICELLO 1 - ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS			11/01/84
53774	MONTICELLO- ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS			11/01/84
53857	MONTICELLO - ITEM 3.2.3 - POST-MAINTENANCE TESTING -CHANGES TO TECH. SPEC. - ALL SR COMPONENTS			11/01/84
54001	MONTICELLO - ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS			11/01/84
54084	MONTICELLO - ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL TESTING - DIVERSE TRIP FEATURES			11/01/84

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CONDENSED MANAGEMENT REPORT

FACILITY: NINE MILE POINT 1

PLANT LOCATION: 8 MI NE OF OSWEGO, NY
 DOCKET NUMBER: 050-00220
 ARCH/ENGINEER: NIAGARA
 IE INSPECTOR: S. HUDSON

LICENSED POWER: 1850 MWT
 DESIGN POWER: 0620 MWE
 NSSS VENDOR: GE

PROJECT MANAGER: R. HERMANN
 BRANCH CHIEF: D. VASSALLO
 LIC. ASSISTANT: S. NORRIS

TAC #	TAC DESCRIPTION	MULTI-PLANT	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				
13155	NINE MILE POINT 1 - ADEQUACY OF STATION ELECTRIC DISTRIBUTION SYSTEMS VOLTAGES		2	COMPLETE 12/20/83
12307	NINE MILE POINT 1 - HELB AND CONSEQUENTIAL SYSTEM FAILURE NINE MILE POINT		3	01/31/84
43021	NINE MILE POINT 1 - TECHNICAL SPECIFICATIONS DEFINING OPERABILITY FOR SAFETY SYSTEMS		2	02/02/84
03029	NINE MILE POINT 1 - REQUESTED EXEMPTIONS FROM 10CFR50 APPENDIX J		2	02/22/84
51686	NINE MILE POINT 1 NUREG 0737 TECH SPECS			03/09/84
07445	NINE MILE POINT 1 - REVISION TO NDT CURVE (APP G)		8	04/18/84
<u>ACTIVE ACTIONS</u>				
51109	NINE MILE POINT 1 - INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.9)			TARGET
52247	NINE MILE POINT 1 - CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPA#C-10)			
52776	NINE MILE POINT 1 - ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION AND PROCEDURES			
52857	NINE MILE POINT 1 - ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RTS COMPONENTS			
52938	NINE MILE POINT 1 - ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING PROCEDURES/VENDOR RECOMMENDATIONS - RTS COMPONENTS			
53019	NINE MILE PT 1-ITEM 3.1.3 -POST-MAINT TESTING-CHANGES TO TECH SPECS-RTS COMPONENTS		1	05/01/84
08943	NINE MILE POINT 1 - RPS POWER SUPPLY		1	05/30/84
42476	NINE MILE POINT 1 - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21)		1	06/01/84
42578	NINE MILE POINT 1 - LONG TERM REVIEW CONTAINMENT PURGE AND VENT (B-24)		2	06/30/84
42884	NINE MILE POINT 1 - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11		3	06/30/84
08118	NINE MILE POINT 1 - APPENDIX I TECH SPECS IMPLEMENTATION REVIEW		1	09/30/84
07943	NINE MILE POINT 1 - MARK I CONTAINMENT LONG TERM PROGRAM IMPLEMENTATION		2	09/30/84
08063	NINE MILE PT 1-CONTROL OF HEAVY LOADS OVER SPENT FUEL POOL (NUREG 0612)			
TAC #	TAC DESCRIPTION	TMI ACTIONS	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				
47954	NINE MILE POINT 1 - PLANT SHIELDING MODIFICATIONS (II.B.2.2)		1	COMPLETE 12/01/83
48181	NINE MILE PT. 1 -NUREG 0737 II.K.3.31 COMPLIANCE WITH 10 CFR 50.46		1	01/03/84
45842	NINE MILE POINT 1 - NUREG-0737 II.K.3.30, SB LOCA OUTLINE		1	01/03/84
44456	NINE MILE POINT 1 - NUREG-0737 II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS		1	04/17/84
<u>ACTIVE ACTIONS</u>				
51179	NINE MILE POINT 1 - I.D.1 CONTROL ROOM DESIGN REVIEW			TARGET
51259	NINE MILE POINT 1 - I.D.2. SAFETY PARAMETER DISPLAY SYSTEM			
44248	NINE MILE POINT 1 - NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES		1	03/31/84
44318	NINE MILE POINT 1 - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES		1	03/31/84
46320	NINE MILE POINT 1 - NUREG-0737 III.A.2.2, METEOROLOGICAL DATA UPGRADE		1	03/31/84
46460	NINE MILE POINT 1 - NUREG-0737 III.D.3.4, CONTROL ROOM HABITABILITY		1	06/30/84
44597	NINE MILE POINT 1 - RV AND SV TESTING		1	06/30/84
44866	NINE MILE POINT 1 - NUREG-0737 II.E.4.2, CONTAINMENT ISOLATION DEPENDABILITY		1	06/30/84
45599	NINE MILE POINT 1 - NUREG-0737 II.K.3.16, CHALLENGES AND FAILURES OF RELIEF VALVES		1	06/30/84
45695	NINE MILE POINT 1 - NUREG-0737 II.K.3.18, ADS ACTUATION STUDY		1	06/30/84
41032	NINE MILE-REACTOR VESSEL WATER LEVEL INSTRUMENTATION		1	06/30/84
45961	NINE MILE POINT 1 - NUREG-0737 III.A.1.2, TECHNICAL SUPPORT CENTER		1	09/30/84

DATA AS OF - 04/30/84

CONDENSED MANAGEMENT REPORT

(CONTINUATION)

NINE MILE POINT 1

TAC # TAC DESCRIPTIONIMI ACTIONSPRIORITYCRITICAL DATEACTIVE ACTIONS (CONTINUATION)

46032 NINE MILE POINT 1 - NUREG-0737 III.A.1.2, OPERATIONAL SUPPORT CENTER
 46104 NINE MILE POINT 1 - NUREG-0737 III.A.1.2, EMERGENCY OPERATIONS FACILITY

1

TARGET

09/30/84

1

09/30/84

TAC # TAC DESCRIPTIONPLANT SPECIFICPRIORITYCRITICAL DATECOMPLETED ACTIONS

07443 NINE MILE POINT 1 - ADDITION OF N2 ISOLATION VALVES
 53286 NINE MILE PT - 1 - TS CHANGE - FIRE PROTECTION
 52687 NINE MILE PT.1 - REVIEW - PROPOSED SRO MANNING AND, IF NEC., REQUEST FOR TIME EXTENSION
 42294 NINE MILE POINT 1 - REACTOR BUILDING CRACK
 07442 NINE MILE POINT 1 - SFP MODIFICATION
 52684 NINE MILE 1 - TS CHANGE - ADMIN CONTROLS SECTION
 52685 NINE MILE 1 - TS CHANGE - APPENDIX H-REPORTING REQUIREMENTS
 52698 NINE MILE PT-1 - TECH SPECS - ISI, IST, AUG. ISI

1

COMPLETE

11/30/83

12/20/83

12/28/83

1

12/29/83

1

02/01/84

03/20/84

04/18/84

04/18/84

ACTIVE ACTIONS

54436 NINE MILE PT. 1 - LIMITED CODE REVISION TO APPROVED ISI PROGRAM PER 10 CFR (2)(4)(IV)
 54129 NINE MILE PT 1 - REVIEW OF RECOUPLING SCREEHOUSE REACTOR BLDG CODE
 54628 NINE MILE PT - 1 - EVALUATION OF STUB TUBE REPAIRS
 53389 NINE MILE POINT 1 - SCHEDULAR EXEMPTION APPENDIX R - HIGH RADIATION LOGIC CIRCUITING
 54130 NINE MILE PT 1 - TS DELETION OF AUTOMATIC ISOLATION OF EMERG COND. ON HIGH RAD
 53270 NINE MILE PA-1 TECH SPEC REVIEW AND INTERPRETATION MSIV INST CHANNEL
 53447 NINE MILE PT-1 - AMENDMENT REQUEST TO INCREASE MCPR LIMITS
 52686 NINE MILE PT. 1 - EXEMPTION FROM HDROGEN RECOMBINER RULE
 52683 NINE MILE 1 - TS CHANGE - RAD MUT INVENTORY REQUIREMENTS
 54187 NINE MILE PT - TS - ADMIN CONTROL SECTION, OVERTIME, ETC.
 54188 NINE MILE PT - 1-TS-ADD CHANNEL CAL. REQUIREMENTS FOR MSIV AND TURBINE STOP VALVE INSTRS.
 42424 NINE MILE POINT 1 - SNUBBER ADDITIONS AND DELETIONS
 51334 NINE MILE PT. - UNIT-1 - PLANT SPECIFIC EVALUATION OF CORE SPRAY PERFORMANCE
 54152 NINE MILE PT-1 - PLANT SPECIFIC APPENDIX J. REVIEW
 54751 NINE MILE 1 - TECH. SPEC. - TRIPLE LOW REACTOR WATER LEVEL SETPOINT
 54752 NINE MILE - 1, TECH. SPEC. - ADS AND OVERPRESSURIZATION VALVE SET POINTS AND TESTING
 54753 NINE MILE 1 - TECH. SPEC. - INT. OF DIESEL GENERATORS ON LOSS OF OR DEGRADED VOLTAGE
 54210 NINE MILE PT. UNIT 1 - T.S. CHANGE FOR CAPSULE SURVEILLANCE PROGRAM PER APPENDIX H
 54548 NINE MILE POINT 1 - TECHNICAL SPECIFICATIONS COVERED BY GENERIC LETTERS 83-36, 83-37 FOR
 NUREG-0737
 53390 NINE MILE POINT 1 - ADDITIONAL EXEMPTION REQUEST FROM APPENDIX R (GENERIC LETTER 83-33)

2

TARGET

04/15/84

05/01/84

05/14/84

05/15/84

05/15/84

05/15/84

05/15/84

05/15/84

05/15/84

05/31/84

05/31/84

06/01/84

06/30/84

06/30/84

07/01/84

07/31/84

07/31/84

08/15/84

08/31/84

09/30/84

09/30/84

ANTICIPATED ACTIONS

53609 NINE MILE POINT 1 - ITEM 1.2 - POST TRIP REVIEW-DATA AND INFORMATION CAPABILITY
 53692 NINE MILE POINT 1 - ITEM 2.2 - EQUIPMENT CLASSIFICATION ANDVENDOR INTERFACE - ALL SR COMPONENTS
 53775 NINE MILE PT 1- ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS
 - ALL SR COMPONENTS
 53858 NINE MILE POINT 1 - ITEM 3.2.3 - POST-MAINTENANCE TESTING -CHANGES TO TECH. SPEC. - ALL SR
 COMPONENTS
 54002 NINE MILE POINT 1 - ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES
 & TEST INTERVALS
 54085 NINE MILE POINT 1 - ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL TESTING - DIVERSE TRIP FEATURES

INITIATION

11/01/84

11/01/84

11/01/84

11/01/84

11/01/84

11/01/84

DATA AS OF - 04/30/84

CONDENSED MANAGEMENT REPORT

FACILITY: NORTH ANNA 1

PLANT LOCATION: 40 MI NW OF RICHMOND, VA
 DOCKET NUMBER: 050-00338
 ARCH/ENGINEER: S&W
 IE INSPECTOR: T. WEBSTER

LICENSED POWER: 2775 MW
 DESIGN POWER: 0907 MWE
 NSSS VENDOR: WEST

PROJECT MANAGER: L. ENGLE
 BRANCH CHIEF: J. MILLER
 LIC. ASSISTANT: P. KREUTZER

TAC #	TAC DESCRIPTION	MULTI-PLANT	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				
47152	NORTH ANNA 1	NATURAL CIRCULATION COOLDOWN (GENERIC LTR 81-21)	2	<u>COMPLETE</u> 10/19/83
49742	NORTH ANNA 1	"NUREG 0737 TECH SPECS (GENERIC LETTER 82-16 82-16 OR 83-02)"	1	01/12/84
<u>ACTIVE ACTIONS</u>				
51110	NORTH ANNA 1	- INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)	1	<u>TARGET</u>
52248	NORTH ANNA 1	- CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPA#C-10)		
52777	NORTH ANNA 1	- ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION AND PROCEDURES		
52858	NORTH ANNA 1	- ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RTS COMPONENTS		
52939	NORTH ANNA 1	- ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING PROCEDURES/VENDOR RECOMMENDATIONS - RTS COMPONENTS		
53020	NORTH ANNA 1	- ITEM 3.1.3 - POST-MAINTENANCE TESTING-CHNGS TO TECH SPECS-RTS COMPONENTS		
53081	NORTH ANNA 1	- REACTOR TRIP SYSTEM RELIABILITY - VENDOR RELATED MODIFICATION		
53134	NORTH ANNA 1	- ITEMS 4.2.1 & 4.2.2 - PREVENTATIVE MAINT PROGRAM FOR REACTOR TRIP BREAKERS - MAINTENANCE & TRENDING		
53181	NORTH ANNA 1	- ITEM 4.3 - AUTOAMTIC ACTUATION OF SHUNT TRIP ATTACHMENT FOR W AND B & W PLANTS		
42497	NORTH ANNA 1	- ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21)	1	06/30/83
42895	NORTH ANNA 1	- MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11	2	07/30/83
13109	NORTH ANNA 1	- ADEQUACY OF STATION ELECTRIC DISTRIBUTION SYSTEM VOLTAGES	2	07/30/83
47112	NA-1:	CONTROL OF HEAVY LOADS - NUREG 0612	2	08/01/83
49665	NORTH ANNA 1	- REACTOR COOLANT PUMP TRIP		07/01/84

TAC #	TAC DESCRIPTION	IMI ACTIONS	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				
44388	NORTH ANNA 1	- NUREG-0737 II.B.1, RCS HIGH POINT VENTS	1	<u>COMPLETE</u> 10/09/83
45240	NORTH ANNA 1	- NUREG-0737 II.K.3.1, AUTO PORV ISOLATION	1	10/19/83
45286	NORTH ANNA 1	- NUREG-0737 II.K.3.2, REPORT ON PORV FAILURES		10/19/83
47955	NORTH ANNA 1	- PLANT SHIELDING MODIFICATIONS (II.B.2.2)	1	11/14/83
47808	NORTH ANNA 1	- NUREG 0737 ITEM II K2.17 POTENTIAL FOR VOIDING IN RCS	1	01/24/84
44457	NORTH ANNA 1	- NUREG-0737 II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS	1	02/27/84
46461	NORTH ANNA 1	- NUREG-0737 III.D.3.4, CONTROL ROOM HABITABILITY	1	03/20/84
<u>ACTIVE ACTIONS</u>				
51180	NORTH ANNA 1	- I.D.1 CONTROL ROOM DESIGN REVIEW		<u>TARGET</u>
51260	NORTH ANNA 1	- I.D.2. SAFETY PARAMETER DISPLAY SYSTEM		
45150	NORTH ANNA 1	- NUREG-0737 II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION		
46321	NORTH ANNA 1	- NUREG-0737 III.A.2.2, METEOROLOGICAL DATA UPGRADE	1	06/01/83
45843	NORTH ANNA 1	- NUREG-0737 II.K.3.30, SB LOCA OUTLINE	1	06/30/83
48182	NORTH ANNA 1	-NUREG 0737 II.K.3.31 COMPLIANCE WITH 10 CFR 50.46	1	09/30/83
44598	NORTH ANNA 1	- RV AND SV TESTING	1	12/31/83
45962	NORTH ANNA 1	- NUREG-0737 III.A.1.2, TECHNICAL SUPPORT CENTER	1	09/30/84
46033	NORTH ANNA 1	- NUREG-0737 III.A.1.2, OPERATIONAL SUPPORT CENTER	1	09/30/84
46105	NORTH ANNA 1	- NUREG-0737 III.A.1.2, EMERGENCY OPERATIONS FACILITY	1	09/30/84
46894	NORTH ANNA 1	- II K2.13 THERMAL MECHANICAL REPORT	1	09/30/84
44249	NORTH ANNA 1	- NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES	1	11/30/84

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CONDENSED MANAGEMENT REPORT

(CONTINUATION)

NORTH ANNA 1

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>IMI ACTIONS</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>ACTIVE ACTIONS (CONTINUATION)</u>				
44319	NORTH ANNA 1 - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES		1	TARGET 11/30/84
<u>PLANT SPECIFIC</u>				
<u>TAC #</u>	<u>TAC DESCRIPTION</u>		<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
52472	NA-1: AMEND LICENSE TO INCORPORATE OLD DOMINIOM ELECTRIC COOPERATIVE AS PARTIAL OWNER OF NORTH ANNA, UNIT 1			COMPLETE 11/18/83
52313	NA-1: REQUEST INTERP 10 CFR 20.202(B)(2) AND 20.203(B) REPOSTING OF RADIATION AREAS			01/27/84
53483	NA-1: REVISE TS TABLE 6.2.1 PER GENERIC LETTER 82-12			02/01/84
52671	NA-1: EXEMPTION REQUEST TO 10CFR50 APPENDIX R, CONTROLL ROOM AREA			02/23/84
54158	NA-1: REVISE TS FOR RCS LEAKAGE DETECTION SYSTEM			02/28/84
52550	NA-1: INCREASE IN FIRE PROTECTION SURVEILLANCE REQUIREMENTS			02/28/84
53526	NA-1: REVISE TS LIMITS FOR QUADRANT POWER TILT RATIO TO ALLOW FULL-CORE FLUX MAPPING			03/06/84
53251	NA-1: REVISE T.S. 3.3-3 FOR CONSISTENCY WITH NA-2 TS 3.3-3			03/06/84
52480	NA-1: AMEND TS IN TABLE 3.6.1 FOR CLARIFYING CHECK VALVE TESTING REQUIREMENTS			03/13/84
52574	NA-1: REVISING TS BY CHANGING FRACTIONAL POWER MULTIPLIER FROM 0.2 TO 0.3 FOR 587.8 F TAV		2	03/13/84
49524	NA-1: AMEND LIC FOR INCREASING RCS TAV TO 587.8 (PHASE 2, POWER UPRATE)			04/09/84
53262	NORTH ANNA 1 - EXTEND BURNUP TO 45,000 MWD/MTV			04/09/84
53517	NA: 1 REVISE TS TO DESIGNATE PORV & SAFETY VALVES AS PART OF "RCS"			04/13/84
54267	NA-1: EXEMPTION REQUEST TO 10 CFR PART 50.44(C)(3)(III)			04/16/84
53519	NA-1: REVISE TS FOR TURBINE OVERSPEED PROTECTION OPERABILITY TO 31 DAYS			04/17/84
54296	NA-1: RELIEF REQUEST: INSERVICE TESTING - PUMPS AND VALVES			
<u>ACTIVE ACTIONS</u>				
52053	NA-1: ADMIN CHANGES TO T.S. REORGANIZATION OF NUC OPS DEPT., QUALITY ASS, EMERG PLN & SECURITY DEPT			TARGET 09/22/83
53252	NA-1: AMEND T.S. BY ADDING EXEMPTION FOR ENGINEERING EVAL FOR ANY FAILURE OF SG/RCP SUPPORT SNUBBERS			02/15/84
53528	NA-1: PM BRIEFING BOOK-HEADQUARTES OPERATION-CENTER			02/28/84
53521	NA-1: REVISE TS FOR ALLOWING 1 OUT OF 2 HX RECOMBINERS TO BEINOPERABLE			03/15/84
53522	NA-1: REVISE TS FOR REACTOR TRIP SYS INTERLOCK SETPOINT P-7 FOR 10% FULL LOAD TURBINE IMPULSE CHAMBER PRESSURE			03/20/84
54789	NA-1: ISSUE SER AND EIA FOR SPENT FUEL AND TRANSSHIPMENT HEARING			05/11/84
54788	NA-1: EXEMPTION REQUEST TO INTERVAL BETWEEN EMERGENCY EXERCISE			05/15/84
54472	NA-1: RELIEF FROM ASME XI VOL. EXH. ACC AND BORON TANK			05/16/84
52474	NA-1: EVALUATE 5 YEAR REPORT ON SETTLEMENT AND GROUNDWATER LEVEL AT SERVICE WATER RESERVOIR		1	05/30/84
52476	NA-1: AMEND TS TERMINATING MEASUREMENTS FOR TORBIDITY & SUSPENDED SOLIDS FROM DRAIN SYS UNDER SER WATER PMP HSE		1	05/30/84
49525	NA-1: EVALUATE NA MAIN TRANSFORMER FAULTS			06/01/84
54845	NORTH ANNA 1 - CORE RELOAD METHODOLOGY AUDIT			06/15/84
54500	NA-2: REVISE TS PER GEN.LTR. 83-30			06/15/84
54505	NA-1: REVISE TS FOR LER REPORTING PER 10CFR PART 50.72 & 50.73			06/30/84
52478	NA-1: AMEND TS BY REDUCING LENGTH OF TIME FOR CORE SURVEILLANCE REPORT FROM 60 TO 30 DAYS		1	06/30/84
53524	NA-1: REVISE TS FOR LCD FOR SURVEILLANCE IN MODES 3.4 AND 5		1	07/30/84
54465	NA-1: DELETE TECH SPEC REQUIREMENTS FOR SG SUPPORTS			09/30/84
54549	NORTH ANNA 1 - TECHNICAL SPECIFICATIONS COVERED BY GENERIC LETTERS 83-36, 83-37 FOR NUREG-0737			09/30/84
54400	NORTH ANNA 1- RCS VENTS TECHNICAL SPECIFICATIONS NUREG 073 ITEM II.B.1			12/30/84
52414	NA-1: INSERVICE TESTING (PUMPS & VALVES)		1	12/31/84
48868	NA-1: AMEND LICENSE FOR RECEIPT AND STORAGE OF SPENT FUEL FROM SURRY 1 & 2		1	12/31/84
48870	NA-1: AMEND T.S. FOR AUGMENTED SPENT FUEL STORAGE			

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CONDENSED MANAGEMENT REPORT

(CONTINUATION)

NORTH ANNA 1

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>ACTIVE ACTIONS</u> (CONTINUATION)				<u>TARGET</u>
54832	NA-1: APPENDIX R REANALYSIS EXEMPTION REQUESTS			06/30/85
<u>ANTICIPATED ACTIONS</u>				<u>INITIATION</u>
53693	NORTH ANNA 1 - ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS			11/01/84
53776	NORTH ANNA 1- ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS			11/01/84
53859	NORTH ANNA 1 - ITEM 3.2.3 - POST-MAINTENANCE TESTING -CHANGES TO TECH. SPEC. - ALL SR COMPONENTS			11/01/84
53921	-ITEMS 4.2.3 & 4.2.4-PREVENTATIVE MAINT. PROGRAM FOR REACTOR TRIP BREAKERS-LIFE TESTING & REPLACEMENT			11/01/84
54003	NORTH ANNA 1 - ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS			11/01/84
54086	NORTH ANNA 1 - ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL TESTING - DIVERSE TRIP FEATURES			11/01/84
53610	NORTH ANNA 1 - ITEM 1.2 - POST TRIP REVIEW-DATA AND INFORMATION CAPABILITY			11/01/84

DATA AS OF - 04/30/84

CONDENSED MANAGEMENT REPORT

FACILITY: NORTH ANNA 2

PLANT LOCATION: 40 MI NW OF RICHMOND, VA
 DOCKET NUMBER: 050-00339
 ARCH/ENGINEER: S&W
 IE INSPECTOR:

LICENSED POWER: 2775 MWT
 DESIGN POWER: 0907 MWE
 NSSS VENDOR: WEST

PROJECT MANAGER: L. ENGLE
 BRANCH CHIEF: J. MILLER
 LIC. ASSISTANT: P. KREUTZER

TAC #	TAC DESCRIPTION	MULTI-PLANT	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				
47153	NORTH ANNA 2	NATURAL CIRCULATION COOLDOWN (GENERIC LTR 81-21)	2	<u>COMPLETE</u> 10/19/83
49743	NORTH ANNA 2	- "NUREG 0737 TECH SPECS (GENERIC LETTER 82-16 82-16 OR 83-02)"		01/12/84
52249	NORTH ANNA 2	- CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPA#C-10)		01/26/84
<u>ACTIVE ACTIONS</u>				
51111	NORTH ANNA 2	- INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)		<u>TARGET</u>
52778	NORTH ANNA 2	- ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION AND PROCEDURES		
52859	NORTH ANNA 2	- ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RTS COMPONENTS		
52940	NORTH ANNA 2	- ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING PROCEDURES/VENDOR RECOMMENDATIONS - RTS COMPONENTS		
53021	NORTH ANNA 2	- ITEM 3.1.3 - POST-MAINTENANCE TESTING-CHNGS TO TECH SPECS-RTS COMPONENTS		
53082	NORTH ANNA 2	- REACTOR TRIP SYSTEM RELIABILITY - VENDOR RELATED MODIFICATIONS		
53135	NORTH ANNA 2	- ITEMS 4.2.1 & 4.2.2 - PREVENTATIVE MAINT PROGRAM FOR REACTOR TRIP BREAKERS - MAINTENANCE & TRENDING		
53182	NORTH ANNA 2	- ITEM 4.3 - AUTOAMTIC ACTUATION OF SHUNT TRIP ATTACHMENT FOR W AND B & W PLANTS	1	07/30/82
42540	NORTH ANNA 2	- ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21)	2	07/01/83
47113	NA-2	CONTROL OF HEAVY LOADS - NUREG-0612	2	07/30/83
48331	NORTH ANNA 2	- ADEQUACY OF STATION ELECTRIC DISTRIBUTION VOLTAGES		07/01/84
49666	NORTH ANNA 2	- REACTOR COOLANT PUMP TRIP		
TAC #	TAC DESCRIPTION	TMI ACTIONS	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				
44389	NORTH ANNA 2	- NUREG-0737 II.B.1, RCS HIGH POINT VENTS	1	<u>COMPLETE</u> 10/09/83
45241	NORTH ANNA 2	- NUREG-0737 II.K.3.1, AUTO PORV ISOLATION		10/19/83
45287	NORTH ANNA 2	- NUREG-0737 II.K.3.2, REPORT ON PORV FAILURES	1	10/19/83
47956	NORTH ANNA 2	- PLANT SHIELDING MODIFICATIONS (II.B.2.2)	1	11/14/83
45647	NORTH ANNA 2	- NUREG-0737 II.K.3.17, ECC SYSTEM OUTAGES	1	12/30/83
47809	NORTH ANNA 2	- NUREG 0737 ITEM II K2.17 POTENTIAL FOR VOIDING IN RCS	1	01/24/84
44458	NORTH ANNA 2	- NUREG-0737 II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS	1	02/27/84
46462	NORTH ANNA 2	- NUREG-0737 III.D.3.4, CONTROL ROOM HABITABILITY	1	03/20/84
<u>ACTIVE ACTIONS</u>				
45151	NORTH ANNA 2	- NUREG-0737 II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION		<u>TARGET</u>
51181	NORTH ANNA 2	- I.D.1 CONTROL ROOM DESIGN REVIEW		
51261	NORTH ANNA 2	- I.D.2. SAFETY PARAMETER DISPLAY SYSTEM		
44599	NORTH ANNA 2	- RV AND 5V TESTING	1	10/31/82
46322	NORTH ANNA 2	- NUREG-0737 III.A.2.2, METEOROLOGICAL DATA UPGRADE	1	06/01/83
45844	NORTH ANNA 2	- NUREG-0737 II.K.3.30, SB LOCA OUTLINE	1	06/30/83
48183	NORTH ANNA 2	-NUREG 0737 II.K.3.31 COMPLIANCE WITH 10 CFR 50.46		09/30/83
46895	NORTH ANNA 2	- II K2.13 THERMAL MECHANICAL REPORT	1	09/30/84
45963	NORTH ANNA 2	- NUREG-0737 III.A.1.2, TECHNICAL SUPPORT CENTER	1	09/30/84
46034	NORTH ANNA 2	- NUREG-0737 III.A.1.2, OPERATIONAL SUPPORT CENTER	1	09/30/84
46106	NORTH ANNA 2	- NUREG-0737 III.A.1.2, EMERGENCY OPERATIONS FACILITY	1	09/30/84
44250	NORTH ANNA 2	- NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES	1	11/30/84

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CONDENSED MANAGEMENT REPORT

(CONTINUATION)

NORTH ANNA 2

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>IMI ACTIONS</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>ACTIVE ACTIONS (CONTINUATION)</u>				
44320	NORTH ANNA 2 - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES		1	TARGET 11/30/84
<u>PLANT SPECIFIC</u>				
<u>COMPLETED ACTIONS</u>				
51540	NA-2: PLANT UPGRADE, PHASE 1, INCREASE T AVERAGE			COMPLETE 10/19/83
52473	NA-2: AMEND LICENSE TO INCORPORATE OLD DOMINION ELECTRIC COOPERATIVE AS PARTIAL OWNER OF NORTH ANNA, UNIT 2			11/18/83
52314	NA-2: REQUEST INTERP 10 CFR 20.202(B)(2) AND 20.203(B) REPOSTING OF RADIATION AREAS			01/27/84
53484	NA-2: REVISE T.S. TABLE 6.2.1 PER GENERIC LETTER 82-12			02/01/84
52672	NA-2: EXEMPTION REQUEST TO 10CFR50 APPENDIX R, CONTROL ROOM			02/23/84
52551	NA-2: INCREASE IN FIRE PROTECTION SURVEILLANCE REQUIREMENTS			02/28/84
53527	NA-2: REVISE TS LIMITS FOR QUADRANT POWER TILT RATIO TO ALLOW TOLL-CORE FLUX-MAPPING			02/28/84
54159	NA-2: REVISE TS FOR RCS LEAKAGE DETECTION SYSTEM			02/28/84
52481	NA-2: AMEND TS IN TABLE 3.6.1 FOR CLARIFYING CHECK VALVE TESTING REQUIREMENTS			03/06/84
53518	NA-2: REVISE TS TO DESIGNATE PORV & SAFETY VALVES AS PART OF "RCS"			04/09/84
53263	NORTH ANNA 2 - EXTEND BURNUP TO 45,000 MWD/MTU			04/09/84
54268	NA-2: EXEMPTION REQUEST TO 10 CFR PART 50.44(C)(3)(III)			04/13/84
53520	NA-2: REVISE TS FOR TURBINE OVERSPEED PROTECTION OPERABILITY TO 31 DAYS			04/16/84
54297	NA-2: RELIEF REQUEST: INSERVICE TESTING - PUMPS AND VALVES			04/17/84
<u>ACTIVE ACTIONS</u>				
46921	NORTH ANNA 2 - CONTROL ROOM HUMAN FACTORS REVIEW			TARGET 10/01/82
52052	NA-2: ADMIN CHANGES TO T.S. REORGANIZATION OF NUC OPS DEPT. QUALITY ASS., EMERG PLN & SECURITY DEPT			09/22/83
46734	NA-2: FOLLOWUP ON JULY 3, 1981 TRANS FIRE RECURRING TRANS FAULTS		2	12/30/83
53253	NA-2: AMEND T.S. BY ADDING EXEMPTION FOR ENGINEERING EVAL. FOR FAILURE OF SG/RCP SUPPORT SNUBBERS			02/15/84
53529	NA-2: PM BRIEFING BOOK-HEADQUARTERS-OPERATION CENTER			02/28/84
53523	NA-2: REVISE TS FOR REACTOR TRIPS SYS INTERLOCK SETPOINT P-7 FOR 10% FULL LOAD TURBINE IMPULSE CHAMBER PRESSURE			03/20/84
54790	NA-2: ISSUE SER AND EIA FOR SPENT FUEL AND TRANSSHIPMENT HEARING			05/11/84
54787	NA-2: EXEMPTION REQUEST TO INTERVAL BETWEEN EMERGENCY EXERCISE			05/15/84
54473	NA-2: RELIEF FROM ASME XI VOL. EXM ACC AND BORON TANK			05/16/84
52475	NA-2: EVALUATE 5 YEAR REPORT ON SETTLEMENT AND GROUNDWATER LEVEL AT SERVICE WATER RESERVOIR			05/30/84
52477	NA-2: AMEND TS TERMINATING MEASUREMENTS FOR TURBIDITY & SUSPENDED SOLIDS FROM DRAINSYS UNDER SER WATER PMP HSE			05/30/84
54846	NORTH ANNA 2 - CORE RELOAD METHODOLOGY AUDIT			06/01/84
54501	NA-2: REVISE TS PER GEN. LTR. 83-30			06/15/84
54506	NA-2: REVISE TS FOR LER REPORTING PER 10CFR PART 50.72 & 50.73			06/15/84
53525	NA-2: REVISE TS FOR LCO FOR SURVEILLANCE IN MODES 3, 4 AND 5			06/30/84
54165	NA-2: REG. REVIEW-VITAL AREA VALIDATION			06/30/84
52479	NA-2: AMEND TS BY REDUCING LENGTH OF TIME FOR CORE SURVEILLANCE REPORT FROM 30 TO 60 DAYS		2	06/30/84
54464	NA-2: DELETE TECH. SPEC. REQUIREMENTS FOR SG SUPPORTS		1	07/30/84
54401	NORTH ANNA 2- RCS VENTS TECHNICAL SPECIFICATIONS NUREG 073 ITEM II.B.1			09/30/84
54550	NORTH ANNA 2 - TECHNICAL SPECIFICATIONS COVERED BY GENERIC LETTERS 83-36, 83-37 FOR NUREG-0737			09/30/84
52575	NA-2: REVISE TS BY CHANGING FRACTIONAL POWER MULTIPLIER FROM 0.2 TO 0.3 FOR 587.8 F TAV			11/30/84
49523	NA-2: AMEND LIC FOR INCREASE RCS TAV TO 587.8 (PHASE 2- POWER INCREASE)		2	11/30/84
48867	NA-2: AMEND LICENSE FOR RECEIPT AND STORAGE OF SPENT FUEL FROM SURRY 1 & 2		2	12/30/84
48869	NA-2: AMEND T.S. FOR AUGMENTED SPENT FUEL STORAGE			12/30/84

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CONDENSED MANAGEMENT REPORT

(CONTINUATION)

NORTH ANNA 2

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>ANTICIPATED ACTIONS</u>				<u>INITIATION</u>
54858	NA-2: INSERVICE TESTING - PUMPS AND VALVES			05/01/84
53611	NORTH ANNA 2 - ITEM 1.2 - POST TRIP REVIEW-DATA AND INFORMATION CAPABILITY			11/01/84
53694	NORTH ANNA 2 - ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS			11/01/84
53777	NORTH ANNA 2- ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS			11/01/84
53860	NORTH ANNA 2 - ITEM 3.2.3 - POST-MAINTENANCE TESTING -CHANGES TO TECH. SPEC. - ALL SR COMPONENTS			11/01/84
53922	-ITEMS 4.2.3 & 4.2.4-PREVENTATIVE MAINT. PROGRAM FOR REACTOR TRIP BREAKERS-LIFE TESTING & REPLACEMENT			11/01/84
54004	NORTH ANNA 2 - ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS			11/01/84
54087	NORTH ANNA 2 - ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL TESTING - DIVERSE TRIP FEATURES			11/01/84

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CONDENSED MANAGEMENT REPORT

FACILITY: OCONEE 1

PLANT LOCATION: 30 MI W OF GREENVILLE, SC
 DOCKET NUMBER: 050-00269
 ARCH/ENGINEER: DUKE/BECH
 IE INSPECTOR: J. BRYANT

LICENSED POWER: 2568 MWT
 DESIGN POWER: 0887 MWE
 NSSS VENDOR: B&W

PROJECT MANAGER: H. NICOLARAS
 BRANCH CHIEF: J. STOLZ
 LIC. ASSISTANT: R. INGRAM

TAC #	TAC DESCRIPTION	MULTI-PLANT	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				
08119	OCONEE 1 - APPENDIX I TECH SPEC IMPLEMENTATION REVIEW		3	<u>COMPLETE</u> 01/16/84
12323	OCONEE 1 - HELB AND CONSEQUENTIAL SYSTEM FAILURE OCONEE 1		3	01/31/84
<u>ACTIVE ACTIONS</u>				
52779	OCONEE 1 - ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION AND PROCEDURES			<u>TARGET</u>
52860	OCONEE 1 - ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RTS COMPONENTS			
52941	OCONEE 1 - ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING PROCEDURES/VENDOR RECOMMENDATIONS - RTS COMPONENTS			
53022	OCONEE 1 - ITEM 3.1.3 - POST-MAINTENANCE TESTING-CHANGES TO TECH SPECS-RTS COMPONENTS			
53083	OCONEE 1 - REACTOR TRIP SYSTEM RELIABILITY- VENDOR RELATED MODIFICATIONS			
53136	OCONEE 1 - ITEMS 4.2.1 & 4.2.2 - PREVENTATIVE MAINTENANCE PROGRAM FOR REACTOR TRIP BREAKERS - MAINTENANCE & TRENDING			
47183	OCONEE 1 NATURAL CIRCULATION COOLDOWN (GENERIC LTR 81-21)		2	04/01/84
49744	OCONEE 1 -"NUREG 0737 TECH SPECS (GENERIC LETTER 82-16 82-16 OR 83-02)"		2	05/31/84
42904	OCONEE 1 - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11		2	05/31/84
51112	OCONEE 1 - INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)		2	06/15/84
49659	OCONEE 1 - REACTOR COOLANT PUMP TRIP			07/01/84
43643	OCONEE 1 - SEISMIC QUALIFICATION OF THE AUXILIARY FEEDWATER SYSTEM		1	09/30/84
42509	OCONEE 1 - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21)		1	03/31/85
52250	OCONEE 1 - CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPA#C-10)		2	09/30/85
TAC #	TAC DESCRIPTION	TMI ACTIONS	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				
44109	OCONEE 1 - NUREG-0737 I.A.1.3.1, SHIFT MANNING OVERTIME LIMITS			<u>COMPLETE</u> 10/28/83
44390	OCONEE 1 - NUREG-0737 II.B.1, RCS HIGH POINT VENTS		1	11/02/83
44180	OCONEE 1 - NUREG-0737 I.A.2.1.4, UPGRADING OF RO AND SRO TRAINING		1	11/18/83
44530	OCONEE 1 - NUREG-0737 II.B.4.1, TRAINING FOR MITIGATING CORE		1	11/18/83
45270	OCONEE 1 - NUREG-0737 II.K.3.1, AUTO PORV ISOLATION		1	11/22/83
45316	OCONEE 1 - NUREG-0737 II.K.3.2, REPORT ON PORV FAILURES		1	11/22/83
45152	OCONEE 1 - NUREG-0737 II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION		1	11/23/83
44459	OCONEE 1 - NUREG-0737 II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS		1	01/11/84
<u>ACTIVE ACTIONS</u>				
51182	OCONEE 1 - I.D.1 CONTROL ROOM DESIGN REVIEW		2	<u>TARGET</u> 06/15/84
51262	OCONEE 1 - I.D.2. SAFETY PARAMETER DISPLAY SYSTEM		2	06/15/84
45845	OCONEE 1 - NUREG-0737 II.K.3.30, SB LOCA OUTLINE		1	09/30/84
45964	OCONEE 1 - NUREG-0737 III.A.1.2, TECHNICAL SUPPORT CENTER		1	09/30/84
46035	OCONEE 1 - NUREG-0737 III.A.1.2, OPERATIONAL SUPPORT CENTER		1	09/30/84
46107	OCONEE 1 - NUREG-0737 III.A.1.2, EMERGENCY OPERATIONS FACILITY		1	09/30/84
46323	OCONEE 1 - NUREG-0737 III.A.2.2, METEOROLOGICAL DATA UPGRADE		1	09/30/84
46463	OCONEE 1 - NUREG-0737 III.D.3.4, CONTROL ROOM HABITABILITY		1	09/30/84
45199	OCONEE 1 - NUREG-0737 II.K.2.13, THERMAL-MECHANICAL REPORT		1	09/30/84
44600	OCONEE 1 - RV AND SV TESTING		1	10/30/84
44251	OCONEE 1 - NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES		1	11/30/84

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CONDENSED MANAGEMENT REPORT

(CONTINUATION)

OCONEE 1

TAC #	TAC DESCRIPTION	TMI ACTIONS	PRIORITY	CRITICAL DATE
<u>ACTIVE ACTIONS</u> (CONTINUATION)				
44321	OCONEE 1 - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES		1	TARGET 11/30/84
48184	OCONEE 1 -NUREG 0737 II.K.3.31 COMPLIANCE WITH 10 CFR 50.46		1	09/30/85

TAC #	TAC DESCRIPTION	PLANT SPECIFIC	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				
51032	OCONEE 1 - EXEMPTION TO MULTIPLE CRISIS MANAGEMENT CENTER STAFF DRILLS			COMPLETE 01/06/84
48954	OCONEE 1 - ISI RELIEF REQUEST OF 10/4/82			02/14/84
52612	OCONEE 1 - REQUALIFICATION PROGRAM		2	03/15/84
51315	OCONEE 1 - REFUELING OUTAGE SURVEILLANCE REQUIREMENTS			03/28/84

TAC #	TAC DESCRIPTION	PLANT SPECIFIC	PRIORITY	CRITICAL DATE
<u>ACTIVE ACTIONS</u>				
48225	OCONEE 1 - EFW TORNADO PROTECTION		2	TARGET
53183	OCONEE 1 - ITEM 4.3 - AUTOMATIC ACTUATION OF SHUNT TRIP ATTACHMENT FOR W AND B & W PLANTS			
54144	OCONEE 1 - OVERTIME LIMITS			
53341	OCONEE 1 - ISI/IST EXEMPTION			02/28/84
52649	OCONEE 1 - FIRE PROTECTION EXEMPTION REQUEST (LIGHTING)			03/15/84
49568	OCONEE 1 - RCS LEAK TEST T.S. CHANGES		1	04/15/84
51894	OCONEE 1 - HYDROGEN ANALYZER PENETRATION T.S. CHANGES		1	04/15/84
51897	OCONEE 1 - B & PV CODE T.S. CHANGES		1	04/15/84
52674	OCONEE 1 - FIRE PROTECTION EXEMPTION (BARRIER/CABLE)			05/04/84
54206	OCONEE 1 - ECCW TECH. SPEC. CHANGE			05/18/84
54211	OCONEE 1 - ADMIN. T.S. CHANGE OF 2/13/84			05/31/84
52201	OCONEE 1 - CONFIRMATORY ORDER AMEND REQUEST 8/26/83		1	05/31/84
52406	OCONEE 1 - ISI RELIEF REQUEST OF 8/15/83		2	05/31/84
51049	OCONEE 1 - STARTUP TRANSFORMERS AND ALLOWABLE DEGRADED CONDITIONS		2	05/31/84
49827	OCONEE 1 - SMALL BREAK LOCA PROCEDURES AND MAINTAINING PROPER SG LEVEL		2	06/01/84
51776	OCONEE 1 - EXEMPTION FOR LOCATION OF EOF		1	06/29/84
55012	OCONEE UNIT 1 - WASTE GAS HOLUDP TANKS TECH. SPEC. REVIEW			07/16/84
55015	OCONEE UNIT 1 - TECH. SPEC. REVIEW OF "REPORTABLE OCCURRENCE" REQS			07/31/84
49974	OCONEE 1 - STA BACHELOR'S DEGREE OR EQUIVALENT			08/15/84
54402	OCONEE 1 - RCS VENTS TECHNICAL SPECIFICATIONS NUREG 073 ITEM II.B.1			09/30/84
54551	OCONEE 1 - TECHNICAL SPECIFICATIONS COVERED BY GENERIC LETTERS 83-36, 83-37 FOR NUREG-0737			09/30/84
49234	OCONEE 1 - IST PROGRAM REVISION AND REQUEST FOR RELIEF		1	10/10/84

TAC #	TAC DESCRIPTION	PLANT SPECIFIC	PRIORITY	CRITICAL DATE
<u>ANTICIPATED ACTIONS</u>				
53612	OCONEE 1 - ITEM 1.2 - POST TRIP REVIEW-DATA AND INFORMATION CAPABILITY			INITIATION 11/01/84
53695	OCONEE 1 - ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS			11/01/84
53778	OCONEE 1 - ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS			11/01/84
53861	OCONEE 1 - ITEM 3.2.3 - POST-MAINTENANCE TESTING -CHANGES TO TECH. SPEC. - ALL SR COMPONENTS			11/01/84
53923	-ITEMS 4.2.3 & 4.2.4-PREVENTATIVE MAINT. PROGRAM FOR REACTOR TRIP BREAKERS-LIFE TESTING & REPLACEMENT			11/01/84
53955	OCONEE 1 - ITEM 4.4 - IMPROVEMENTS IN MAINTENANCE AND TEST PROCEDURES FOR B & W PLANTS			11/01/84
54005	OCONEE 1 - ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS			11/01/84
54088	OCONEE 1 - ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL TESTING - DIVERSE TRIP FEATURES			11/01/84

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CONDENSED MANAGEMENT REPORT

FACILITY: OCONEE 2

PLANT LOCATION: 30 MI W OF GREENVILLE, SC
 DOCKET NUMBER: 050-00270
 ARCH/ENGINEER: DUKE/BECH
 IE INSPECTOR: J. BRYANT

LICENSED POWER: 2568 MWT
 DESIGN POWER: 0887 MWE
 NSSS VENDOR: B&W

PROJECT MANAGER: H. NICOLARAS
 BRANCH CHIEF: J. STOLZ
 LIC. ASSISTANT: R. INGRAM

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>MULTI-PLANT</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
49617	OCONEE 2 - PLANS FOR PREVENTING EXCEEDING PTS SCREENING CRITERION		1	<u>COMPLETE</u> 10/28/83
08120	OCONEE 2 - APPENDIX I TECH SPEC IMPLEMENTATION REVIEW		3	01/16/84
12324	OCONEE 2 - HELB AND CONSEQUENTIAL SYSTEM FAILURE OCONEE 2		3	01/31/84
<u>ACTIVE ACTIONS</u>				
47184	OCONEE 2 NATURAL CIRCULATION COOLDOWN (GENERIC LTR 81-21)		2	<u>TARGET</u>
52780	OCONEE 2 - ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION AND PROCEDURES			
52861	OCONEE 2 - ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RTS COMPONENTS			
52942	OCONEE 2 - ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING PROCEDURES/VENDOR RECOMMENDATIONS - RTS COMPONENTS			
53023	OCONEE 2-ITEM 3.1.3 -POST-MAINTENANCE TESTING-CHANGES TO TECH SPECS-RTS COMPONENTS			
53084	OCONEE 2 - REACTOR TRIP SYSTEM RELIABILITY- VENDOR RELATED MODIFICATIONS			
53137	OCONEE 2 - ITEMS 4.2.1 & 4.2.2 - PREVENTATIVE MAINTENANCE PROGRAM FOR REACTOR TRIP BREAKERS - MAINTENANCE & TRENDING			
53184	OCONEE 2 - ITEM 4.3 - AUTOMATIC ACTUATION OF SHUNT TRIP ATTACHMENT FOR W AND B & W PLANTS			
49745	OCONEE 2 - NUREG 0737 TECH SPECS (GENERIC LTR 82-16)		2	05/31/84
42905	OCONEE 2 - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11		2	05/31/84
51113	OCONEE 2 - INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)		2	06/15/84
49660	OCONEE 2 - REACTOR COOLANT PUMP TRIP		2	07/01/84
43644	OCONEE 2 - SEISMIC QUALIFICATION OF THE AUXILIARY FEEDWATER SYSTEM		1	09/30/84
42510	OCONEE 2 - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21)		1	03/31/85
52251	OCONEE 2 - CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPA#C-10)		2	09/30/85
<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>TMI ACTIONS</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
44110	OCONEE 2 - NUREG-0737 I.A.1.3.1, SHIFT MANNING OVERTIME LIMITS			<u>COMPLETE</u> 10/28/83
44391	OCONEE 2 - NUREG-0737 II.B.1, RCS HIGH POINT VENTS		1	11/02/83
44181	OCONEE 2 - NUREG-0737 I.A.2.1.4, UPGRADING OF RO AND SRO TRAINING		1	11/18/83
44531	OCONEE 2 - NUREG-0737 II.B.4.1, TRAINING FOR MITIGATING CORE		1	11/18/83
45271	OCONEE 2 - NUREG-0737 II.K.3.1, AUTO PORV ISOLATION		1	11/22/83
45317	OCONEE 2 - NUREG-0737 II.K.3.2, REPORT ON PORV FAILURES		1	11/22/83
45153	OCONEE 2 - NUREG-0737 II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION		1	11/23/83
44460	OCONEE 2 - NUREG-0737 II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS		1	01/11/84
<u>ACTIVE ACTIONS</u>				
51183	OCONEE 2 - I.D.1 CONTROL ROOM DESIGN REVIEW		2	<u>TARGET</u> 06/15/84
51263	OCONEE 2 - I.D.2. SAFETY PARAMETER DISPLAY SYSTEM		2	06/15/84
45846	OCONEE 2 - NUREG-0737 II.K.3.30, SB LOCA OUTLINE		1	09/30/84
45965	OCONEE 2 - NUREG-0737 III.A.1.2, TECHNICAL SUPPORT CENTER		1	09/30/84
46036	OCONEE 2 - NUREG-0737 III.A.1.2, OPERATIONAL SUPPORT CENTER		1	09/30/84
46108	OCONEE 2 - NUREG-0737 III.A.1.2, EMERGENCY OPERATIONS FACILITY		1	09/30/84
46324	OCONEE 2 - NUREG-0737 III.A.2.2, METEOROLOGICAL DATA UPGRADE		1	09/30/84
46464	OCONEE 2 - NUREG-0737 III.D.3.4, CONTROL ROOM HABITABILITY		1	09/30/84
45200	OCONEE 2 - NUREG-0737 II.K.2.13, THERMAL-MECHANICAL REPORT		1	09/30/84

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CONDENSED MANAGEMENT REPORT

(CONTINUATION)

OCONEE 2

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>IMI ACTIONS</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>ACTIVE ACTIONS</u>		(CONTINUATION)		<u>TARGET</u>
44601	OCONEE 2 - RV AND SV TESTING		1	10/30/84
44252	OCONEE 2 - NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES		1	11/30/84
44322	OCONEE 2 - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES		1	11/30/84
48185	OCONEE 2 - NUREG 0737 II.K.3.31 COMPLIANCE WITH 10 CFR 50.46		1	09/30/85
<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				<u>COMPLETE</u>
52312	OCONEE 2 - TECH SPECS FOR CYCLE 7 RELOAD		1	11/23/83
51036	OCONEE 2 - EXEMPTION TO MULTIPLE CRISIS MANAGEMENT CENTER STAFF DRILLS			01/06/84
48955	OCONEE 2 - ISI RELIEF REQUEST OF 10/4/82			02/14/84
52613	OCONEE 2 REQUALIFICATION PROGRAM		2	03/15/84
<u>ACTIVE ACTIONS</u>				<u>TARGET</u>
48226	OCONEE 2 - EFW TORNADO PROTECTION		2	
53342	OCONEE 3 - ISI/IST EXEMPTION			02/28/84
52650	OCONEE 2 - FIRE PROTECTION EXEMPTION REQUEST (LIGHTING)			03/15/84
49569	OCONEE 2 - RCS LEAK TEST T.S. CHANGES		1	04/15/84
51895	OCONEE 2 - HYDROGEN ANALYZER PENETRATION T.S. CHANGES		1	04/15/84
51898	OCONEE 2 - B & PV CODE T.S. CHANGES		1	04/15/84
52407	OCONEE 2 - ISI RELIEF REQUEST OF 8/15/83		2	04/20/84
54145	OCONEE 2 - OVERTIME LIMITS			05/01/84
52675	OCONEE 2 - FIRE PROTECTION EXEMPTION (BARRIER/CABLE)			05/04/84
54207	OCONEE 2 - ECCW TECH. SPEC. CHANGE			05/18/84
54212	OCONEE 2 - ADMIN. T.S. CHANGE OF 2/13/84			05/31/84
52202	OCONEE 2 - CONFIRMATORY ORDER AMEND. REQUEST 8/26/83		1	05/31/84
51050	OCONEE 2 - STARTUP TRANSFORMERS AND ALLOWABLE DEGRADED CONDITIONS		2	05/31/84
49828	OCONEE 2 - SMALL BREAK LOCA PROCEDURES AND MAINTAINING PROPER SG LEVEL		2	06/01/84
55006	OCONEE UNIT 2 - LETTER ON SNUBBER SURVEILLANCE SCHEDULE			06/15/84
51777	OCONEE 2 - EXEMPTION FOR LOCATION OF EOF		1	06/29/84
55013	OCONEE UNIT 2 - WASTE GAS HOLDUP TANKS TECH. SPEC. REVIEW			07/16/84
55016	OCONEE UNIT 2 - TECH. SPEC. REVIEW OF "REPORTABLE OCCURRENCE" REQS			07/31/84
49972	OCONEE 2 - STA BACHELOR'S DEGREE OR EQUIVALENT			08/15/84
54403	OCONEE 2 - RCS VENTS TECHNICAL SPECIFICATIONS NUREG 073 ITEM II.B.1			09/30/84
54552	OCONEE 2 - TECHNICAL SPECIFICATIONS COVERED BY GENERIC LETTERS 83-36, 83-37 FOR NUREG-0737			09/30/84
49235	OCONEE 2 - IST PROGRAM REVISION AND REQUEST FOR RELIEF		1	10/10/84
<u>ANTICIPATED ACTIONS</u>				<u>INITIATION</u>
53613	OCONEE 2 - ITEM 1.2 - POST TRIP REVIEW-DATA AND INFORMATION CAPABILITY			11/01/84
53696	OCONEE 2 - ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS			11/01/84
53779	OCONEE 2 - ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS			11/01/84
53862	OCONEE 2 - ITEM 3.2.3 - POST-MAINTENANCE TESTING -CHANGES TO TECH. SPEC. - ALL SR COMPONENTS			11/01/84
53924	-ITEMS 4.2.3 & 4.2.4-PREVENTATIVE MAINT. PROGRAM FOR REACTOR TRIP BREAKERS-LIFE TESTING & REPLACEMENT			11/01/84
53956	OCONEE 2 - ITEM 4.4 - IMPROVEMENTS IN MAINTENANCE AND TEST PROCEDURES FOR B & W PLANTS			11/01/84
54006	OCONEE 2 - ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS			11/01/84
54089	OCONEE 2 - ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL TESTING - DIVERSE TRIP FEATURES			11/01/84

DATA AS OF - 04/30/84

CONDENSED MANAGEMENT REPORT

FACILITY: OCONEE 3

PLANT LOCATION: 30 MI W OF GREENVILLE, SC
 DOCKET NUMBER: 050-00287
 ARCH/ENGINEER: DUKE/BECH
 IE INSPECTOR: J. BRYANT

LICENSED POWER: 2568 MWT
 DESIGN POWER: 0887 MWE
 NSSS VENDCR: B&W

PROJECT MANAGER: H. NICOLARAS
 BRANCH CHIEF: J. STOLZ
 LIC. ASSISTANT: R. INGRAM

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>MULTI-PLANT</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
08121	OCONEE 3 - APPENDIX I-ALARA		3	<u>COMPLETE</u> 01/16/84
12325	OCONEE 3 - HELB AND CONSEQUENTIAL SYSTEM FAILURE OCONEE 3		3	01/31/84
<u>ACTIVE ACTIONS</u>				
47185	OCONEE 3 NATURAL CIRCULATION COOLDOWN (GENERIC LTR 81-21)		2	<u>TARGET</u>
52781	OCONEE 3 - ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION AND PROCEDURES			
52862	OCONEE 3 - ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RTS COMPONENTS			
52943	OCONEE 3 - ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING PROCEDURES/VENDOR RECOMMENDATIONS - RTS COMPONENTS			
53024	OCONEE 3-ITEM 3.1.3 -POST-MAINTENANCE TESTING-CHANGES TO TECH SPECS-RTS COMPONENTS			
53085	OCONEE 3 - REACTOR TRIP SYSTEM RELIABILITY- VENDOR RELATED MODIFICATIONS			
53138	OCONEE 3 - ITEMS 4.2.1 & 4.2.2 - PREVENTATIVE MAINTENANCE PROGRAM FOR REACTOR TRIP BREAKERS - MAINTENANCE & TRENDING			
53185	OCONEE 3 - ITEM 4.3 - AUTOMATIC AUTUATION OF SHUNT TRIP ATTACHMENT FOR W AND B & W PLANTS			
51114	OCONEE 3 - INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)		2	06/15/83
49746	OCONEE 3 - NUREG 0737 TECH SPECS (GENERIC LTR 82-16)		2	05/31/84
42906	OCONEE 3 - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11		2	05/31/84
49661	OCONEE 3 - REACTOR COOLANT PUMP TRIP			07/01/84
43645	OCONEE 3 - SEISMIC QUALIFICATION OF THE AUXILIARY FEEDWATER SYSTEM		1	09/30/84
42511	OCONEE 3 - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21)		1	03/31/85
52252	OCONEE 3 - CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPA#C-10)		2	09/30/85
<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>TMI ACTIONS</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
44111	OCONEE 3 - NUREG-0737 I.A.1.3.1, SHIFT MANNING OVERTIME LIMITS			<u>COMPLETE</u> 10/28/83
44392	OCONEE 3 - NUREG-0737 II.B.1, RCS HIGH POINT VENTS		1	11/02/83
44182	OCONEE 3 - NUREG-0737 I.A.2.1.4, UPGRADING OF RO AND SRO TRAINING		1	11/18/83
44532	OCONEE 3 - NUREG-0737 II.B.4.1, TRAINING FOR MITIGATING CORE		1	11/18/83
45272	OCONEE 3 - NUREG-0737 II.K.3.1, AUTO PORV ISOLATION		1	11/22/83
45318	OCONEE 3 - NUREG-0737 II.K.3.2, REPORT ON PORV FAILURES		1	11/22/83
45154	OCONEE 3 - NUREG-0737 II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION		1	11/23/83
44467	OCONEE 3 - NUREG-0737 II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS		1	01/11/84
<u>ACTIVE ACTIONS</u>				
51184	OCONEE 3 - I.D.1 CONTROL ROOM DESIGN REVIEW		2	06/15/84
51264	OCONEE 3 - I.D.2. SAFETY PARAMETER DISPLAY SYSTEM		2	06/15/84
45201	OCONEE 3 - NUREG-0737 II.K.2.13, THERMAL-MECHANICAL REPORT		1	09/30/84
45847	OCONEE 3 - NUREG-0737 II.K.3.30, SB LOCA OUTLINE		1	09/30/84
45966	OCONEE 3 - NUREG-0737 III.A.1.2, TECHNICAL SUPPORT CENTER		1	09/30/84
46037	OCONEE 3 - NUREG-0737 III.A.1.2, OPERATIONAL SUPPORT CENTER		1	09/30/84
46109	OCONEE 3 - NUREG-0737 III.A.1.2, EMERGENCY OPERATIONS FACILITY		1	09/30/84
46325	OCONEE 3 - NUREG-0737 III.A.2.2, METEOROLOGICAL DATA UPGRADE		1	09/30/84
46465	OCONEE 3 - NUREG-0737 III.D.3.4, CONTROL ROOM HABITABILITY		1	09/30/84
46602	OCONEE 3 - RV AND SV TESTING		1	10/30/84

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CONDENSED MANAGEMENT REPORT

(CONTINUATION)

OCONEE 3

TAC #	TAC DESCRIPTION	TMI ACTIONS	PRIORITY	CRITICAL DATE
<u>ACTIVE ACTIONS</u> (CONTINUATION)				
44253	OCONEE 3 - NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES		1	TARGET 11/30/84
44323	OCONEE 3 - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES		1	11/30/84
48186	OCONEE 3 -NUREG 0737 II.K.3.31 COMPLIANCE WITH 10 CFR 50.46		1	09/30/85

TAC #	TAC DESCRIPTION	PLANT SPECIFIC	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				
51034	OCONEE 3 - EXEMPTION TO MULTIPLE CRISIS MANAGEMENT CENTER STAFF DRILLS			COMPLETE 01/06/84
52447	OCONEE 3 - APSR POSITION LIMITS		1	01/30/84
48956	OCONEE 3 - ISI RELIEF REQUEST OF 10/4/82			02/14/84
52614	OCONEE 3 REQUALIFICATION PROGRAM		2	03/15/84

TAC #	TAC DESCRIPTION	PLANT SPECIFIC	PRIORITY	CRITICAL DATE
<u>ACTIVE ACTIONS</u>				
48227	OCONEE 3 - EFW TORNADO PROTECTION		2	TARGET 02/28/84
53343	OCONEE 2 - ISI/IST EXEMPTION			03/15/84
52651	OCONEE 3 - FIRE PROTECTION EXEMPTION REQUEST (LIGHTING)			04/15/84
49570	OCONEE 3 - RCS LEAK TEST T.S. CHANGES		1	04/15/84
51896	OCONEE 3 - HYDROGEN ANALYZER PENETRATION T.S. CHANGES		1	04/15/84
51899	OCONEE 3 - B & PV CODE T.S. CHANGES		1	04/15/84
52408	OCONEE 3 - ISI RELIEF REQUEST OF 8/15/83		2	04/20/84
54205	OCONEE 3 - TECH SPECS FOR CYCLE & RELOAD			04/30/84
54146	OCONEE 3 - OVERTIME LIMITS			05/01/84
52676	OCONEE 3 - FIRE PROTECTION EXEMPTION (BARRIER/CABLE)			05/04/84
54208	OCONEE 3 - ECCW TECH. SPEC. CHANGE			05/18/84
54213	OCONEE 3 - ADMIN. T.S. CHANGE OF 2/13/84			05/31/84
52203	OCONEE 3 - CONFIRMATORY ORDER AMEND. REQUEST 8/26/83		1	05/31/84
51051	OCONEE 3 - STARTUP TRANSFORMERS AND ALLOWABLE DEGRADED CONDITIONS		2	05/31/84
49829	OCONEE 3- SMALL BREAK LOCA PROCEDURES AND MAINTAINING PROPER SG LEVEL		2	06/01/84
51778	OCONEE 3 - EXEMPTION FOR LOCATION OF EOF		1	06/29/84
55014	OCONEE UNIT 3 - WASTE GAS HOLDUP TANKS TECH. SPEC. REVIEW			07/16/84
55017	OCONEE UNIT 3 - TECH. SPEC. REVIEW OF "REPORTABLE OCCURRENCE" REQS			07/31/84
49978	OCONEE 3 - STA BACHELOR'S DEGREE OR EQUIVALENT			08/15/84
54404	OCONEE 3- RCS VENTS TECHNICAL SPECIFICATIONS NUREG 073 ITEM II.B.1			09/30/84
54553	OCONEE 3 - TECHNICAL SPECIFICATIONS COVERED BY GENERIC LETTERS 83-36, 83-37 FOR NUREG-0737			09/30/84
49236	OCONEE 3 - IST PROGRAM REVISION AND REQUEST FOR RELIEF		1	10/10/84

TAC #	TAC DESCRIPTION	PLANT SPECIFIC	PRIORITY	CRITICAL DATE
<u>ANTICIPATED ACTIONS</u>				
53614	OCONEE 3 - ITEM 1.2 - POST TRIP REVIEW-DATA AND INFORMATION CAPABILITY			INITIATION 11/01/84
53697	OCONEE 3 - ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS			11/01/84
53780	OCONEE 3- ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS			11/01/84
53863	OCONEE 3 - ITEM 3.2.3 - POST-MAINTENANCE TESTING -CHANGES TO TECH. SPEC. - ALL SR COMPONENTS			11/01/84
53925	-ITEMS 4.2.3 & 4.2.4-PREVENTATIVE MAINT. PROGRAM FOR REACTOR TRIP BREAKERS-LIFE TESTING & REPLACEMENT			11/01/84
53957	OCONEE 3 - ITEM 4.4 - IMPROVEMENTS IN MAINTENANCE AND TEST PROCEDURES FOR B & W PLANTS			11/01/84
54007	OCONEE 3 - ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS			11/01/84
54090	OCONEE 3 - ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL TESTING - DIVERSE TRIP FEATURES			11/01/84

DATA AS OF - 04/30/84

CONDENSED MANAGEMENT REPORT

FACILITY: OYSTER CREEK 1

PLANT LOCATION: 9 MI S OF TOMS RIVER, NJ
 DOCKET NUMBER: 050-00219
 ARCH/ENGINEER: B&R
 IE INSPECTOR: C. COWGILL

LICENSED POWER: 1930 MWT
 DESIGN POWER: 0650 MWE
 NSSS VENDOR: GE

PROJECT MANAGER: J. LOMBARDO
 BRANCH CHIEF: D. CRUTCHFIELD
 LIC. ASSISTANT: H. SMITH

TAC #	TAC DESCRIPTION	MULTI-PLANT	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				
07944	OYSTER CREEK - MARK I CONTAINMENT LONG TERM PROGRAM IMPLEMENTATION		1	COMPLETE 01/13/84
<u>ACTIVE ACTIONS</u>				
52782	OYSTER CREEK 1 - ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION AND PROCEDURES			TARGET
52863	OYSTER CREEK 1 - ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RTS COMPONENTS			
52944	OYSTER CREEK 1 - ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING PROCEDURES/VENDOR RECOMMENDATIONS - RTS COMPONENTS			
53025	OYSTER CREEK 1-ITEM 3.1.3 -POST-MAINT TESTING-CHANGES TO TECH SPECS-RTS COMPONENTS			
51115	OYSTER CREEK 1 - INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)			05/01/84
51687	OYSTER CREEK NUREG 0737 TECH SPECS			06/01/84
08440	OYSTER CREEK - HYDRAULIC SNUBBERS		3	06/01/84
42524	OYSTER CREEK - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21)		1	06/01/84
42613	OYSTER CREEK - LONG TERM REVIEW CONTAINMENT PURGE AND VENT (B-24)		1	06/01/84
08439	OYSTER CREEK - MECHANICAL SNUBBERS		2	06/01/84
46668	OYSTER CREEK - IMPLEMENTATION OF NUREG-0313, REV. 1		2	06/30/84
08100	OYSTER CREEK - APPENDIX I TECH SPEC IMPLEMENTATION REVIEW		3	08/15/84
43717	OYSTER CREEK - SAFETY CONCERNS ASSOCIATED WITH PIPE BREAKS IN THE BWR SCRAM SYSTEM		2	10/01/84
42914	OYSTER CREEK - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11		2	11/01/84
52253	OYSTER CREEK 1 - CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPA#C-10)			09/30/85
<u>TAC # TAC DESCRIPTION TMI ACTIONS PRIORITY CRITICAL DATE</u>				
<u>COMPLETED ACTIONS</u>				
44112	OYSTER CREEK - NUREG-0737 I.A.1.3.1, SHIFT MANNING OVERTIME LIMITS			COMPLETE 11/28/83
45848	OYSTER CREEK - NUREG-0737 II.K.3.30, SB LOCA OUTLINE		1	12/21/83
48187	OYSTER CREEK -NUREG 0737 II.K.3.31 COMPLIANCE WITH 10 CFR 50.46		1	12/21/83
45802	OYSTER CREEK - NUREG-0737 II.K.3.29, PERFORMANCE OF ISOLATION CONDENSER		1	02/01/84
<u>ACTIVE ACTIONS</u>				
47960	OYSTER CREEK - PLANT SHIELDING MODIFICATIONS (II.B.2.2)		1	06/01/84
46466	OYSTER CREEK - NUREG-0737 III.D.3.4, CONTROL ROOM HABITABILITY		1	06/01/84
46326	OYSTER CREEK - NUREG-0737 III.A.2.2, METEOROLOGICAL DATA UPGRADE		1	08/01/84
44254	OYSTER CREEK - NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES		1	08/01/84
44324	OYSTER CREEK - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES		1	08/01/84
44462	OYSTER CREEK - NUREG-0737 II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS		1	08/01/84
44603	OYSTER CREEK 1 - RV AND SV TESTING		1	08/01/84
45155	OYSTER CREEK - NUREG-0737 II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION		1	08/01/84
45600	OYSTER CREEK - NUREG-0737 II.K.3.16, CHALLENGES AND FAILURES OF RELIEF VALVES		1	08/01/84
45696	OYSTER CREEK - NUREG-0737 II.K.3.18, ADS ACTUATION STUDY		1	08/01/84
45720	OYSTER CREEK - NUREG-0737 II.K.3.19, INTERLOCK RECIRC PUMP MODIFICATION		1	08/01/84
51185	OYSTER CREEK 1 - I.D.1 CONTROL ROOM DESIGN REVIEW			08/01/84
51265	OYSTER CREEK 1 - I.D.2. SAFETY PARAMETER DISPLAY SYSTEM			08/01/84
45967	OYSTER CREEK - NUREG-0737 III.A.1.2, TECHNICAL SUPPORT CENTER		1	09/30/84
46038	OYSTER CREEK - NUREG-0737 III.A.1.2, OPERATIONAL SUPPORT CENTER		1	09/30/84
46110	OYSTER CREEK - NUREG-0737 III.A.1.2, EMERGENCY OPERATIONS FACILITY		1	09/30/84

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CONDENSED MANAGEMENT REPORT

(CONTINUATION)

OYSTER CREEK 1

TAC #	TAC DESCRIPTION	PLANT SPECIFIC	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				
51304	OYSTER CREEK - REVIEW LICENSEE'S ADMINISTRATIVE ORGANIZATION CHANGE REQUEST TO THE T.S.		2	<u>COMPLETE</u> 01/12/84
51726	OYSTER CREEK - REVIEW REQUEST TO DEFER REPLACEMENT OF CORE SPRAY SPARGER FROM CYCLE 10 TO CYCLE 11 REFUELING OUTAGE			01/26/84
52460	OYSTER CREEK - REVIEW TS CHANGE REQUEST TO THE NEUTRON MONITORING SYSTEM			01/30/84
51402	OYSTER CREEK - REVIEW REQUEST FOR AMENDMENT TO ISOLATION CONDENSER TECH. SPECS.		1	02/06/84
43588	OYSTER CREEK - NEW CORE SPRAY SPARGER DESIGN AND PIPING MODIFICATIONS		1	02/17/84
49393	OYSTER CREEK - HYDROSTATIC LOADS			02/23/84
52337	OYSTER CREEK - REVIEW REQUEST FOR APPROVAL OF PROPOSED DISPOSAL METHODS			03/08/84
<u>ACTIVE ACTIONS</u>				
51314	OYSTER CREEK - REVIEW OF NO JET PUMP CORE SPRAY PERFORMANCE		1	<u>TARGET</u> 06/01/84
52461	OYSTER CREEK - REVIEW FIRE PROTECTION EXEMPTION REQUESTS		2	06/01/84
48787	OYSTER CREEK - REVIEW REQUEST FOR INCREASED SPENT FUEL STORAGE CAPACITY		1	06/01/84
42646	OYSTER CREEK - GENERAL ELECTRIC RELOAD FUEL APPLICATION		1	06/01/84
11270	OYSTER CREEK - INSERVICE TESTING (IST)			06/15/84
52517	OYSTER CREEK - REVIEW TS CHANGE REQUEST FOR SECONDARY CONTAINMENT INTEGRITY			06/15/84
52518	OYSTER CREEK - REVIEW TS CHANGE REQUEST FOR HIGH DRYWELL PRESSURE TRIP SETPOINT			06/15/84
52519	OYSTER CREEK - REVIEW TS CHANGE REQUEST FOR SCRAM SYSTEM RELIABILITY			06/15/84
52549	OYSTER CREEK - REVIEW TS CHANGE REQUEST FOR ISI			07/01/84
52336	OYSTER CREEK - REVIEW FINAL ENVIRONMENTAL STATEMENT (FES) FOR POL/FTL CONVERSION			08/01/84
52516	OYSTER CREEK - REVIEW TS CHANGE REQUEST FOR GRID UNVOLTAGE PROTECTITON SYSTEM AND STATION BATTERIES		1	08/01/84
49410	OYSTER CREEK - BATTERY STATUS ALARMS		1	08/01/84
49411	OYSTER CREEK - VENTILATION SYSTEMS		1	08/01/84
49412	OYSTER CREEK - PRIMARY COOLANT ACTIVITY		1	08/01/84
49413	OYSTER CREEK - MAIN STEAM ISOLATION VALVE MAINT.		1	08/01/84
49496	OYSTER CREEK - FINAL RULE ON INTERIM REQUIREMENTS RELATED TO H2 CONTROL - REVIEW OF EXEMPTION REQUEST		1	08/01/84
53411	OYSTER CREEK - REVIEW ADMINISTRATIVE TS CHANGE REQUEST FOR CLARITY & FORMAT ONLY		1	08/01/84
49392	OYSTER CREEK - WIND LOADS		1	08/01/84
49394	OYSTER CREEK - TORNADO MISSILE DAMAGE		1	08/01/84
49395	OYSTER CREEK - TURBINE MISSILE		1	08/01/84
49397	OYSTER CREEK - EMERGENCY CONDENSER ISOLATION		1	08/01/84
49398	OYSTER CREEK - SEISMIC DESIGN		1	08/01/84
49399	OYSTER CREEK - DESIGN CODES AND STANDARDS		1	08/01/84
49400	OYSTER CREEK - THERMAL OVERLOAD		1	08/01/84
49401	OYSTER CREEK - LEAKAGE DETECTION		1	08/01/84
49404	OYSTER CREEK - WATER PURITY SPECIFICATION		1	08/01/84
49405	OYSTER CREEK - EMERGENCY CONDENSER LOGIC TESTING		1	08/01/84
49406	OYSTER CREEK - BUS TRANSFERS		1	08/01/84
49407	OYSTER CREEK - RPS TESTING		1	08/01/84
49408	OYSTER CREEK - NEUTRON MONITORING ISOLATION		1	08/01/84
11268	OYSTER CREEK - FULL TERM OPERATING LICENSE			09/30/84
54554	OYSTER CREEK 1 - TECHNICAL SPECIFICATIONS COVERED BY GENERIC LETTERS 83-36, 83-37 FOR NUREG-0737			09/30/84
<u>ANTICIPATED ACTIONS</u>				
53615	OYSTER CREEK 1 - ITEM 1.2 - POST TRIP REVIEW-DATA AND INFORMATION CAPABILITY			<u>INITIATION</u> 11/01/84
53698	OYSTER CREEK 1 - ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS			11/01/84
53781	OYSTER CREEK 1 - ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS			11/01/84

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CONDENSED MANAGEMENT REPORT

(CONTINUATION)

OYSTER CREEK 1

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>ANTICIPATED ACTIONS (CONTINUATION)</u>				
53864	OYSTER CREEK 1 - ITEM 3.2.3 - POST-MAINTENANCE TESTING -CHANGES TO TECH. SPEC. - ALL SR COMPONENTS			<u>INITIATION</u> 11/01/84
54008	OYSTER CREEK 1 - ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS			11/01/84
54091	OYSTER CREEK 1 - ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL TESTING - DIVERSE TRIP FEATURES			11/01/84

DATA AS OF - 04/30/84

CONDENSED MANAGEMENT REPORT

FACILITY: PALISADES

PLANT LOCATION: 5 MI S OF SOUTH HAVEN, MI
 DOCKET NUMBER: 050-00255
 ARCH/ENGINEER: BECH
 IE INSPECTOR: B. JORGENSEN

LICENSED POWER: 2530 MWT
 DESIGN POWER: 0805 MWE
 NSSS VENDOR: COMB

PROJECT MANAGER: W. PAULSON
 BRANCH CHIEF: D. CRUTCHFIELD
 LIC. ASSISTANT: H. SMITH

TAC #	TAC DESCRIPTION	MULTI-PLANT	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				
47129	PALISADES - CONTROL OF HEAVY LOADS AT NUCLEAR POWER PLANTS (NUREG 0612)		2	11/09/83
47186	PALISADES NATURAL CIRCULATION COOLDOWN (GENERIC LTR 81-21)		2	12/15/83
49747	PALISADES - NUREG 0737 TECH SPECS (GENERIC LTR 82-16)		3	03/02/84
42764	PALISADES - ESF RESET CONTROL DESIGN DEFICIENCY - I&E BULLETIN 80-06.		2	03/07/84
46850	PALISADES - PWR MSLB WITH CONTINUED FEEDWATER ADDITION		2	04/11/84
42614	PALISADES - LONG TERM REVIEW CONTAINMENT PURGE AND VENT (B-24)		1	04/18/84
<u>TARGET</u>				
<u>ACTIVE ACTIONS</u>				
51116	PALISADES - INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)		1	06/30/84
42517	PALISADES - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21)		2	06/30/84
42915	PALISADES - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11		3	06/30/84
06779	PALISADES - FILTER TECH SPECS			07/01/84
49700	PALISADES - REACTOR COOLANT PUMP TRIP		3	07/15/84
08148	PALISADES - APPENDIX I TECH SPEC IMPLEMENTATION REVIEW		2	07/15/84
42130	PALISADES - DECAY HEAT REMOVAL CAPABILITY TECH SPECS		2	07/31/84
43052	PALISADES - TECHNICAL SPECIFICATIONS DEFINING OPERABILITY FOR SAFETY SYSTEMS		2	08/30/84
08621	PALISADES - REVIEW OF ASYMMETRIC LOCA LOADS SUBMITTAL			12/31/84
53139	PALISADES - ITEMS 4.2.1 & 4.2.2 - PREVENTATIVE MAINTENANCE PROGRAM FOR REACTOR TRIP BREAKERS - MAINTENANCE & TRENDING			12/31/84
52254	PALISADES - CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPA#C-10)			12/31/84
52783	PALISADES - ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION AND PROCEDURES			12/31/84
52864	PALISADES - ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RTS COMPONENTS			12/31/84
52945	PALISADES - ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING PROCEDURES/VENDOR RECOMMENDATIONS - RTS COMPONENTS			12/31/84
53026	PALISADES-ITEM 3.1.3 -POST-MAINTENANCE TESTING-CHANGES TO TECH SPECS-RTS COMPONENTS			12/31/84
53086	PALISADES - REACTOR TRIP SYSTEM RELIABILITY - VENDOR RELATED MODIFICATIONS			12/31/84
<u>TM1 ACTIONS</u>				
<u>COMPLETED ACTIONS</u>				
47810	PALISADES - NUREG 0737 ITEM II K2.17 POTENTIAL FOR VOIDING IN RCS		1	10/19/83
45265	PALISADES - NUREG-0737 II.K.3.1, AUTO PORV ISOLATION		1	10/20/83
45311	PALISADES - NUREG-0737 II.K.3.2, REPORT ON PORV FAILURES		1	10/20/83
44394	PALISADES - NUREG-0737 II.B.1, RCS HIGH POINT VENTS		1	11/07/83
44873	PALISADES - NUREG-0737 II.E.4.2, CONTAINMENT ISOLATION DEPENDABILITY		1	04/18/84
<u>TARGET</u>				
<u>ACTIVE ACTIONS</u>				
44604	PALISADES - RV AND SV TESTING		1	05/01/84
44463	PALISADES - NUREG-0737 II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS		1	06/30/84
45156	PALISADES - NUREG-0737 II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION		1	06/30/84
45849	PALISADES - NUREG-0737 II.K.3.30, JB LOCA OUTLINE		1	06/30/84
46327	PALISADES - NUREG-0737 III.A.2.2, METEOROLOGICAL DATA UPGRADE		1	06/30/84
48188	PALISADES -NUREG 0737 II.K.3.31 COMPLIANCE WITH 10 CFR 50.46		1	06/30/84
44666	PALISADES - NUREG-0737 II.E.1.1, AFW SYSTEM EVALUATION		1	07/31/84
46896	PALISADES - II K2.13 THERMAL MECHANICAL REPORT		1	09/30/84

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CONDENSED MANAGEMENT REPORT

(CONTINUATION)

PALISADES

TAC #	TAC DESCRIPTION	TMI ACTIONS	PRIORITY	CRITICAL DATE
<u>ACTIVE ACTIONS</u> (CONTINUATION)				
45968	PALISADES - NUREG-0737 III.A.1.2, TECHNICAL SUPPORT CENTER		1	TARGET 09/30/84
46039	PALISADES - NUREG-0737 III.A.1.2, OPERATIONAL SUPPORT CENTER		1	09/30/84
46111	PALISADES - NUREG-0737 III.A.1.2, EMERGENCY OPERATIONS FACILITY		1	09/30/84
48448	PALISADES - NUREG 0737, II.K.3.25 POWER ON PUMP SEALS			09/30/84
44255	PALISADES - NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES		1	11/30/84
44325	PALISADES - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES		1	11/30/84
51186	PALISADES - I.D.1 CONTROL ROOM DESIGN REVIEW			06/30/85
51266	PALISADES - I.D.2. SAFETY PARAMETER DISPLAY SYSTEM			06/30/85
<u>COMPLETED ACTIONS</u>				
49858	PALISADES - INTEGRATED ASSESSMENT SUPPLEMENT (POST SEP)			COMPLETE 11/07/83
52532	PALISADES - REVISIONS TO GUARD TRAIN. PLAN			11/07/83
52290	PALISADES - T.S. CHANGE R.V. SURVEILLANCE SPECIMEN SCHEDULE			02/28/84
53543	PALISADES - STEAM GENERATOR INSPECTION AND REPAIR			02/28/84
54298	PALISADES - VARIANCE FROM THE DISPLAY CRITERION FOR NOBLE GAS EFFLUENT MONITOR (II.F.J.1)			03/07/84
54477	PALISADES - PROPOSED T/S CHANGES - REVISED ACCIDENT & TRANSIENT ANALYSES DUE TO STEAM GENERATOR TUBE PLUGGING			04/12/84
<u>ACTIVE ACTIONS</u>				
54844	PALISADES - APPENDIX R EXEMPTION REQUEST			TARGET 05/25/84
54811	PALISADES - ANALYSIS OF AXIAL POWER DISTRIBUTION LIMITS (XN-NF-78-16, SUPPLEMENT 1)			05/31/84
53507	PALISADES - EXEMPTION REQUEST FROM REQUIREMENT TO PERFORM AIR LOCK LEAK TEST AT PA			05/31/84
54166	PALISADES - DELETE SPARE HPSI PUMP FROM SECTION 5.33 OF TECH. SPECS.			05/31/84
54269	PALISADES - JUSTIFICATION OF XNE CORRELATION FOR PALISADES			05/31/84
11218	PALISADES - FULL TERM OPERATING LICENSE		1	05/31/84
52511	PALISADES - CHANGE IN BASIS FOR LIMITING SAFETY SYSTEM SETTINGS BASED ON REVISED ROD WITHDRAWAL ANALYSIS			05/31/84
48725	PALISADES - SEISMIC REVIEW OF PIPING <2 1/2" & ELECTRICAL COMPONENTS			06/30/84
49327	PALISADES PLANT - PROPOSED TECHNICAL SPECIFICATION CHANGE REQUEST: SECTION 6 ADMIN CONTROLS		3	06/30/84
46574	PALISADES - DISCONNECTION OF BOTH REDUNDANT BATTERIES FROM DC BUSES		1	06/30/84
42320	PALISADES - RELIEF REQUEST FOR INSERVICE PUMP TESTING PROGRAM			06/30/84
53557	PALISADES - REQUEST TO DELETE REQUIREMENTS OF NOVEMBER 9 1979 ORDER FOR MODIFICATION OF LICENSE			06/30/84
54795	PALISADES - ADDITION OF 408V DISTRIBUTION BUSES - TECH SPEC. CHANGE			07/15/84
53508	PALISADES - STEAM GENERATOR INSPECTION AND REPAIR			07/31/84
53545	PALISADES - TECH. SPEC. CHANGE OPERABILITY OF LEAKAGE		1	07/31/84
43269	PALISADES - ISI UPDATE SEC. XI 77578 ADDENDA DATED 11/12/80			07/31/84
48726	PALISADES - DESIGN CODE ASSESSMENT			07/31/84
54843	PALISADES - EXEMPTION REQUEST - COUNTY AND STATE PARTICIPATION IN ANNUAL EMERGENCY EXERCISE			08/03/84
54555	PALISADES - TECHNICAL SPECIFICATIONS COVERED BY GENERIC LETTERS 83-36, 83-37 FOR NUREG-0737			09/30/84
54405	PALISADES - RCS VENTS TECHNICAL SPECIFICATIONS NUREG 073 ITEM II.B.1			09/30/84
07733	PALISADES - HYDROGEN RECOMBINER TESTING REQUIREMENTS		1	09/30/84
52490	PALISADES - MSIV COST/BENEFIT ANALYSIS OF TWO STEAM GENERATOR BLOWDOWN			10/30/84
<u>ANTICIPATED ACTIONS</u>				
55007	PALISADES - HYDROTEST RELIEF REQUEST			INITIATION 05/14/84
53616	PALISADES - ITEM 1.2 - POST TRIP REVIEW-DATA AND INFORMATION CAPABILITY			11/01/84
53699	PALISADES - ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR VENDOR INTERFACE - ALL SR COMPONENTS			11/01/84
53782	PALISADES - ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS			11/01/84

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CONDENSED MANAGEMENT REPORT

(CONTINUATION)

PALISADES

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>ANTICIPATED ACTIONS (CONTINUATION)</u>				
53865	PALISADES - ITEM 3.2.3 - POST-MAINTENANCE TESTING -CHANGES TO TECH. SPEC. - ALL SR COMPONENTS			<u>INITIATION</u> 11/01/84
53926	-ITEMS 4.2.3 & 4.2.4-PREVENTATIVE MAINT. PROGRAM FOR REACTOR TRIP BREAKERS-LIFE TESTING & REPLACEMENT			11/01/84
54009	PALISADES - ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS			11/01/84
54092	PALISADES - ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL TESTING - DIVERSE TRIP FEATURES			11/01/84

DATA AS OF - 04/30/84

CONDENSED MANAGEMENT REPORT

FACILITY: PEACH BOTTOM 2

PLANT LOCATION: 19 MI S OF LANCASTER, PA
 DOCKET NUMBER: 050-00277
 ARCH/ENGINEER: BECH
 IE INSPECTOR: A. BLOUGH

LICENSED POWER: 3293 MWT
 DESIGN POWER: 1065 MWE
 NSSS VENDOR: GE

PROJECT MANAGER: G. GEARS
 BRANCH CHIEF: J. STOLZ
 LIC. ASSISTANT: R. INGRAM

TAC #	TAC DESCRIPTION	MULTI-PLANT	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				
08503	PEACH BOTTOM 2 - CONTAINMENT LEAK TESTING - APP. J (GENERIC)		2	<u>COMPLETE</u> 03/01/84
07945	PEACH BOTTOM 2 - MARK I CONTAINMENT LONG TERM PROGRAM IMPLEMENTATION		1	03/06/84
11258	PEACH BOTTOM 2 - IST		1	03/30/84
42605	PEACH BOTTOM 2 - LONG TERM REV CONTAINMENT PURGE AND VENT (B-24)		1	04/25/84
<u>ACTIVE ACTIONS</u>				
52255	PEACH BOTTOM 2 - CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPA#C-10)			<u>TARGET</u>
52784	PEACH BOTTOM 2 - ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION AND PROCEDURES			
52865	PEACH BOTTOM 2 - ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RTS COMPONENTS			
52946	PEACH BOTTOM 2 - ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING PROCEDURES/VENDOR RECOMMENDATIONS - RTS COMPONENTS			
53027	PEACH BOTTOM 2 - ITEM 3.1.3 - POST-MAINT TESTING - CHANGES TO TECH SPECS - RTS COMPONENTS		1	04/30/84
42503	PEACH BOTTOM 2 - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21)		2	04/30/84
08501	PEACH BOTTOM 2 - SURV. OF MECHANICAL SNUBBERS			06/29/84
51688	PEACH BOTTOM 2 - NUREG 0737 TECH SPECS		2	06/29/84
46671	PEACH BOTTOM 2 - IMPLEMENTATION OF NUREG-0313, REV. 1		1	06/30/84
08935	PEACH BOTTOM 2 - RPS POWER SUPPLY		3	06/30/84
08044	PEACH BOTTOM 2 - APPENDIX I TECH SPEC IMPLEMENTATION REVIEW		2	07/02/84
42922	PEACH BOTTOM 2 - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11		2	09/28/84
43728	PEACH BOTTOM 2 - SAFETY CONCERNS ASSOCIATED WITH PIPE BREAKS IN THE BWR SCRAM SYSTEM			12/28/84
51117	PEACH BOTTOM 2 - INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)			
TAC #	TAC DESCRIPTION	TMI ACTIONS	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				
44605	PEACH BOTTOM 2 - RV AND SV TESTING		1	<u>COMPLETE</u> 11/04/83
44664	PEACH BOTTOM 2 - NUREG-0737 II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS		1	02/08/84
45850	PEACH BOTTOM 2 - NUREG-0737 II.K.3.30, SB LOCA OUTLINE		1	02/28/84
45189	PEACH BOTTOM 2 - NUREG 0737 II.K.3.31 COMPLIANCE WITH 10 CFR 50.46		1	02/28/84
<u>ACTIVE ACTIONS</u>				
51187	PEACH BOTTOM 2 - I.D.1 CONTROL ROOM DESIGN REVIEW			<u>TARGET</u>
44874	PEACH BOTTOM 2 - NUREG-0737 II.E.4.2, CONTAINMENT ISOLATION DEPENDABILITY		1	05/31/84
44256	PEACH BOTTOM 2 - NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES		1	06/01/84
44326	PEACH BOTTOM 2 - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES		1	06/01/84
45697	PEACH BOTTOM 2 - NUREG-0737 II.K.3.18, ADS ACTUATION STUDY		1	06/30/84
48275	PEACH BOTTOM 2 - NUREG 0737 II.K.3.28, VERIFY QUALIFICATION OF ACCUMULATORS ON ADS VALVES		1	06/30/84
45969	PEACH BOTTOM 2 - NUREG-0737 III.A.1.2, TECHNICAL SUPPORT CENTER		1	09/30/84
46040	PEACH BOTTOM 2 - NUREG-0737 III.A.1.2, OPERATIONAL SUPPORT CENTER		1	09/30/84
46112	PEACH BOTTOM 2 - NUREG-0737 III.A.1.2, EMERGENCY OPERATIONS FACILITY		1	09/30/84
46328	PEACH BOTTOM 2 - NUREG-0737 III.A.2.2, METEOROLOGICAL DATA UPGRADE		1	09/30/84
45157	PEACH BOTTOM 2 - NUREG-0737 II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION		1	12/14/84
51267	PEACH BOTTOM 2 - I.D.2. SAFETY PARAMETER DISPLAY SYSTEM			12/29/84
45601	PEACH BOTTOM 2 - NUREG-0737 II.K.3.16, CHALLENGES AND FAILURES OF RELIEF VALVES		1	12/30/84

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CONDENSED MANAGEMENT REPORT

(CONTINUATION)

PEACH BOTTOM 2

TAC #	TAC DESCRIPTION	PLANT SPECIFIC	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				
48111	PEACH BOTTOM 1 - TECH SPECS RELATED TO IMPLEMENTATION OF NUREG-0737		3	<u>COMPLETE</u> 10/04/83
52371	PEACH BOTTOM 2 - ANNUAL EXERCISE EXEMPTION REQUEST			11/03/83
51784	PEACH BOTTOM 2 - JCO CONCERNING RHR PIPE CRACKS			11/30/83
51552	PEACH BOTTOM 2 - RECIRCULATION PIPE CRACKS		1	11/30/83
49194	PEACH BOTTOM 2 - GEOLOGICAL ASSESSMENT OF ALLEGED FAULTING AT SITE		3	12/06/83
51385	PEACH BOTTOM 2 - TS FOR PERIODIC LEAK INTEGRITY TESTING			12/12/83
52377	PEACH BOTTOM 2 - ALT. SHUTDOWN CAPABILITY ASSESSMENT		1	01/26/84
48469	PEACH BOTTOM 2 - ROD BLOCK SYSTEM TECH. SPEC. CHANGE		2	02/10/84
49693	PEACH BOTTOM 2 - DELETION OF APP. B WATER QUAL. REG. AND AQUATIC MONITORING		3	02/24/84
51404	PEACH BOTTOM 2 - NOISE MONITORING PROGRAM - APP. B TS AMENDMENT		2	02/24/84
53331	PEACH BOTTOM 2 - ONE-TIME EXEMPTION FROM LOCAL LEAK RATE TEST			02/28/84
49031	PEACH BOTTOM 2 - MK 1 TORUS T.S. CHANGES LONG TERM PROGRAM		2	03/13/84
48874	PEACH BOTTOM 2 - REVIEW OF HEALTH PHYSICS ORGANIZATIONAL CHANGE & GENERAL ORG. CHANGES IN T.S.			03/21/84
51019	PEACH BOTTOM 2 - HPCI STEAM LINE LOW-PRESSURE		2	03/21/84
51354	PEACH BOTTOM 2 - FOUR MISC. TECH SPEC CHANGES		2	03/28/84
48404	PEACH BOTTOM 2 - DEGRADED GRID VOLTAGE TECH. SPECS.			04/11/84
<u>ACTIVE ACTIONS</u>				
54800	PEACH BOTTOM 2: REVIEW OF PROPOSED TECH. SPEC. REQUIREMENTS ON AIR SUPPLY SYSTEM LEAKAGE			<u>TARGET</u>
54823	PEACH BOTTOM 2: REVISED APP. J. TECH. SPECS. REQUEST			
54830	PEACH BOTTOM 2 - TECH. SPEC. CHANGE ON AUDIT OF EMERG. PLAN PROCEDURES			
53351	PEACH BOTTOM 2 - TS CHANGE TO PERMIT TEMP. BYPASS OF HYDROGEN TRIP			04/30/84
53485	PEACH BOTTOM 2 - 2 EXEMPTION REQUEST FROM ISI-CLASS II ECCS			05/01/84
54617	PEACH BOTTOM 2 - EMERGENCY PREPAREDNESS EXERCISE EXEMPTION REQUEST			05/15/84
54367	PEACH BOTTOM 2 - TS CHANGE TO DELETE DRYWELL AIR MONITORING SYSTEM			05/31/84
53273	PEACH BOTTOM 2 - TS CHANGE RELATED TO SDV AND RWCU INSTRUMENTATION			05/31/84
54488	PEACH BOTTOM 2: FIRE BARRIER PENETRATIONS T/S CHANGE			06/22/84
52681	PEACH BOTTOM 2 - TS CHANGE FOR COOLANT LEAKAGE AND I-131 LIMITS			06/30/84
54766	PEACH BOTTOM 2 - OPERABILITY OF PURGE/VENT VALVES			06/30/84
47375	PEACH BOTTOM 2 - SUPPRESSION CHAMBER HIGH LEVEL TRIP SETTING		3	06/30/84
47377	PEACH BOTTOM 2 - SURVEILLANCE REQS FOR THE PCIS/LPC1 INTERLOCK PRESSURE SWITCH		3	06/30/84
47379	PEACH BOTTOM 2 - FUEL OIL SYSTEMS FOR STANDBY DIESEL GENERATORS		3	06/30/84
47537	PEACH BOTTOM 2 - CORE SPRAY PUMP DISCHARGE PRESSURE INTERLOCK		3	06/30/84
47539	PEACH BOTTOM 2 - REACTOR LEVEL SURVEILLANCE INSTRUMENTS		3	06/30/84
51741	PEACH BOTTOM 2 - REVIEW OF APP R, III F AND III M EXEMPTION REQUESTS		2	07/02/84
54793	PEACH BOTTOM 2 - TS CHANGE ON REQUIREMENTS FOR TESTING EMERGENCY BATTERIES			07/20/84
51017	PEACH BOTTOM 2 WETWELL/DRYWELL VACUUM BREAKERS		3	07/30/84
46787	PEACH BOTTOM 2 - TECH SPEC CHANGE RELATED TO VERIF. OF DRYWELL-TO-TORUS VAC BREAKER CLOSURE		3	07/31/84
46789	PEACH BOTTOM 2 - ENVIRONM TECH SPEC CHANGE RELATED TO A REPORTING PERIOD		3	07/31/84
54828	PEACH BOTTOM 2: TECH. SPEC. CHANGES CONCERNING REACTOR WATER LEVEL SETPOINT			08/31/84
54556	PEACH BOTTOM 2 - TECHNICAL SPECIFICATIONS COVERED BY GENERIC LETTERS 83-36, 83-37 FOR NUREG-0737			09/30/84
54359	PEACH BOTTOM 2: REVIEW OF STRUCTURAL STEEL			09/30/84
<u>ANTICIPATED ACTIONS</u>				
53617	PEACH BOTTOM 2 - ITEM 1.2 - POST TRIP REVIEW-DATA AND INFORMATION CAPABILITY			<u>INITIATION</u> 11/01/84
53700	PEACH BOTTOM 2 - ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS			11/01/84
53783	PEACH BOTTOM 2 - ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS			11/01/84
53866	PEACH BOTTOM 2 - ITEM 3.2.3 - POST-MAINTENANCE TESTING -CHANGES TO TECH. SPEC. - ALL SR COMPONENTS			11/01/84

TECHNICAL ASSIGNMENT CONTROL SYSTEM
 CONDENSED MANAGEMENT REPORT
 PEACH BOTTOM 2

DATA AS OF - 04/30/84

(CONTINUATION)

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>ANTICIPATED ACTIONS (CONTINUATION)</u>				
54010	PEACH BOTTOM 2 - ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS			<u>INITIATION</u> 11/01/84
54093	PEACH BOTTOM 2 - ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL TESTING - DIVERSE TRIP FEATURES			11/01/84

DATA AS OF - 04/30/84

CONDENSED MANAGEMENT REPORT

FACILITY: PEACH BOTTOM 3

PLANT LOCATION: 19 MI S OF LANCASTER, PA
 DOCKET NUMBER: 050-00278
 ARCH/ENGINEER: BECH
 IE INSPECTOR: A. BLOUGH

LICENSED POWER: 3293 MW
 DESIGN POWER: 1065 MWE
 NSSS VENDOR: GE

PROJECT MANAGER: G. GEARS
 BRANCH CHIEF: J. STOLZ
 LIC. ASSISTANT: R. INGRAM

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>MULTI-PLANT</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
08504	PEACH BOTTOM 3 - CONTAINMENT LFAK TESTING - APP. J (GENERIC)		2	<u>COMPLETE</u> 03/01/84
07946	PEACH BOTTOM 3 - MARK I CONTAINMENT LONG TERM PROGRAM IMPLEMENTATION		1	03/06/84
11257	PEACH BOTTOM 3 - IST		1	03/30/84
42606	PEACH BOTTOM 3 - LONG TERM REVIEW CONTAINMENT PURGE AND VENT (B-24)		1	04/25/84
<u>ACTIVE ACTIONS</u>				
52256	PEACH BOTTOM 3 - CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPA#C-10)			<u>TARGET</u>
52785	PEACH BOTTOM 3 - ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION AND PROCEDURES			
52866	PEACH BOTTOM 3 - ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RTS COMPONENTS			
52947	PEACH BOTTOM 3 - ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING PROCEDURES/VENDOR RECOMMENDATIONS - RTS COMPONENTS			
53028	PEACH BOTTOM 3 - ITEM 3.1.3 - POST-MAINT TESTING-CHANGES TO TECH SPECS-RTS COMPONENTS			
42504	PEACH BOTTOM 3 - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21)		1	04/30/84
51689	PEACH BOTTOM 3 NUREG 0737 TECH SPECS			06/29/84
08122	PEACH BOTTOM 3 - APPENDIX I TECH SPEC IMPLEMENTATION REVIEW		3	06/30/84
08500	PEACH BOTTOM 3 - SURV. OF MECHANICAL SNUBBERS		2	06/30/84
08936	PEACH BOTTOM 3 - RPS POWER SUPPLY		1	06/30/84
46672	PEACH BOTTOM 3 - IMPLEMENTATION OF NUREG-0313, REV. 1		2	06/30/84
42923	PEACH BOTTOM 3 - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11		2	07/02/84
43732	PEACH BOTTOM 3 - SAFETY CONCERNS ASSOCIATED WITH PIPE BREAKS IN THE BWR SCRAM SYSTEM		2	09/28/84
51118	PEACH BOTTOM 3 - INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)			12/28/84
<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>TMI ACTIONS</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
44606	PEACH BOTTOM 3 - RV AND SV TESTING		1	<u>COMPLETE</u> 11/04/83
44465	PEACH BOTTOM 3 - NUREG-0737 II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS		1	02/05/84
45851	PEACH BOTTOM 3 - NUREG-0737 II.K.3.30, SB LOCA OUTLINE		1	02/28/84
48190	PEACH BOTTOM 3 - NUREG 0737 II.K.3.31 COMPLIANCE WITH 10 CFR 50.46		1	02/28/84
<u>ACTIVE ACTIONS</u>				
51188	PEACH BOTTOM 3 - I.D.1 CONTROL ROOM DESIGN REVIEW			<u>TARGET</u>
51268	PEACH BOTTOM 3 - I.D.2. SAFETY PARAMETER DISPLAY SYSTEM			
44875	PEACH BOTTOM 3 - NUREG-0737 II.E.4.2, CONTAINMENT ISOLATION DEPENDABILITY		1	02/28/84
44257	PEACH BOTTOM 3 - NUREG-0737 I.C.1.2.A, ADEQUATE CORE COOLING GUIDELINES		1	05/31/84
44327	PEACH BOTTOM 3 - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES		1	06/01/84
45698	PEACH BOTTOM 3 - NUREG-0737 II.K.3.18, ADS ACTUATION STUDY		1	06/01/84
48276	PEACH BOTTOM 3 - NUREG 0737 II.K.3.28, VERIFY QUALIFICATION OF ACCUMULATORS ON ADS VALVES		1	06/30/84
45970	PEACH BOTTOM 3 - NUREG-0737 III.A.1.2, TECHNICAL SUPPORT CENTER		1	06/30/84
46041	PEACH BOTTOM 3 - NUREG-0737 III.A.1.2, OPERATIONAL SUPPORT CENTER		1	09/30/84
46113	PEACH BOTTOM 3 - NUREG-0737 III.A.1.2, EMERGENCY OPERATIONS FACILITY		1	09/30/84
46329	PEACH BOTTOM 3 - NUREG-0737 III.A.2.2, METEOROLOGICAL DATA UPGRADE		1	09/30/84
45158	PEACH BOTTOM 3 - NUREG-0737 II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION		1	09/30/84
45602	PEACH BOTTOM 3 - NUREG-0737 II.K.3.16, CHALLENGES AND FAILURES OF RELIEF VALVES		1	12/14/84
				12/30/84

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(CONTINUATION)

PEACH BOTTOM 3

TAC #	TAC DESCRIPTION	PLANT SPECIFIC	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				<u>COMPLETE</u>
48112	PEACH BOTTOM 2	- TECH SPECS RELATED TO IMPLEMENTATION OF NUREG-0737	3	10/04/83
52372	PEACH BOTTOM 3	- ANNUAL EXERCISE EXEMPTION REQUEST		11/03/83
49195	PEACH BOTTOM 3	- GEOLOGICAL ASSESSMENT OF ALLEGED FAULTING AT SITE	3	12/06/83
51386	PEACH BOTTOM 3	- TS FOR PERIODIC LEAK INTEGRITY TESTING		12/12/83
52378	PEACH BOTTOM 3	- ALT. SHUTDOWN CAPABILITY ASSESSMENT	2	01/26/84
48470	PEACH BOTTOM 3	- ROD BLOCK SYSTEM TECH. SPEC. CHANGE	2	02/10/84
49604	PEACH BOTTOM 3	- DELETION OF APP. B WATER QUAL. REG. AND AQUATIC MONITORING	3	02/24/84
51405	PEACH BOTTOM 3	- NOISE MONITORING PROGRAM - APP. B TS AMENDMENT	2	02/24/84
49032	PEACH BOTTOM 3	- MK 1 TORUS T.S. CHANGES LONG TERM PROGRAM		03/13/84
48875	PEACH BOTTOM 2	- REVIEW OF HEALTH PHYSICS ORGANIZATIONAL CHANGE & GENERAL ORG. CHANGES IN T.S.		03/21/84
51020	PEACH BOTTOM 3	- HPCI STEAM LINE LOW PRESSURE TECH. SPEC.	2	03/21/84
48405	PEACH BOTTOM 3	- DEGRADED GRID VOLTAGE TECH. SPECS.		04/11/84
<u>ACTIVE ACTIONS</u>				<u>TARGET</u>
54801	PEACH BOTTOM 3	: REVIEW OF PROPOSED TECH. SPEC. REQUIREMENTS ON AIR SUPPLY SYSTEM LEAKAGE		
54824	PEACH BOTTOM 3	: REVISED APP. J TECH. SPECS. REQUEST		
54831	PEACH BOTTOM 3	- TECH. SPEC. CHANGE ON AUDIT OF EMERG. PLAN PROCEDURES		
51355	PEACH BOTTOM 3	- FOUR MISC. TECH SPEC CHANGES	2	04/30/84
54618	PEACH BOTTOM 3	- EMERGENCY PREPAREDNESS EXERCISE EXEMPTION REQUEST		05/15/84
54368	PEACH BOTTOM 3	- T/S CHANGE TO DELETE DRYWELL AIR MONITORING SYSTEM		05/31/84
53274	PEACH BOTTOM 3	- TS CHANGE RELATED TO SDV AND RWCU INSTRUMENTATION		05/31/84
54489	PEACH BOTTOM 3	: FIRE BARRIER PENETRATIONS T/S CHANGE		06/22/84
51018	PEACH BOTTOM 3	WETWELL/DRYWELL VACUUM BREAKERS	3	06/30/84
47376	PEACH BOTTOM 3	- SUPPRESSION CHAMBER HIGH LEVEL TRIP SETTING	3	06/30/84
47378	PEACH BOTTOM 3	- SURVEILLANCE REQS FOR THE PCIS/LPC1 INTERLOCK PRESSURE SWITCH	3	06/30/84
47380	PEACH BOTTOM 3	- FUEL OIL SYSTEMS FOR STANDBY DIESEL GENERATORS	3	06/30/84
47538	PEACH BOTTOM 3	- CORE SPRAY PUMP DISCHARGE PRESSURE INTERLOCK	3	06/30/84
47540	PEACH BOTTOM 3	- REACTOR LEVEL SURVEILLANCE INSTRUMENTS	3	06/30/84
52682	PEACH BOTTOM 3	- TS CHANGE FOR COOLANT LEAKAGE AND I-131 LIMITS		06/31/84
51742	PEACH BOTTOM 3	- REVIEW OF APP R, III F AND III M EXEMPTION REQUESTS	2	07/02/84
54794	PEACH BOTTOM 3	- TS CHANGE ON REQUIREMENTS FOR TESTING EMERGENCY BATTERIES		07/20/84
46788	PEACH BOTTOM 3	- TECH SPEC CHANGE RELATED TO VERIF OF DRYWELL-TO-TORUS VAC. BREAKER CLOSURE	3	07/31/84
46790	PEACH BOTTOM 3	- ENVIRONM TECH SPEC CHANGE RELATED TO A REPORTING PERIOD	3	07/31/84
54829	PEACH BOTTOM 3	- TECH. SPEC. CHANGES CONCERNING REACTOR WATER LEVEL SETPOINTS		08/31/84
54360	PEACH BOTTOM 3	: REVIEW OF STRUCTURAL STEEL		09/30/84
54557	PEACH BOTTOM 3	- TECHNICAL SPECIFICATIONS COVERED BY GENERIC LETTERS 83-36, 83-37 FOR NUREG-0737		09/30/84
<u>ANTICIPATED ACTIONS</u>				<u>INITIATION</u>
54767	PEACH BOTTOM 3	- OPERABILITY OF PURGE/VENT VALVES		10/19/84
53618	PEACH BOTTOM 3	- ITEM 1.2 - POST TRIP REVIEW-DATA AND INFORMATION CAPABILITY		11/01/84
53701	PEACH BOTTOM 3	- ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS		11/01/84
53784	PEACH BOTTOM 3	- ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS		11/01/84
53867	PEACH BOTTOM 3	- ITEM 3.2.3 - POST-MAINTENANCE TESTING -CHANGES TO TECH. SPEC. - ALL SR COMPONENTS		11/01/84
54011	PEACH BOTTOM 3	- ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS		11/01/84
54094	PEACH BOTTOM 3	- ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL TESTING - DIVERSE TRIP FEATURES		11/01/84

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CONDENSED MANAGEMENT REPORT

FACILITY: PILGRIM 1

PLANT LOCATION: 4 MI SE OF PLYMOUTH, MASS
 DOCKET NUMBER: 050-00293
 ARCH/ENGINEER: BECH
 IE INSPECTOR: J. JOHNSON

LICENSED POWER: 1998 MWT
 DESIGN POWER: 0655 MWE
 NSSS VENDOR: GE

PROJECT MANAGER: P. LEECH
 BRANCH CHIEF: D. VASSALLO
 LIC. ASSISTANT: S. NORRIS

TAC #	TAC DESCRIPTION	MULTI-PLANT	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				
12185	PILGRIM 1 HIGH ENERGY LINE BREAK & CONSEQUENTIAL SYSTEM FAILURE		2	<u>COMPLETE</u> 01/31/84
42579	PILGRIM 1 - LONG TERM REVIEW CONTAINMENT PURGE AND VENT (B-24)		1	04/11/84
<u>ACTIVE ACTIONS</u>				
52867	PILGRIM 1 - ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RTS COMPONENTS			
52948	PILGRIM 1 - ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING PROCEDURES/VENDOR RECOMMENDATIONS - RTS COMPONENTS			
53029	PILGRIM 1 - ITEM 3.1.3 - POST-MAINTENANCE TESTING-CHNGS TO TECH SPECS-RTS COMPONENTS			
51119	PILGRIM 1 - INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)			
52257	PILGRIM 1 - CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPA#C-10)			
52786	PILGRIM 1 - ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION AND PROCEDURES			
51690	PILGRIM 1 NUREG 0737 TECH SPECS			
10817	PILGRIM 1 - CONTROL OF HEAVY LOADS OVER SPENT FUEL POOL (NUREG 0612)		2	04/30/84
08841	PILGRIM 1 - BWR FEEDWATER & CONTROL ROD DRIVE NOZZLE CRACKING		1	04/31/84
51643	PILGRIM 1 - SUPPLEMENTAL REQUESTS FOR EXEMPTION REGARDING 10CFR 50.48 AND APPENDIX R			05/15/84
08123	PILGRIM 1 - APPENDIX I TECH SPEC IMPLEMENTATION REVIEW		3	06/15/84
07948	PILGRIM 1 - MARK I CONTAINMENT LONG TERM PROGRAM IMPLEMENTATION		1	06/30/84
10825	PILGRIM 1 - BWR SINGLE LOOP OPERATION		3	06/30/84
46673	PILGRIM 1 - IMPLEMENTATION OF NUREG-0313, REV. 1		2	06/30/84
42477	PILGRIM 1 - ENVIRONMENTAL QUALIFICATIONS OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21)		1	08/31/84
42885	PILGRIM 1 - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11		2	09/30/84
TAC #	TAC DESCRIPTION	TMI ACTIONS	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				
45852	PILGRIM 1 - NUREG-0737 II.K.3.30, SB LOCA OUTLINE		1	<u>COMPLETE</u> 12/29/83
48191	PILGRIM 1 - NUREG 0737 II.K.3.31 COMPLIANCE WITH 10 CFR 50.46		1	12/29/83
47625	PILGRIM 1 - NUREG 0737 ITEM II F1.4 CONTAINMENT PRESSURE INSTRUMENT		1	03/23/84
47696	PILGRIM 1 - NUREG 0737 ITEM II F1.5 CONTAINMENT WATER LEVEL MONITOR		1	03/23/84
47767	PILGRIM 1 - NUREG 0737 ITEM II F1.6 CONTAINMENT HYDROGEN MONITOR		1	03/23/84
<u>ACTIVE ACTIONS</u>				
51269	PILGRIM 1 - I.D.2. SAFETY PARAMETER DISPLAY SYSTEM		1	12/30/83
46330	PILGRIM 1 - NUREG-0737 III.A.2.2, METEOROLOGICAL DATA UPGRADE		1	03/31/84
44607	PILGRIM 1 - RV AND SV TESTING		1	04/30/84
44328	PILGRIM 1 - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES		1	04/30/84
44258	PILGRIM 1 - NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES		1	06/30/84
44876	PILGRIM 1 - NUREG-0737 II.E.4.2, CONTAINMENT ISOLATION DEPENDABILITY		1	06/30/84
45159	PILGRIM 1 - NUREG-0737 II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION		1	06/30/84
45603	PILGRIM 1 - NUREG-0737 II.K.3.16, CHALLENGES AND FAILURES OF RELIEF VALVES		1	06/30/84
47313	PILGRIM 1 - CORRECT PERFORMANCE OF OPERATING ACTIVITIES (API.C.6)		1	06/30/84
47858	PILGRIM 1 - NUREG - 0737 II. 3.22B, RCIC SUCTION MODIFICATIONS		1	06/30/84
48271	PILGRIM 1 - NUREG 0737 II.K.3.28, VERIFY QUALIFICATION OF ACCUMULATORS ON ADS VALVES		1	06/30/84
45971	PILGRIM 1 - NUREG-0737 III.A.1.2, TECHNICAL SUPPORT CENTER		1	09/30/84
46042	PILGRIM 1 - NUREG-0737 III.A.1.2, OPERATIONAL SUPPORT CENTER		1	09/30/84

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CONDENSED MANAGEMENT REPORT

(CONTINUATION)

PILGRIM 1

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>TMI ACTIONS</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>ACTIVE ACTIONS (CONTINUATION)</u>				
46114	PILGRIM 1 - NUREG-0737 III.A.1.2, EMERGENCY OPERATIONS FACILITY		1	<u>TARGET</u> 09/30/84
44466	PILGRIM 1 - NUREG-0737 II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS		1	09/30/84
51189	PILGRIM 1 - I.D.1 CONTROL ROOM DESIGN REVIEW			12/30/84
<u>PLANT SPECIFIC</u>				
<u>COMPLETED ACTIONS</u>				
49128	PILGRIM - APPENDIX R FIRE PROTECTION ALTERNATIVE SHUTDOWN REVIEW			<u>COMPLETE</u> 11/02/83
49909	PILGRIM - PROPOSED T.S. CHANGE TO ADD ACTION STATEMENT RE: JET PUMP FLOW MISMATCH			11/09/83
51888	PILGRIM - REQUEST TO EXPAND THE OPERATING REGION OF PILGRIM'S POWER/FLOW MAP			11/22/83
51560	PILGRIM - INCREASED SAFETY RELIEF VALVE SIMMER MARGIN TECH SPEC			12/29/83
52471	PILGRIM - SHIFT MANNING EXTENSION			12/30/83
48573	PILGRIM 1 - ASSESSMENT OF MANAGEMENT IMPROVEMENT PLAN			03/22/84
53423	PILGRIM - ALARA PROGRAM FOR PIPE REPLACEMENT			04/03/84
52688	PILGRIM - TS AMENDMENT CONCERNING APPROVAL OF PROCEDURES			04/24/84
52689	PILGRIM-T.S. AMENDMENT REFLECTING BEC ORGANIZATIONAL CHNGS			04/24/84
<u>ACTIVE ACTIONS</u>				
52438	PILGRIM - LONG TERM SCHEDULING PROGRAM		3	<u>TARGET</u> 03/31/84
43804	PILGRIM - APPENDIX J EXEMPTION REQUESTS			04/30/84
52690	PILGRIM - TS CHANGE RE CRD WEEKLY SURVEILLANCE			05/15/84
52033	PILGRIM - REQUEST FRO SCHEDULAR EXTENSION FOR COMPLETION OF CLASS II & III HYDROTESTS			05/30/84
53416	PILGRIM - APPENDIX R EXEMPTION REQUESTS #11-15			05/31/84
54609	PILGRIM - TS CHANGE REMOVING DETAILS OF SECTION XI ISI PROGRAM			06/01/84
54691	PILGRIM - OPERABILITY OF NEW 8 INCH ISOLATION VALVES			06/15/84
48767	PILGRIM - TS CHANGE REQUEST TO ALLOW CONTROLLED EFFLUENT RELEASE FROM OTHER THAN RADWASTE FACILITY			06/15/84
51381	PILGRIM 1 - INSERVICE INSPECTION - 2ND TEN YEAR INTERVAL			06/30/84
54610	PILGRIM - TS CHANGES RELATIVE TO FIRE PROTECTION			06/30/84
53217	PILGRIM-RELOAD 6 FOR CYCLE 7 OPERATION			07/06/84
54608	PILGRIM - TS CHANGE TO ALLOW INCREASED CORE FLOW			07/06/84
53218	PILGRIM - RECIRCULATION SYSTEM INSPECTION AND REPAIR			08/05/84
52032	PILGRIM - INSERVICE TESTING PROGRAM AND REQUESTS FOR RELIEF - 2ND TEN YEAR INTERVAL			08/30/84
54558	PILGRIM 1 - TECHNICAL SPECIFICATIONS COVERED BY GENERIC LETTERS 83-36, 83-37 FOR NUREG-0737			09/30/84
<u>ANTICIPATED ACTIONS</u>				
53619	PILGRIM 1 - ITEM 1.2 - POST TRIP REVIEW-DATA AND INFORMATION CAPABILITY			<u>INITIATION</u> 11/01/84
53702	PILGRIM 1 - ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS			11/01/84
53785	PILGRIM 1- ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS			11/01/84
53868	PILGRIM 1 - ITEM 3.2.3 - POST-MAINTENANCE TESTING -CHANGES TO TECH. SPEC. - ALL SR COMPONENTS			11/01/84
54012	PILGRIM 1 - ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS			11/01/84
54095	PILGRIM 1 - ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL TESTING - DIVERSE TRIP FEATURES			11/01/84

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CONDENSED MANAGEMENT REPORT

FACILITY: POINT BEACH 1

PLANT LOCATION: 15 MI N OF MANITOWOC, WISC
 DOCKET NUMBER: 050-00266
 ARCH/ENGINEER: BECH
 IE INSPECTOR: R. HAGUE

LICENSED POWER: 1518 MWT
 DESIGN POWER: 0497 MWE
 NSSS VENDOR: WEST

PROJECT MANAGER: T. COLBURN
 BRANCH CHIEF: J. MILLER
 LIC. ASSISTANT: P. KREUTZER

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>MULTI-PLANT</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
47147	POINT BEACH 1 NATURAL CIRCULATION COOLDOWN (GENERIC LTR 81-21)		2	<u>COMPLETE</u> 11/08/83
49748	POINT BEACH 1 - "NUREG 0737 TECH SPECS (GENERIC LETTER 82-16 DR 83-02)"		1	12/28/83
12271	POINT BEACH 1 - ENVIRON. QUAL. OF CONTROL SYSTEMS POINT BEACH 1		3	01/31/84
07710	POINT BEACH 1 - ASYMMETRIC LOCA LOADS (GENERIC) (USI A-02)		2	02/01/84
07727	POINT BEACH 1-CONTROL OF HEAVY LOADS OVER SPENT FUEL POOL POWER PLANTS (NUREG 0612)		2	03/27/84

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>MULTI-PLANT</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>ACTIVE ACTIONS</u>				
52787	POINT BEACH 1 - ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION AND PROCEDURES		1	<u>TARGET</u>
52868	POINT BEACH 1 - ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RTS COMPONENTS		1	
52949	POINT BEACH 1 - ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING PROCEDURES/VENDOR RECOMMENDATIONS - RTS COMPONENTS		1	
53030	POINT BEACH 1-ITEM 3.1.3 -POST-MAINTENANCE TESTING-CHNGS TO TECH SPECS-RTS COMPONENTS		1	
53087	POINT BEACH 1 - REACTOR TRIP SYSTEM RELIABILITY - VENDOR RELATED MODIFICATIONS		1	
53140	POINT BEACH 1 - ITEMS 4.2.1 & 4.2.2 - PREVENTATIVE MAINT PROGRAM FOR REACTOR TRIP BREAKERS - MAINTENANCE & TRENDING		1	
53186	POINT BEACH 1 - ITEM 4.3 - AUTOAMTIC ACTUATION OF SHUNT TRIP ATTACHMENT FOR W AND B & W PLANTS		1	
42498	POINT BEACH 1 - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21)		1	04/15/84
48247	POINT BEACH 1 - APPENDIX R FIRE PROTECTION REVIEW		1	04/30/84
07712	POINT BEACH 1 - APPENDIX I TECH SPEC IMPLEMENTATION REVIEW		3	07/31/84
06828	POINT BEACH 1 - UPI ECCS INJECTION		3	07/31/84
51120	POINT BEACH 1 - INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)		1	10/30/84
49656	POINT BEACH 1 - REACTOR COOLANT PUMP TRIP		3	10/31/84
43669	POINT BEACH 1 - SEISMIC QUALIFICATION OF THE AUXILIARY FEEDWATER SYSTEM		1	10/31/84
52258	POINT BEACH 1 - CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPA#C-10)		3	07/30/85

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>TMI ACTIONS</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
44467	POINT BEACH 1 - NUREG-0737 II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS		1	<u>COMPLETE</u> 11/03/83
47965	POINT BEACH 1 - PLANT SHIELDING MODIFICATIONS (II.B.2.2)		1	11/03/83
47811	POINT BEACH 1 - NUREG 0737 ITEM II K2.17 POTENTIAL FOR VOIDING IN RCS		1	01/24/84

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>TMI ACTIONS</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>ACTIVE ACTIONS</u>				
45853	POINT BEACH 1 - NUREG-0737 II.K.3.30, SB LOCA OUTLINE		1	<u>TARGET</u> 07/31/84
51270	POINT BEACH 1 - I.D.2. SAFETY PARAMETER DISPLAY SYSTEM		1	07/31/84
46331	POINT BEACH 1 - NUREG-0737 III.A.2.2, METEOROLOGICAL DATA UPGRADE		1	08/01/84
44608	POINT BEACH 1 - RV AND SV TESTING		1	08/30/84
46897	POINT BEACH 1 - II K2.13 THERMAL MECHANICAL REPORT		1	09/30/84
45972	POINT BEACH 1 - NUREG-0737 III.A.1.2, TECHNICAL SUPPORT CENTER		1	09/30/84
46043	POINT BEACH 1 - NUREG-0737 III.A.1.2, OPERATIONAL SUPPORT CENTER		1	09/30/84
46115	POINT BEACH 1 - NUREG-0737 III.A.1.2, EMERGENCY OPERATIONS FACILITY		1	09/30/84
44259	POINT BEACH 1 - NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES		1	11/30/84
44329	POINT BEACH 1 - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES		1	11/30/84
51190	POINT BEACH 1 - I.D.1 CONTROL ROOM DESIGN REVIEW		1	04/30/85
45160	POINT BEACH 1 - NUREG-0737 II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION		1	06/30/85

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CONDENSED MANAGEMENT REPORT

(CONTINUATION)

POINT BEACH 1

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>TMI ACTIONS</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>ACTIVE ACTIONS</u> (CONTINUATION)				
48192	POINT BEACH 1 -NUREG 0737 II.K.3.31 COMPLIANCE WITH 10 CFR 50.46		1	<u>TARGET</u> 06/30/85
<u>PLANT SPECIFIC</u>				
<u>COMPLETED ACTIONS</u>				
48929	POINT BEACH 1 - SPENT FUEL POOL POISON SURVEILLANCE AND PORV OPERABILITY		3	<u>COMPLETE</u> 10/05/83
51597	POINT BEACH 1 - ADMINISTRATIVE TECH SPEC CHANGES		3	10/06/83
48375	POINT BEACH 1 - LOSS OF VOLTAGE RELAY TIME DELAY		1	10/31/83
52693	POINT BEACH 1-RESPONSE TO SEN. GLENN RE: TRANSFER OF SPENT FUEL TO POINT BEACH - GREEN TICKET		1	11/28/83
53271	POINT BEACH 1 - AUXILIARY FEEDWATE AUTO INITIATION T.S		2	12/29/83
53345	POINT BEACH 1 - RCS VENTS SCHEDULAR EXEMPTION			12/30/83
52692	POINT BEACH 1 - RETURN TO POWER OPERATION AT EITHER 2200 PSIA		2	12/30/83
53212	POINT BEACH 1 - EQUIPMENT QUALIFICATION DEADLINE EXTENSION		1	01/03/84
52663	POINT BEACH 1 - ADMINISTRATIVE CHANGES TO LOC'S		2	02/02/84
52502	POINT BEACH 1 - SHIFT STAFFING EXEMPTION REQUEST		1	03/26/84
48762	POINT BEACH 1 - 2ND 10-YEAR INTERVAL ISI RELIEF		3	03/29/84
53400	POINT BEACH - SURVEILLANCE TESTS FOR INTEGRITY OF OF RECOVERY SYSTEMS		1	04/06/84
54589	POINT BEACH 1 - DEBRIS IN CRDM-STUCK CONTROL ROD			04/06/84
<u>ACTIVE ACTIONS</u>				
47494	POINT BEACH 1 - INSTRUMENT POWER SUPPLY TECH SPEC		2	<u>TARGET</u> 02/28/84
54356	POINT BEACH 1 - CRDM SUPPORT PIN CRACKING		1	04/20/84
51042	POINT BEACH 1 - TIA, REGION III, AUX FW SYSTEM OPERABILITY		1	04/28/84
51301	POINT BEACH 1 - INSERVICE TESTING		1	04/30/84
54446	POINT BEACH - REACTOR VESSEL HOT LEG NOZZLE SLAG INCLUSIONS		1	04/30/84
54253	POINT BEACH 1 - REPEAL OF CONFIRMATORY ORDERS		3	05/31/84
51550	POINT BEACH 1 - FIRE PROTECTION TECH SPECS		2	05/31/84
52665	POINT BEACH 1 - CONTAINMENT TYPE A TESTING (APPENDIX J)		2	06/15/84
54200	POINT BEACH 1 - REPORTING REQUIREMENTS T.S. GL 83-43		2	06/30/84
54596	POINT BEACH 1 - NUREG-0737 ORDER MODIFICATION		1	06/30/84
54591	POINT BEACH 1 - HEAVY LOADS HANDLING T.S.		2	08/30/84
54592	POINT BEACH 1 - HYDRAULIC SNUBBER T.S.		2	08/30/84
54595	POINT BEACH 1 - ADMIN T.S.		2	08/30/84
54559	POINT BEACH 1 - TECHNICAL SPECIFICATIONS COVERED BY GENERIC LETTERS 83-36, 83-37 FOR NUREG-0737		2	09/30/84
54406	POINT BEACH 1- RCS VENTS TECHNICAL SPECIFICATIONS NUREG 073 ITEM II.B.1		2	09/30/84
49969	POINT BEACH 1 - WESTINGHOUSE OPTIMIZED FUEL DESIGN		3	09/30/84
<u>ANTICIPATED ACTIONS</u>				
54989	POINT BEACH 1 - CONTAINMENT TENDON SURVEILLANCE			<u>INITIATION</u> 05/09/84
53620	POINT BEACH 1 - ITEM 1.2 - POST TRIP REVIEW-DATA AND INFCRMATION CAPABILITY			11/01/84
53703	POINT BEACH 1 - ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS			11/01/84
53786	POINT BEACH 1- ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS			11/01/84
53869	POINT BEACH 1 - ITEM 3.2.3 - POST-MAINTENANCE TESTING -CHANGES TO TECH. SPEC. - ALL SR COMPONENTS			11/01/84
53927	-ITEMS 4.2.3 & 4.2.4-PREVENTATIVE MAINT. PROGRAM FOR REACTOR TRIP BREAKERS-LIFE TESTING & REPLACEMENT			11/01/84
54013	POINT BEACH 1 - ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS			11/01/84
54096	POINT BEACH 1 - ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL TESTING - DIVERSE TRIP FEATURES			11/01/84

DATA AS OF - 04/30/84

CONDENSED MANAGEMENT REPORT

FACILITY: POINT BEACH 2

PLANT LOCATION: 15 MI N OF MANITOWOC, WISC
 DOCKET NUMBER: 050-00301
 ARCH/ENGINEER: BECH
 IE INSPECTOR: R. HAGUE

LICENSED POWER: 1518 MWT
 DESIGN POWER: 0497 MWE
 NSSS VENDOR: WEST

PROJECT MANAGER: T. COLBURN
 BRANCH CHIEF: J. MILLER
 LIC. ASSISTANT: P. KREUTZER

TAC #	TAC DESCRIPTION	MULTI-PLANT	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				
47148	POINT BEACH 2 NATURAL CIRCULATION COOLDOWN (GENERIC LTR 81-21)		2	<u>COMPLETE</u> 11/08/83
49749	POINT BEACH 2 - "NUREG 0737 TECH SPECS (GENERIC LETTER 82-16 OR 83-02)"		1	12/28/83
12272	POINT BEACH 2 - ENVIRON. QUAL. OF CONTROL SYSTEMS POINT BEACH 2		3	01/31/84
08158	POINT BEACH 2 - GENERIC - ASSYMETRIC LOCA LOADS (USI A-02)		2	02/01/84
08074	POINT BEACH 2-CONTROL OF HEAVY LOADS OVER SPENT FUEL POOL (NUREG 0612)		2	03/27/84
<u>ACTIVE ACTIONS</u>				
52788	POINT BEACH 2 - ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION AND PROCEDURES		1	<u>TARGET</u>
52869	POINT BEACH 2 - ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RTS COMPONENTS		1	
52950	POINT BEACH 2 - ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING PROCEDURES/VENDOR RECOMMENDATIONS - RTS COMPONENTS		1	
53031	POINT BEACH 2-ITEM 3.1.3 -POST-MAINT TESTING-CHANGES TO TECH SPECS-RTS COMPONENTS		1	
53088	POINT BEACH 2 - REACTOR TRIP SYSTEM RELIABILITY - VENDOR RELATED MODIFICATIONS		1	
53141	POINT BEACH 2 - ITEMS 4.2.1 & 4.2.2 - PREVENTATIVE MAINT PROGRAM FOR REACTOR TRIP BREAKERS - MAINTENANCE & TRENDING		1	
53187	POINT BEACH 2 - ITEM 4.3 - AUTOAMTIC ACTUATION OF SHUNT TRIP ATTACHMENT FOR W AND B & W PLANTS		1	
42499	POINT BEACH 2 - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-2')		1	04/15/84
48246	POINT BEACH 2 - APPENDIX R FIRE PROTECTION REVIEW		1	04/30/84
08150	POINT BEACH 2 - GENERIC - APPENDIX I		3	07/31/84
08349	POINT BEACH 2 - UPI ECCS INJECTION		3	07/31/84
43670	POINT BEACH 2 - SEISMIC QUALIFICATION OF THE AUXILIARY FEEDWATER SYSTEM		1	10/31/84
49657	POINT BEACH 2 - REACTOR COOLANT PUMP TRIP		3	10/31/84
51121	POINT BEACH 2 - INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)		1	10/31/84
52259	POINT BEACH 2 - CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPA#C-10)		3	07/30/85

TAC #	TAC DESCRIPTION	TMI ACTIONS	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				
47966	POINT BEACH 2 - PLANT SHIELDING MODIFICATIONS (II.B.2.2)		1	<u>COMPLETE</u> 11/03/83
44468	POINT BEACH 2 - NUREG-0737 II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS		1	11/03/83
47812	POINT BEACH 2 - NUREG 0737 ITEM II K2.17 POTENTIAL FOR VOIDING IN RCS		1	01/24/84
<u>ACTIVE ACTIONS</u>				
51271	POINT BEACH 2 - I.D.2. SAFETY PARAMETER DISPLAY SYSTEM		1	<u>TARGET</u> 07/30/84
45854	POINT BEACH 2 - NUREG-0737 II.K.3.30, SB LOCA OUTLINE		1	07/31/84
46332	POINT BEACH 2 - NUREG-0737 III.A.2.2, METEOROLOGICAL DATA UPGRADE		1	08/01/84
44609	POINT BEACH 2 - RV AND SV TESTING		1	08/31/84
46898	POINT BEACH 2 - II K2.13 THERMAL MECHANICAL REPORT		1	09/30/84
45973	POINT BEACH 2 - NUREG-0737 III.A.1.2, TECHNICAL SUPPORT CENTER		1	09/30/84
46044	POINT BEACH 2 - NUREG-0737 III.A.1.2, OPERATIONAL SUPPORT CENTER		1	09/30/84
46116	POINT BEACH 2 - NUREG-0737 III.A.1.2, EMERGENCY OPERATIONS FACILITY		1	09/30/84
44260	POINT BEACH 2 - NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES		1	11/30/84
44330	POINT BEACH 2 - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES		1	11/30/84
51191	POINT BEACH 2 - I.D.1 CONTROL ROOM DESIGN REVIEW		1	04/30/85
45161	POINT BEACH 2 - NUREG-0737 II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION		1	06/30/85

DATA AS OF - 04/30/84

CONDENSED MANAGEMENT REPORT

(CONTINUATION)

POINT BEACH 2

TAC #	TAC DESCRIPTION	TMI ACTIONS	PRIORITY	CRITICAL DATE
ACTIVE AC. NS	(CONTINUATION)			TARGET
48193	POINT BEACH 2 -NUREG 073 II.K.3.31 COMPLIANCE WITH 10 CFR 50.46		1	06/30/85

TAC #	TAC DESCRIPTION	PLANT SPECIFIC	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				
48930	POINT BEACH 2 - SPENT FUEL POOL POISON SURVEILLANCE AND PORV OPERABILITY		3	COMPLETE 10/05/83
51596	POINT BEACH 2 - ADMINISTRATIVE TECH SPEC CHANGES		3	10/06/83
48376	POINT BEACH 2 - LOSS OF VOLTAGE RELAY TIME DELAY		1	10/31/83
52694	POINT BEACH 2 - RESPONSE TO SEN GLENN RE: TRANSFER OF SPENT FUEL TO POINT BEACH - GREEN TICKET		1	11/28/83
52421	POINT BEACH 2 - COMPONENT COOLING WATER HEAT EXCHANGER T.S.		1	12/12/83
53272	POINT BEACH 2 - AUXILIARY FEEDWATER AUTO INITIATION T.S.		2	12/29/83
53346	POINT BEACH 2 - RCS VENTS SCHEDULAR EXEMPTION		2	12/30/83
53213	POINT BEACH 2 - EQUIPMENT QUALIFICATION DEADLINE EXTENSION		1	01/03/84
52666	POINT BEACH 2 - ADMINISTRATIVE CHANGES TO LCO'S		2	02/02/84
52503	POINT BEACH 2 - SHIFY STAFFING EXEMPTION REQUEST		1	03/26/84
49549	POINT BEACH 2 - 2ND 10-YEAR INTERVAL ISI RELIEF		3	03/29/84
53401	POINT BEACH 2 - SURVEILLANCE TESTS FOR INTEGRITY OF RECOVERY SYSTEMS		1	04/06/84

ACTIVE ACTIONS				TARGET
51043	POINT BEACH 2 - TIA, REGION III, AUX FW SYSTEM OPERABILITY		1	04/28/84
47495	POINT BEACH 2 - INSTRUMENT POWER SUPPLY T.S.		2	04/28/84
51302	POINT BEACH 2 - INSERVICE TESTING		1	04/30/84
51551	POINT BEACH 2 - FIRE PROTECTION TECH SPECS		2	05/31/84
52666	POINT BEACH 2 - CONTAINMENT TYPE A TESTING (APPENDIX J)		2	06/15/84
54201	POINT BEACH 2 - REPORTING REQUIREMENTS T.S. GL 83-43		2	06/30/84
54597	POINT BEACH 2 - NUREG-0737 ORDER MODIFICATION		1	06/30/84
54594	POINT BEACH 2 - ADMIN T.S.		2	08/30/84
54590	POINT BEACH 2 - HEAVY LOADS HANDLING T.S.		2	08/30/84
54593	POINT BEACH 2 - HYDRAULIC SNUBBER T.S.		2	08/30/84
54407	POINT BEACH 2 - RCS VENTS TECHNICAL SPECIFICATIONS NUREG 073 ITEM II.B.1		2	09/30/84
54560	POINT BEACH 2 - TECHNICAL SPECIFICATIONS COVERED BY GENERIC LETTERS 83-36, 83-37 FOR NUREG-0737		2	09/30/84
49970	POINT BEACH 2 - WESTINGHOUSE OPTIMIZED FUEL DESIGN		3	09/30/84

ANTICIPATED ACTIONS				INITIATION
54990	POINT BEACH 2 - CONYAINMENT TENDON SURVEILLANCE			05/09/84
53621	POINT BEACH 2 - ITEM 1.2 - POST TRIP REVIEW-DATA AND INFORMATION CAPABILITY			11/01/84
53704	POINT BEACH 2 - ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS			11/01/84
53787	POINT BEACH 2 - ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS			11/01/84
53870	POINT BEACH 2 - ITEM 3.2.3 - POST-MAINTENANCE TESTING -CHANGES TO TECH. SPEC. - ALL SR COMPONENTS			11/01/84
53928	-ITEMS 4.2.3 & 4.2.4-PREVENTATIVE MAINT. PROGRAM FOR REACTOR TRIP BREAKERS-LIFE TESTING & REPLACEMENT			11/01/84
54014	POINT BEACH 2 - ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS			11/01/84
54097	POINT BEACH 2 - ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL TESTING - DIVERSE TRIP FEATURES			11/01/84

DATA AS OF - 04/30/84

CONDENSED MANAGEMENT REPORT

FACILITY: PRAIRIE ISLAND 1

PLANT LOCATION: 28 MI SE OF MINNEAPOLIS, MINN
 DOCKET NUMBER: 050-00282
 ARCH/ENGINEER: FPI
 IE INSPECTOR: C. FEIERABEND

LICENSED POWER: 1650 MWT
 DESIGN POWER: 0530 MWE
 NSSS VENDOR: WEST

PROJECT MANAGER: D. DIANNI
 BRANCH CHIEF: J. MILLER
 LIC. ASSISTANT: P. KREUTZER

TAC # TAC DESCRIPTIONMULTI-PLANTPRIORITYCRITICAL DATECOMPLETED ACTIONS

49949 PRAIRIE ISLAND UNIT NO 1 - FIRE PROTECTION EXEMPTION

COMPLETE
01/09/84TARGETACTIVE ACTIONS

52260 PRAIRIE ISLAND 1 - CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPA#C-10)
 52789 PRAIRIE ISLAND 1 - ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION AND PROCEDURES
 52870 PRAIRIE ISLAND 1 - ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RTS COMPONENTS
 52951 PRAIRIE ISLAND 1 - ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING PROCEDURES/VENDOR
 RECOMMENDATIONS - RTS COMPONENTS
 53032 PRAIRIE IS 1-ITEM 3.1.3 -POST-MAINT TESTING-CHANGES TO TECH SPECS-RTS COMPONENTS
 53089 PRAIRIE ISLAND 1 - REACTOR TRIP SYSTEM RELIABILITY- VENDOR RELATED MODIFICATIONS
 53142 PRAIRIE ISLAND 1 - ITEMS 4.2.1 & 4.2.2- PREVENTATIVE MAINT PROGRAM FOR REACTOR TRIP BREAKERS -
 MAINTENANCE & TRENDING
 53188 PRAIRIE ISLAND 1 - ITEM 4.3 - AUTOAMTIC ACTUATION OF SHUNT TRIP ATTACHMENT FOR W AND B & W
 PLANTS
 51122 PRAIRIE ISLAND 1 - INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)
 42500 PRAIRIE ISLAND 1 - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT
 (80-CLI-21,
 08731 PRAIRIE ISLAND 1 - REVIEW OF ASYMMETRIC LOCA LOADS SUBMITTAL
 08351 PRAIRIE ISLAND 1 - WESTINGHOUSE UPPER PLENUM ECCS INJECTION - LONG RANGE
 49663 PRAIRIE ISLAND 1 - REACTOR COOLANT PUMP TRIP

1

04/20/84

2

05/20/84

3

09/30/84

10/30/84

TAC # TAC DESCRIPTIONTMI ACTIONSPRIORITYCRITICAL DATECOMPLETED ACTIONS

47967 PRAIRIE ISLAND 1 - PLANT SHIELDING MODIFICATIONS (II.B.2.2)
 44469 PRAIRIE ISLAND 1 - NUREG-0737 II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS
 47813 PRAIRIE ISLAND 1 - NUREG 0737 ITEM II K2.17 POTENTIAL FOR VOIDING IN RCS

1

COMPLETE

10/12/83

1

11/23/83

1

12/30/83

TARGETACTIVE ACTIONS

46333 PRAIRIE ISLAND 1 - NUREG-0737 III.A.2.2, METEOROLOGICAL DATA UPGRADE
 44610 PRAIRIE ISLAND 1 - RV AND SV TESTING
 51192 PRAIRIE ISLAND 1 - I.D.1 CONTROL ROOM DESIGN REVIEW
 45162 PRAIRIE ISLAND 1 - NUREG-0737 II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION
 45974 PRAIRIE ISLAND 1 - NUREG-0737 III.A.1.2, TECHNICAL SUPPORT CENTER
 46045 PRAIRIE ISLAND 1 - NUREG-0737 III.A.1.2, OPERATIONAL SUPPORT CENTER
 46117 PRAIRIE ISLAND 1 - NUREG-0737 III.A.1.2, EMERGENCY OPERATIONS FACILITY
 46899 PRAIRIE ISLAND 1 - II K2.13 THERMAL MECHANICAL REPORT
 44261 PRAIRIE ISLAND 1 - NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES
 44331 PRAIRIE ISLAND 1 - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES
 45855 PRAIRIE ISLAND 1 - NUREG-0737 II.K.3.30, SB LOCA OUTLINE
 48194 PRAIRIE ISLAND 1 -NUREG 0737 II.K.3.31 COMPLIANCE WITH 10 CFR 50.46
 51272 PRAIRIE ISLAND 1 - I.D.2. SAFETY PARAMETER DISPLAY SYSTEM

1

2

2

08/05/84

1

09/30/84

1

09/30/84

1

09/30/84

1

09/30/84

2

11/30/84

2

11/30/84

1

09/30/85

1

09/30/85

10/30/86

DATA AS OF - 04/30/84

CONDENSED MANAGEMENT REPORT

(CONTINUATION)

PRAIRIE ISLAND 1

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				<u>COMPLETE</u>
51906	PRAIRIE ISLAND 1 - ISI RELIEF REQUEST RCP CASING WELDS			10/12/83
51940	PRAIRIE ISLAND UNIT 1 FQ LIMIT CHANGE DUE TO REVISED LOCA ANA			12/28/83
52405	PRAIRIE ISLAND UNIT 1 ISI/IST 10YR EXTENSION			12/28/83
51485	PI-1 IST 7 OPEN ISSUES			01/31/84
51991	PI-1 TS CHANGE REQUEST ON 0737 ITEMS & MISC CHANGES			02/21/84
54408	PRAIRIE ISLAND 1- RCS VENTS TECHNICAL SPECIFICATIONS NUREG 073 ITEM II.B.1			03/27/84
<u>ACTIVE ACTIONS</u>				<u>TARGET</u>
53559	PRAIRIE ISLAND 1 FIRE PROTECTION EXEMPTION (2) BASED ON GL83-33			05/30/84
53458	PRAIRIE ISLAND UNIT 1 TS CHANGE REQUEST RESPONDING TO GL83-37			07/01/84
53456	PRAIRIE ISLAND UNIT 1 ISI/IST; 2ND 10 YEAR PROGRAM			07/15/84
<u>ANTICIPATED ACTIONS</u>				<u>INITIATION</u>
53622	PRAIRIE ISLAND 1 1 - ITEM 1.2 - POST TRIP REVIEW-DATA AND INFORMATION CAPABILITY			11/01/84
53705	PRAIRIE ISLAND 1 - ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS			11/01/84
53788	PRAIRIE IS 1- ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS			11/01/84
53871	PRAIRIE ISLAND 1 - ITEM 3.2.3 - POST-MAINTENANCE TESTING -CHANGES TO TECH. SPEC. - ALL SR COMPONENTS			11/01/84
53929	-ITEMS 4.2.3 & 4.2.4-PREVENTATIVE MAINT. PROGRAM FOR REACTOR TRIP BREAKERS-LIFE TESTING & REPLACEMENT			11/01/84
54015	PRAIRIE ISLAND 1 - ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS			11/01/84
54098	PRAIRIE ISLAND 1 - ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL TESTING - DIVERSE TRIP FEATURES			11/01/84

DATA AS OF - 04/30/84

CONDENSED MANAGEMENT REPORT

FACILITY: PRAIRIE ISLAND 2

PLANT LOCATION: 28 MI SE OF MINNEAPOLIS, MINN
 DOCKET NUMBER: 050-00306
 ARCH/ENGINEER: FPI
 IE INSPECTOR: C. FEIERABEND

LICENSED POWER: 1650 MWT
 DESIGN POWER: 0530 MWE
 NSSS VENDOR: WEST

PROJECT MANAGER: D. DIANNI
 BRANCH CHIEF: J. MILLER
 LIC. ASSISTANT: P. KREUTZER

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>MULTI-PLANT</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
49950	PRAIRIE ISLAND UNIT NO 2 - FIRE PROTECTION 4 EXEMPTIONS			<u>COMPLETE</u> 01/09/84
<u>ACTIVE ACTIONS</u>				
51123	PRAIRIE ISLAND 2 - INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)			<u>TARGET</u>
52261	PRAIRIE ISLAND 2 - CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPA#C-10)			
52790	PRAIRIE ISLAND 2 - ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION AND PROCEDURES			
52871	PRAIRIE ISLAND 2 - ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RTS COMPONENTS			
52952	PRAIRIE ISLAND 2 - ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING PROCEDURES/VENDOR RECOMMENDATIONS - RTS COMPONENTS			
53033	PRAIRIE IS 2-ITEM 3.1.3 -POST-MAINTENANCE TESTING-CHNGS TO TECH SPECS-RTS COMPONENTS			
53090	PRAIRIE ISLAND 2 - REACTOR TRIP SYSTEM RELIABILITY- VENDOR RELATED MODIFICATIONS			
53143	PRAIRIE ISLAND 2 - ITEMS 4.2.1 & 4.2.2- PREVENTATIVE MAINT PROGRAM FOR REACTOR TRIP BREAKERS - MAINTENANCE & TRENDING			
53189	PRAIRIE ISLAND 2 - ITEM 4.3 - AUTOAMTIC ACTUATION OF SHUNT TRIP ATTACHMENT FOR W AND B & W PLANTS			
42501	PRAIRIE ISLAND 2 - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21)		1	04/20/84
08732	PRAIRIE ISLAND 2 - REVIEW OF ASYMMETRIC LOCA LOADS SUBMITTAL		2	05/20/84
08471	PRAIRIE ISLAND 2 - WESTINGHOUSE UPPER PLENUM ECCS INJECTION - LONG RANGE		3	09/30/84
49664	PRAIRIE ISLAND 2 - REACTOR COOLANT PUMP TRIP			10/30/84
<u>TMI ACTIONS</u>				
<u>COMPLETED ACTIONS</u>				
47968	PRAIRIE ISLAND 2 - PLANT SHIELDING MODIFICATIONS		1	<u>COMPLETE</u> 10/12/83
44470	PRAIRIE ISLAND 2 - NUREG-0737 II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS		1	11/23/83
47814	PRAIRIE ISLAND 2 - NUREG 0737 ITEM II K2.17 POTENTIAL FOR VOIDING IN RCS		1	12/30/83
<u>ACTIVE ACTIONS</u>				
46334	PRAIRIE ISLAND 2 - NUREG-0737 III.A.2.2, METEOROLOGICAL DATA UPGRADE		1	
44611	PRAIRIE ISLAND 2 - RV AND SV TESTING		1	
51193	PRAIRIE ISLAND 2 - I.D.1 CONTROL ROOM DESIGN REVIEW			
45975	PRAIRIE ISLAND 2 - NUREG-0737 III.A.1.2, TECHNICAL SUPPORT CENTER		1	09/30/84
46046	PRAIRIE ISLAND 2 - NUREG-0737 III.A.1.2, OPERATIONAL SUPPORT CENTER		1	09/30/84
46118	PRAIRIE ISLAND 2 - NUREG-0737 III.A.1.2, EMERGENCY OPERATIONS FACILITY		1	09/30/84
46900	PRAIRIE ISLAND 2 - II K2.13 THERMAL MECHANICAL REPORT		1	09/30/84
44332	PRAIRIE ISLAND 2 - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES		1	11/30/84
44262	PRAIRIE ISLAND 2 - NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES		1	06/05/85
45163	PRAIRIE ISLAND 2 - NUREG-0737 II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION		1	06/05/85
45856	PRAIRIE ISLAND 2 - NUREG-0737 II.K.3.30, SB LOCA OUTLINE		1	09/30/85
48195	PRAIRIE ISLAND 2 -NUREG 0737 II.K.3.31 COMPLIANCE WITH 10 CFR 50.46		1	09/30/85
51273	PRAIRIE ISLAND 2 - I.D.2. SAFETY PARAMETER DISPLAY SYSTEM			01/01/87

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CONDENSED MANAGEMENT REPORT

(CONTINUATION)

PRAIRIE ISLAND 2

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
51905	PRAIRIE ISLAND UNIT 2 - ISI RELIEF REQUEST RCP CASING WELDS			<u>COMPLETE</u> 10/12/83
51941	PRAIRIE IS UNIT 2 FQ LIMIT CHANGE DUE REVISED LOCA ANA			12/28/83
51484	PI-1 IST 7 OPEN ISSUES			01/31/84
51992	PI-2 TS CHANGE REQUEST ON 0737 ITEMS & MISCHANGE			02/21/84
54409	PRAIRIE ISLAND 2- RCS VENTS TECHNICAL SPECIFICATIONS NUREG 073 ITEM II.B.1			03/27/84
<u>ACTIVE ACTIONS</u>				
53558	PRAIRIE ISLAND 2 FIRE PROTECTION EXEMPTION (2) BASED ON GL83-33			<u>TARGET</u> 05/30/84
53459	PRAIRIE ISLAND UNIT 2 TS CHANGE REQUEST RESPONDING TO GL83-37			07/01/84
53457	PRAIRIE ISLAND UNIT NO 2 ISI/IST; 2ND 10 YR PROGRAM			07/15/84
<u>ANTICIPATED ACTIONS</u>				
53623	PRAIRIE ISLAND 2 - ITEM 1.2 - POST TRIP REVIEW-DATA AND INFORMATION CAPABILITY			<u>INITIATION</u> 11/01/84
53706	PRAIRIE ISLAND 2 - ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS			11/01/84
53789	PRAIRIE IS 2- ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS			11/01/84
53872	PRAIRIE ISLAND 2 - ITEM 3.2.3 - POST-MAINTENANCE TESTING -CHANGES TO TECH. SPEC. - ALL SR COMPONENTS			11/01/84
53930	-ITEMS 4.2.3 & 4.2.4-PREVENTATIVE MAINT. PROGRAM FOR REACTOR TRIP BREAKERS-LIFE TESTING & REPLACEMENT			11/01/84
54016	PRAIRIE ISLAND 2 - ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS			11/01/84
54099	PRAIRIE ISLAND 2 - ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL TESTING - DIVERSE TRIP FEATURES			11/01/84

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CONDENSED MANAGEMENT REPORT

FACILITY: QUAD CITIES 1

PLANT LOCATION: 20 MI NE OF MOLINE, ILL
 DOCKET NUMBER: 050-00254
 ARCH/ENGINEER: S&L
 IE INSPECTOR: N. CHRISSTOMOS

LICENSED POWER: 2511 MWT
 DESIGN POWER: 0789 MWE
 NSSS VENDOR: GE

PROJECT MANAGER: R. BEVAN
 BRANCH CHIEF: D. VASSALLO
 LIC. ASSISTANT: S. NORRIS

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>MULTI-PLANT</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
08933	QUAD CITIES 1 - RPS POWER SUPPLY		1	<u>COMPLETE</u> 12/30/83
<u>ACTIVE ACTIONS</u>				
52262	QUAD CITIES 1 - CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPA#C-10)			
52791	QUAD CITIES 1 - ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION AND PROCEDURES			
52872	QUAD CITIES 1 - ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RTS COMPONENTS			
52953	QUAD CITIES 1 - ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING PROCEDURES/VENDOR RECOMMENDATIONS - RTS COMPONENTS			
53034	QUAD CITIES 1 - ITEM 3.1.3 - POST-MAINTENANCE TESTING-CHNGS TO TECH SPECS-RTS COMPONENTS		2	05/31/83
46669	QUAD CITIES 1 - IMPLEMENTATION OF NUREG-0313, REV. 1		1	07/30/83
42478	QUAD CITIES 1 - ENVIRONMENTAL QUALIFICATIONS OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21)		1	07/30/83
42580	QUAD CITIES 1 - LONG TERM REVIEW CONTAINMENT PURGE AND VENT (B-24)		1	08/31/83
42886	QUAD CITIES 1 - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11		2	10/01/83
12310	QUAD CITIES 1 - HELB AND CONSEQUENTIAL SYSTEM FAILURE QUAD CITIES 1		3	11/01/83
08124	QUAD CITIES 1 - APP I TECH SPEC IMPLEMENTATION REVIEW		3	12/01/83
51124	QUAD CITIES 1 - INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)			03/01/84
51691	QUAD CITIES 1 NUREG 0737 TECH SPECS		1	06/30/84
07947	QUAD CITIES 1 - MARK I CONTAINMENT LONG TERM PROGRAM IMPLEMENTATION			
<u>COMPLETED ACTIONS</u>				
44400	QUAD CITIES 1 - NUREG-0737 II.B.1, RCS HIGH POINT VENTS		1	<u>COMPLETE</u> 11/17/83
45857	QUAD CITIES 1 - NUREG-0737 II.K.3.30, SB LOCA OUTLINE		1	12/28/83
47859	QUAD CITIES 1 - NUREG - 0737 II.K.3.22B, RCIC SUCTION MODIFICATIONS		1	12/29/83
<u>ACTIVE ACTIONS</u>				
46335	QUAD CITIES 1 - NUREG-0737 III.A.2.2, METEOROLOGICAL DATA UPGRADE		1	<u>TARGET</u> 06/01/83
45604	QUAD CITIES 1 - NUREG-0737 II.K.3.16, CHALLENGES AND FAILURES OF RELIEF VALVES		1	08/25/83
45700	QUAD CITIES 1 - NUREG-0737 II.K.3.18, ADS ACTUATION STUDY		1	08/30/83
45164	QUAD CITIES 1 - NUREG-0737 II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION		1	09/30/83
48196	QUAD CITIES 1 - NUREG 0737 II.K.3.31 COMPLIANCE WITH 10 CFR 50.46		1	09/30/83
48272	QUAD CITIES 1 - NUREG 0737 II.K.3.28, VERIFY QUALIFICATION OF ACCUMULATORS ON ADS VALVES		1	09/30/83
44613	QUAD CITIES 1 - RV AND SV TESTING		1	10/31/83
44263	QUAD CITIES 1 - NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES		1	11/30/83
44333	QUAD CITIES 1 - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES		1	11/30/83
51194	QUAD CITIES 1 - I.D.1 CONTROL ROOM DESIGN REVIEW		2	06/30/84
51274	QUAD CITIES 1 - I.D.2. SAFETY PARAMETER DISPLAY SYSTEM			06/30/84
45976	QUAD CITIES 1 - NUREG-0737 III.A.1.2, TECHNICAL SUPPORT CENTER		1	09/30/84
46047	QUAD CITIES 1 - NUREG-0737 III.A.1.2, OPERATIONAL SUPPORT CENTER		1	09/30/84
<u>COMPLETED ACTIONS</u>				
52362	QUAD CITIES 1 - PROPOSED T.S. RE RPS POWER MONITORING SYSTEM			<u>COMPLETE</u> 02/14/84

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CONDENSED MANAGEMENT REPORT

(CONTINUATION)

QUAD CITIES 1

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS (CONTINUATION)</u>				<u>COMPLETE</u>
52318	QUAD CITIES 1 - T.S. AMENDMENT RE S/R VALVE POSITION INDICATION			02/24/84
53364	QUAD CITIES 1 - REQUESTED EXEMPTION FROM REQMTS OF APPX FOR ILRT			04/10/84
54304	QUAD CITIES 1 - CONTROL RAD. CONT. MATERIALS - DEMINIMUS LEVELS			04/13/84
52145	QUAD CITIES UNIT 1 - IMPLEMENTATION OF FW NOZZLE AND CRD-RL NOZZLE WORK			04/16/84
<u>ACTIVE ACTIONS</u>				<u>TARGET</u>
51520	QUAD CITIES 1 - ECONOMIC GENERATION CONTROL		2	07/29/83
49939	QUAD CITIES 1 - TIA-TS 48 VOLT D.C.		3	07/30/83
52080	QUAD CITIES 1 - T/A ON INSERVICE INSPECTION PROGRAM REQUIREMENTS			08/31/83
48301	QUAD CITIES 1 - PROPOSED TS CHANGES RE APP. H AND G METAL SURVEILLANCE & MIN TEMP REQ		3	09/30/83
51746	QUAD CITIES UNIT 1 CRD SCRAM TIME TESTING - TS CHANGE			09/30/83
52304	QUAD CITIES 1 - INSERVICE TESTING PROGRAM, PUMPS AND VALVES, 2ND DECADE			11/15/83
51744	QUAD CITIES UNIT 1 CHANGE TO TS ON CTMT VAC BKRS AND DG TEST RQMTS			12/30/83
52302	QUAD CITIES - 1 INSERVICE INSPECTION PROGRAM - 2ND DECADE			03/01/84
49196	QUAD CITIES 1 - CONTAINMENT OXYGEN CONTENT TS CHANGE		3	04/01/84
54361	QUAD CITIES-1 TECH SPEC CHANGES IN MAPLHGR CURVES			06/01/84
54616	QUAD CITIES 1 - REQUEST TO RESCIND ORDER RELATED TO IEB 80-17-SDV MODIFICATIONS			06/01/84
54362	QUAD CITIES-1 TECH SPEC CHANGES FOR SCRAM DISCHARGE VOLUME MODS			08/01/84
54561	QUAD CITIES 1 - TECHNICAL SPECIFICATIONS COVERED BY GENERIC LETTERS 83-36, 83-37 FOR NUREG-0737			09/30/84
<u>ANTICIPATED ACTIONS</u>				<u>INITIATION</u>
55008	QUAD CITIES UNIT 1 - 125V DC BATTERY OPERABILITY			05/11/84
53624	QUAD CITIES 1 - ITEM 1.2 - POST TRIP REVIEW-DATA AND INFORMATION CAPABILITY			11/01/84
53707	QUAD CITIES 1 - ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS			11/01/84
53790	QUAD CITIES 1- ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS			11/01/84
53873	QUAD CITIES 1 - ITEM 3.2.3 - POST-MAINTENANCE TESTING -CHANGES TO TECH. SPEC. - ALL SR COMPONENTS			11/01/84
54017	QUAD CITIES 1 - ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS			11/01/84
54100	QUAD CITIES 1 - ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL TESTING - DIVERSE TRIP FEATURES			11/01/84

DATA AS OF - 04/30/84

CONDENSED MANAGEMENT REPORT

FACILITY: QUAD CITIES 2

PLANT LOCATION: 20 MI NE OF MOLINE, ILL
 DOCKET NUMBER: 050-00265
 ARCH/ENGINEER: S&L
 IE INSPECTOR: N. CHRISSTIMOS

LICENSED POWER: 2511 MW
 DESIGN POWER: 0789 MWE
 NSSS VENDOR: GE

PROJECT MANAGER: R. BEVAN
 BRANCH CHIEF: D. VASSALLO
 LIC. ASSISTANT: S. NORRIS

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>MULTI-PLANT</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
08934	QUAD CITIES 2 - RPS POWER SUPPLY		1	COMPLETE 12/30/83
<u>ACTIVE ACTIONS</u>				
52792	QUAD CITIES 2 - ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION AND PROCEDURES			TARGET
52873	QUAD CITIES 2 - ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RTS COMPONENTS			
52954	QUAD CITIES 2 - ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING PROCEDURES/VENDOR RECOMMENDATIONS - RTS COMPONENTS			
53035	QUAD CITIES 2-ITEM 3.1.3 -POST-MAINTENANCE TESTING-CHNGS TO TECH SPECS-RTS COMPONENTS			
52263	QUAD CITIES 2 - CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPA#C-10)			
12311	QUAD CITIES 2 - HELB AND CONSEQUENTIAL SYSTEM FAILURE QUAD CITIES 2		3	05/31/83
46670	QUAD CITIES 2 - IMPLEMENTATION OF NUREG-0313, REV. 1		2	05/31/83
42479	QUAD CITIES 2 - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21)		1	07/30/83
42581	QUAD CITIES 2 - LONG TERM REVIEW CONTAINMENT PURGE AND VENT (B-24)		1	07/30/83
42887	QUAD CITIES 2 - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11		2	08/31/83
43725	QUAD CITIES 2 - SAFETY CONCERNS ASSOCIATED WITH PIPE BREAKS IN THE BWR SCRAM SYSTEM		2	10/31/83
08125	QUAD CITIES 2 - APP I TECH SPEC IMPLEMENTATION REVIEW		3	11/01/83
51125	QUAD CITIES 2 - INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)			12/01/83
51692	QUAD CITIES 2 NUREG 0737 TECH SPECS			03/01/84
07949	QUAD CITIES 2 - MARK I CONTAINMENT LONG TERM PROGRAM IMPLEMENTATION		1	06/30/84
<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>IMI ACTIONS</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
44401	QUAD CITIES 2 - NUREG-0737 II.B.1, RCS HIGH POINT VENTS		1	COMPLETE 11/17/83
47860	QUAD CITIES 2 - NUREG - 0737 II.K.3.22B, RCIC SUCTION MODIFICATIONS		1	12/28/83
45858	QUAD CITIES 2 - NUREG-0737 II.K.3.30, SB LOCA OUTLINE		1	12/28/83
<u>ACTIVE ACTIONS</u>				
44472	QUAD CITIES 2 - NUREG-0737 II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS		1	09/30/82
46336	QUAD CITIES 2 - NUREG-0737 III.A.2.2, METEOROLOGICAL DATA UPGRADE		1	06/01/83
45605	QUAD CITIES 2 - NUREG-0737 II.K.3.16, CHALLENGES AND FAILURES OF RELIEF VALVES		1	08/25/83
45701	QUAD CITIES 2 - NUREG-0737 II.K.3.18, ADS ACTUATION STUDY		1	08/30/83
45165	QUAD CITIES 2 - NUREG-0737 II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION		1	09/30/83
48197	QUAD CITIES 2 -NUREG 0737 II.K.3.31 COMPLIANCE WITH 10 CFR 50.46		1	09/30/83
44614	QUAD CITIES 2 - RV AND SV TESTING		1	10/31/83
44264	QUAD CITIES 2 - NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES		1	11/30/83
44334	QUAD CITIES 2 - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES		1	11/30/83
51195	QUAD CITIES 2 - I.D.1 CONTROL ROOM DESIGN REVIEW			06/30/84
51275	QUAD CITIES 2 - I.D.2. S FETY PARAMETER DISPLAY SYSTEM			06/30/84
45977	QUAD CITIES 2 - NUREG-0737 III.A.1.2, TECHNICAL SUPPORT CENTER		1	09/30/84
46048	QUAD CITIES 2 - NUREG-0737 III.A.1.2, OPERATIONAL SUPPORT CENTER		1	09/30/84
<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
<u>COMPLETE</u>				

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CONDENSED MANAGEMENT REPORT

(CONTINUATION)

QUAD CITIES 2

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u> (CONTINUATION)				<u>COMPLETE</u>
52025	QUAD CITIES 2 - PROPOSED OPERATING LIMIT MCPR REDUCTION			01/24/84
52315	QUAD CITIES 2 - PROPOSED CHANGES IN SAFETY-RELIEF VALVE SETPOINTS			01/24/84
52363	QUAD CITIES 2 - PROPOSED T.S. RE RPS POWER MONITORING SYSTEM			02/14/84
52319	QUAD CITIES 2 - T.S. AMENDMENT RE S/R VALVE POSITION INDICATION			02/24/84
52094	QUAD CITIES 2 - LARGE PIPE CRACK			02/28/84
51747	QUAD CITIES UNIT 2 CRD SCRAM TIME TESTING - TS CHANGE			04/12/84
54305	QUAD CITIES 2 - CONTROL RAD. CONT. MATERIALS - DEMINIMUS LEVELS			04/13/84
52146	QUAD CITIES UNIT 2 - IMPLEMENTATION OF FW NOZZLE AND CRD-RL NOZZLE WORK			04/16/84
<u>ACTIVE ACTIONS</u>				<u>TARGET</u>
51521	QUAD CITIES 2 - ECONOMIC GENERATION CONTROL		2	07/29/83
49940	QUAD CITIES 2 - TIA-TS 48 VOLT D.C.		3	07/30/83
52081	QUAD CITIES 2 - T/A ON INSERVICE INSPECTION PROGRAM REQUIREMENTS			08/31/83
48302	QUAD CITIES 2 - PROPOSED TS CHANGES RE APP. H & G, METAL SURVEILLANCE AND MIN. TEMP. REQ		2	09/30/83
51745	QUAD CITIES UNIT 2 CHANGE TO TS ON CTMT VAC BKRS AND DG TEST ROOMS			12/30/83
52303	QUAD CITIES 2- INSERVICE INSPECTION PROGRAM - 2ND DECADE			03/01/84
49197	QUAD CITIES 2 - CONTAINMENT OXYGEN CONTENT TS CHANGE		3	04/01/84
52305	QUAD CITIES 2 - INSERVICE TESTING PROGRAM, PUMPS AND VALVES, 2ND DECADE			05/01/84
54562	QUAD CITIES 2 - TECHNICAL SPECIFICATIONS COVERED BY GENERIC LETTERS 83-36, 83-37 FOR NUREG-0737			09/30/84
<u>ANTICIPATED ACTIONS</u>				<u>INITIATION</u>
55009	QUAD CITIES UNIT 2 - 125V DC BATTERY OPERABILITY			05/11/84
53625	QUAD CITIES 2 - ITEM 1.2 - POST TRIP REVIEW-DATA AND INFORMATION CAPABILITY			11/01/84
53708	QUAD CITIES 2 - ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS			11/01/84
53791	QUAD CITIES 2- ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS			11/01/84
53874	QUAD CITIES 2 - ITEM 3.2.3 - POST-MAINTENANCE TESTING -CHANGES TO TECH. SPEC. - ALL SR COMPONENTS			11/01/84
54018	QUAD CITIES - ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS			11/01/84
54101	QUAD CITIES 2 - ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL TESTING - DIVERSE TRIP FEATURES			11/01/84

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CONDENSED MANAGEMENT REPORT

FACILITY: RANCHO SECO 1

PLANT LOCATION: 25 MI SE OF SACRAMENTO, CA
 DOCKET NUMBER: 050-00312
 ARCH/ENGINEER: BECH
 IE INSPECTOR: H. CANTER

LICENSED POWER: 2772 MWT
 DESIGN POWER: 0918 MWE
 NSSS VENDOR: B&W

PROJECT MANAGER: S. MINER
 BRANCH CHIEF: J. STOLZ
 LIC. ASSISTANT: R. INGRAM

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>MULTI-PLANT</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
42601	RANCHO SECO - LONG TERM REVIEW CONTAINMENT PURGE AND VENT (B-24)		1	<u>COMPLETE</u> 10/25/83
08079	RANCHO SECO 1-CONTROL OF HEAVY LOADS OVER SPENT FUEL POOL (NUREG 0612)		1	10/25/83
42918	RANCHO SECO - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11		2	12/03/83
49616	RANCHO SECO - PLANS FOR PREVENTING EXCEEDING PTS SCREENING CRITERION		1	12/28/83
08602	RANCHO SECO - REVIEW OF ASYMMETRIC LOCA LOADS SUBMITTAL		2	01/16/84
12326	RANCHO SECO - HELB AND CONSEQUENTIAL SYSTEM FAILURE RANCHO SECO		3	01/31/84
12616	QUALITY ASSURANCE REQUIREMENTS REGARDING DIESEL GENERATOR FUEL OIL		3	01/31/84
08153	RANCHO SECO - APPENDIX I TECH SPEC IMPLEMENTATION REVIEW		1	02/22/84
<u>ACTIVE ACTIONS</u>				
52793	RANCHO SECO - ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION AND PROCEDURES			<u>TARGET</u>
52874	RANCHO SECO - ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RTS COMPONENTS			
52955	RANCHO SECO - ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING PROCEDURES/VENDOR RECOMMENDATIONS - RTS COMPONENTS			
53036	RANCHO SECO-ITEM 3.1.3 -POST-MAINTENANCE TESTING-CHANGES TO TECH SPECS-RTS COMPONENTS			
53091	RANCHO SECO - REACTOR TRIP SYSTEM RELIABILITY - VENDOR RELATED MODIFICATIONS			
53144	RANCHO SECO - ITEMS 4.2.1 & 4.2.2 - PREVENTATIVE MAINT PROGRAM FOR REACTOR TRIP BREAKERS - MAINTENANCE & TRENDING			
06710	RANCHO SECO - REACTOR COOLANT SYSTEM OVERPRESSURIZATION PROTECTION		2	06/15/84
11288	RANCHO SECO - INSERVICE TESTING		1	06/28/84
43042	RANCHO SECO - TECHNICAL SPECIFICATIONS DEFINING OPERABILITY FOR SAFETY SYSTEMS		2	06/29/84
46855	RANCHO SECO - PWR MSLB WITH CONT'D FEEDWATER ADDITION		1	07/30/84
53190	RANCHO SECO - ITEM 4.3 - AUTOAMTIC ACTUATION OF SHUNT TRIP ATTACHMENT FOR W AND B & W PLANTS			07/30/84
42507	RANCHO SECO - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21)		1	08/30/84
42126	RANCHO SECO - DECAY HEAT REMOVAL CAPABILITY TECH SPECS		3	09/30/84
49680	RANCHO SECO - REACTOR COOLANT PUMP TRIP		2	09/30/84
51126	RANCHO SECO 1 - INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)		1	09/30/84
52264	RANCHO SECO - CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPA#C-10)			09/30/84
43606	RANCHO SECO - SNUBBER INSERVICE SURVEILLANCE TECHNICAL SPECIFICATIONS		3	11/15/84
47170	RANCHO SECO 1 NATURAL CIRCULATION COOLDOWN (GENERIC LTR 81-21)		2	01/31/85
<u>TAC # TAC DESCRIPTION MULTI-PLANT PRIORITY CRITICAL DATE</u>				
<u>COMPLETED ACTIONS</u>				
44673	RANCHO SECO - NUREG-0737 II.E.1.1, AFII SYSTEM EVALUATION		1	<u>COMPLETE</u> 10/05/83
45273	RANCHO SECO - NUREG-0737 II.K.3.1, AUTO PORV ISOLATION		1	10/26/83
45319	RANCHO SECO - NUREG-0737 II.K.3.2, REPORT ON PORV FAILURES		1	10/26/83
<u>ACTIVE ACTIONS</u>				
44615	RANCHO SECO 1 - RV AND SV TESTING		1	<u>TARGET</u> 07/16/84
45859	RANCHO SECO - NUREG-0737 II.K.3.30, SB LOCA OUTLINE		1	09/30/84
45166	RANCHO SECO - NUREG-0737 II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION		1	09/30/84
45202	RANCHO SECO - NUREG-0737 II.K.2.13, THERMAL-MECHANICAL REPORT		1	09/30/84
46265	RANCHO SECO 1 - III.A.2.1 EMERGENCY PLAN UPGRADE TO MEET RULE		1	09/30/84
46337	RANCHO SECO - NUREG-0737 III.A.2.2, METEOROLOGICAL DATA UPGRADE		1	09/30/84

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CONDENSED MANAGEMENT REPORT

(CONTINUATION)

RANCHO SECO 1

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>TMI ACTIONS</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>ACTIVE ACTIONS</u> (CONTINUATION)				
48198	RANCHO SECO	-NUREG 0737 II.K.3.31 COMPLIANCE WITH 10 CFR 50.46	1	<u>TARGET</u> 09/30/84
51196	RANCHO SECO	1 - I.D.1 CONTROL ROOM DESIGN REVIEW	1	09/30/84
51276	RANCHO SECO	1 - I.D.2. SAFETY PARAMETER DISPLAY SYSTEM	1	09/30/84
44265	RANCHO SECO	- NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES	1	11/30/84
44335	RANCHO SECO	- NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES	1	11/30/84
45978	RANCHO SECO	- NUREG-0737 III.A.1.2, TECHNICAL SUPPORT CENTER	1	01/18/85
46049	RANCHO SECO	- NUREG-0737 III.A.1.2, OPERATIONAL SUPPORT CENTER	1	01/18/85
46121	RANCHO SECO	- NUREG-0737 III.A.1.2, EMERGENCY OPERATIONS FACILITY	1	01/18/85
<u>COMPLETED ACTIONS</u>				
48907	RANCHO SECO	- SEPTEMBER 30, 1982 ASLAB ORDER	1	<u>COMPLETE</u> 10/05/83
51330	RANCHO SECO	- EXEMPTION REQUEST FROM NUREG 0737 ORDER	2	10/30/83
51432	RANCHO SECO	- CORE BARREL BOLTS	1	11/28/83
49309	RANCHO SECO	- REVIEW OF GENERIC HPI NOZZLE CRACKING IN B & W PLANTS	2	12/30/83
52641	RANCHO SECO	- IMPLEMENTATION OF APPEAL BOARD DECISION (ALAB-746)	2	01/03/84
49241	RANCHO SECO	- SPENT FUEL POOL EXPANSION	2	01/20/84
52292	RANCHO SECO	- HIGH RANGE RADIATION MONITOR		03/16/84
52713	RANCHO SECO	- SALP REPORT FOR 1983 (THRU NOV 30)		04/23/84
<u>ACTIVE ACTIONS</u>				
52606	RANCHO SECO	- SUPPLEMENT NO 1 TO NUREG 0737 TECH SPEC AMEND		<u>TARGET</u>
54835	RANCHO SECO	- REACTOR VESSEL HEAD VENT		
54349	RANCHO SECO	- INTEGRATED SCHEDULES		
54351	RANCHO SECO	-SUPPLEMENT NO. 2 TO NUREG 0737 TECH SPECS (AMMEND #83 SUPPL #2)	1	
54352	RANCHO SECO	-NUREG 0737 TECH SPEC - GL83-37 (AMMEND #100)	1	
51696	RANCHO SECO	- MINI PURGE SYSTEM	2	
52307	RANCHO SECO	- AUX FEEDWATER HEADER INSPECTION	2	11/30/83
52561	RANCHO SECO	- 10CFR50.54 SHIFT STAFFING EXTENSION REQUEST	2	04/30/84
54441	RANCHO SECO	- ISI INTERVAL CLARIFICATION	1	05/30/84
54487	RANCHO SECO	- NNI FAILURE INVESTIGATION	2	05/30/84
54271	RANCHO SECO	- CONTAINMENT RAD MONITORS SURVEILLANCE (AMEND 108)	2	05/30/84
42049	RANCHO SECO	- LCD FOR DG OPERABILITY	2	05/30/84
49703	RANCHO SECO	- TECH SPEC PRESSURE TEMP LIMITS	1	05/30/84
52291	RANCHO SECO	- ALTERNATE SHUTDOWN CAPABILITY		06/29/84
51695	RANCHO SECO	- ALLOWABLE LEAKAGE FOR REACTOR BUILDING SPRAY AND DECAY HEAT REMOVAL		06/29/84
53329	RANCHO SECO	- LOCAL LEAK RATE TESTING FOR PURGE AND 12 INCHES CONTAINMENT VALVES		06/29/84
53510	RANCHO SECO	- CYCLE 7 RELOAD REPORT VOLUME 1	2	06/29/84
49825	RANCHO SECO	- SMALL BREAK LOCA PROCEDURES AND MAINTAINING PROPER SG LEVEL		05/30/84
49575	RANCHO SECO	- IAEA SAFEGUARDS INSPECTIONS	2	06/30/84
53444	RANCHO SECO	- APPENDIX R EXEMPTION REQUEST FOR AUX BUILDING	2	06/30/84
54836	RANCHO SECO	- SECURITY PLAN AMENDMENT (#12)	2	07/01/84
53388	RANCHO SECO	- 12 "REACTOR BUILDING EQUALIZING VALVES OPERABILITY		07/09/84
51697	RANCHO SECO	- ORGANIZATIONAL CHANGES	2	07/29/84
49574	RANCHO SECO	- REACTOR BUILDING POST TENSIONING TENDON SURVEILLANCE	2	07/30/84
52607	RANCHO SECO	- NUCLEAR INSTRUMENT CALIBRATION	2	08/28/84
52656	RANCHO SECO	- AFW SYSTEM INTERNAL MISSLES	2	09/28/84
54410	RANCHO SECO	- RCS VENTS TECHNICAL SPECIFICATIONS NUREG 073 ITEM II.B.1	2	09/30/84
48129	RANCHO SECO	- REACTOR COOLANT PUMP SEAL DAMAGE	3	09/30/84

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CONDENSED MANAGEMENT REPORT

(CONTINUATION)

RANCHO SECO 1

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>ACTIVE ACTIONS</u> (CONTINUATION)				
13099	RANCHO SECO - SEISMIC ASSESSMENT OF FOOTHILLS FAULT SYSTEM		3	<u>TARGET</u> 09/30/84
43190	RANCHO SECO - AUXILIARY FEEDWATER SYSTEM UPGRADE		1	09/30/84
42933	RANCHO SECO - ATMOSPHERIC DUMP VALVE CIRCUITRY MODIFICATION		1	09/30/84
53509	RANCHO SECO - NUREG 0630 IMPLEMENTATION		2	10/17/84
52605	RANCHO SECO - INSTRUMENT SURVEILLANCE REQUIREMENTS		2	11/15/84
51007	RANCHO SECO - ISI LICENSE AMENDMENT		2	11/15/84
<u>ANTICIPATED ACTIONS</u>				
53626	RANCHO SECO - ITEM 1.2 - POST TRIP REVIEW-DATA AND INFORMATION CAPABILITY			<u>INITIATION</u> 11/01/84
53709	RANCHO SECO - ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS			11/01/84
53792	RANCHO SECO- ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS			11/01/84
53875	RANCHO SECO - ITEM 3.2.3 - POST-MAINTENANCE TESTING -CHANGES TO TECH. SPEC. - ALL SR COMPONENTS			11/01/84
53931	-ITEMS 4.2.3 & 4.2.4-PREVENTATIVE MAINT. PROGRAM FOR REACTOR TRIP BREAKERS-LIFE TESTING & REPLACEMENT			11/01/84
53958	RANCHO SECO - ITEM 4.4 - IMPROVEMENTS IN MAINTENANCE AND TEST PROCEDURES FOR B & W PLANTS			11/01/84
54019	RANCHO SECO - ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS			11/01/84
54102	RANCHO SECO - ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL TESTING - DIVERSE TRIP FEATURES			11/01/84

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CONDENSED MANAGEMENT REPORT

FACILITY: ROBINSON 2

PLANT LOCATION: 5 MI NW OF HARTSVILLE, SC
 DOCKET NUMBER: 050-00261
 ARCH/ENGINEER: EBASCO
 IE INSPECTOR: S. WEISS

LICENSED POWER: 2300 MWT
 DESIGN POWER: 0700 MWE
 NSSS VENDOR: WEST

PROJECT MANAGER: G. REQUA
 BRANCH CHIEF: S. VARGA
 LIC. ASSISTANT: C. PARRISH

TAC #	TAC DESCRIPTION	MULTI-PLANT	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				
43604	ROBINSON - EXEMPTION TO APPENDIX R FROM COLD SHUTDOWN WITHIN 72 HOURS		1	<u>COMPLETE</u> 11/22/83
43605	ROBINSON - EXEMPTION TO APPENDIX R ON CABLE PENETRATION TEMPERATURE		1	11/25/83
49610	ROBINSON 2 - PLANS FOR PREVENTING EXCEEDING PTS SCREENING CRITERION		1	12/05/83
42560	ROBINSON 2 - LONG TERM REVIEW CONTAINMENT PURGE AND VENT (B-24)		1	12/05/83
12275	ROBINSON 2 - HELB AND CONSEQUENTIAL SYSTEM FAILURE H.B. ROBINSON 2		3	01/31/84
08522	ROBINSON 2 - REVIEW OF ASYMMETRIC LOCA LOADS SUBMITTAL		2	02/01/84
08511	ROBINSON 2 - TECH SPEC SURVEILLANCE REQMS. FOR MECHANICAL SNUBBERS		2	03/25/84

TAC #	TAC DESCRIPTION	MULTI-PLANT	PRIORITY	CRITICAL DATE
<u>ACTIVE ACTIONS</u>				
52265	ROBINSON 2 - CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPA#C-10)			<u>TARGET</u>
52794	ROBINSON 2 - ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION AND PROCEDURES			
52875	ROBINSON 2 - ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RTS COMPONENTS			
52956	ROBINSON 2 - ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING PROCEDURES/VENDOR RECOMMENDATIONS - RTS COMPONENTS			
53037	ROBINSON 2-ITEM 3.1.3 -POST-MAINTENANCE TESTING-CHANGES TO TECH SPECS-RTS COMPONENTS			
53092	ROBINSON 2 - REACTOR TRIP SYSTEM RELIABILITY - VENDOR RELATED MODIFICATIONS			
53145	ROBINSON 2 - ITEMS 4.2.1 & 4.2.2 - PREVENTATIVE MAINT PROGRAM FOR REACTOR TRIP BREAKERS - MAINTENANCE & TRENDING			
53191	ROBINSON 2 - ITEM 4.3 - AUTOAMTIC ACTUATION OF SHUNT TRIP ATTACHMENT FOR W AND B & W PLANTS			
08126	ROBINSON 2 - APPENDIX I TECH SPEC IMPLEMENTATION REVIEW		3	04/20/83
43636	ROBINSON 2 - SEISMIC QUALIFICATION OF THE AUXILIARY FEEDWATER SYSTEM		1	04/27/84
08080	ROBINSON 2 -CONTROL OF HEAVY LOADS OVER SPENT FUEL POOL (NUREG 0612)		2	06/15/84
43381	ROBINSON - EXEMPTION FOR FIRE PROTECTION OIL COLLECTION SYSTEM		2	06/15/84
49753	ROBINSON 2 - "NUREG 0737 TECH SPECS (GENERIC LETTER 82-16 82-16 OR 83-02)"			06/25/84
51127	ROBINSON 2 - INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)		1	06/29/84
42466	ROBINSON 2 - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21)		1	06/30/84
12978	ROBINSON 2 - ADEQUACY OF STATION ELECTRIC. DISTR SYS. VOLTAGES		2	10/28/84
49690	ROBINSON 2 - REACTOR COOLANT PUMP TRIP (GL 83 10D)		2	10/28/84

TAC #	TAC DESCRIPTION	TMI ACTIONS	PRIORITY	CRITICAL DATE
<u>ACTIVE ACTIONS</u>				
44336	ROBINSON 2 - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES		1	<u>TARGET</u>
45167	ROBINSON 2 - NUREG-0737 II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION		1	
45860	ROBINSON 2 - NUREG-0737 II.K.3.30, SB LOCA OUTLINE		1	
45979	ROBINSON 2 - NUREG-0737 III.A.1.2, TECHNICAL SUPPORT CENTER		1	
46050	ROBINSON 2 - NUREG-0737 III.A.1.2, OPERATIONAL SUPPORT CENTER		1	
46122	ROBINSON 2 - NUREG-0737 III.A.1.2, EMERGENCY OPERATIONS FACILITY		1	
46478	ROBINSON 2 - NUREG-0737 III.D.3.4, CONTROL ROOM HABITABILITY		1	
44474	ROBINSON 2 - NUREG-0737 II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS		1	03/30/84
47633	ROBINSON 2 - NUREG 0737 ITEM II F1.4 CONTAINMENT PRESSURE INSTRUMENT		1	03/30/84
47704	ROBINSON 2 - NUREG 0737 ITEM II F1.5 CONTAINMENT WATER LEVEL MONITOR		1	03/30/84
47775	ROBINSON 2 - NUREG 0737 ITEM II F1.6 CONTAINMENT HYDROGEN MONITOR		1	03/30/84
44687	ROBINSON 2 - NUREG-0737 II.E.1.2.1, AFW SAFETY GRADE AUTO INITIATION		1	04/13/84
44728	ROBINSON 2 - NUREG-0737 II.E.1.2.2, AFW SAFETY GRADE FLOW INDICATION		1	04/13/84

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CONDENSED MANAGEMENT REPORT

(CONTINUATION)

ROBINSON 2

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>IMI ACTIONS</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>ACTIVE ACTIONS (CONTINUATION)</u>				
46338	ROBINSON 2 - NUREG-0737 III.A.2.2, METEOROLOGICAL DATA UPGRADE		1	<u>TARGET</u> 09/30/84
48199	ROBINSON 2 - NUREG 0737 II.K.3.31 COMPLIANCE WITH 10 CFR 50.46		1	09/30/84
51197	ROBINSON 2 - I.D.1 CONTROL ROOM DESIGN REVIEW			09/30/84
51277	ROBINSON 2 - I.D.2. SAFETY PARAMETER DISPLAY SYSTEM			09/30/84
46901	ROBINSON 2 - II K2.13 THERMAL MECHANICAL REPORT		1	10/01/84
44616	ROBINSON 2 - RV AND SV TESTING		1	10/31/84
<u>PLANT SPECIFIC</u>				
<u>TAC #</u>	<u>TAC DESCRIPTION</u>		<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
43828	ROBINSON 2 - ALTERNATE SHUTDOWN SYSTEM		1	<u>COMPLETE</u> 11/22/83
53230	H.B. ROBINSON 2 - HEARING PREPARATION REPAIR			02/10/84
48877	H. B. ROBINSON - STEAM GENERATOR REPAIR		1	02/28/84
49438	H. B. ROBINSON #2 GENERIC LETTERS 82-17 & 82-23		1	02/28/84
52553	H.B. ROBINSON 2 - MISCL TECH SPECS			03/28/84
52632	H.B. ROBINSON 2 - REVISE TEC SPEC SURVEILLANCE REQUIREMENTS			03/30/84
<u>ACTIVE ACTIONS</u>				
52696	H.B. ROBINSON 2 - T/S ORGANIZATION CHANGE		3	<u>TARGET</u>
52697	H. B. ROBINSON 2 - VENT T/S CHANGE CONTAINMENT PURGE			
52615	H.B. ROBINSON-2 PTS FLUX REDUCTION PROGRAM			
52616	H.B. ROBINSON 2 SAMPLING OF REACTOR VESSEL HEAD WELDS			
53336	H. B. ROBINSON 2 - APPENDIX J TEST PROGRAM EXEMPTIONS			
54484	H. B. ROBINSON - 2 WASTE DISPOSAL PROCEDURE			
54619	H.B. ROBINSON 2 - REPORTING REQUIREMENT TEC. SPECS. (6C 83-43)			
13024	ROBINSON 2 - ENVIRONMENTAL TECHNICAL SPECIFICATIONS		3	
54192	ROBINSON 2 - SEALING FACTORS - RADWASTE			02/24/84
51796	HB ROBINSON 2 - REVISE TEC SPECS TO BE CONSISTANT WITH PRESENT SA		3	03/28/84
51967	H.B. ROBINSON 2 - REVISE ISI TEC SPEC TO CORRECT ERROR		3	03/28/84
49294	H.B. ROBINSON 2 - LOSS OF FEEDWATER TRANSIENT		2	03/28/84
49223	H. B. ROBINSON 2 - REVIEW AFW PROTECTION FROM TORNADO MISSILES		3	03/30/84
52440	H.B. ROBINSON 2 - SECOND 10 YR ISI PROGRAM		1	04/13/84
52617	H.B. ROBINSON 2 HEATUP & COOLDOWN CURVE TS REVISION		2	04/20/84
51966	H.B. ROBINSON 2 - REVISE ISI TEC SPECS TO INCORPORATE SECT XI OF ASME CWE			04/20/84
53482	H.B. ROBINSON 2 AFW INITIATION T/S REVISION		3	04/20/84
52695	H.B. ROBINSON 2 - CYCLE 10 OPERATIONS			04/21/84
11215	ROBINSON 2 - INSERVICE TESTING - PUMPS AND VALVES		1	04/27/84
51339	H. B. ROBINSON 2 - EXEMPTION TO APPENDIX R, III.G.2		1	05/31/84
54202	H.B. ROBINSON-2 - APPENDIX R SEC III G COLD SHUTDOWN			05/31/84
54994	H.B. ROBINSON - 2 EXEMPTION FROM 10CFR50 APPENDIX E SECTION IV.F.1.			06/01/84
53481	H.B. ROBINSON 2 USE OF 10 SPARE SPACES FOR FUEL STORAGE		2	06/20/84
51962	H.B. ROBINSON 2 - CHANGE TECH SPECS - DEDICATED SHUTDOWN SYSTEM		3	06/25/84
53326	H. B. ROBINSON-2 - DEDICATED SHUTDOWN SYSTEM TECHNICAL SPECIFICATIONS			06/29/84
53229	H.B. ROBINSON 2 - FLUX REDUCTION REPORT & CAPSULE T REPORT			09/30/84
54563	ROBINSON 2 - TECHNICAL SPECIFICATIONS COVERED BY GENERIC LETTERS 83-36, 83-37 FOR NUREG-0737			09/30/84
54411	ROBINSON 2- RCS VENTS TECHNICAL SPECIFICATIONS NUREG 073 ITEM II.B.1			09/30/84
<u>ANTICIPATED ACTIONS</u>				
54896	H.B. ROBINSON 2 - ABT FOR AFW INITIATION/FLOW			<u>INITIATION</u> 05/03/84
53627	ROBINSON 2 - ITEM 1.2 - POST TRIP REVIEW-DATA AND INFORMATION CAPABILITY			11/01/84

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CONDENSED MANAGEMENT REPORT

(CONTINUATION)

ROBINSON 2

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>ANTICIPATED ACTIONS (CONTINUATION)</u>				
53710	ROBINSON 2 - ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS			<u>INITIATION</u> 11/01/84
53793	ROBINSON 2- ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS			11/01/84
53876	ROBINSON 2 - ITEM 3.2.3 - POST-MAINTENANCE TESTING -CHANGES TO TECH. SPEC. - ALL SR COMPONENTS			11/01/84
53932	-ITEMS 4.2.3 & 4.2.4-PREVENTATIVE MAINT. PROGRAM FOR REACTOR TRIP BREAKERS-LIFE TESTING & REPLACEMENT			11/01/84
54020	ROBINSON 2 - ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS			11/01/84
54103	ROBINSON 2 - ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL TESTING - DIVERSE TRIP FEATURES			11/01/84

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CONDENSED MANAGEMENT REPORT

FACILITY: SALEM 1

PLANT LOCATION: 20 MI S OF WILMINGTON, DEL
 DOCKET NUMBER: 050-00272
 ARCH/ENGINEER: PUBSERVE
 IE INSPECTOR: L. NORRHOLM

LICENSED POWER: 3338 MWT
 DESIGN POWER: 1090 MWE
 NSSS VENDOR: WEST

PROJECT MANAGER: D. FISCHER
 BRANCH CHIEF: S. VARGA
 LIC. ASSISTANT: C. PARRISH

TAC #	TAC DESCRIPTION	MULTI-PLANT	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				
12276	SALEM 1 - HELB AND CONSEQUENTIAL SYSTEM FAILURE SALEM 1		3	<u>COMPLETE</u> 01/31/84
<u>ACTIVE ACTIONS</u>				
52876	SALEM 1 - ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RTS COMPONENTS			<u>TARGET</u>
52957	SALEM 1 - ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING PROCEDURES/VENDOR RECOMMENDATIONS - RTS COMPONENTS			
53038	SALEM 1-ITEM 3.1.3 -POST-MAINTENANCE TESTING-CHANGES TO TECH SPECS-RTS COMPONENTS			
53093	SALEM 1 - REACTOR TRIP SYSTEM RELIABILITY - VENDOR RELATED MODIFICATIONS			
53146	SALEM 1 - ITEMS 4.2.1 & 4.2.2 - PREVENTATIVE MAINTENANCE PROGRAM FOR REACTOR TRIP BREAKERS - MAINTENANCE & TRENDING			
53192	SALEM 1 - ITEM 4.3 - AUTOAMTIC ACTUATION OF SHUNT TRIP ATTACHMENT FOR W AND B & W PLANTS			
52795	SALEM 1 - ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION AND PROCEDURES			
52710	SALEM 1 - ADD SURVEILLANCE TESTING ON RTB'S			05/01/84
49754	SALEM 1 - "NUREG 0737 TECH SPECS (GENERIC LETTER 82-16 82-16 OR 83-02)"			05/01/84
51128	SALEM 1 - INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)			05/01/84
08154	SALEM 1 - APPENDIX I TECH SPEC IMPLEMENTATION REVIEW		3	05/01/84
49669	SALEM 1 - REACTOR COOLANT PUMP TRIP			07/01/84
52266	SALEM 1 - CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPA#C-10)			09/30/84
08824	SALEM 1 - REVIEW OF ASYMMETRIC LOCA LOADS SUBMITTAL		2	09/30/84
08081	SALEM 1 -CONTROL OF HEAVY LGADS OVER SPENT FUEL POOL (NUREG 0612)		2	09/30/84
42467	SALEM 1 - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21)		1	09/30/84
<u>TMI ACTIONS</u>				
<u>COMPLETED ACTIONS</u>				
45247	SALEM 1 - NUREG-0737 II.K.3.1, AUTO PORV ISOLATION		1	<u>COMPLETE</u> 10/06/83
45293	SALEM 1 - NUREG-0737 II.K.3.2, REPORT ON PORV FAILURES		1	10/06/83
44476	SALEM 1 - NUREG-0737 II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS		1	11/04/83
47817	SALEM 1 - NUREG 0737 ITEM II K2.17 POTENTIAL FOR VOIDING IN RCS		1	01/30/84
<u>ACTIVE ACTIONS</u>				
46340	SALEM 1 - NUREG-0737 III.A.2.2, METEOROLOGICAL DATA UPGRADE		1	<u>TARGET</u> 06/01/84
46480	SALEM 1 - NUREG-0737 III.D.3.4, CONTROL ROOM HABILABILITY		1	06/01/84
45862	SALEM 1 - NUREG-0737 II.K.3.30, SB LOCA OUTLINE		1	06/30/84
45169	SALEM 1 - NUREG-0737 II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION		1	06/30/84
44618	SALEM 1 - RV AND SV TESTING		1	09/30/84
45981	SALEM 1 - NUREG-0737 III.A.1.2, TECHNICAL SUPPORT CENTER		1	09/30/84
46052	SALEM 1 - NUREG-0737 III.A.1.2, OPERATIONAL SUPPORT CENTER		1	09/30/84
46124	SALEM 1 - NUREG-0737 III.A.1.2, EMERGENCY OPERATIONS FACILITY		1	09/30/84
46903	SALEMS 1 - II K2.13 THERMAL MECHANICAL REPORT		1	09/30/84
48201	SALEM 1 -NUREG 0737 II.K.3.31 COMPLIANCE WITH 10 CFR 50.46		1	09/30/84
51198	SALEM 1 - I.D.1 CONTROL ROOM DESIGN REVIEW			09/30/84
44268	SALEM 1 - NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES		1	11/30/84
44338	SALEM 1 - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES		1	11/30/84
51278	SALEM 1 - I.D.2. SAFETY PARAMETER DISPLAY SYSTEM			12/30/84

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CONDENSED MANAGEMENT REPORT

(CONTINUATION)

SALEM 1

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				<u>COMPLETE</u>
52322	SALEM 1 - EXTENDED I.L.R.T INTERVAL FOR UNIT 1			10/31/83
49296	SALEM 1 - MODIFY FUEL ENRICHMENT			11/22/83
52706	SALEM 1 - RADIOLOGICAL EFFLUENT CONTROLS			02/28/84
52708	SALEM 1 - NUCLEAR DEPT. REORGANIZATION			02/28/84
<u>ACTIVE ACTIONS</u>				<u>TARGET</u>
54953	SALEM 1 - SECURITY PLAN CHANGES			
52704	SALEM 1 - REVISE AIRLOCK TECH SPEC			01/20/84
47102	SALEM 1 - NUCLEAR DEPARTMENT REORGANIZATION		2	05/01/84
52323	SALEM 1 - REV. 0 TO TRAINING PROCEDURE TP-305			05/15/84
52702	SALEM 1 - ADD MANUAL INITIATION OF AUX FEED			06/01/84
51881	SALEM 1 - 10" PRESSURE/VACUUM VALVE FOR CONTAINMENT VENTING VENTING			06/01/84
51570	SALEM 1 - AUTO SWITCHOVER MODIFICATION			06/15/84
48993	SALEM 1 RCS INVENTORY BALANCE			06/15/84
48991	SALEM 1 MSIV SOLENOID VALVES			06/25/84
47122	SALEM 1 - ENVIRONMENTAL PROTECTION PLAN		2	07/01/84
49908	SALEM - ATWS EVENT FOLLOWUP			07/31/84
53539	SALEM 1 - ADDITIONAL APPENDIX R EXEMPTIONS			08/01/84
54412	SALEM 1- RCS VENTS TECHNICAL SPECIFICATIONS NUREG 073 ITEM II.B.1			09/30/84
54564	SALEM 1 - TECHNICAL SPECIFICATIONS COVERED BY GENERIC LETTERS 83-36, 83-37 FOR NUREG-0737			09/30/84
49269	SALEM UNIT 1 - REVISED ROD EXCHANGE METHODOLOGY			09/30/84
<u>ANTICIPATED ACTIONS</u>				<u>INITIATION</u>
53628	SALEM 1 - ITEM 1.2 - POST TRIP REVIEW-DATA AND INFORMATION CAPABILITY			11/01/84
53711	SALEM 1 - ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS			11/01/84
53794	SALEM 1- ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS			11/01/84
53877	SALEM 1 - ITEM 3.2.3 - POST-MAINTENANCE TESTING -CHANGES TO TECH. SPEC. - ALL SR COMPONENTS			11/01/84
53933	-ITEMS 4.2.3 & 4.2.4-PREVENTATIVE MAINT. PROGRAM FOR REACTOR TRIP BREAKERS-LIFE TESTING & REPLACEMENT			11/01/84
54021	SALEM 1 - ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS			11/01/84
54104	SALEM 1 - ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL CAPABILITY			11/01/84

DATA AS OF - 04/30/84

CONDENSED MANAGEMENT REPORT

FACILITY: SALEM 2

PLANT LOCATION: 20 MI S OF WILMINGTON, DEL
 DOCKET NUMBER: 050-00311
 ARCH/ENGINEER: PUBSERVE
 IE INSPECTOR:

LICENSED POWER: 3411 MW
 DESIGN POWER: 1115 MWE
 NSSS VENDOR: WEST

PROJECT MANAGER: D. FISCHER
 BRANCH CHIEF: S. VARGA
 LIC. ASSISTANT: C. PARRISH

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>MULTI-PLANT</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>ACTIVE ACTIONS</u>				
52796	SALEM 2 - ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION AND PROCEDURES			<u>TARGET</u>
52877	SALEM 2 - ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RTS COMPONENTS			
52958	SALEM 2 - ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING PROCEDURES/VENDOR RECOMMENDATIONS - RTS COMPONENTS			
53039	SALEM 2-ITEM 3.1.3 -POST-MAINTENANCE TESTING-CHANGES TO TECH SPECS-RTS COMPONENTS			
53094	SALEM 2 - REACTOR TRIP SYSTEM RELIABILITY - VENDOR RELATED MODIFICATIONS			
53147	SALEM 2 - ITEMS 4.2.1 & 4.2.2 - PREVENTATIVE MAINTENANCE PROGRAM FOR REACTOR TRIP BREAKERS - MAINTENANCE & TRENDING			
53193	SALEM 2 - ITEM 4.3 - AUTOAMTIC ACTUATION OF SHUNT TRIP ATTACHMENT FOR W ANDB & W PLANTS			
48327	SALEM 2 - APPENDIX I TECHNICAL SPECIFICATIONS REVIEW		3	05/01/84
49755	SALEM 2 -"NUREG 0737 TECH SPECS (GENERIC LETTER 82-16 82-16 OR 83-02)"			05/01/84
51129	SALEM 2 - INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)			05/01/84
49670	SALEM 2 - REACTOR COOLANT PUMP TRIP			07/01/84
42573	SALEM 2 - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21)		1	09/30/84
52267	SALEM 2 - CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPA#C-10)			09/30/84
46815	SALEM 2 -CONTROL OF HEAVY LOADS OVER SPENT FUEL POOL (NUREG 0612)		2	09/30/84
<u>COMPLETED ACTIONS</u>				
45248	SALEM 2 - NUREG-0737 II.K.3.1, AUTO PORV ISOLATION		1	<u>COMPLETE</u> 10/06/83
45294	SALEM 2 - NUREG-0737 II.K.3.2, REPORT ON PORV FAILURES		1	10/06/83
44477	SALEM 2 - NUREG-0737 II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS		1	11/04/83
47818	SALEM 2 - NUREG 0737 ITEM II K2.17 POTENTIAL FOR VOIDING IN RCS		1	01/30/84
<u>ACTIVE ACTIONS</u>				
46341	SALEM 2 - NUREG-0737 III.A.2.2, METEOROLOGICAL DATA UPGRADE		1	<u>TARGET</u> 06/01/84
45863	SALEM 2 - NUREG-0737 II.K.3.30, SB LOCA OUTLINE		1	06/30/84
45170	SALEM 2 - NUREG-0737 II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION		1	06/30/84
44619	SALEM 2 - RV AND SV TESTING		1	09/30/84
45982	SALEM 2 - NUREG-0737 III.A.1.2, TECHNICAL SUPPORT CENTER		1	09/30/84
46053	SALEM 2 - NUREG-0737 III.A.1.2, OPERATIONAL SUPPORT CENTER		1	09/30/84
46125	SALEM 2 - NUREG-0737 III.A.1.2, EMERGENCY OPERATIONS FACILITY		1	09/30/84
46904	SALEM 2 - II K2.13 THERMAL MECHANICAL REPORT		1	09/30/84
48202	SALEM 2 -NUREG 0737 II.K.3.31 COMPLIANCE WITH 10 CFR 50.46		1	09/30/84
51199	SALEM 2 - I.D.1 CTRLR ROOM DESIGN REVIEW			09/30/84
44269	SALEM 2 - NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES		1	11/30/84
44339	SALEM 2 - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES		1	11/30/84
51279	SALEM 2 - I.D.2. SAFETY PARAMETER DISPLAY SYSTEM			12/30/84
<u>COMPLETED ACTIONS</u>				
49295	SALEM 2 - MODIFY FUEL ENRICHMENT			<u>COMPLETE</u> 11/22/83
48988	SALEM UNIT 2 SURVEILLANCE OF SNUBBERS			01/27/84

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CONDENSED MANAGEMENT REPORT

(CONTINUATION)

SALEM 2

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS (CONTINUATION)</u>				<u>COMPLETE</u>
52707	SALEM 2 - RADIOLOGICAL EFFLUENT CONTROLS			02/28/84
52709	SALEM 2 - NUCLEAR DEPT. REORGANIZATION			02/28/84
<u>ACTIVE ACTIONS</u>				<u>TARGET</u>
54468	SALEM UNIT 2. SURVEILLANCE CAPSULE REPORT TIME EXTENSION			
54954	SALEM 2 - SECURITY PLAN CHANGES			
46713	SALEM 2 - FIRE PROTECTION REVIEW OPEN ITEMS		3	04/20/84
52712	SALEM 2 - FIRE PROTECTION LICENSE CONDITIONS			04/20/84
52705	SALEM 2 - REVISE AIRLOCK TECH SPEC			04/30/84
52711	SALEM 2 - ADD SURVEILLANCE TESTING ON RTB'S			05/01/84
47103	SALEM 2 - NUCLEAR DEPARTMENT REORGANIZATION		3	05/01/84
51882	SALEM 1 - 10" PRESSURE/VACUUM VALVE FOR CONTAINMENT VENTING			06/01/84
52707	SALEM 2 - ADD MANUAL INITIATION OF AUX FEED			06/01/84
53541	SALEM 2 - CHANGE CONTROL ROOM POSITIVE PRESSURE TEST VALVE			06/15/84
51571	SALEM 2 - AUTO SWITCHOVER MODIFICATION			06/15/84
48994	SALEM 2 RCS INVENTORY BALANCE			06/15/84
48992	SALEM 2 MSIV SOLENOID VALVES			06/25/84
47123	SALEM 2 - ENVIRONMENTAL PROTECTION PLAN		3	07/01/84
53540	SALEM 2 - ADDITIONAL APPENDIX R EXEMPTIONS			08/01/84
46922	SALEM 2 - CONTROL ROOM HUMAN FACTORS REVIEW			09/30/84
47887	SALEM 2 - SUMPT TEST REPORT		3	09/30/84
54565	SALEM 2 - TECHNICAL SPECIFICATIONS COVERED BY GENERIC LETTERS 83-36, 83-37 FOR NUREG-0737			09/30/84
54413	SALEM 2 - RCS VENTS TECHNICAL SPECIFICATIONS NUREG 073 ITEM II.B.1			09/30/84
<u>ANTICIPATED ACTIONS</u>				<u>INITIATION</u>
52327	SALEM 2 - REV. 0 TO TRAINING PROCEDURE TP-305			10/01/84
53629	SALEM 2 - ITEM 1.2 - POST TRIP REVIEW-DATA AND INFORMATION CAPABILITY			11/01/84
53712	SALEM 2 - ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS			11/01/84
53795	SALEM 2 - ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS			11/01/84
53878	SALEM 2 - ITEM 3.2.3 - POST-MAINTENANCE TESTING -CHANGES TO TECH. SPEC. - ALL SR COMPONENTS			11/01/84
53934	-ITEMS 4.2.3 & 4.2.4-PREVENTATIVE MAINT. PROGRAM FOR REACTOR TRIP BREAKERS-LIFE TESTING & REPLACEMENT			11/01/84
54022	SALEM 2 - ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS			11/01/84
54105	SALEM 2 - ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL CAPABILITY			11/01/84

DATA AS OF - 04/30/84

CONDENSED MANAGEMENT REPORT

FACILITY: SAN ONOFRE 1

PLANT LOCATION: 5 MI S OF SAN CLEMENTE, CA
 DOCKET NUMBER: 050-00206
 ARCH/ENGINEER: BECH
 IE INSPECTOR: L. MILLER

LICENSED POWER: 1347 MWT
 DESIGN POWER: 0436 MWE
 NSSS VENDOR: WEST

PROJECT MANAGER: W. PAULSON
 BRANCH CHIEF: D. CRUTCHFIELD
 LIC. ASSISTANT: H. SMITH

TAC #	TAC DESCRIPTION	MULTI-PLANT	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				
13178	SAN ONOFRE 1 - REACTOR VESSEL CAVITY SEAL RING MISSILE POTENTIAL		2	<u>COMPLETE</u> 02/01/84
42615	SAN ONOFRE 1 - LONG TERM REVIEW CONTAINMENT PURGE AND VENT (B-24)		1	02/17/84
<u>ACTIVE ACTIONS</u>				
52248	SAN ONOFRE 1 - CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPA#C-10)			<u>TARGET</u>
52797	SAN ONOFRE 1 - ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION AND PROCEDURES			
52878	SAN ONOFRE 1 - ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RTS COMPONENTS			
53040	SAN ONOFRE 1 - ITEM 3.1.3 - POST-MAINTENANCE TESTING - CHNGS TO TECH SPECS - RTS COMPONENTS			
53095	SAN ONOFRE 1 - REACTOR TRIP SYSTEM RELIABILITY - VENDOR RELATED MODIFICATIONS			
53148	SAN ONOFRE 1 - ITEMS 4.2.1 & 4.2.2 - PREVENTATIVE MAINT PROGRAM FOR REACTOR TRIP BREAKERS - MAINTENANCE & TRENDING			
53194	SAN ONOFRE 1 - ITEM 4.3 - AUTOAMTIC ACTUATION OF SHUNT TRIP ATTACHMENT FOR W AND B & W PLANTS			
51130	SAN ONOFRE 1 - INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)		1	12/31/83
11231	SAN ONOFRE 1 - INSERVICE TESTING (IST)		1	04/30/84
08101	SAN ONOFRE 1 - APPENDIX I TECH SPEC IMPLEMENTATION REVIEW		3	04/30/84
05082	SAN ONOFRE 1 - CONTROL OF HEAVY LOADS OVER SPENT FUEL POOL (NUREG 0612)		2	06/30/84
42916	SAN ONOFRE 1 - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11		2	06/30/84
42516	SAN ONOFRE 1 - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21)		1	06/30/84
49756	SAN ONOFRE 1 - "NUREG 0737 TECH SPECS (GENERIC LETTER 82-16 OR 83-02)"		1	06/30/84
46518	SAN ONOFRE 1 - FIRE PROTECTION TECH SPECS		1	06/30/84
49685	SAN ONOFRE 1 - REACTOR COOLANT PUMP TRIP		2	07/01/84
42131	SAN ONOFRE 1 - DECAY HEAT REMOVAL CAPABILITY TECH SPECS		2	09/30/84
TAC #	TAC DESCRIPTION	TMI ACTIONS	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				
47819	SAN ONOFRE 1 - NUREG 0737 ITEM II K2.17 POTENTIAL FOR VOIDING IN RCS		1	<u>COMPLETE</u> 12/19/83
46482	SAN ONOFRE 1 - NUREG-0737 III.D.3.4, CONTROL ROOM HABITABILITY		1	02/03/84
46128	SAN ONOFRE 1 - NUREG-0737 I.A.1.3.1, SHIFT MANNING OVERTIME LIMITS			02/17/84
47637	SAN ONOFRE 1 - NUREG 0737 ITEM II F1.4 CONTAINMENT PRESSURE INSTRUMENT		1	04/16/84
47708	SAN ONOFRE 1 - NUREG 0737 ITEM II F1.5 CONTAINMENT WATER LEVEL MONITOR		1	04/16/84
47779	SAN ONOFRE 1 - NUREG 0737 ITEM II F1.4 CONTAINMENT HYDROGEN MONITOR		1	04/16/84
<u>ACTIVE ACTIONS</u>				
44620	SAN ONOFRE 1 - RV AND SV TESTING		1	<u>TARGET</u> 04/30/84
45171	SAN ONOFRE 1 - NUREG-0737 II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION		1	06/30/84
45864	SAN ONOFRE 1 - NUREG-0737 II.K.3.30, SB LOCA OUTLINE		1	06/30/84
44478	SAN ONOFRE 1 - NUREG-0737 II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS		1	06/30/84
46342	SAN ONOFRE 1 - NUREG-0737 III.A.2.2, METEOROLOGICAL DATA UPGRADE		1	06/30/84
46906	SAN ONOFRE 1 - II K2.13 THERMAL MECHANICAL REPORT		1	09/30/84
45983	SAN ONOFRE 1 - NUREG-0737 III.A.1.2, TECHNICAL SUPPORT CENTER		1	09/30/84
46054	SAN ONOFRE 1 - NUREG-0737 III.A.1.2, OPERATIONAL SUPPORT CENTER		1	09/30/84
46126	SAN ONOFRE 1 - NUREG-0737 III.A.1.2, EMERGENCY OPERATIONS FACILITY		1	09/30/84
48203	SAN ONOFRE 1 - NUREG 0737 II.K.3.31 COMPLIANCE WITH 10 CFR 50.46		1	09/30/84
44270	SAN ONOFRE 1 - NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES		1	11/30/84

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(CONTINUATION)

SAN ONOFRE 1

TAC #	TAC DESCRIPTION	TMI ACTIONS	PRIORITY	CRITICAL DATE
<u>ACTIVE ACTIONS</u> (CONTINUATION)				
44340	SAN ONOFRE 1 - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES		1	TARGET 11/30/84
51200	SAN ONOFRE 1 - I.D.1 CONTROL ROOM DESIGN REVIEW		1	12/31/85
51280	SAN ONOFRE 1 - I.D.2. SAFETY PARAMETER DISPLAY SYSTEM		1	12/31/85
<u>PLANT SPECIFIC</u>				
<u>COMPLETED ACTIONS</u>				
48142	SAN ONOFRE 1 - OPERABILITY OF PWR SOURCES DURING SHUTDOWN & ELIM OF AUX SALT WTR PUMP FR OPER CONSIDERATION FOR SIS		2	COMPLETE 10/28/83
51312	SAN ONOFRE UNIT 1 - REVISED SECURITY PLAN		2	10/31/83
48753	SAN ONOFRE UNIT NO. 1 - MODIFICATION OF TECH. SPECS. REGARDING COOLANT ACTIVITY SAMPLING REQUIREMENTS		2	12/06/83
47835	SAN ONOFRE UNIT 1 - STEAM GENERATOR INSPECTION PROGRAM		1	02/07/84
52165	SAN ONOFRE UNIT NO. 1 - MODIFY TABLE 3.5.2-2 AND PORTIONS OF SECTION 5 OF TECH SPECS.			02/17/84
52352	SAN ONOFRE UNIT NO. 1 - TECH. SPEC. CHANGE - SPHERE PURGE AIR SUPPLY AND OUTLET VALVES			02/17/84
48035	SAN ONOFRE UNIT 1 - REVISION OF EQUATION TO CALCULATE RADIOACTIVE AIRBORNE EFFLUENT RELEASE RATE LIMIT		2	03/01/84
52630	SAN ONOFRE UNIT NO. 1 - REEVALUATION OF SPENT FUEL STORAGE RACKS			04/17/84
52351	SAN ONOFRE UNIT NO. 1 - REVISE TECH. SPECS. REGARDING BORON CONCENTRATION CHANGES			04/23/84
<u>ACTIVE ACTIONS</u>				
52959	SAN ONOFRE 1 - ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING PROCEDURES/VENDOR RECOMMENDATIONS - RTS COMPONENTS			TARGET
53440	SAN ONOFRE 1 - PROPOSED CHANGE NO 129 - APPENDIX B T.S. 13 T.S. (12/5/83)			
54127	VENT VALVE OPERABILITY AUXILIARY OPERATOR REQ'TS			
54376	SAN ONOFRE 1 - INTEGRATED LIVING SCHEDULE			
54377	SAN ONOFRE 1 - LICENSE CONDITION FOR STEAM GENERATOR INSPECTION			
54854	SAN ONOFRE 1 - EMERGENCY EXERCISE EXEMPTION			04/23/84
54749	SAN ONOFRE 1 - RHR MAINTENANCE			05/31/84
52166	SAN ONOFRE UNIT NO. 1 - INVESTIGATION OF POTENTIAL DIESEL GENERATOR CRANKSHAFT PROBLEMS			06/05/84
52350	SAN ONOFRE UNIT NO. 1 - APPENDIX J TECHNICAL SPECIFICATIONS		1	06/30/84
47988	SAN ONOFRE UNIT #1 - ADEQUACY OF SEPARATION CRITERIA USED IN TMI MODIFICATIONS		2	06/30/84
10853	SAN ONOFRE 1 - CHANGE TESTING OF STATION BATTERY SYSTEM			07/15/84
54880	SAN ONOFRE 1 - DEDICATED SHUTDOWN SYSTEM FOR APPENDIX R		1	09/01/84
11232	SAN ONOFRE 1 - FULL TERM OPERATING LICENSE			09/30/84
51739	SAN ONOFRE 1: STATION DIST. VOLTAGE VERIF. TEST			09/30/84
53332	SAN ONOFRE UNIT NO. 1 - REVISIONS TO TECHNICAL SPECIFICATIONS 3.7, 4.4.B, 4.4.E AND 4.4.F			09/30/84
54414	SAN ONOFRE 1 - RCS VENTS TECHNICAL SPECIFICATIONS NUREG 073 ITEM II.B.1			09/30/84
54566	SAN ONOFRE 1 - TECHNICAL SPECIFICATIONS COVERED BY GENERIC LETTERS 83-36, 83-37 FOR NUREG-0737			09/30/84
<u>ANTICIPATED ACTIONS</u>				
54893	SAN ONOFRE 1 - ORGANIZATION CHANGE			INITIATION 05/02/84
54991	SAN ONOFRE 1 - OVERPRESSURE MITIGATION SYSTEM TECHNICAL SPECIFICATION			05/10/84
53879	SAN ONOFRE 1 - ITEM 3.2.3 - POST-MAINTENANCE TESTING -CHANGES TO TECH. SPEC. - ALL SR COMPONENTS			11/01/84
53935	-ITEMS 4.2.3 & 4.2.4-PREVENTATIVE MAINT. PROGRAM FOR REACTOR TRIP BREAKERS-LIFE TESTING & REPLACEMENT			11/01/84
54023	SAN ONOFRE 1 - ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS			11/01/84
54106	SAN ONOFRE 1 - ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL TESTING - DIVERSE TRIP FEATURES			11/01/84

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CONDENSED MANAGEMENT REPORT

(CONTINUATION)

SAN ONOFRE 1

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>ANTICIPATED ACTIONS (CONTINUATION)</u>				
53630	SAN ONOFRE 1 - ITEM 1.2 - POST TRIP REVIEW-DATA AND INFORMATION CAPABILITY			<u>INITIATION</u> 11/01/84
53713	SAN ONOFRE 1 - ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS			11/01/84
53796	SAN ONOFRE 1- ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS			11/01/84

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CONDENSED MANAGEMENT REPORT

FACILITY: SAN ONOFRE 2

PLANT LOCATION: 5 MI S OF SAN CLEMENTE, CA
 DOCKET NUMBER: 050-00361
 ARCH/ENGINEER: BECH
 IE INSPECTOR: D. KIRSCH

LICENSED POWER: 3410 MWT
 DESIGN POWER: 1070 MWE
 NSSS VENDOR: COMB

PROJECT MANAGER: H. ROOD
 BRANCH CHIEF: G. KNIGHTON
 LIC. ASSISTANT: J. LEE

TAC #	TAC DESCRIPTION	MULTI-PLANT	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				
51131	SAN ONOFRE 2 - INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)			COMPLETE 04/30/84
<u>ACTIVE ACTIONS</u>				
52269	SAN ONOFRE 2 - CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPA#C-10)			TARGET
52373	SAN ONOFRE 2 - CONTROL OF HEAVY LOADS (PHASE I)			
52798	SAN ONOFRE 2 - ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION AND PROCEDURES			
52879	SAN ONOFRE 2 - ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RTS COMPONENTS			
52960	SAN ONOFRE 2 - ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING PROCEDURES/VENDOR RECOMMENDATIONS - RTS COMPONENTS			
53041	SAN ONOFRE 2-ITEM 3.1.3 -POST-MAINTENANCE TESTING-CHNGS TO TECH SPECS-RTS COMPONENTS			
53096	SAN ONOFRE 2 - REACTOR TRIP SYSTEM RELIABILITY - VENDOR RELATED MODIFICATIONS			
53149	SAN ONOFRE 2 - ITEMS 4.2.1 & 4.2.2 - PREVENTATIVE MAINT PROGRAM FOR REACTOR TRIP BREAKERS - MAINTENANCE & TRENDING			
42528	SAN ONOFRE 2 - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21)		1	12/30/82
49757	SAN ONOFRE 2-"NUREG 0737 TECH SPECS (GENERIC LETTER 82-16 82-16 OR 83-02)"			09/30/83
49691	SAN ONOFRE 2 - REACTOR COOLANT PUMP TRIP			07/01/84
<u>TMI ACTIONS</u>				
<u>COMPLETED ACTIONS</u>				
51664	SONGS 2 - RCS VOIDING - LIC. COND.2.C(19)P (TMI II.K.2.17)			COMPLETE 03/08/84
<u>ACTIVE ACTIONS</u>				
54652	SAN ONOFRE 2 - NUREG 0737 III.A.1.2 TECHNICAL SUPPORT CENTER			
54653	SAN ONOFRE 2 - NUREG-0737 III.A.1.2 OPERATIONAL SUPPORT CENTER			
54654	SAN ONOFRE 2 - NUREG-0737 III.A.1.2 EMERGENCY OPERATIONS FACILITY			
54663	SAN ONOFRE 2 - INSTRUMENTS FOR DETECTION ON INADEQUATE CORE COOLING (II.F.2)			
51201	SAN ONOFRE 2 - I.D.1 CONTROL ROOM DESIGN REVIEW			
51281	SAN ONOFRE 2 - I.D.2. SAFETY PARAMETER DISPLAY SYSTEM			
51588	SAN ONOFRE 2 - RV AND SV TESTING (II.D.1)			
51666	SONGS 2 - SBLOCA MODEL & ANALYSIS - LIC. COND. 2.C.19(R) - (TMI II.K.3.31)			05/26/84
54980	SAN ONOFRE 2, II.K.2.13, THERMAL MECHANICAL REPORT			06/30/84
<u>PLANT SPECIFIC</u>				
<u>COMPLETED ACTIONS</u>				
51773	SONGS 2 - SALP EVALUATION			COMPLETE 10/01/83
51833	SONGS 2 - CHG T.S. 3.6.1.4 BASES - PCN 29			10/01/83
51838	SONGS 2 - CHG T.S. SEC. 7 - NAT. CIRC A SERIES TESTS-PCN 7 - AMDT 4			10/05/83
51837	SONGS 2 - CHG T.S. TABLE 3.8-2 MOV OVL0AD BYPASS PCN 2 - AMDT 4			10/15/83
51830	SONGS 2 - REVIEW PASS MONTHLY REPORTS LIC COND. 2.C.(19)I			10/15/83
51839	SONGS 2 - TECH SPEC CORRECTIONS - PCN 10 AMDT 4			10/22/83
51840	SONGS 2 - T.S. SEC 6, ADMIN.-PCN 11-AMDT 4			10/22/83
51841	SONGS 2 - T.S. 4.6.1.3 - PCN 13 - AMDT 4			10/22/83
51842	SONGS 2 - T.S. TABLE 3.12-1 - PCN 12 AMDT 4			10/22/83

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CONDENSED MANAGEMENT REPORT

(CONTINUATION)

SAN ONOFRE 2

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u> (CONTINUATION)				
51846	SONGS 2 - LIC COND. 2(C)8- PCN 22 - AMDT5			<u>COMPLETE</u> 10/24/83
51845	SONGS 2 - FIRE PROTECTION T.S. - PCN 24 -AMDT 7			10/27/83
51843	SONGS 2 - CHG T.S. 3.3.3.9 - PCN 28 - AMDT 6			10/30/83
51844	SONGS 2 - CHG PASS DATE - PCN 42 - AMDT 8			11/05/83
51626	SONGS 2 - CHG T.S. 2.2.1, 3.2.4, 4.2.4.4, CHG DNBR PENALTY FACTORS (PC33)			11/09/83
52527	SONGS 2 - RTB FAILURES DURING SURVEILLANCE TEST			11/10/83
52624	SONGS 2 AND 3 TRIP BREAKER UY ARMATURE JAMMING EVALUATE, ETC			01/15/84
54263	SONGS 2 - REVIEW EMERGENCY PREPAREDNESS EXEMPTION REQUEST			02/28/84
51657	SONGS 2 - WASTE SOLIDIFICATION PROCESS CONTROL PROGRAM - LICENSE CONDITION 2.C.(16)			04/25/84
51832	SONGS 2 - CHG T.S. 6.9.1.1.3.6 - PCN 21			04/30/84
51821	SONGS 2 - CHG SURV INTERNAL TO 25 MOS			04/30/84
51835	SONGS 2 - CHG T.S. 4.5.2.F - PCN 64 - HPSI REQTS			04/30/84
51836	SONGS 2 - DELETE T.S. 3.0.4 - PCN 54			04/30/84
52626	SONGS 2/3 - EVALUATE T.S. CHANGE ON OVERTIME PLANT STAFF - PCN 37			04/30/84
51621	SONGS 2 - CHG T.S. 6.12.3, CLARIFY ACCESS CONTROL TO HIGH RAD. AREAS IN CONTAINMENT (PC44)			04/30/84
51649	SONGS 2 - HIGH BURNUP FISSION GAS RELEASE - LICENSE CONDITION 2.C.(6)			04/30/84
51614	SONGS 2 - CHG. T.S. 4.4.5.2.2.C/D, CLARIFY RCPB VALVE TEST REQUIREMENTS (PC9)			04/30/84
<u>ACTIVE ACTIONS</u>				
51316	SAN ONOFRE 2 - HIGH FREQUENCY OF NUISANCE ALARMS			<u>TARGET</u> 04/30/83
51424	SAN ONOFRE 2 - IN-CONTAINMENT H2 MONITORS			05/20/83
51616	SONGS 2 - CHG. T.S.3.4.1.4.1, ALLOW BOTH TRAINS SDCS TO BE OUT IF ONE RCP IS ON-MODE 5 ONLY (PC15)			06/25/83
51618	SONGS 2 - CHG T.S.2.2.1, TABLE 2.2-1 TO CLARIFY SETPOINT VERIFICATION PROCEDURE, RC FLOW-LOW (PC-34)			06/25/83
51619	SONGS 2 - CHG T.S. 6.8.2/6.8.3 TO MODIFY APPROVAL AUTHORITY FOR PROCEDURE CHANGES (PC43)			06/25/83
51613	SONGS 2-CHG T.S. TABLE 4.3-2, CHANGE ESFAS SUBGROUP RELAY SURVEILLANCE INTERVAL FROM 6 MOS TO 18 MOS (PC4)			06/25/83
51623	SONGS 2 - CHG T.S. 3.3.2(TABLE 3.3.-5) AND 4.7.1.2.1.A, CHG RESPONSE TIME LIMIT FOR AFW (PC36)			06/25/83
51627	SONGS 2 - CHG T.S. 3/4.3.3.7, 3/4.7.8.2 TO REFLECT EXTRA FIRE PROTECTION EQPT. (PC72)			06/25/83
51653	SONGS 2 - CONTROL SYSTEM FAILURES DUE TO HELBA OR EQPT. FAILURES. LIC. COND. 2.C.(12)			06/26/83
51834	SONGS 2 - EDIT T.S. TO AGREE WITH UNIT 3 - PCN 56			07/21/83
51662	SONGS 2 - RCS VENTS - LICENSE CONDITION 2.C.19(H) - (TMI II.B.1)			07/26/83
51826	SONGS 2 - REVIEW 6/30/83 PORV SUBMITTAL LIC COND 2.C(24)			07/30/83
51828	SONGS 2 - REVIEW EOF TMI III A.2. - LIC COND. 2.C.(19)S			07/30/83
52567	SONGS 2 BORON FLOWPATH ADEQUACY/ISOLATION			11/10/83
54282	SONGS 2 - EVALUATE RTB SHUNT DEVICE INOPERABILITY			04/02/84
54284	SONGS 2 - ISSUE ORDER CONFIRMING E.P. COMMITMENTS			04/02/84
54814	SONGS 2 - REVIEW REVISED FHA - ISSUE SER			04/25/84
54510	SONGS 3 - EVALUATE LOSS OF CONTAINMENT SPRAY EVENT OF 3-15-84			04/27/84
54638	SAN ONOFRE 2 - CRITICALITY ACCIDENT REQMT'S			05/01/84
54635	SAN ONOFRE 2 - CONTAINMENT ISOLATION VALVES			05/15/84
54816	SONGS 2 - AMEND. LIC. PER IAEA FACILITY AGREEMENT			05/25/84
51661	SONGS 2 - SPECIAL L.P.TESTS - LICENSE CONDITION 2.C.(19)G - (TMI I.G.1)			05/26/84
54257	SONGS 2 - EVALUATE CPC THERMAL POWER DECALIBRATION			05/30/84
54694	SONGS 2 - PCN 8 - REVISE TECH. SPEC. 3.6-1 - CONT. ISO. VALVES			06/12/84
54697	SONGS - 2-PCN 41 - TECH. SPEC. 3/4.2.8 - PRESSURIZER			06/12/84
54699	SONGS 2-PCN 75 - TECH. SPEC. 4.8.1.1.2.C - FUEL OIL RETEST			06/12/84
54738	SONGS 2-PCN 155 VENTS - TECH. SPEC. 3/4.4.10			06/12/84
54700	SONGS 2-PCN 83 - TECH. SPEC. FIG. 6.2-1 - SCE ORG. CHART			06/12/84

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CONDENSED MANAGEMENT REPORT

(CONTINUATION)

SAN ONOFRE 2

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>ACTIVE ACTIONS</u> (CONTINUATION)				<u>TARGET</u>
54702	SONGS 2-PCN 84 - REVISE E.Q. LIC. COND. 2.C.(5)			06/12/84
54705	SONGS 2-PCN 85 - TECH. SPEC. TABLE 3.3-2, RPIS RESP. TIME			06/12/84
54706	SONGS 2-PCN 89 - TECH. SPEC. 3/4.4.4 - S.G. INSPECTION			06/12/84
54708	SONGS 2-PCN 91 - DELETE TECH. SPEC. 4.8 1.2.D.6 PER 83-30			06/12/84
54710	SONGS 2-PCN 97 - TECH. SPEC. 3/4.6.4-H. MONITOR SURV.			06/12/84
54712	SONGS 2-PCN 98 - F.P. LIC. COND. 2.C.(14)			06/12/84
54714	SONGS 2-PCN 99 - TECH. SPEC. TABLE 3.3-3 - CPIS ACTION			06/12/84
54716	SONGS 2-PCN 100 - TECH. SPEC. 3/4.3.3.1 - RAD. MON. ALARM			06/12/84
54718	SONGS 2-PCN 101 - TECH. SPEC. 3/4.3.3.6 - ACC. MON. INST. TIME			06/12/84
54720	SONGS 2-PCN 102 - TECH. SPEC. 3/4.3.3.9 - EFFLUENT MON			06/12/84
54722	SONGS 2-PCN 103 - TECH. SPEC. 3/4.11 - NUREG 0472 CHANGES			06/12/84
54724	SONGS 2-PCN 109 - TECH. SPEC. 4.2.3.C - BURNUP - 31 DAYS			06/12/84
54726	SONGS 2-PCN 110 - TECH. SPEC. 4.3.3.2.4.A - CHANNEL CHECK			06/12/84
54728	SONGS 2-PCN 131 - TECH. SPEC. 3.3.3.8 - SALT WATER DILUTION			06/12/84
54730	SONGS 2-PCN 135 - TECH. SPEC. TABLE 3.3-4 - TGIS SETPOINTS			06/12/84
54733	SONGS 2-PCN 145 - CHG. F.P. LIC. COND. 2.C.(14)			06/12/84
54735	SONGS 2-PCN 146 - CHG. F.P. TECH. SPECS.			06/18/84
54780	SONGS 2 - CHG. OSRC REVIEW OF PCP,ODCM/ETC - PCN-79			06/18/84
54782	SONGS 2 - DELETE L.C.2.C(19)F - PCN 82			06/18/84
54784	SONGS 2 - CHG PURGE LIMIT TO 3000 HRS - PCN 144			06/18/84
54812	SONGS 2 - REVIEW FISSION GAS PRESSURE ANALYSIS			06/25/84
54817	SONGS 2 - EVALUATE OPERATING EXPERIENCE			06/25/84
54819	SONGS 2 - PCN 138 - TECH. SPEC. 3.1.3.1/6/7 - CHANGE TECH. SPEC. PER REVIEW CPC SOFTWARE			06/25/84
54860	SONGS 2 - PCN 108 - TECHNICAL SPECIFICATION 3.8-1(TBL) CONTAINMENT PENETRATION OVERCURRENT PROTECTION			06/30/84
54862	SONGS 2 - PCN 111 - TECH. SPEC. 3.11.2 - AUX. BOILER RADWASTE, INCINERATE WASTE OIL			06/30/84
54864	SONGS 2 - PCN 114 - TECHNICAL SPECIFICATION 4.6.1.6 - CONTAINMENT TENDON SURVEILLANCE			06/30/84
54866	SONGS 2 - NPF 119 - TECH. SPEC. B 3.2.3 - AZIMUTHAL POWERTILT			06/30/84
54868	SONGS 2 - PCN 123 - TECH. SPEC. 2.0 - DNBR LOW-BASIS			06/30/84
54870	SONGS 2 - PCN 125 - TECH. SPEC. 3.6.2.2 - SPRAY CHEMICAL REQUIRMENTS			09/01/84
54148	SONGS 2 - IN SERVICE INSPECTION, SER INPUT			09/30/84
54567	SAN ONOFRE 2 - TECHNICAL SPECIFICATIONS COVERED BY GENERIC LETTERS 83-36, 83-37 FOR NUREG-0737			11/01/84
51847	SONGS 2 - AFW PUMP LUBE OIL COOLERS - LIC. COND. 2.C(25)			11/26/84
51659	SONGS 2 - EVALUATE EMERGENCY PROCEDURES/GUIDELINES-LICENSE CONDITION 2.C.(19).D - (TMI-I.C.1)			02/16/85
51647	SONGS 2 - CONTAINMENT TENDON SURVEILLANCE - LICENSE CONDITION 2.C.(4)			05/26/85
51655	SONGS 2 - TURBINE DISC INSPECTION - LICENSE CONDITION 2.C.(15)			05/26/85
51656	SONGS 3 - TURBINE DISC INSPECTION - LICENSE CONDITION 2.C.(13)			05/26/88
51651	SONGS 2 - LOW TEMPERATURE OVERPRESSURE PROTECTION - LICENSE CONDITION 2.C.(7)			
<u>ANTICIPATED ACTIONS</u>				<u>INITIATION</u>
54872	SONGS 2 - PCN 127 - TECH. SPEC. 3.7.10 - LCO FOR EMERGENCY CHILLED WATER SYSTEM			05/01/84
54874	SONGS 2 - PCN 134 - TECH. SPEC. 2.2.2 - CPC ADDRESSABLE CONSTANTS			05/01/84
54876	SONGS 2 - PCN-130 - L.C.2.C(4) - CONTAINMENT TENDON SURVEILLANCE			05/01/84
53631	SAN ONOFRE 2 - ITEM 1.2 - POST TRIP REVIEW-DATA AND INFORMATION CAPABILITY			11/01/84
53714	SAN ONOFRE 2 - ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS			11/01/84
53797	SAN ONOFRE 2- ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS			11/01/84
53880	SAN ONOFRE 2 - ITEM 3.2.3 - POST-MAINTENANCE TESTING -CHANGES TO TECH. SPEC. - ALL SR COMPONENTS			11/01/84
53936	-ITEMS 4.2.3 & 4.2.4-PREVENTATIVE MAINT. PROGRAM FOR REACTOR TRIP BREAKERS-LIFE TESTING & REPLACEMENT			11/01/84

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T E C H N I C A L A S S I G N M E N T C O N T R O L S Y S T E M

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C O N D E N S E D M A N A G E M E N T R E P O R T

(CONTINUATION)

SAN ONOFRE 2

TAC # TAC DESCRIPTION

PLANT SPECIFIC

PRIORITY

CRITICAL DATE

ANTICIPATED ACTIONS (CONTINUATION)

54024	SAN ONOFRE 2 - ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS		
54107	SAN ONOFRE 2 - ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL TESTING - DIVERSE TRIP FEATURES		

INITIATION

11/01/84

11/01/84

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CONDENSED MANAGEMENT REPORT

FACILITY: SAN ONOFRE 3

PLANT LOCATION: 5 MI S OF SAN CLEMENTE, CA
 DOCKET NUMBER: 050-00362
 ARCH/ENGINEER: BECH
 IE INSPECTOR: H. CANTER

LICENSED POWER: 3390 MWT
 DESIGN POWER: 1070 MWE
 NSSS VENDOR: COMB

PROJECT MANAGER: H. ROOD
 BRANCH CHIEF: G. KNIGHTON
 LIC. ASSISTANT: J. LEE

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>MULTI-PLANT</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
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COMPLETED ACTIONS

51132	SAN ONOFRE 3 - INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)			<u>COMPLETE</u> 04/30/84
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ACTIVE ACTIONS

12135	SAN ONOFRE 3, EMERGENCY PLAN REVIEW		1	<u>TARGET</u>
52270	SAN ONOFRE 3 - CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPA#C-10)			
52374	SAN ONOFRE 3 - CONTROL OF HEAVY LOADS (PHASE I)			
52799	SAN ONOFRE 3 - ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION AND PROCEDURES			
52880	SAN ONOFRE 3 - ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RTS COMPONENTS			
52961	SAN ONOFRE 3 - ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING PROCEDURES/VENDOR RECOMMENDATIONS - RTS COMPONENTS			
53042	SAN ONOFRE 3-ITEM 3.1.3 -POST-MAINTENANCE TESTING-CHNGS TO TECH SPECS-RTS COMPONENTS			
53097	SAN ONOFRE 3 - REACTOR TRIP SYSTEM RELIABILITY - VENDOR RELATED MODIFICATIONS			
53150	SAN ONOFRE 3 - ITEMS 4.2.1 & 4.2.2 - PREVENTATIVE MAINT PROGRAM FOR REACTOR TRIP BREAKERS - MAINTENANCE & TRENDING			
42529	SAN ONOFRE 3 - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21)		1	07/30/82
49692	SAN ONOFRE 3 - REACTOR COOLANT PUMP TRIP			07/01/84

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>TMI ACTIONS</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
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COMPLETED ACTIONS

51665	SONGS 3 - 48 HP. TEST OF AFW PUMPS - LIC. COND. 2.C.(17).F - (TMI II.E.1.1)			<u>COMPLETE</u> 03/08/84
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ACTIVE ACTIONS

51202	SAN ONOFRE 3 - I.D.1 CONTROL ROOM DESIGN REVIEW			
51282	SAN ONOFRE 3 - I.D.2. SAFETY PARAMETER DISPLAY SYSTEM			
51589	SAN ONOFRE 3 - RV AND SV TESTING (II.D.1)			
54757	SAN ONOFRE-3 - NUREG-0737 III.A.1.2 TECHNICAL SUPPORT CENTER			
54758	SAN ONOFRE - NUREG-0737 III.A.1.2 OPERATIONAL SUPPORT CENTER			
54759	SAN ONOFRE-3 - NUREG-0737 III.A.1.2 EMERGENCY OPERATIONS FACILITY			
51667	SONGS 3 - SBLUCA MODEL AND ANALYSIS - LIC. COND. 2.C.(17)I - (TMI II.K.3.31)			05/26/84
54981	SAN ONOFRE 3, II.K.2.13, THERMAL MECHANICAL REPORT			06/30/84
51663	SONGS 3 - I.C.C. INSTRUMENTATION - LIC. COND. 2.C.(17)H - (TMI II.F.2)			05/26/85

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
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COMPLETED ACTIONS

51772	SONGS 3 - SALP EVALUATION			<u>COMPLETE</u> 10/01/83
51628	SONGS 3 - CHG T.S. 3/4.3.3.7, 3/4.7.8.2 TO REFLECT EXTRA FIRE PROTECTION EQPT (PC72)			10/12/83
51831	SONGS 3 - REVIEW PASS MONTHLY REPORTS - LIC COND. 2.C(17)D			10/15/83
51823	SONGS 3 - T.S. TABLE 3.3-5 PCN 60 AMDT 1			11/05/83
52388	SONGS 3 - DELAY LOSS OF OFFSITE POWER TEST			11/15/83
52627	SONGS 3 - TRIP BREAKER UV ARMATURE JAMMING - EVALUATE ETC.			12/15/83
54264	SONGS 3 - REVIEW EMERGENCY PREPAREDNESS EXEMPTION REQUEST			02/28/84
51658	SONGS 3 - WASTE SOLIDIFICATION PROCESS CONTROL PROGRAM - LICENSE CONDITION 2.C.(14)			04/25/84
51622	SONGS 3 - CHG T.S. 6.12.3, CLARIFY ACCESS CONTROL TO HIGH RAD AREAS IN CONTAINMENT (PC44)			04/30/84

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CONDENSED MANAGEMENT REPORT

(CONTINUATION)

SAN ONOFRE 3

TAC #	TAC DESCRIPTION	PLANT SPECIFIC	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u> (CONTINUATION)				<u>COMPLETE</u>
51650	SONGS 3 - HIGH BURNUP FISSION GAS RELEASE - LICENSE CONDITION 2.C.(6)			04/30/84
51615	SONGS 3-CHG. T.S.4.4.5.2.2 C/D. CLARIFY RCPB VALVE TEST REQUIREMENTS (PC9)			04/30/84
52625	SONGS 2/3 - EVALUATE T.S. CHANGE ON OVERTIME LIMITS FOR PLANT STAFF			04/30/84
51824	SONGS 3 - T.S. 4.5.2.F-HP5I PUMP REQTS PCN 64			04/30/84
51825	SONGS 3 - DELETE T.S. 3.0.4- PCN 54			04/30/84
51822	SONGS 3 - CHG SURV INTERVAL TO 25MOS			04/30/84
<u>ACTIVE ACTIONS</u>				<u>TARGET</u>
51317	SAN ONOFRE 3 - HIGH FREQUENCY OF NUISANCE ALARMS			04/30/83
51425	SAN ONOFRE 3 - IN-CONTAINMENT H2 MONITORS			05/20/83
51617	SONGS 3 - CHG T.S.3.4.1.4.1, ALLOW BOTH TRAINS SDCS TO BE OUT IF ONE RCP IS ON-MODE 5 ONLY (PC15)			06/25/83
51620	SONGS 3 - CHG T.S. 6.8.2/6.8.3 TO MODIFY APPROVAL AUTHORITY FOR PROCEDURE CHANGES (PC43)			06/25/83
51624	SONGS 3 - CHG T.S. 3.3.2 (TABLE 3.3-5) AND 4.7.1.2.1.A, CHG RESPONSE TIME LIMIT FOR AFW (PC36)			06/25/83
51625	SONGS 3 - CHG T.S.2.2.1, 3.2.4, 4.2.4.4, CHG DNBR PENALTY FACTORS (PC33)			06/25/83
51654	SONGS 3 - CONTROL SYS. FAILURES DUE TO HELBA OR EQPT. FAILURES - LIC. COND. 2.C.(10)			06/26/83
51827	SONGS 3 - REVIEW 6/30/83 PORV SUBMITTAL LIC COND 2.C.(19)			07/30/83
51829	SONGS 3 - REVIEW EOF - TMI III.A.2. - LIC COND. 2.C.(17)J			07/30/83
52560	SONGS 3 STARTUP TESTING - EVALUATE CHANGES			01/03/84
54283	SONGS 3 - EVALUATE RTB SHUNT DEVICE INOPERABILITY			04/02/84
54285	SONGS 3 - ISSUE ORDER CONFIRMING E.P. COMMITMENTS			04/02/84
54637	SAN ONOFRE 3 - CRITICALITY ACCIDENT REQUIREMENT			05/01/84
54636	SAN ONOFRE 3 - CONTAINMENT ISOLATION VALVES			05/15/84
54256	SONGS 3 - EVALUATE CPC THERMAL POWER DECALIBRATION			05/30/84
54695	SONGS 3 - PCN 8 - REVISE TECH. SPEC. 3.6.1 - CONT. ISO. VALVES			06/12/84
54696	SONGS 3 - PCN 42 - TECH. SPEC. 3/4.2.8 - PRESSURIZER			06/12/84
54698	SONGS 3-PCN 75 - TECH. SPEC. 4.8.1.1.2.C - FUEL OIL RETEST			06/12/84
54701	SONGS 3-PCN 83 - TECH. SPEC. FIG. 6.2-1 - SCE ORG. CHART			06/12/84
54703	SONGS 3-PCN 84 - REVISE E.Q. LIC. COND. 2.C.(5)			06/12/84
54704	SONGS 3-PCN 85 - TECH. SPEC. TABLE 3.3-2, RPIS RESP. TIME			06/12/84
54707	SONGS 3-PCN 89 - TECH. SPEC. 3/4.4.4 - S.G. INSPECTION			06/12/84
54729	SONGS 3-PCN 131 - TECH. SPEC. 3.3.3.8 - SALT WATER DILUTION			06/12/84
54731	SONGS 3-PCN 135 - TECH. SPEC. TABLE 3.3-4 - TGIS SETPOINTS			06/12/84
54732	SONGS 3-PCN 140 - TECH. SPEC. 4.5.2 - DELETE HV-0396			06/12/84
54734	SONGS 3-PCN 145 - CHG. F.P. LIC. COND. 2.C.(12)			06/12/84
54736	SONGS 3-PCN 146 - CHG. F.P. TECH. SPECS.			06/12/84
54737	SONGS 3-PCN 155 VENTS - TECH. SPEC. 3/4.4.10			06/12/84
54709	SONGS 3-PCN 91 - DELETE TECH. SPEC. 4.8.1.2.D.6 PER 83-30			06/12/84
54713	SONGS 3-PCN 98 - F.P. LIC. COND. 2.C.(14)			06/12/84
54715	SONGS 3-PCN 99 - TECH. SPEC. TABLE 3.3-3 - CPIS ACTION			06/12/84
54717	SONGS 3-PCN 100 - TECH. SPEC. 3/4.3.3.1 - RAD. MON. ALARMS			06/12/84
54719	SONGS 3-PCN 101 - TECH. SPEC. 3/4.3.3.6 - ACC. MON. INST. TIME			06/12/84
54721	SONGS 3-PCN 102 - TECH. SPEC. 3/4.3.3.9 - EFFLUENT MON			06/12/84
54723	SONGS 3-PCN 103 - TECH. SPEC. 3/4.11 - NUREG 0472 CHANGES			06/12/84
54725	SONGS 3-PCN 109 - TECH. SPEC. 4.2.3.C - BURNUP - 31 DAYS			06/12/84
54727	SONGS 3-PCN 110 - TECH. SPEC. 4.3.3.2.4.A - CHANNEL CHECK			06/12/84
54781	SONGS 3 - CHG OSRC REVIEW OF PCP/ODCM/ETC - PCN79			06/18/84
54783	SONGS 2 - DELETE L.C.2.C(17)F - PCN 82			06/18/84
54785	SONGS 3 - CHG PURGE LIMIT TO 3000 HRS - PCN 144			06/18/84
54711	SONGS 3-PCN 97 - TECH. SPEC. 3/4.6.4 - H2 MONITOR SURV.			06/21/84

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(CONTINUATION)

SAN ONOFRE 3

TAC #	TAC DESCRIPTION	PLANT SPECIFIC	PRIORITY	CRITICAL DATE
<u>ACTIVE ACTIONS</u>				<u>TARGET</u>
(CONTINUATION)				
54813	SONGS 3 - REVIEW FISSION GAS PRESSURE ANALYSIS			06/25/84
54818	SONGS 3 - EVALUATE OPERATING EXPERIENCE			06/25/84
54820	SONGS 3 - PCN 138 - TECH. SPEC. 3.1.3.1/6/7 - CHANGE TECH. SPEC. PER REVIEW CPC SOFTWARE			06/25/84
54861	SONGS 2 - PCN 108 - TECHNICAL SPECIFICATION 3.8-1(TBL) CONTAINMENT PENETRATION OVERCURRENT PROTECTION			06/30/84
54863	SONGS 3 - PCN 111 - TECHNICAL SPECIFICATION 3.11.2 - AUX. BOILER RADWASTE, INCINERATE WASTE OIL			06/30/84
54865	SONGS 3 - PCN 114 - TECH. SPEC. 4.6.1.6 - CONTAINMENT TENDON SURVEILLANCE			06/30/84
54867	SONGS 3 - NPF 119 - TECH. SPEC. B3.2.3 - AZIMUTHAL POWER TILT			06/30/84
54869	SONGS 2 - PCN 123 - TECH. SPEC. 2.0 - DNBR LOW-BASIS			06/30/84
54871	SONGS 2 - PCN 125 - TECH. SPEC. 3.6.2.2 - SPRAY CHEMICAL REQUIRMENTS			06/30/84
54815	SONGS 3 - REVIEW REVISED FHA - ISSUE SER			07/25/84
54149	SONGS 3 - IN SERVICE INSPECTION - SER INPUT			09/01/84
54568	SAN ONOFRE 3 - TECHNICAL SPECIFICATIONS COVERED BY GENERIC LETTERS 83-36, 83-37 FOR NUREG-0737			09/30/84
51648	SONGS 3 - CONTAINMENT TENDON SURVEILLANCE - LICENSE CONDITION 2.C.(4)			11/15/84
51660	SONGS 3 - EMERGENCY PROCEDURES/GUIDELINES-LICENSE CONDITION 2.C.(17)A - (TMI I.C.1)			05/26/85
51848	SONGS 3 - AFW LUBE OIL COOLERS - LIC COND. 2.C.(20)			06/01/85
51652	SONGS 3 - LOW TEMPERATURE OVERPRESSURE PROTECTION - LICENSE CONDITION 2.C.(7)			05/26/88
<u>ANTICIPATED ACTIONS</u>				<u>INITIATION</u>
54873	SONGS 2 - PCN 127 - TECH. SPEC. 3.7.10 - LCD FOR EMERGENCY CHILLED WATER SYSTEM			05/01/84
54875	SONGS 2 - PCN 134 - TECH. SPEC. 2.2.2 - CPC ADDRESSABLE CONSTANTS			05/01/84
54877	SONGS 2 - PCN-130 - L.C.2.C(4) - CONTAINMENT TENDON SURVEILLANCE			05/01/84
53632	SAN ONOFRE 3 - ITEM 1.2 - POST TRIP REVIEW-DATA AND INFORMATION CAPABILITY			11/01/84
53715	SAN ONOFRE 3 - ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS			11/01/84
53798	SAN ONOFRE 3 - ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS			11/01/84
53881	SAN ONOFRE 3 - ITEM 3.2.3 - POST-MAINTENANCE TESTING -CHANGES TO TECH. SPEC. - ALL SR COMPONENTS			11/01/84
53937	-ITEMS 4.2.3 & 4.2.4-PREVENTATIVE MAINT. PROGRAM FOR REACTOR TRIP BREAKERS-LIFE TESTING & REPLACEMENT			11/01/84
54025	SAN ONOFRE 3 - ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS			11/01/84
54108	SAN ONOFRE 3 - ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL TESTING - DIVERSE TRIP FEATURES			11/01/84

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CONDENSED MANAGEMENT REPORT

FACILITY: SEQUOYAH 1

PLANT LOCATION: 9.5 MI NE OF CHATTANOOGA, TN
 DOCKET NUMBER: 050-00327
 ARCH/ENGINEER: TVA
 IE INSPECTOR: F. FORD

LICENSED POWER: 3411 MWT
 DESIGN POWER: 1148 MWE
 NSSS VENDOR: WEST

PROJECT MANAGER: C. STAHL
 BRANCH CHIEF: E. ADENSAM
 LIC. ASSISTANT: M. DUNCAN

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>MULTI-PLANT</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
51789	SEQUOYAH 1 - NATURAL CIRCULATION COOLDOWN			<u>COMPLETE</u> 12/30/83
<u>ACTIVE ACTIONS</u>				
51133	SEQUOYAH 1 - INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)			<u>TARGET</u>
52271	SEQUOYAH 1 - CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPA#C-10)			
52800	SEQUOYAH 1 - ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION AND PROCEDURES			
52881	SEQUOYAH 1 - ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RTS COMPONENTS			
52962	SEQUOYAH 1 - ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING PROCEDURES/VENDOR RECOMMENDATIONS - RTS COMPONENTS			
53043	SEQUOYAH 1-ITEM 3.1.3 -POST-MAINTENANCE TESTING-CHANGES TO TECH SPECS-RTS COMPONENTS			
53098	SEQUOYAH 1 - REACTOR TRIP SYSTEM RELIABILITY - VENDOR RELATED MODIFICATIONS			
53151	SEQUOYAH 1 - ITEMS 4.2.1 & 4.2.2 - PREVENTATIVE MAINT PROGRAM FOR REACTOR TRIP BREAKERS - MAINTENANCE & TRENDING			
53195	SEQUOYAH 1 - ITEM 4.7 - AUTOAMTIC ACTUATION OF SHUNT TRIP ATTACHMENT FOR W AND B & W PLANTS			
47130	SEQUOYAH 1 - CONTROL OF HEAVY LOADS AT NUCLEAR POWER PLANTS (NUREG 0612)		2	01/00/84
49758	SEQUOYAH 1 -"NUREG 0737 TECH SPECS (GENERIC LETTER 82-16 82-16 OR 83-02)"			01/00/84
47064	SEQUOYAH 1/2 - FIRE PROTECTION		1	01/00/84
42535	SEQUOYAH 1 - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21)		1	06/30/84
49694	SEQUOYAH 1 - REACTOR COOLANT PUMP TRIP			07/01/84
<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>TMI ACTIONS</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
46271	SEQUOYAH 1 - III.A.2.1 EMERGENCY PLAN UPGRADE TO MEET RULE MEET RULE		1	<u>COMPLETE</u> 12/30/83
46343	SEQUOYAH 1 - NUREG-0737 III.A.2.2, METEOROLOGICAL DATA UPGRADF		1	12/30/83
44408	SEQUOYAH 1 - NUREG-0737 II.B.1, RCS HIGH POINT VENTS		1	12/30/83
51283	SEQUOYAH 1 - I.D.2. SAFETY PARAMETER DISPLAY SYSTEM			12/30/83
<u>ACTIVE ACTIONS</u>				
44621	SEQUOYAH 1 - RV AND SV TESTING		1	<u>TARGET</u> 02/15/83
45250	SEQUOYAH 1 - NUREG-0737 II.K.3.1, AUTO PORV ISOLATION		1	02/15/83
45865	SEQUOYAH 1 - NUREG-0737 II.K.3.30, SB LOCA OUTLINE		1	06/30/83
47820	SEQUOYAH 1 - NUREG 0737 ITEM II K2.17 POTENTIAL FOR VOIDING IN RCS		1	06/30/83
44341	SEQUOYAH 1 - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES		1	06/30/83
48204	SEQUOYAH 1 -NUREG 0737 II.K.3.31 COMPLIANCE WITH 10 CFR 50.46		1	09/30/83
45172	SEQUOYAH 1 - NUREG-0737 II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION		1	12/00/83
44479	SEQUOYAH 1 - NUREG-0737 II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS		1	12/31/83
51203	SEQUOYAH 1 - I.D.1 CONTROL ROOM DESIGN REVIEW			01/00/84
45984	SEQUOYAH 1 - NUREG-0737 III.A.1.2, TECHNICAL SUPPORT CENTER		1	09/30/84
46055	SEQUOYAH 1 - NUREG-0737 III.A.1.2, OPERATIONAL SUPPORT CENTER		1	09/30/84
46127	SEQUOYAH 1 - NUREG-0737 III.A.1.2, EMERGENCY OPERATIONS FACILITY		1	09/30/84
<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
<u>COMPLETE</u>				

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(CONTINUATION)

SEQUOYAH 1

TAC #	TAC DESCRIPTION	PLANT SPECIFIC	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u> (CONTINUATION)				<u>COMPLETE</u>
51461	SEQUOYAH 1 - CHEMICAL FEED LINE CHANGE			12/30/83
51466	SEQUOYAH 1 - ADMINISTRATIVE CHANGE TO RADIOLOGICAL EMERGENCY PLAN			12/30/83
51467	SEQUOYAH 1 - LIMITING CONDITION REVISION TO CO2 SYSTEM			12/30/83
51470	SEQUOYAH 1 - ADMINISTRATIVE CHANGE TO CENTRAL ROOM COMMAND			12/30/83
51760	SEQUOYAH 1 - TEMPORARY TECH SPEC CHANGE TO PERFORMING SHUTDOWN BOARD MODIFICATIONS			12/30/83
51763	SEQUOYAH 1 - VOLUME REDUCTION SYSTEM TO PROCESS			12/30/83
51970	SEQUOYAH 1 - DG SURVEILLANCE REQ'DS			12/30/83
52130	SEQUOYAH UNIT 1 - DNB PARAMETER CHANGES			12/30/83
<u>ACTIVE ACTIONS</u>				<u>TARGET</u>
54277	SEQUOYAH 1 - BRITISH FUEL ASSEMBLIES			06/30/83
51457	SEQUOYAH 1 - LOW TEMPERATURE OVERPRESSURE PROTECTION		3	07/31/83
47360	SEQUOYAH 1/2 - TECHNICAL SPECIFICATION REVISIONS			09/26/83
52134	SEQUOYAH UNIT 1 - CONTAINMENT ISOLATION VALVES			12/00/83
49171	SEQUOYAH 1 CONTAINMENT AIR TEMPERATURE AND PURGE LINES			01/00/84
51459	SEQUOYAH 1 - DIESEL GENERATOR RELIABILITY			01/00/84
51463	SEQUOYAH 1 - ROD DROP PROTECTION			01/00/84
52125	SEQUOYAH UNIT 1 - REVISION TO FUSE SURVEILLANCE REQUIREMENTS			01/00/84
52128	SEQUOYAH UNIT 1 - PRESSURIZER SPRAY NOZZLE			01/00/84
52195	SEQUOYAH UNIT 1 - SUBCOOLING MARGIN MONITOR			01/00/84
53334	SEQUOYAH-UNIT 1 - AFW PUMP ENDURANCE TESTS			01/20/84
53391	SEQUOYAH 1 - POST ACCIDENT SAMPLING			01/31/84
52659	SEQUOYAH 1 - SNUBBER SPECIFICATIONS			01/31/84
52661	SEQUOYAH 1 - DG TIMERS		3	02/00/84
47065	SEQUOYAH 1/2 TASK INTERFACE AGREEMENT(81-15)			02/17/84
53393	SEQUOYAH 1 - MOV THERMAL OVERLOAD PROTECTION			02/24/84
53403	SEQUOYAH 2 - ASME CODE RELIEF FOR HYDROSTATIC TESTING			03/14/84
54273	SEQUOYAH 1 - SLAP REVIEWS			03/14/84
54274	SEQUOYAH 2 - SALP REVIEWS			03/15/84
53494	SEQUOYAH 1 - CONTAINMENT PURGE LINES			03/30/84
54214	SEQUOYAH UNIT 1 - RV MATERIAL ANALYSIS			05/21/84
54884	SEQUOYAH (UNIT 1 & 2) - EMERGENCY EXERCISE EXEMPTION REQUEST			06/30/84
54278	SEQUOYAH 2 - BRITISH FUEL ASSEMBLIES			06/30/84
54275	SEQUOYAH 1 - 1ST REVIEW			06/30/84
54276	SEQUOYAH 2 - 1ST PROGRAM			06/30/84
52132	SEQUOYAH UNIT 1 - H2 IGNITER SURVEILLANCE			06/30/84
46917	SEQUOYAH 1 - CONTROL ROOM HUMAN FACTORS REVIEW			06/30/84
54415	SEQUOYAH 1- RCS VENTS TECHNICAL SPECIFICATIONS NUREG 073 ITEM II.B.1			09/30/84
54569	SEQUOYAH 1 - TECHNICAL SPECIFICATIONS COVERED BY GENERIC LETTERS 83-36, 83-37 FOR NUREG-0737			09/30/84
<u>ANTICIPATED ACTIONS</u>				<u>INITIATION</u>
53633	SEQUOYAH 1 - ITEM 1.2 - POST TRIP REVIEW-DATA AND INFORMATION CAPABILITY			11/01/84
53716	SEQUOYAH 1 - ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS			11/01/84
53799	SEQUOYAH 1- ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS			11/01/84
53882	SEQUOYAH 1 - ITEM 3.2.3 - POST-MAINTENANCE TESTING -CHANGES TO TECH. SPEC. - ALL SR COMPONENTS			11/01/84
53938	-ITEMS 4.2.3 & 4.2.4-PREVENTATIVE MAINT. PROGRAM FOR REACTOR TRIP BREAKERS-LIFE TESTING & REPLACEMENT			11/01/84
54026	SEQUOYAH 1 - ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS			11/01/84

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(CONTINUATION)

SEQUOYAH 1

TAC # TAC DESCRIPTION

PLANT SPECIFIC

PRIORITY

CRITICAL DATE

ANTICIPATED ACTIONS (CONTINUATION)

54109 SEQUOYAH 1 - ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL TESTING - DIVERSE TRIP FEATURES

INITIATION
11/01/84

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(CONTINUATION)

SEQUOYAH 2

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS (CONTINUATION)</u>				<u>COMPLETE</u>
52131	SEQUOYAH UNIT 2 - DNB PARAMETER CHANGES			12/30/83
51046	SEQUOYAH 2 - UHI ACCUMULATOR WATER LEVEL SETPOINT AND TOLERANCES			12/30/83
51458	SEQUOYAH 2 - LOW TEMPERATURE OVERPRESSURE PROTECTION			12/30/83
51460	SEQUOYAH 2 - DIESEL GENERATOR RELIABILITY			12/30/83
51465	SEQUOYAH 2 - ADMINISTRATIVE CHANGE TO RADIOLOGICAL EMERGENCY PLAN			12/30/83
51468	SEQUOYAH 2 - LIMITING CONDITION REVISION TO CO2 SYSTEM			12/30/83
51469	SEQUOYAH 2 - ADMINISTRATIVE CHANGE TO PHYSICAL SECURITY PLAN			12/30/83
51761	SEQUOYAH 2 - TEMPORARY TECH SPEC CHANGE TO PERFORM SHUTDOWN BOARD MODIFICATIONS			12/30/83
51762	SEQUOYAH 2 - VOLUME REDUCTION SYSTEM TO PROCESS LOW-LEVEL RADIOACTIVE WASTE			12/30/83
<u>ACTIVE ACTIONS</u>				<u>TARGET</u>
51971	SEQUOYAH 2 - FUEL RELOAD			10/01/82
46918	SEQUOYAH 2 - CONTROL ROOM HUMAN FACTORS REVIEW			07/01/83
48254	SEQUOYAH 2 - TECH. SPEC. CHANGES ON PLANT ORGANIZATION			07/30/83
49172	SEQUOYAH 2 CONTAINMENT AIR TEMPERATURE AND PURGE LINES			09/26/83
52135	SEQUOYAH UNIT 2 - CONTAINMENT ISOLATION VALVES			09/26/83
52129	SEQUOYAH UNIT 2 - PRESSURIZER SPRAY NOZZLE			10/16/83
52196	SEQUOYAH UNIT 2 - SUBCOOLING MARGIN MONITOR			01/13/84
53392	SEQUOYAH 2 POST ACCIDENT SAMPLING			01/20/84
53335	SEQUOYAH UNIT 2 - AFW PUMP ENDURANCE TESTS			01/31/84
52660	SEQUOYAH 2 - SNUBBER SPECIFICATIONS			01/31/84
52662	SEQUOYAH 2 - DG TIMERS			02/17/84
53394	SEQUOYAH 2 - MOV THERMAL OVERLOAD PROTECTION			02/24/84
53404	SEQUOYAH 1 - ASME CODE RELIEF FOR HYDROSTATIC TESTING			03/15/84
53495	SEQUOYAH 2 - CONTAINMENT PURGE LINES			05/30/84
51972	SEQUOYAH 2 - DG SUEVILLANCE REG'DS			06/30/84
52133	SEQUOYAH UNIT 2 - H2 IGNITER SUEVILLANCE			06/30/84
51441	SEQUOYAH 2 - R & D H2 CONTROL			09/00/84
51440	SEQUOYAH 2 - ENVIRONMENTAL QUALIFICATION			09/30/84
54416	SEQUOYAH 2 - RCS VENTS TECHNICAL SPECIFICATIONS NUREG 073 ITEM II.B.1			09/30/84
54570	SEQUOYAH 2 - TECHNICAL SPECIFICATIONS COVERED BY GENERIC LETTERS 83-36, 83-37 FOR NUREG-0737			09/30/84
<u>ANTICIPATED ACTIONS</u>				<u>INITIATION</u>
53634	SEQUOYAH 2 - ITEM 1.2 - POST TRIP REVIEW-DATA AND INFORMATION CAPABILITY			11/01/84
53717	SEQUOYAH 2 - ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS			11/01/84
53800	SEQUOYAH 2- ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS			11/01/84
53883	SEQUOYAH 2 - ITEM 3.2.3 - POST-MAINTENANCE TESTING -CHANGES TO TECH. SPEC. - ALL SR COMPONENTS			11/01/84
53939	-ITEMS 4.2.3 & 4.2.4-PREVENTATIVE MAINT. PROGRAM FOR REACTOR TRIP BREAKERS-LIFE TESTING & REPLACEMENT			11/01/84
54027	SEQUOYAH 2 - ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS			11/01/84
54110	SEQUOYAH 2 - ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL TESTING - DIVERSE TRIP FEATURES			11/01/84

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CONDENSED MANAGEMENT REPORT

FACILITY: ST LUCIE 1

PLANT LOCATION: 12 MI SE OF FT. PIERCE, FLA
 DOCKET NUMBER: 050-00335
 ARCH/ENGINEER: EBASCO
 IE INSPECTOR: S. ELROD

LICENSED POWER: 2700 MWT
 DESIGN POWER: 0830 MWE
 NSSS VENDOR: COMB

PROJECT MANAGER: D. SELLS
 BRANCH CHIEF: J. MILLER
 LIC. ASSISTANT: P. KREUTZER

TAC #	TAC DESCRIPTION	MULTI-PLANT	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				
12319	ST. LUCIE 1 - HELB AND CONSEQUENTIAL SYSTEM FAILURE ST. LUCIE 1		3	<u>COMPLETE</u> 01/31/84
49760	ST. LUCIE 1 - "NUREG 0737 TECH SPECS (GENERIC LETTER 82-16 82-16 OR 83-02)"			03/02/84
51325	ST. LUCIE 1 - THERMAL SHIELD DAMAGE			03/14/84
<u>ACTIVE ACTIONS</u>				
51135	ST. LUCIE 1 - INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)			<u>TARGET</u>
52273	ST. LUCIE 1 - CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MFA#C-10)			
52802	ST. LUCIE 1 - ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION AND PROCEDURES			
52883	ST. LUCIE 1 - ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RTS COMPONENTS			
52964	ST. LUCIE 1 - ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING PROCEDURES/VENDOR RECOMMENDATIONS - RTS COMPONENTS			
53045	ST. LUCIE 1-ITEM 3.1.3 -POST-MAINTENANCE TESTING-CHANGES TO TECH SPECS-RTS COMPONENTS			
53100	ST. LUCIE 1 - REACTOR TRIP SYSTEM RELIABILITY - VENDOR RELATED MODIFICATIONS			
53153	ST. LUCIE 1 - ITEMS 4.2.1 & 4.2.2 - PREVENTATIVE MAINT PROGRAM FOR REACTOR TRIP BREAKERS - MAINTENANCE & TRENDING			
11108	ST. LUCIE 1 - FIRE PROTECTION SER SUPPLEMENT		1	05/01/84
43852	ST LUCIE 1 - EXEMPTION FROM APP. R SECT. 111G		1	05/01/84
43853	ST LUCIE 1 - EXEMPTION FROM APPENDIX R SECT. 111A		1	05/01/84
42900	ST. LUCIE 1 - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11		2	05/01/84
08292	ST LUCIE 1 - 10CFR 50.55 A(6) INSERVICE TESTING		1	05/01/84
08083	ST LUCIE 1 -CONTROL OF HEAVY LOADS OVER SPENT FUEL POOL (NUREG 0612)		2	06/01/84
08923	ST. LUCIE 1 - REVIEW OF ASYMETRIC LOCA LOADS SUBMITTAL		2	06/01/84
42492	ST. LUCIE 1 - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21)		1	07/01/84
49693	ST. LUCIE 1 - REACTOR COOLANT PUMP TRIP			07/01/84
<u>TMI ACTIONS</u>				
<u>COMPLETED ACTIONS</u>				
47816	ST. LUCIE 1- NUREG 0737 ITEM II.K.2.17 POTENTIAL FOR VOIDING IN RCS		1	<u>COMPLETE</u> 10/05/83
45266	ST. LUCIE 1 - NUREG-0737 II.K.3.1, AUTO PORV ISOLATION		1	10/06/83
45312	ST. LUCIE 1 - NUREG-0737 II.K.3.2, REPORT ON PORV FAILURES		1	10/06/83
<u>ACTIVE ACTIONS</u>				
45168	ST. LUCIE 1 - NUREG-0737 II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION		1	<u>TARGET</u> 10/30/83
46339	ST. LUCIE 1 - NUREG-0737 III.A.2.2, METEOROLOGICAL DATA UPGRADE		1	11/30/83
48200	ST. LUCIE 1 -NUREG 0737 II.K.3.31 COMPLIANCE WITH 10 CFR 50.46		1	11/30/83
45861	ST. LUCIE 1 - NUREG-0737 II.K.3.30, SB LOCA OUTLINE		1	01/30/84
51205	ST. LUCIE 1 - I.D.1 CONTROL ROOM DESIGN REVIEW			06/01/84
51285	ST. LUCIE 1 - I.D.2. SAFETY PARAMETER DISPLAY SYSTEM			06/01/84
46905	ST. LUCIE 1 - II.K.2.13 THERMAL MECHANICAL REPORT		1	09/30/84
45980	ST. LUCIE 1 - NUREG-0737 III.A.1.2, TECHNICAL SUPPORT CENTER		1	10/30/84
46051	ST. LUCIE 1 - NUREG-0737 III.A.1.2, OPERATIONAL SUPPORT CENTER		1	10/30/84
46123	ST. LUCIE 1 - NUREG-0737 III.A.1.2, EMERGENCY OPERATIONS FACILITY		1	10/30/84
44617	ST LUCIE 1 - RV AND SV TESTING		1	10/30/84
44267	ST. LUCIE 1 - NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES		1	11/30/84

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CONDENSED MANAGEMENT REPORT

(CONTINUATION)

ST LUCIE 1

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>TMI ACTIONS</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>ACTIVE ACTIONS (CONTINUATION)</u>				
44337	ST. LUCIE 1 - NUREG-0737 I.C.1.3.1, ABNORMAL TRANSIENT OPERATOR GUIDELINES		1	<u>TARGET</u> 11/30/84
<u>COMPLETED ACTIONS</u>				
<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
52556	ST LUCIE 1 - LICENSE AMENDMENT DEALING WITH MINIMUM SHIFT CREW COMPOSITION			<u>COMPLETE</u> 02/24/84
49853	ST. LUCIE 1 - LIC. AMDT. FOR LINEAR HEAT RATE CHANGE			03/01/84
48640	ST. LUCIE 1 - TRANSFER TO EXXON FUEL			03/01/84
49483	ST. LUCIE 1 - LIC. AMDT. FOR CYCLE 6 RELOAD			03/01/84
52410	ST. LUCIE 1 - LICENSE AMENDMENT TO DELETE CONTAINMENT LEAKAGE PATH TEST FOR TAP 2 OF PENETRATION 25			03/14/84
52299	ST LUCIE 1 - LICENSE AMENDMENT TO MODIFY PEAKING FACTORS AND PEAKING PENALTIES			04/25/84
<u>ACTIVE ACTIONS</u>				
51044	ST LUCIE 1 - LIC AMDT FOR RELIEF FROM TS REQmnt			<u>TARGET</u> 10/30/83
52031	ST. LUCIE 1 - CONTAINMENT PURGE VALVE OPERABILITY			11/01/83
48641	ST. LUCIE 1 - EXEMPTION FROM APPENDIX R, III G SCHEDULAR			11/15/83
48924	ST. LUCIE 1 - DIESEL GENERATOR SURVEILLANCE REQ.			12/15/83
52558	ST. LUCIE 1 - ALTERNATE SHUTDOWN SUBMITTAL			01/16/84
48795	ST. LUCIE 1 - REANALYSES OF RCP SEIZED ROTOR AND LOSS OF ALL NON-EMERGENCY AC POWER			01/30/84
53538	ST. LUCIE - LICENSE AMENDMENT TO DELETE APPENDIX B-PART 1 FROM THE TECHNICAL SPECIFICATIONS			04/13/84
43869	ST. LUCIE 1 - FIRE PROTECTION TECH SPEC		3	04/15/84
48622	ST. LUCIE 1 - 10 CFR 50.55 A(6) - INSERVICE INSPECTION		1	06/01/84
43241	ST. LUCIE 1 - STATION BLACKOUT		1	06/01/84
54417	ST. LUCIE 1 - RCS VENTS TECHNICAL SPECIFICATIONS NUREG 0737 NUREG 073 ITEM IT.B.1			09/30/84
54571	ST. LUCIE 1 - TECHNICAL SPECIFICATIONS COVERED BY GENERIC LETTERS 83-36, 83-37 FOR NUREG-0737			09/30/84
<u>ANTICIPATED ACTIONS</u>				
53635	ST. LUCIE 1 - ITEM 1.2 - POST TRIP REVIEW-DATA AND INFORMATION CAPABILITY			<u>INITIATION</u> 11/01/84
53718	ST. LUCIE 1 - ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS			11/01/84
53801	ST. LUCIE 1 - ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS			11/01/84
53884	ST. LUCIE 1 - ITEM 3.2.3 - POST-MAINTENANCE TESTING -CHANGES TO TECH. SPEC. - ALL SR COMPONENTS			11/01/84
53940	-ITEMS 4.2.3 & 4.2.4-PREVENTATIVE MAINT. PROGRAM FOR REACTOR TRIP BREAKERS-LIFE TESTING & REPLACEMENT			11/01/84
54028	ST. LUCIE 1 - ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS			11/01/84
54111	ST. LUCIE 1 - ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL TESTING - DIVERSE TRIP FEATURES			11/01/84

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CONDENSED MANAGEMENT REPORT

FACILITY: ST LUCIE 2

PLANT LOCATION: 12 MI SE OF FT. PIERCE, FLA
 DOCKET NUMBER: 050-00389
 ARCH/ENGINEER: EBASCO
 IE INSPECTOR: S. ELROD

LICENSED POWER: 2560 MWT
 DESIGN POWER: 0804 MWE
 NSSS VENDOR: COMB

PROJECT MANAGER: D. SELLS
 BRANCH CHIEF: J. MILLER
 LIC. ASSISTANT: P. KREUTZER

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>MULTI-PLANT</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>ACTIVE ACTIONS</u>				
52965	ST. LUCIE 2 - ITEMS 3.1.1 & 3.1.2 -- POS MAIN: ENHANCE TESTING PROCEDURES/VENDOR RECOMMENDATIONS - RTS COMPONENTS			<u>TARGET</u>
53046	ST. LUCIE 2 - ITEM 3.1.3 - POST-MAINTENANCE TESTING - CHANGES TO TECH SPECS - RTS COMPONENTS			
53101	ST. LUCIE 2 - REACTOR TRIP SYSTEM RELIABILITY - VENDOR RELATED MODIFICATIONS			
53154	ST. LUCIE 2 - ITEMS 4.2.1 & 4.2.2 - PREVENTATIVE MAINT PROGRAM FOR REACTOR TRIP BREAKERS - MAINTENANCE & TRENDING			
51136	ST. LUCIE 2 - INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)			
52274	ST. LUCIE 2 - CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPA#C-10)			
52803	ST. LUCIE 2 - ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION AND PROCEDURES			
52884	ST. LUCIE 2 - ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RTS COMPONENTS			
<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>TMI ACTIONS</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>ACTIVE ACTIONS</u>				
51206	ST. LUCIE 2 - I.D.1 CONTROL ROOM DESIGN REVIEW			
51286	ST. LUCIE - I.D.2. SAFETY PARAMETER DISPLAY SYSTEM			
51605	ST. LUCIE 2 - RV AND SV TESTING			
54982	ST. LUCIE 2, II.K.2.13, THERMAL MECHANICAL REPORT			06/30/84
<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
51493	ST. LUCIE 2 - INSERVICE INSPECTION PROGRAM			<u>COMPLETE</u>
51782	ST LUCIE 2 - TURBINE OVERSPEED PROTECTION: VALVE TESTING CYCLES			10/16/83
51491	ST. LUCIE 2 - REVIEW DETAILED CONTROL RM. DESIGN REPORT			12/20/83
52557	ST LUCIE 2 - LICENSE AMENDMENT DEALING WITH MINIMUM SHIFT CREW COMPOSITION			02/09/84
51532	ST. LUCIE 2 - BARRIER FOR HIGH ENERGY EQUIP			02/24/84
51421	ST. LUCIE 2 - FLAME IMPINGEMENT SHIELDS INSIDE CONTAINMENT			03/27/84
<u>ACTIVE ACTIONS</u>				
52559	ST. LUCIE 2 - INSERVICE INSPECTION AND TESTING FOR CLASS 1,2, AND 3 COMPONENTS			<u>TARGET</u>
51422	ST. LUCIE 2 - STEAM GENERATOR PRESSURE/TEMPERATURE LIMITATION			04/29/83
51474	ST. LUCIE 2 - EVALUATE FP & L REPORT ON COLD SHUTDOWN REQR.			05/06/83
51488	ST. LUCIE 2 - ENVIRONMENTAL QUAL OF MECH. & ELECT. EQUIP			05/20/83
51489	ST. LUCIE 2 - REVIEW AND APPROVED REVISED PHYS. SECURITY PLAN			05/20/83
51492	ST. LUCIE 2 - REVIEW ANAL. OF CONTINUOUS CONT. PURGE SYST.			05/20/83
51864	ST. LUCIE 2 - TS ADMINISTRATION AND EDITORIAL CHANGES - I L-83-349, 6/17/83			08/31/83
51530	ST. LUCIE 2 - EMERGENCY RESPONSE CAPABILITY			10/30/83
51529	ST. LUCIE 2 - PASS CORE DAMAGE EST. PROCEDURE			12/30/83
54266	ST. LUCIE 2 - LICENSE AMDT TO MODIFY ISI REQUIREMENT FOR TWO MECHANICAL SNUBBERS			03/16/84
51490	ST. LUCIE 2 - REVIEW INADEQUATE CORE COOL. REPT.			03/30/84
53387	ST LUCIE 2 - AMENDS FIGURES IN TECHNICAL SPECIFICATIONS ASSOCIATED WITH AXIAL SHAPE INDEX			04/15/84
54513	ST. LUCIE 2 - PRESSURIZER HEATER TRANSFORMER BARRIER DESIGN			05/25/84
53428	ST. LUCIE 2 - FIRE PROTECTION, COMPLIANCE WITH APPENDIX R			06/01/84
54485	ST. LUCIE - 2, BORON MIXING TEST REPORT - CEN-259			06/15/84

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CONDENSED MANAGEMENT REPORT

(CONTINUATION)

ST LUCIE 2

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>ACTIVE ACTIONS</u> (CONTINUATION)				<u>TARGET</u>
54745	ST LUCIE 2 - TURBINE DISC INTEGRITY			06/30/84
54746	ST. LUCIE 2 - FIRE PROTECTION FEATURES			07/31/84
54463	ST. LUCIE 2 - LICENSE AMDT. TO RERACK THE SPENT FUEL POOL TO INCREASE THE SPENT FUEL STORAGE CAPACITY			08/10/84
54572	ST. LUCIE 2 - TECHNICAL SPECIFICATIONS COVERED BY GENERIC LETTERS 83-36, 83-37 FOR NUREG-0737			09/30/84
51531	ST. LUCIE 2 - HEAVY LOADS			12/30/84
51533	ST. LUCIE 2 - AXIAL GROWTH			12/30/84
<u>ANTICIPATED ACTIONS</u>				<u>INITIATION</u>
53636	ST. LUCIE 2 - ITEM 1.2 - POST TRIP REVIEW-DATA AND INFORMATION CAPABILITY			11/01/84
53719	ST. LUCIE 2 - ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS			11/01/84
53802	ST. LUCIE 2 - ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS			11/01/84
53885	ST. LUCIE 2 - ITEM 3.2.3 - POST-MAINTENANCE TESTING -CHANGES TO TECH. SPEC - ALL SR COMPONENTS			11/01/84
53941	-ITEMS 4.2.3 & 4.2.4-PREVENTATIVE MAINT. PROGRAM FOR REACTOR TRIP BREAKERS-LIFE TESTING & REPLACEMENT			11/01/84
54029	ST. LUCIE 2 - ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS			11/01/84
54112	ST. LUCIE 2 - ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL TESTING - DIVERSE TRIP FEATURES			11/01/84

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CONDENSED MANAGEMENT REPORT

FACILITY: SUMMER 1

PLANT LOCATION: 26 MI NW OF COLUMBIA, SC
 DOCKET NUMBER: 050-00395
 ARCH/ENGINEER: GIL
 IE INSPECTOR: .

LICENSED POWER: 2775 MWT
 DESIGN POWER: 0900 MWE
 NSSS VENDOR: WEST

PROJECT MANAGER: J. HOPKINS
 BRANCH CHIEF: E. ADENSAM
 LIC. ASSISTANT: M. DUNCAN

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>MULTI-PLANT</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
53197	SUMMER 1 - ITEM 4.3 - AUTOAMTIC ACTUATION OF SHUNT TRIP ATTACHMENT FOR W AND B & W PLANTS			<u>COMPLETE</u> 11/23/83
49761	SUMMER 1 - NUREG 0737 TECH SPECS (GENERIC LTR 82-16)			04/20/84
<u>ACTIVE ACTIONS</u>				
51137	SUMMER 1 - INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)			<u>TARGET</u>
52804	SUMMER 1 - ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION AND PROCEDURES			
52885	SUMMER 1 - ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RTS COMPONENTS			
52966	SUMMER 1 - ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING PROCEDURES/VENDOR RECOMMENDATIONS - RTS COMPONENTS			
53047	SUMMER 1-ITEM 3.1.3 -POST-MAINTENANCE TESTING-CHANGES TO TECH SPECS-RTS COMPONENTS			
53102	SUMMER 1 - REACTOR TRIP SYSTEM RELIABILITY- VENDOR RELATED MODIFICATIONS			
53155	SUMMER 1 - ITEMS 4.2.1 & 4.2.2 - PREVENTATIVE MAINTENANCE PROGRAM FOR REACTOR TRIP BREAKERS - MAINTENANCE & TRENDING			
52275	SUMMER 1 - CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPA#C-10)			
52376	SUMMER 1 - CONTROL OF HEAVY LOADS (PHASE I)			
<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>TMI ACTIONS</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
53549	V.C. SUMMER - VOIDING IN THE RCS NUREG-0737 II.K.2.17			<u>COMPLETE</u> 02/15/84
<u>ACTIVE ACTIONS</u>				
51207	SUMMER 1 - I.D.1 CONTROL ROOM DESIGN REVIEW			<u>TARGET</u>
51287	SUMMER 1 - I.D.2. SAFETY PARAMETER DISPLAY SYSTEM			
51606	SUMMER 1 - RV AND SV TESTING			
54983	SUMMER 1, II.K.2.13, THERMAL MECHANICAL REPORT			06/30/84
49464	SUMMER 1 - INSTRUMENTS FOR DETECTION ON INADEQUATE CORE COOLING (II.F.2)			09/30/84
<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
51356	V. C. SUMMER - FIRE SUPPRESSION SYSTEMS			<u>COMPLETE</u> 10/12/83
52054	V.C. SUMMER - ORGANIZATIONAL CHANGES			10/31/83
49975	V.C. SUMMER - SEISMIC CONFIRMATORY PROGRAM			11/16/83
52093	SUMMER 1 - RELIEF FROM ASME CODE/SECTION XI HYDROSTATIC TESTING REQUIREMENTS FOR SUMMER #1 FEEDWATER SYSTEM			11/21/83
49208	V. C. SUMMER - ATTCH. 5 MISC			12/23/83
52057	V.C. SUMMER - RADIATION MONITORS - T.S. REQTS			01/05/84
52058	V.C. SUMMER - T.S. SURVEILLANCE - SERVICE WATER			02/03/84
53553	V. C. SUMMER - GL81-27 REMINDER			02/06/84
53552	V. C. SUMMER - EMERG. PLANNING EXERCISE EXEMPTION			03/20/84
52055	V.C. SUMMER - T.S. SURVEILLANCE TENDONS			04/16/84
53281	V.C. SUMMER - FWIV TECH. SPECS.			04/16/84
52338	V.C. SUMMER - CONTROL ROOM AIR FLOW - T.S 3.7.6			04/25/84

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CONDENSED MANAGEMENT REPORT

(CONTINUATION)

SUMMER 1

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>ACTIVE ACTIONS</u>				<u>TARGET</u>
52413	V. C. SUMMER - SEISMIC MONITORING PROGRAM			
53317	V. C. SUMMER - SPENT FUEL POOL RERACK AND NEW 4.3% ENRICHED FUEL RACKS			
52059	V.C. SUMMER - GL 82-33 RESPONSE			
51568	V. C. SUMMER - REACTOR TRIP BREAKERS			
53550	V.C. SUMMER - RX BLDG SUMP ISO VALVES-CONT. PENET. COND. OVERCURRENT PROT. DEVICES			
53551	V. C. SUMMER - GL83-37 RESPONSE - HYDROGEN MONITOR T.S.			
53406	V.C. SUMMER - AUTO SHUT TRIP TECH SPECS			
51630	V.C. SUMMER - 25 MILE BACKUP EOF			04/30/84
52124	V. C. SUMMER - L. C. (10) LOW TEMP. OVER PRESS. PROT. SYS.			05/30/84
49539	V.C. SUMMER - REQUALIFICATION PROGRAM			05/30/84
49976	V.C. SUMMER - ISI/PUMP RELIEF REQUESTS			06/30/84
52056	V.C. SUMMER - OVERPOWER & OVERTEMP. DELTA T SETPTS			06/30/84
53282	V.C. SUMMER - T.S. MICH. SNUBBERS			06/30/84
53283	V.C. SUMMER - RBCU FAN MOTORS EDDY CURRTEENT BRAKE- CONT. PENET. COND. OVER CURRENT PROT. DEVICES			06/30/84
49203	V. C. SUMMER - SW INTAKE STRUCTURE			07/30/84
54467	V.C. SUMMER - RHR SYS. POWER LOCKOUTS (L.C. 15)			07/31/84
49209	V. C. SUMMER - TYPE C TESTS			12/30/84
<u>ANTICIPATED ACTIONS</u>				<u>INITIATION</u>
54899	V. C. SUMMER - STM. GEN. INSPECTION PORTS (L.C. 13)			08/17/84
53637	SUMMER 1 - ITEM 1.2 - POST TRIP REVIEW-DATA AND INFORMATION CAPABILITY			11/01/84
53720	SUMMER 1 - ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS			11/01/84
53803	SUMMER 1- ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS			11/01/84
53886	SUMMER 1 - ITEM 3.2.3 - POST-MAINTENANCE TESTING -CHANGES TO TECH. SPEC. - ALL SR COMPONENTS			11/01/84
53942	-ITEMS 4.2.3 & 4.2.4-PREVENTATIVE MAINT. PROGRAM FOR REACTOR TRIP BREAKERS-LIFE TESTING & REPLACEMENT			11/01/84
54030	SUMMER 1 - ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS			11/01/84
54113	SUMMER 1 - ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL TESTING - DIVERSE TRIP FEATURES			11/01/84

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CONDENSED MANAGEMENT REPORT

FACILITY: SURRY 1

PLANT LOCATION: 17 MI NW OF NEWPORT NEWS, VA
 DOCKET NUMBER: 050-00280
 ARCH/ENGINEER: S&W
 IE INSPECTOR: D. BURKE

LICENSED POWER: 2441 MWT
 DESIGN POWER: 0788 MWE
 NSSS VENDOR: WEST

PROJECT MANAGER: D. NEIGHBORS
 BRANCH CHIEF: S. VARGA
 LIC. ASSISTANT: C. PARRISH

TAC #	TAC DESCRIPTION	MULTI-PLANT	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				
12976	SURRY 1 - ADEQUACY OF STATION ELECTRIC. DISTR. SYS VOLTAGES		2	COMPLETE 10/14/83
47167	SURRY 1 NATURAL CIRCULATION COOLDOWN (GENERIC LTR 81-21)		2	10/19/83
11091	SURRY 1 - FIRE PROTECTION SER SUPPLEMENT		1	01/17/84
08617	SURRY 1 - FILTER TECH SPECS		3	01/17/84
08521	SURRY 1 - REVIEW OF ASYMMETRIC LOCA LOADS SUBMITTAL		2	02/01/84
12277	SURRY 1 - HELB AND CONSEQUENTIAL SYSTEM FAILURE SURRY 1		3	02/02/84
11974	SURRY 1 - REACTOR CAVITY SEAL RING GENERIC ISSUE		2	02/10/84
49762	SURRY 1 - "NUREG 0737 TECH SPECS (GENERIC LETTER 82-16 82-16 OR 83-02)"			03/28/84
<u>ACTIVE ACTIONS</u>				
52805	SURRY 1 - ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION AND PROCEDURES			TARGET
52886	SURRY 1 - ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RTS COMPONENTS			
52967	SURRY 1 - ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING PROCEDURES/VENDOR RECOMMENDATIONS - RTS COMPONENTS			
53048	SURRY 1-ITEM 3.1.3 -POST-MAINTENANCE TESTING-CHANGES TO TECH SPECS-RTS COMPONENTS			
53103	SURRY 1 - REACTOR TRIP SYSTEM RELIABILITY - VENDOR RELATED MODIFICATIONS			
53156	SURRY 1 - ITEMS 4.2.1 & 4.2.2 - PREVENTATIVE MAINTENANCE PROGRAM FOR REACTOR TRIP BREAKERS - MAINTENANCE & TRENDING			
53198	SURRY 1 - ITEM 4.3 - AUTOAMTIC ACTUATION OF SHUNT TRIP ATTACHMENT FOR W AND B & W PLANTS			
42468	SURRY 1 - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21)		1	12/31/83
08620	SURRY 1 - FUEL HANDLING ACCIDENT INSIDE CONTAINMENT		2	02/01/84
08102	SURRY 1 - APPENDIX I TECH SPEC IMPLEMENTATION REVIEW		3	03/01/84
08084	SURRY 1 -CONTROL OF HEAVY LOADS OVER SPENT FUEL POOL (NUREG 0612)		2	06/01/84
51138	SURRY 1 - INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)			06/01/84
49681	SURRY 1 - REACTOR COOLANT PUMP TRIP			07/01/84
42867	SURRY 1 - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11		2	08/31/84
52276	SURRY 1 - CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPA#C-10)			09/30/85
<u>TAC # TAC DESCRIPTION TMI ACTIONS PRIORITY CRITICAL DATE</u>				
<u>COMPLETED ACTIONS</u>				
47978	SURRY 1 - PLANT SHIELDING MODIFICATIONS (II.B.2.2)		1	COMPLETE 10/31/83
47821	SURRY 1 - NUREG 0737 ITEM II K2.17 POTENTIAL FOR VOIDING IN RCS		1	01/09/84
44480	SURRY 1 - NUREG-0737 II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS		1	02/27/84
<u>ACTIVE ACTIONS</u>				
45173	SURRY 1 - NUREG-0737 II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION		1	TARGET 12/31/83
45866	SURRY 1 - NUREG-0737 II.K.3.30, SB LOCA OUTLINE		1	12/31/83
48206	SURRY 1 -NUREG 0737 II.K.3.31 COMPLIANCE WITH 10 CFR 50.46		1	12/31/83
44622	SURRY 1 - RV AND SV TESTING		1	03/31/84
45985	SURRY 1 - NUREG-0737 III.A.1.2, TECHNICAL SUPPORT CENTER		1	09/30/84
46056	SURRY 1 - NUREG-0737 III.A.1.2, OPERATIONAL SUPPORT CENTER		1	09/30/84
46128	SURRY 1 - NUREG-0737 III.A.1.2, EMERGENCY OPERATIONS FACILITY		1	09/30/84
46344	SURRY 1 - NUREG-0737 III.A.2.2, METEOROLOGICAL DATA UPGRADE		1	09/30/84
46907	SURRY 1 - II K2.13 THERMAL MECHANICAL REPORT		1	09/30/84

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(CONTINUATION)

SURRY 1

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>TMI ACTIONS</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>ACTIVE ACTIONS</u> (CONTINUATION)				
51208	SURRY 1 - I.D.1 CONTROL ROOM DESIGN REVIEW			<u>TARGET</u> 09/30/84
51288	SURRY 1 - I.D.2. SAFETY PARAMETER DISPLAY SYSTEM			09/30/84
44272	SURRY 1 - NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES		1	11/30/84
44342	SURRY 1 - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES		1	11/30/84
<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
51369	SURRY 1 - CONTAINMENT ISOLATION VALVES TECH SPECS			<u>COMPLETE</u> 11/14/83
48968	SURRY 1 - ISI SECOND TEN YEAR INTERVAL			01/24/84
52293	SURRY 1 - REVISE SHUBBER TECH SPECS			01/31/84
52295	SURRY 1 - ADD HEALTH PHYSICS SUPERVISOR TO T.S.6.4.B.1.B			01/31/84
52358	SURRY 1 - REDUCE BORON CONCENTRATION IN BIT			02/24/84
49552	SURRY 1 - ADDENDUM TO FIRST TEN YEAR ISI - RELIEF REQUEST			02/28/84
53260	SURRY 1 - EXTEND BURNUP TO 45,000 MWD/MTU			04/09/84
43115	SURRY 1 - FIRE PUMP TECH SPEC 4.18.B.1.F(2)		1	04/20/84
<u>ACTIVE ACTIONS</u>				
53535	SURRY 1 - REGULATORY EFFECTIVENESS REVIEW			<u>TARGET</u>
51371	SURRY 1 - REACTOR VESSEL COOLANT LEVEL MONITOR			12/31/83
48118	SURRY 1 - REORGANIZATION			02/01/84
49144	SURRY 1 - 1ST SECOND TEN YEAR INTERVAL			03/01/84
43113	SURRY 1 - HIGH ENERGY BREAK INSPECTION PROGRAM T.S. 4.15-A		1	03/15/84
53359	SURRY 1 - ASME XI RELIEF REQUEST (12/83)			04/01/84
53533	SURRY 1 - ADEQUACY OF STATION ELECTRIC DISTRIBUTION SYSTEM VOLTAGES (FOLLOWUP ON B-48)			04/01/84
54847	SURRY 1 - CORE RELOAD METHODOLOGY AUDIT			06/01/84
49493	SURRY 1 - CHARGING PUMP TECH SPECS			06/01/84
54626	SURRY 1 - ASME XI RELIEF (3/26/84) -HYDROSTATIC TEST			06/15/84
54742	SURRY 1 - AFWS - CHANGE OPERABLE TO AVAILABLE			06/30/84
53492	SURRY 1 - LIFT ORDER TO SHOW CAUSE DATED MARCH 13, 1979			06/30/84
54418	SURRY 1 - RCS VENTS TECHNICAL SPECIFICATIONS NUREG 073 ITEM II.B.1			09/30/84
54573	SURRY 1 - TECHNICAL SPECIFICATIONS COVERED BY GENERIC LETTERS 83-36, 83-37 FOR NUREG-0737			09/30/84
49148	SURRY 1 - DRY CASK INDEPENDENT SPENT FUEL STORAGE INSTALLATION			12/31/84
<u>ANTICIPATED ACTIONS</u>				
53638	SURRY 1 - ITEM 1.2 - POST TRIP REVIEW-DATA AND INFORMATION CAPABILITY			<u>INITIATION</u> 11/01/84
53721	SURRY 1 - ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS			11/01/84
53804	SURRY 1 - ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS			11/01/84
53887	SURRY 1 - ITEM 3.2.3 - POST-MAINTENANCE TESTING -CHANGES TO TECH. SPEC. - ALL SR COMPONENTS			11/01/84
53943	-ITEMS 4.2.3 & 4.2.4-PREVENTATIVE MAINT. PROGRAM FOR REACTOR TRIP BREAKERS-LIFE TESTING & REPLACEMENT			11/01/84
54031	SURRY 1 - ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS			11/01/84
54114	SURRY 1 - ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL CAPABILITY			11/01/84

DATA AS OF - 04/30/84

CONDENSED MANAGEMENT REPORT

FACILITY: SURRY 2

PLANT LOCATION: 17 MI NW OF NEWPORT NEWS, VA
 DOCKET NUMBER: 050-00281
 ARCH/ENGINEER: S&W
 IE INSPECTOR: D. BURKE

LICENSED POWER: 2441 MWT
 DESIGN POWER: 0788 MWE
 NSSS VENDOR: WEST

PROJECT MANAGER: D. NEIGHBORS
 BRANCH CHIEF: S. VARGA
 LIC. ASSISTANT: C. PARRISH

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>MULTI-PLANT</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
12977	SURRY 2 - ADEQUACY OF STATION ELECTRIC. DISTR. SYS. VOLTAGES		2	<u>COMPLETE</u> 10/19/83
47168	SURRY 2 NATURAL CIRCULATION COOLDOWN (GENERIC LTR 81-21)		2	10/19/83
08616	SURRY 2 - FILTER TECH SPECS		3	01/17/84
11092	SURRY 2 - FIRE PROTECTION SER SUPPLEMENT		1	01/17/84
12278	SURRY 2 - HELB AND CONSEQUENTIAL SYSTEM FAILURE SURRY 2		3	02/02/84
11975	SURRY 2 - REACTOR CAVITY SEAL RING GENERIC ISSUE		2	02/10/84
49763	SURRY 2 - "NUREG 0737 TECH SPECS (GENERIC LETTER 82-16 82-16 OR 83-02)"			03/28/84
<u>ACTIVE ACTIONS</u>				
52806	SURRY 2 - ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION AND PROCEDURES			<u>TARGET</u>
52887	SURRY 2 - ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RTS COMPONENTS			
52968	SURRY 2 - ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING PROCEDURES/VENDOR RECOMMENDATIONS - RTS COMPONENTS			
53049	SURRY 2-ITEM 3.1.3 -POST-MAINTENANCE TESTING-CHANGES TO TECH SPECS-RTS COMPONENTS			
53104	SURRY 2 - REACTOR TRIP SYSTEM RELIABILITY - VENDOR RELATED MODIFICATIONS			
53157	SURRY 2 - ITEMS 4.2.1 & 4.2.2 - PREVENTATIVE MAINTENANCE PROGRAM FOR REACTOR TRIP BREAKERS - MAINTENANCE & TRENDING			
53199	SURRY 2 - ITEM 4.3 - AUTOAMTC ACTUATION OF SHUNT TRIP ATTACHMENT FOR W AND B & W PLANTS			
08520	SURRY 2 - REVIEW OF ASYMMETRIC LOCA LOADS SUBMITTAL		2	
42469	SURRY 2 - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21)		1	12/31/83
08619	SURRY 2 - FUEL HANDLING ACCIDENT INSIDE CONTAINMENT		2	02/01/84
08103	SURRY 2 - APPENDIX I TECH SPEC IMPLEMENTATION REVIEW		3	03/01/84
08085	SURRY 2 -CONTROL OF HEAVY LOADS OVER SPENT FUEL POOL (NUREG 0612)		2	06/01/84
51139	SURRY 2 - INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)			06/01/84
49682	SURRY 2 - REACTOR COOLANT PUMP TRIP			07/01/84
42868	SURRY 2 - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11		2	08/31/84
52277	SURRY 2 - CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPA#C-10)			09/30/85
<u>TAC # TAC DESCRIPTION IMI ACTIONS PRIORITY CRITICAL DATE</u>				
<u>COMPLETED ACTIONS</u>				
47979	SURRY 2 - PLANT SHIELDING MODIFICATIONS (II.B.2.2)		1	<u>COMPLETE</u> 10/31/83
47822	SURRY 2 - NUREG 0737 ITEM II K2.17 POTENTIAL FOR VOIDING IN RCS		1	01/09/84
44481	SURRY 2 - NUREG-0737 II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS		1	02/27/84
<u>ACTIVE ACTIONS</u>				
44343	SURRY 2 - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES		1	<u>TARGET</u> 08/15/83
45174	SURRY 2 - NUREG-0737 II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION		1	12/31/83
45867	SURRY 2 - NUREG-0737 II.K.3.30, SB LOCA OUTLINE		1	12/31/83
48207	SURRY 2 -NUREG 0737 II.K.3.31 COMPLIANCE WITH 10 CFR 50.46		1	12/31/83
44623	SURRY 2 - RV AND SV TESTING		1	03/31/84
45986	SURRY 2 - NUREG-0737 III.A.1.2, TECHNICAL SUPPORT CENTER		1	09/30/84
46057	SURRY 2 - NUREG-0737 III.A.1.2, OPERATIONAL SUPPORT CENTER		1	09/30/84
46129	SURRY 2 - NUREG-0737 III.A.1.2, EMERGENCY OPERATIONS FACILITY		1	09/30/84
46345	SURRY 2 - NUREG-0737 III.A.2.2, METEOROLOGICAL DATA UPGRADE		1	09/30/84

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CONDENSED MANAGEMENT REPORT

(CONTINUATION)

SURRY 2

TAC #	TAC DESCRIPTION	IMI ACTIONS	PRIORITY	CRITICAL DATE
<u>ACTIVE ACTIONS</u> (CONTINUATION)				
46908	SURRY 2 - II K2.13 THERMAL MECHANICAL REPORT		1	TARGET 09/30/84
51209	SURRY 2 - I.D.1 CONTROL ROOM DESIGN REVIEW			09/30/84
51289	SURRY 2 - I.D.2. SAFETY PARAMETER DISPLAY SYSTEM			09/30/84
44273	SURRY 2 - NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES		1	11/30/81
<u>TAC # TAC DESCRIPTION PLANT SPECIFIC</u>				
<u>COMPLETED ACTIONS</u>				
51370	SURRY 2 - CONTAINMENT ISOLATION VALVES TECH SPECS			COMPLETE 11/14/83
52294	SURRY 2 - REVISE SHUBBER TECH SPECS			01/31/84
52296	SURRY 2 - ADD HEALTH PHYSICS SUPERVISOR TO TECH SPEC 6.4.B.1.B			01/31/84
52359	SURRY 2 - REDUCE BORON CONCENTRATION IN BIT			02/24/84
49553	SURRY 2 - ADDENDUM TO FIRST TEN YEAR ISS - RELIEF REQUEST			02/28/84
53261	SURRY 2 - EXTEND BURNUP TO 45,000 MWD/MTU			04/09/84
43116	SURRY 2 - FIRE PUMP TECH SPEC 4.18.B.1.F(2)		1	04/20/84
<u>ACTIVE ACTIONS</u>				
53536	SURRY 2 - REGULATORY EFFECTIVENESS REVIEW			TARGET
51887	SURRY 2 RELIEF FROM IWA - 24--(A) OF ASME SECTION XI			08/15/83
51372	SURRY 2 - REACTOR VESSEL COOLANT LEVEL MONITOR			12/31/83
48119	SURRY 2 - REORGANIZATION			02/01/84
49143	SURRY 2 - ISI - SECOND TEN YEAR INTERVAL			03/01/84
49145	SURRY 2 - IST - SECOND TEN YEAR INTERVAL			03/01/84
43114	SURRY 2 - HIGH ENERGY BREAK INSPECTION PROGRAM TECH SPEC 4.15-A		1	03/01/84
53360	SURRY 2 - ASME XI RELIEF REQUEST (12/83)			04/01/84
53534	SURRY 2 - ADEQUACY OF STATION ELECTRIC DISTRIBUTION SYSTEM VOLTAGES (FOLLOWUP OF B-48)			04/01/84
54848	SURRY 2 - CORE RELOAD METHODOLOGY AUDIT			06/01/84
49494	SURRY 2 - CHARGING PUMP TECH. SPECS			06/01/84
54799	SURRY 2 - ASME XI RELIEF (4/12/84) - HYDROSTATIC TEST			06/15/84
54743	SURRY 2 - AFWS - CHANGE OPERABLE TO AVAILABLE			06/30/84
54419	SURRY 2 - RCS VENTS TECHNICAL SPECIFICATIONS NUREG 073 ITEM II.B.1			09/30/84
54574	SURRY 2 - TECHNICAL SPECIFICATIONS COVERED BY GENERIC LETTERS 83-36, 83-37 FOR NUREG-0737			09/30/84
49149	SURRY 2 - DRY CASK INDEPENDENT SPENT FUEL STORAGE INSTALLATION			12/31/84
<u>ANTICIPATED ACTIONS</u>				
53639	SURRY 2 - ITEM 1.2 - POST TRIP REVIEW-DATA AND INFORMATION CAPABILITY			INITIATION 11/01/84
53722	SURRY 2 - ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS			11/01/84
53805	SURRY 2 - ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS			11/01/84
53888	SURRY 2 - ITEM 3.2 - POST-MAINTENANCE TESTING -CHANGES TO TECH. SPEC. - ALL SR COMPONENTS			11/01/84
53944	-ITEMS 4.2.3 & 4.2 - PREVENTATIVE MAINT. PROGRAM FOR REACTOR TRIP BREAKERS-LIFE TESTING & REPLACEMENT			11/01/84
54032	SURRY 2 - ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS			11/01/84
54115	SURRY 2 - ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL CAPABILITY			11/01/84

DATA AS OF - 04/30/84

CONDENSED MANAGEMENT REPORT

FACILITY: SUSQUEHANNA 1

PLANT LOCATION: 7 MI NE OF BERWICK, PA
 DOCKET NUMBER: 050-00387
 ARCH/ENGINEER: BECH
 IE INSPECTOR: G. RHOADES

LICENSED POWER: 3293 MW
 DESIGN POWER: 1065 MWE
 NSSS VENDOR: GE

PROJECT MANAGER: R. PERCH
 BRANCH CHIEF: A. SCHWENCER
 LIC. ASSISTANT: E. HYLTON

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>MULTI-PLANT</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>ACTIVE ACTIONS</u>				
52807	SUSQUEHANNA 1 - ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION AND PROCEDURES			<u>TARGET</u>
52888	SUSQUEHANNA 1 - ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RTS COMPONENTS			
52969	SUSQUEHANNA 1 - ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING PROCEDURES/VENDOR RECOMMENDATIONS - RTS COMPONENTS			
53050	SUSQUEHANNA 1-ITEM 3.1.3 -POST-MAINT TESTING-CHANGES TO TECH SPECS-RTS COMPONENTS			
51360	SUSQUEHANNA 1 - INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)			
52278	SUSQUEHANNA 1 - CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPA#C-10)			
<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>TMI ACTIONS</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
53321	MCGUIRE UNIT 2: NUREG-0737, II.F.2.3 INADEQUATE CORE COOLING INSTRUMENTATION; REF: TAC #45146			<u>COMPLETE</u> 03/30/84
<u>ACTIVE ACTIONS</u>				
49461	SUSQUEHANNA 1 - INSTRUMENTS FOR DETECTION CN INADEQUATE CORE COOLING (II.F.2)			<u>TARGET</u>
51210	SUSQUEHANNA 1 - I.D.1 CONTROL ROOM DESIGN REVIEW			
51290	SUSQUEHANNA 1 - I.D.2. SAFETY PARAMETER DISPLAY SYSTEM			
54584	SUSQUEHANNA 1 - II.D.1 - RV & SV TESTING			
<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
49595	SUSQUEHANNA 1 - REVISION 6 TO EMERGENCY PLAN			<u>COMPLETE</u> 10/31/83
49642	SUSQUEHANNA 1 - EXPAND POWER/FLOW MAP OPERATING AREA			11/02/83
49914	SUSQUEHANNA 1 - FIRE DETECTION INSTRUMENTATION T/S TABLE 3.3.7.9-1			11/02/83
52496	SUSQUEHANNA 1 - PRIMARY CONTAINMENT ISOLATION VALVE SIGNALS - RHR/RWCU			11/18/83
52585	SUSQUEHANNA 1 - ESW PUMP LOAD SEQUENCE TIMERS			12/12/83
52151	SUSQUEHANNA 1 - MONTHLY GRAB SAMPLES			12/12/83
52150	SUSQUEHANNA 1 - FIRE HOSE STATIONS			12/12/83
54157	SUSQUEHANNA 1 - EMERGENCY PLANNING EXERCISE - EXEMPTION REQUEST			03/21/84
53232	SUSQUEHANNA 1 - SRV INPLANT TEST			03/23/84
53325	SUSQUEHANNA-1 - STANDBY GAS TREATMENT SYSTEM - TECHNICAL SPECIFICATION 3.6.5 AND 4.6.5			03/23/84
51980	SUSQUEHANNA 1 - PROPOSED TRANSCO PIPELINE			04/27/84
<u>ACTIVE ACTIONS</u>				
51982	SUSQUEHANNA 1 - INSERVICE INSPECTION PROGRAM			<u>TARGET</u>
51983	SUSQUEHANNA 1 - COMMON ELECTRICAL POWER SOURCES RE MULTIPLE CONTROL SYSTEMS FAILURE			
52048	SUSQUEHANNA 1 - MAIN STEAM LINE RADIATION - HIGH SETPOINT			
51508	SUSQUEHANNA 1 - REACTOR VESSEL MATERIAL SURVEILLANCE PROGRAM - WITHDRAWAL SCHEDULE			
51716	SUSQUEHANNA 1 - SIMULATED LOSS OF OFFSITE POWER TEST			
51958	SUSQUEHANNA 1 - LICENSE AMENDMENT TO INCLUDE CHANGES TO PHYSICAL SECURITY PLAN			
52495	SUSQUEHANNA 1 - SNUBBER INSERVICE VISUAL INSPECTION			
49596	SUSQUEHANNA 1 - REVISION 31 TO FSAR			
49597	SUSQUEHANNA 1 - REVISION 32 TO FSAR			
49256	SUSQUEHANNA 1 - SAMPLE TEST OF PROTECTIVE FUSES			

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CONDENSED MANAGEMENT REPORT

(CONTINUATION)

SUSQUEHANNA 1

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>ACTIVE ACTIONS</u> (CONTINUATION)				<u>TARGET</u>
53564	SUSQUEHANNA 1 - REVISION 7 TO EMERGENCY PLAN			
53323	SUSQUEHANNA-1 - L/C 2.C.28 (B) TMI ITEM I.G.1			
53324	SUSQUEHANNA-1; REPORTING REQUIREMENTS L/C 2.G.			
52586	SUSQUEHANNA 1 - ULTIMATE HEAT SINK T/S 4.7.1.3			
52717	SUSQUEHANNA 1 - PRIMARY CONTAINMENT ISOLATION VALVE SIGNALS - SHUTDOWN COOLING			
54490	SUSQUEHANNA 1 - TECHNICAL SPECIFICATION 3.8.1.1			
52149	SUSQUEHANNA 1 - EQUIPMENT QUALIFICATION FOR SDV PIPE BREAK ENVIRONMENT			
54614	SUSQUEHANNA 1 - HIGH ENERGY LINE BREAKS			
54633	SUSQUEHANNA 1 - EXEMPTION FROM STATE AND LOCAL PARTICIPATION IN ANNUAL EMERGENCY DRILL			
54755	SUSQUEHANNA-1 - SPRAY POND OPERATING LIMITS			
48743	SUSQUEHANNA 1 - PROPOSED AMENDMENT 1 TO NPF-14			
48878	SUSQUEHANNA 1 - CHANGE TO DIESEL GENERATOR SURVEILLANCE REQUIREMENT			
49137	SUSQUEHANNA 1 - MOLDED CASE CIRCUIT BREAKERS			
<u>ANTICIPATED ACTIONS</u>				<u>INITIATION</u>
54949	SUSQUEHANNA 1 - PHYSICAL SECURITY PLAN - CHANGE 5			05/03/84
54951	SUSQUEHANNA 1 - CHUGGING LOADS			05/04/84
54967	SUSQUEHANNA-1 FEEDWATER BYPASS LEAKAGE			05/04/84
53889	SUSQUEHANNA 1 - ITEM 3.2.3 - POST-MAINTENANCE TESTING -CHANGES TO TECH. SPEC. - ALL SR COMPONENTS			11/01/84
54033	SUSQUEHANNA 1 - ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS			11/01/84
54116	SUSQUEHANNA 1 - ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL TESTING - DIVERSE TRIP FEATURES			11/01/84
53640	SUSQUEHANNA 1 - ITEM 1.2 - POST TRIP REVIEW-DATA AND INFORMATION CAPABILITY			11/01/84
53723	SUSQUEHANNA 1 - ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS			11/01/84
53806	SUSQUEHANNA 1- ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS			11/01/84

DATA AS OF - 04/30/84

CONDENSED MANAGEMENT REPORT

FACILITY: SUSQUEHANNA 2

PLANT LOCATION: 7 MI NE OF BERWICK, PA
 DOCKET NUMBER: 050-00388
 ARCH/ENGINEER: BECH
 IE INSPECTOR: G. RHOADES

LICENSED POWER: 000 MWT
 DESIGN POWER: 1052 MWE
 NSSS VENDOR: GE

PROJECT MANAGER: R. PERCH
 BRANCH CHIEF: A. SCHWENCER
 LIC. ASSISTANT: E. HYLTON

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>IMI ACTIONS</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>ACTIVE ACTIONS</u>				
54585	SUSQUEHANNA 2 - II.D.1 - RV & SV TESTING			<u>TARGET</u>
<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>ACTIVE ACTIONS</u>				
54612	SUSQUEHANNA 2 - COMMON ELECTRICAL POWER SOURCES RE MULTIPLE CONTROL SYSTEMS FAILURE			<u>TARGET</u>
54613	SUSQUEHANNA 2 - HIGH ENERGY LINE BREAKS			
54634	SUSQUEHANNA 2 - EXEMPTION FROM STATE AND LOCAL PARTICIPATION IN ANNUAL EMERGENCY DRILL			
54754	SUSQUEHANNA - 2, SPRAY POND OPERATING LIMITS			
54756	SUSQUEHANNA-2 - NITROGEN MAKEUP SYSTEM			
54765	SUSQUEHANNA 2 - INITIAL TEST PROGRAM			
<u>ANTICIPATED ACTIONS</u>				
54950	SUSQUEHANNA 2 - PHYSICAL SECURITY PLAN - CHANGE S			<u>INITIATION</u>
54952	SUSQUEHANNA 2 - CHUGGING LOADS			05/03/84
54968	SUSQUEHANNA-2 - FEEDWATER BYPASS LEAKAGE			05/04/84
				05/04/84

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CONDENSED MANAGEMENT REPORT

FACILITY: THREE MILE ISLAND 1

PLANT LOCATION: 10 MI SE OF HARRISBURG, PA
 DOCKET NUMBER: 050-00289
 ARCH/ENGINEER: GIL
 IE INSPECTOR: R. CONTE

LICENSED POWER: 2535 MWT
 DESIGN POWER: 0819 MWE
 NSSS VENDOR: B&W

PROJECT MANAGER: J. VANVLIET
 BRANCH CHIEF: J. STOLZ
 LIC. ASSISTANT: R. INGRAM

TAC #	TAC DESCRIPTION	MULTI-PLANT	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				
51733	THREE MILE ISLAND 1 - NUREG-0630 CLADDING MODELS FOR LOCA ANALYSIS			<u>COMPLETE</u> 10/28/83
49615	TMI-1 - PLANS FOR PREVENTING EXCEEDING PTS SCREENING CRITERION			12/27/83
42602	THREE MILE ISLAND 1 - LONG TERM REVIEW CONTAINMENT PURGE AND VENT (B-24)		1	01/27/84
08603	THREE MILE ISLAND 1 - REVIEW OF ASYMMETRIC LOCA LOADS SUBMITTAL		2	03/27/84
<u>ACTIVE ACTIONS</u>				
11286	THREE MILE ISLAND 1 - TS FOR HYDRAULIC SNUBBERS		3	<u>TARGET</u>
42127	THREE MILE ISLAND 1 - DECAY HEAT REMOVAL CAPABILITY TECH SPECS		2	
51361	TMI-1 - INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)			
52808	TMI-1 - ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION AND PROCEDURES			
52889	TMI-1 - ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RTS COMPONENTS			
52970	TMI-1 - ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING PROCEDURES/VENDOR RECOMMENDATIONS - RTS COMPONENTS			
53051	TMI-1-ITEM 3.1.3 -POST-MAINTENANCE TESTING-CHANGES TO TECH SPECS-RTS COMPONENTS			
53105	TMI-1 - REACTOR TRIP SYSTEM RELIABILITY - VENDOR RELATED MODIFICATIONS			
53158	TMI-1 - ITEMS 4.2.1 & 4.2.2 - PREVENTATIVE MAINTENANCE PROGRAM FOR REACTOR TRIP BREAKERS - MAINTENANCE & TRENDING			
53200	TMI-1 - ITEM 4.3 - AUTOAMTIC ACTUATION OF SHUNT TRIP ATTACHMENT FOR W AND B & W PLANTS			
11238	THREE MILE ISLAND 1 - FIRE PROTECTION SER SUPPLEMENT		1	05/15/84
42513	THREE MILE ISLAND 1 - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21)		1	05/30/84
47157	TMI-1 NATURAL CIRCULATION COOLDOWN (GENERIC LTR 81-21)		2	05/31/84
49764	TMI-1 - "NUREG 0737 TECH SPECS (GENERIC LETTER 82-16 82-16 OR 83-02)"			06/15/84
43043	THREE MILE ISLAND 1 - TECHNICAL SPECIFICATIONS DEFINING OPERABILITY FOR SAFETY SYSTEMS		2	06/15/84
49698	TMI-1 - REACTOR COOLANT PUMP TRIP			07/01/84
47131	TMI-1 - CONTROL OF HEAVY LOADS AT NUCLEAR POWER PLANTS (NUREG 0612)		2	09/28/84
52279	TMI-1 - CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPA#C-10)			09/28/84
TAC #	TAC DESCRIPTION	TMI ACTIONS	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				
44411	THREE MILE ISLAND 1 - NUREG-0737 II.B.1, RCS HIGH POINT VENTS		1	<u>COMPLETE</u> 10/17/83
45274	THREE MILE ISLAND 1 - NUREG-0737 II.K.3.1, AUTO PORV ISOLATION		1	03/05/84
45320	THREE MILE ISLAND 1 - NUREG-0737 II.K.3.2, REPORT ON PORV FAILURES		1	03/05/84
46274	THREE MILE ISLAND 1- III.A.2.1 EMERGENCY PLAN UPGRADE TO MEET RULE		1	03/09/84
48284	THREE MILE ISLAND 1 - NUREG-0737 II.B.3.2 POST ACCIDENT SAMPLING		1	03/22/84
47641	THREE MILE ISLAND 1 - NUREG 0737 ITEM II F1.4 CONTAINMENT PRESSURE INSTRUMENT		1	04/20/84
47783	THREE MILE IS. 1 - NUREG 0737 ITEM II F1.6 CONTAINMENT HYDROGEN MONITOR		1	04/20/84
<u>ACTIVE ACTIONS</u>				
48283	THREE MILE ISLAND 1 - NUREG-0737 I.C.1.3.A. ATOG		1	<u>TARGET</u>
48286	THREE MILE ISLAND 1 - NUREG-0737 II.K.3.30 SBLOCA METHODS		1	
48287	THREE MILE ISLAND 1 - NUREG-0737 III.A.2.2 METEOROLOGICAL DATA UPGRADE		1	
45987	THREE MILE ISLAND 1 - NUREG-0737 III.A.1.2, TECHNICAL SUPPORT CENTER		1	
46058	THREE MILE ISLAND 1 - NUREG-0737 III.A.1.2, OPERATIONAL SUPPORT CENTER		1	

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CONDENSED MANAGEMENT REPORT

(CONTINUATION)

THREE MILE ISLAND 1

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>TMI ACTIONS</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>ACTIVE ACTIONS</u> (CONTINUATION)				
46130	THREE MILE ISLAND 1 - NUREG-0737 III.A.1.2, EMERGENCY OPERATIONS FACILITY		1	<u>TARGET</u>
47049	THREE MILE ISLAND 1 - NUREG-0737 III.D.3.4 CONTROL ROOM HABITABILITY		1	
51211	TMI-1 - I.D.1 CONTROL ROOM DESIGN REVIEW			
51291	TMI-1 - I.D.2. SAFETY PARAMETER DISPLAY SYSTEM			
47712	THREE MILE IS. 1 - NUREG 0737 ITEM II F1.5 CONTAINMENT WATER LEVEL MONITOR		1	02/28/84
44624	THREE MILE ISLAND 1 - RV AND SV TESTING		1	05/30/84
54979	TMI-1, II.K.2.13 FOLLOW ON, THERMAL MECH. RPT (GENERIC REVIEW)			06/30/84
48208	THREE MILE IS. 1 -NUREG 0737 II.K.3.31 COMPLIANCE WITH 10 CFR 50.46		1	08/30/84
47048	THREE MILE ISLAND 1 - NUREG-0737 II.F.2 INST. TO DETECT ICC		1	09/28/84

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
52008	THREE MILE ISLAND 1 - EFW FLOW INDICATION REVIEW (CERTIFICATION ITEM)			<u>COMPLETE</u>
47894	THREE MILE ISLAND 1 - TSCR 109-HYDROGEN RECOMBINER		3	10/01/83
51968	TMI 1 - EFFLUENT DISCHARGE MONITOR RELOCATION			10/05/83
49620	THREE MILE ISLAND 1 - TSCR NO. 120- RAISING HPI & LPI BYPASS SETPOINTS			10/14/83
51865	TMI-1 CYCLE 5 RELOAD RE-REVIEW			10/19/83
51855	TMI - 1 GPU/B & W LAWSUIT REVIEW			10/28/83
52451	TMI-1 TSCR-97			10/29/83
51740	TMI 1: STATION DIST VOLTAGE VERIF. TEST			11/07/83
52667	TMI-1 - POST ACCIDENT SHIELDING ALTERNATE			01/19/84
53516	THREE MILE ISLAND 1 - NUREG 0737 IMPLEMENTATION REVIEW			01/27/84
47047	THREE MILE ISLAND 1 - APPENDIX I REVIEW CORRECTIONS			02/17/84
53250	THREE MILE ISLAND 1 - ALTERNATE SOLUTION PLANT SHIELDING			03/06/84
53363	THREE MILES ISLAND 1 - FIRE-PROOF CABLE TEST PROGRAM REVIEW			03/14/84
53562	THREE MILE ISLAND 1 - RESPONSE TO COMMISSION ORDER ON DESIGN ISSUES			03/15/84
51527	TMI - ACCEPTANCE REVIEW OF INDEPENDENT AUDITOR FOR TRAINING PROGRAMS			03/19/84
54502	THREE MILE ISLAND 1-COMMISSION REVIEW OF DESIGN ISSUE (REPLY FILING)			04/09/84
53249	THREE MILE ISLAND 1 - EFW FLOW INDICATOR			04/13/84
54599	THREE MILE ISLAND 1 - NUREG-0737, ITEM II.E.1.2 REVIEW			04/25/84
46925	TMI-1 - CONTROL ROOM HUMAN FACTORS REVIEW			04/25/84
48838	THREE MILE ISLAND UNIT 1 - TSCR 116 - RX BLDG PURGE		2	04/26/84
53362	THREE MILE ISLAND 1 - EFW FLOW TEST REVIEW			04/30/84

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>TMI ACTIONS</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>ACTIVE ACTIONS</u>				
53295	THREE MILE ISLAND 1 - STEAM GENERATOR HEARINGS			<u>TARGET</u>
54243	THREE MILE ISLAND 1 - MANAGEMENT INTEGRITY (CERT ITEM 64)			
54600	THREE MILE ISLAND 1 - EMERGENCY FEEDWATER AUTO-INITIATION (II.E.1.2 PART 1)			
54504	THREE MILE ISLAND 1 - BACK-UP INCORE THERMOCOUPLE OPERABILITY (TSCR 130)			
54245	THREE MILE ISLAND 1 - RCS VENTS (TSCR-137)			
46948	TMI-1 TSCR 104 SURVEILLANCE CAPSULES		3	
47484	TMI-1 STEAM GENERATOR TUBE LEAKS		2	
49831	THREE MILE ISLAND 1 - TSCR-123 STEAM GENERATOR REPAIR TECHNICAL SPECIFICATIONS			
53302	THREE MILE ISLAND 1 - PRE-OPERATIONAL TESTING REVIEW			
51412	THREE MILE ISLAND 1 - TSCR 110 AUX BLDG/FHB EXHAUST FLOW			
52387	THREE MILE ISL-1 LONG TERM EFW MODS (NUREG 0737 II.E.1.1)			
52588	THREE MILE ISLAND 1 - TSCR 38 - FUEL CASK DROP			
51057	THREE MILE ISLAND 1- MAIN CONDENSER OFF-GAS IODINE SAMPLER			05/15/84
49619	THREE MILE ISLAND 1 - TSCR NO. 114 - POST-ACCIDENT MONITORING INSTRUMENTATION			05/15/84

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(CONTINUATION)

THREE MILE ISLAND 1

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>ACTIVE ACTIONS</u>				<u>TARGET</u>
48253	THREE MILE ISLAND 1 - APPENDIX R EXEMPTION REVIEW		1	05/15/84
52489	THREE MILE ISLAND 1 - FIRE BRIGADE TRAINING			05/15/84
52003	THREE MILE ISLAND 1 - FIRE PROTECTION SCHEDULAR EXTENSION (9/30/83)			05/15/84
53385	THREE MILE ISLAND 1 - TSCR 97. REV 1 FIRE PROTECTION CHANGES			05/30/84
54244	THREE MILE ISLAND 1 - HEARING-RELATED LICENSE CONDITIONS			05/30/84
49271	THREE MILE ISLAND 1 - ADDITIONAL WATER STORAGE CAPACITY			05/30/84
52006	THREE MILE ISLAND 1 - SUBCOOLING MARGIN INSTRUMENT STRING ERROR (CERTIFICATION ITEM)			05/30/84
52007	THREE MILE ISLAND 1 - BACKUP INCORE THERMOCOUPLE REVIEW (CERTIFICATION ITEM)			05/30/84
53511	THREE MILE ISLAND 1 - UCS 2.206 PETITION			05/30/84
52668	TMI-1 - MANAGER OF TRAINING, TMI (TSCR134)			05/31/84
52589	THREE MILE ISLAND 1 - TSCR 126 - SURVEILLANCE STANDARDS			05/31/84
46929	TMI-1 APPENDIX J REVIEW			05/31/84
49955	THREE MILE ISLAND 1 - TSCR 124 SNUBBER			05/31/84
54246	THREE MILE ISLAND 1 - BYPRODUCT, SOURCE OR SPECIAL NUCLEAR MATERIAL (TSCR 133)			05/31/84
53386	THREE MILE ISLAND 1 - FIRE AREA VS ZONE EXEMPTION REQUEST			05/31/84
43815	TMI-1 TSCR 99 ADMINISTRATIVE CHANGES		3	05/31/84
47050	THREE MILE ISLAND 1 - 1ST REVIEW		1	06/15/84
53399	THREE MILE ISLAND 1 - TSCR 116, REV 1, CONTAINMENT PURGING			06/15/84
54420	TMI-1- RCS VENTS TECHNICAL SPECIFICATIONS NUREG 073 ITEM II.B.1			09/30/84
<u>ANTICIPATED ACTIONS</u>				<u>INITIATION</u>
53959	TMI-1 - ITEM 4.4 - IMPROVEMENTS IN MAINTENANCE AND TEST PROCEDURES FOR B & W PLANTS			11/01/84
54034	TMI-1 - ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS			11/01/84
54117	TMI-1 - ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL CAPABILITY			11/01/84
53641	TMI-1 - ITEM 1.2 - POST TRIP REVIEW-DATA AND INFORMATION CAPABILITY			11/01/84
53724	TMI-1 - ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS			11/01/84
53807	TMI-1- ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS			11/01/84
53890	TMI-1 - ITEM 3.2.3 - POST-MAINTENANCE TESTING -CHANGES TO TECH. SPEC. - ALL SR COMPONENTS			11/01/84
53945	-ITEMS 4.2.3 & 4.2.4-PREVENTATIVE MAINT. PROGRAM FOR REACTOR TRIP BREAKERS-LIFE TESTING & REPLACEMENT			11/01/84

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CONDENSED MANAGEMENT REPORT

FACILITY: TROJAN

PLANT LOCATION: 42 MI N OF PORTLAND, ORE
 DOCKET NUMBER: 050-00344
 ARCH/ENGINEER: BECH
 IE INSPECTOR: G. JOHNSTON

LICENSED POWER: 3411 MWT
 DESIGN POWER: 1130 MWE
 NSSS VENDOR: WEST

PROJECT MANAGER: C. TRAMMELL
 BRANCH CHIEF: J. MILLER
 LIC. ASSISTANT: P. KREUTZER

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>MULTI-PLANT</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
12279	TROJAN - ENV. QUAL. OF CONTROL SYSTEMS TROJAN TROJAN		3	<u>COMPLETE</u> 01/31/84
<u>ACTIVE ACTIONS</u>				
53106	TROJAN - REACTOR TRIP SYSTEM RELIABILITY - VENDOR RELATED MODIFICATIONS			<u>TARGET</u>
53159	TROJAN - ITEMS 4.2.1 & 4.2.2 - PREVENTATIVE MAINTENANCE PROGRAM FOR REACTOR TRIP BREAKERS - MAINTENANCE & TRENDING			
53201	TROJAN - ITEM 4.3 - AUTOAMTIC ACTUATION OF SHUNT TRIP ATTACHMENT FOR W AND B & W PLANTS			
42502	TROJAN - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21)		1	
51362	TROJAN - INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)			
52800	TROJAN - ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION AND PROCEDURES			
52890	TROJAN - ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RTS COMPONENTS			
52971	TROJAN - ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING PROCEDURES/VENDOR RECOMMENDATIONS - RTS COMPONENTS			
53052	TROJAN-ITEM 3.1.3 -POST-MAINTENANCE TESTING-CHANGES TO TECH SPECS-RTS COMPONENTS			
12369	TROJAN - SEISMIC RESTRAINTS/WALL PROBLEM		1	06/30/83
06596	TROJAN - ANALYSIS OF ASYMMETRIC LOCA LOADS AND EFFECT ON REACTOR VESSEL SUPPORTS		1	09/30/83
07721	TROJAN - APPENDIX I TECH SPECS IMPLEMENTATION REVIEW REVIEW		1	09/30/83
10141	TROJAN - TECH SPEC SURVEILLANCE PER FOR MECHANICAL SNUBBERS		1	09/30/83
11298	TROJAN - HYDRAULIC SNUBBERS UPGRADE TECH SPECS		3	09/30/83
49765	TROJAN - "NUREG 0737 TECH SPECS (GENERIC LETTER 82-16 82-16 OR 83-02)"			09/30/83
42594	TROJAN - LONG TERM REVIEW CONTAINMENT PURGE AND VENT (B-24)		1	09/30/83
49697	TROJAN - REACTOR COOLANT PUMP TRIP			07/01/84
47448	TROJAN - 3 LOOP OPERATION (N-1)		3	01/01/85
52280	TROJAN - CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPA#C-10)			09/30/85
<u>TAC # TAC DESCRIPTION TMI ACTIONS PRIORITY CRITICAL DATE</u>				
<u>COMPLETED ACTIONS</u>				
47823	TROJAN - NUREG 0737 ITEM II K2.17 POTENTIAL FOR VOIDING IN RCS		1	<u>COMPLETE</u> 01/03/84
44482	TROJAN - NUREG-0737 II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS		1	01/26/84
<u>ACTIVE ACTIONS</u>				
43091	TROJAN - NUREG-0737, III.A.1.2, TECHNICAL SUPPORT CENTER		1	<u>TARGET</u>
46059	TROJAN - NUREG-0737 III.A.1.2, OPERATIONAL SUPPORT CENTER		1	
46131	TROJAN - NUREG-0737 III.A.1.2, EMERGENCY OPERATIONS FACILITY		1	
46346	TROJAN - NUREG-0737 III.A.2.2, METEOROLOGICAL DATA UPGRADE		1	
51212	TROJAN - I.D.1 CONTROL ROOM DESIGN REVIEW			
51292	TROJAN - I.D.2. SAFETY PARAMETER DISPLAY SYSTEM			
45175	TROJAN - NUREG-0737 II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION		1	06/30/83
45868	TROJAN - NUREG-0737 II.K.3.30, SB LOCA OUTLINE		1	06/30/83
48209	TROJAN -NUREG 0737 II.K.3.31 COMPLIANCE WITH 10 CFR 50.46			12/01/83
46909	TROJAN - II K2.13 THERMAL MECHANICAL REPORT		1	09/30/84
44625	TROJAN - RV AND SV TESTING		1	10/31/84
44274	TROJAN - NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES		1	11/30/84
44344	TROJAN - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES		1	11/30/84

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TROJAN

TAC #	TAC DESCRIPTION	PLANT SPECIFIC	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				<u>COMPLETE</u>
51820	TROJAN - REVISED TESTING OF MSIVS AND PZR SAFETY VALVES			11/01/83
52076	TROJAN - DELETE BORIC ACID TRANSFER PUMPS AND PZR HEATERS FROM APP.R SCOPE (SAFE SHUTDOWN)			11/01/83
42996	TROJAN - SURVEILLANCE SPECIMEN-CAPSULE U ANALYSIS (WCAP - 9469)		3	11/14/83
49630	TROJAN - REVISE PARA. 2.A OF NPF-1 TO REFERENCE UPDATED FSAR			11/22/83
51002	TROJAN - ISI RELIEF REQUEST DATED 3/3/83			12/19/83
52590	TROJAN - ON-SITE & OFF-SITE ORGANIZATIONAL CHANGES LCA 96 DTD 10-4-83			12/22/83
53344	TROJAN - SCHEDULE CHANGE FOR NUREG-0737 ITEMS II.F.1.1 AND II.F.1.2			02/21/84
52077	TROJAN - APP.R EXEMPTION - RCP LUBE OIL COLLECTION SYSTEM			03/05/84
48653	TROJAN - IST RELIEF REQUEST DTD 7-23-82			03/14/84
52591	TROJAN - REVISED FLOW TEST FOR SPRAY ADDITIVE TANK LCA 95. PGE 10-14-83			03/20/84
<u>ACTIVE ACTIONS</u>				<u>TARGET</u>
49636	TROJAN - MOUNT ST. HELENS - SPIRIT LAKE DAM			
55003	TROJAN - MISC. FIRE PROTECTION TECH. SPEC. CHANGES LCA 103. PGE 4-27-84			
54993	TROJAN - DELETE A DG TEST PER GL 83-30 LCA 105. PGE 4-26-84			
53396	TROJAN - COMBINE STA AND SENIOR OPERATOR			
53563	TROJAN - DELETE BORON INJECTION TANK			
54218	TROJAN - LCA 99. REVISED CAPSULE SCHEDULE; CONTAINMENT ISOL. VALVES; CONTAINMENT PEAK PRESS.			
54454	TROJAN - HIGHER FUEL BURNUP			
54750	TROJAN - CYCLE 7 WITH SS RODS. PGE 4-6-84			
54972	TROJAN - LCA 102 TESTING SEALED SOURCES. PGE 4-27-84			
51953	TROJAN - REVIEW CLOW TRICENTRIC 8" BUTTERFLY VALVES FOR ACCEPT AS CONTAINMENT ISOL. VALVES (PGE LTR 1-25-83)			11/30/83
52088	TROJAN - SPENT FUEL POOL RE-RACK (RE-RACK II)			01/30/84
53357	TROJAN - EQ EXTENSIONS PGE LETTER DATED 12-16-83			02/15/84
54992	TROJAN - LCA 101. SHUTDOWN MARGIN VERIFICATION WITH INOPERABLE RODS			07/01/84
54575	TROJAN - TECHNICAL SPECIFICATIONS COVERED BY GENERIC LETTERS 83-36, 83-37 FOR NUREG-0737			09/30/84
54421	TROJAN- RCS VENTS TECHNICAL SPECIFICATIONS NUREG 073 ITEM II.B.1			09/30/84
52528	TROJAN - MOUNT ST. HELENS SURVEILLANCE FY84			09/30/84
<u>ANTICIPATED ACTIONS</u>				<u>INITIATION</u>
53642	TROJAN 1 - ITEM 1.2 - POST TRIP REVIEW-DATA AND INFORMATION CAPABILITY			11/01/84
53725	TROJAN - ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR VENDOR INTERFACE - ALL SR COMPONENTS			11/01/84
53808	TROJAN- ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS			11/01/84
53891	TROJAN - ITEM 3.2.3 - POST-MAINTENANCE TESTING -CHANGES TO TECH. SPEC. - ALL SR COMPONENTS			11/01/84
53946	-ITEMS 4.2.3 & 4.2.4-PREVENTATIVE MAINT. PROGRAM FOR REACTOR TRIP BREAKERS-LIFE TESTING & REPLACEMENT			11/01/84
54035	TROJAN - ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS			11/01/84
54118	TROJAN 1 - ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL TESTING - DIVERSE TRIP FEATURES			11/01/84

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CONDENSED MANAGEMENT REPORT

FACILITY: TURKEY POINT 3

PLANT LOCATION: 25 MI S OF MIAMI, FLA
 DOCKET NUMBER: 050-00250
 ARCH/ENGINEER: BECH
 IE INSPECTOR: R. VOGTIGWELL

LICENSED POWER: 2200 MWT
 DESIGN POWER: 0693 MWE
 NSSS VENDOR: WEST

PROJECT MANAGER: D. MCDONALD
 BRANCH CHIEF: S. VARGA
 LIC. ASSISTANT: C. PARRISH

TAC #	TAC DESCRIPTION	MULTI-PLANT	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				<u>COMPLETE</u>
08784	TURKEY POINT 3 - TECH SPEC SURVEILLANCE REQUIREMENTS FOR MECHANICAL SNUBBERS		2	10/14/83
08088	TURKEY PT 3 - CONTROL OF HEAVY LOADS OVER SPENT FUEL POOL (NUREG 0612)		2	11/01/83
49611	TURKEY POINT 3 - PLANS FOR PREVENTING EXCEEDING PTS SCREENING CRITERION			11/14/83
12280	TURKEY POINT 3 - HELB AND CONSEQUENTIAL SYSTEM FAILURE TURKEY POINT 3		3	02/02/84
08782	TURKEY POINT 3 - REVIEW OF ASYMMETRIC LOCA LOADS SUBMITTAL		2	02/06/84
48552	TURKEY POINT 3 FIRE PROTECTION (APPEND R) EXEMPTION REQUEST			03/27/84
49415	TURKEY POINT 3 - BLOCKING SI SIGNAL DURING COOLDOWN			04/13/84
43849	TURKEY POINT 3 - SAFE SHUTDOWN APPENDIX R TO SDV-1V COUPLING		1	04/16/84
08127	TURKEY POINT 3 - APPENDIX I TECH SPEC IMPLEMENTATION REVIEW		3	04/23/84

ACTIVE ACTIONS

TAC #	TAC DESCRIPTION	MULTI-PLANT	PRIORITY	CRITICAL DATE
<u>ACTIVE ACTIONS</u>				<u>TARGET</u>
52281	TURKEY POINT 3 - CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPA#C-10)			
52891	TURKEY POINT 3 - ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RTS COMPONENTS			
52972	TURKEY POINT 3 - ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING PROCEDURES/VENDOR RECOMMENDATIONS - RTS COMPONENTS			
53053	TURKEY PT 3-ITEM 3.1.3 -POST-MAINTENANCE TESTING-CHANGES TO TECH SPECS-RTS COMPONENTS			
53107	TURKEY POINT 3 - REACTOR TRIP SYSTEM RELIABILITY - VENDOR RELATED MODIFICATIONS			
53160	TURKEY POINT 3 - ITEMS 4.2.1 & 4.2.2 - PREVENTATIVE MAINT PROGRAM FOR REACTOR TRIP BREAKERS - MAINTENANCE & TRENDING			
53202	TURKEY POINT 3 - ITEM 4.3 - AUTOAMTIC ACTUATION OF SHUNT TRIP ATTACHMENT FOR W AND B & W PLANTS			12/16/83
49766	TURKEY POINT 3 - "NUREG 0737 TECH SPECS (GENERIC LETTER 82-16 OR 83-02)"			
42869	TURKEY POINT 3 - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11		2	02/17/84
10057	TURKEY POINT 3 - POTENTIAL EQUIPMENT FAILURES ASSOCIATED WITH GRID VOLTAGE		2	05/25/84
42470	TURKEY POINT 3 - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21)		1	06/01/84
51363	TURKEY POINT 3 - INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)			06/01/84
49678	TURKEY POINT 3 - REACTOR COOLANT PUMP TRIP			07/01/84

TAC #	TAC DESCRIPTION	TMI ACTIONS	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				<u>COMPLETE</u>
44483	TURKEY POINT 3 - NUREG-0737 II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS		1	11/08/83
46487	TURKEY POINT 3 - NUREG-0737 III.D.3.4, CONTROL ROOM HABITABILITY		1	11/25/83
47824	TURKEY POINT 3 - NUREG 0737 ITEM II K2.17 POTENTIAL FOR VOIDING IN RCS		1	01/06/84

ACTIVE ACTIONS

TAC #	TAC DESCRIPTION	TMI ACTIONS	PRIORITY	CRITICAL DATE
<u>ACTIVE ACTIONS</u>				<u>TARGET</u>
48210	TURKEY POINT 3 -NUREG 0737 II.K.3.31 COMPLIANCE WITH 10 CFR 50.46		1	03/16/84
45176	TURKEY POINT 3 - NUREG-0737 II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION		1	03/16/84
45869	TURKEY POINT 3 - NUREG-0737 II.K.3.30, SB LOCA OUTLINE		1	03/16/84
44626	TURKEY POINT 3 - RV AND SV TESTING		1	06/01/84
51213	TURKEY POINT 3 - I.D.1 CONTROL ROOM DESIGN REVIEW			06/01/84
51293	TURKEY POINT 3 - I.D.2. SAFETY PARAMETER DISPLAY SYSTEM			06/01/84
45988	TURKEY POINT 3 - NUREG-0737 III.A.1.2, TECHNICAL SUPPORT CENTER		1	09/30/84
46060	TURKEY POINT 3 - NUREG-0737 III.A.1.2, OPERATIONAL SUPPORT CENTER		1	09/30/84
46132	TURKEY POINT 3 - NUREG-0737 III.A.1.2, EMERGENCY OPERATIONS FACILITY		1	09/30/84
46347	TURKEY POINT 3 - NUREG-0737 III.A.2.2, METEOROLOGICAL DATA UPGRADE		1	09/30/84

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CONDENSED MANAGEMENT REPORT

(CONTINUATION)

TURKEY POINT 3

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>TMI ACTIONS</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>ACTIVE ACTIONS</u> (CONTINUATION)				
44275	TURKEY POINT 3 - NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES		1	TARGET 11/30/84
44345	TURKEY POINT 3 - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES		1	11/30/84
<u>PLANT SPECIFIC</u>				
<u>TAC #</u>	<u>TAC DESCRIPTION</u>		<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
52493	TURKEY POINT UNIT 3 - TECH SPEC AMEND - RESIDUAL HEAT REMOVAL PUMPS			COMPLETE 10/14/83
51768	TURKEY POINT 3 - TECH SPEC AMEND REQUIRING MONTHLY SAFETY SYSTEM WALKDOWNS			10/26/83
52040	TURKEY POINT 3 - SPENT FUEL STORAGE FACILITY EXPANSION			11/16/83
51793	TURKEY POINT 3 - FUEL DESIGN TECH SPEC AMENDMENT FOR OFA AND WABA			12/09/83
52142	TURKEY POINT 3 - REQUEST TO INCREASE THE FH TECH SPEC LIMIT			12/23/83
51766	TURKEY POINT 3 - APPENDIX B TECH SPEC DELETION REQUEST MONITORING PROGRAM			01/04/84
08810	TURKEY POINT 3 - SINGLE FAILURE		1	02/22/84
52500	TURKEY POINT UNIT 3 - ISI RELIEF REQUEST			03/01/84
54280	TURKEY POINT 3 - SCHEDULAR EXEMPTION APP R			03/24/84
54292	TURKEY POINT 3 - ISI EXEMPTION REQUEST			04/25/84
54241	TURKEY POINT 3 - REACTOR VESSEL MATERIAL DATA			04/26/84
<u>ACTIVE ACTIONS</u>				
52810	TURKEY POINT 3 - ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION AND PROCEDURES			TARGET
54480	TURKEY POINT UNIT 3 SPENT FUEL POOL EXPANSION			
54826	TURKEY POINT 3 - ELECT DISTRIBUTION DESIGN REVIEW			
49133	TURKEY POINT 3 - REVISED 1ST PROGRAM			12/30/83
49917	TURKEY POINT 3 - CONTAINMENT VENT/PURGE VALVE OPERABILITY			12/30/83
53452	TURKEY POINT 3 DELETION OF APP B ENVIRONMENTAL PROTECTION LIMITS			02/17/84
52384	TURKEY POINT 3 - DELETION REQUEST FOR APPENDIX B GROUND WATER MONITORING REQUIREMENTS			03/16/84
54428	TURKEY POINT UNIT 3 - NEUTRON SOURCE DATA CY9			05/18/84
54956	TURKEY POINT UNIT 3 - PROPOSED TECHNICAL SPECIFICATION FOR CONTROL OF HEAVY LOADS			06/08/84
54851	TURKEY POINT 3 - PROPOSED TECH. SPEC. - ELECTRICAL			06/29/84
53409	TURKEY POINT 3 ISI RELIEF REQUEST			06/29/84
54675	TURKEY POINT UNIT 3 - FUEL STORAGE LINEAR LOADING INCREASE			07/01/84
54677	TURKEY POINT UNIT 3 - ISI/IST PROPOSED TECH. SPEC.			08/15/84
54973	TURKEY POINT UNIT 3 - INSERVICE INSPECTION (ISI) SECOND 10-YEAR SUMMARY REPORT			09/14/84
54576	TURKEY POINT 3 - TECHNICAL SPECIFICATIONS COVERED BY GENERIC LETTERS 83-36, 83-37 FOR NUREG-0737			09/30/84
54422	TURKEY POINT 3- RCS VENTS TECHNICAL SPECIFICATIONS NUREG 073 ITEM II.B.1			09/30/84
<u>ANTICIPATED ACTIONS</u>				
53643	TURKEY POINT 3 - ITEM 1.2 - POST TRIP REVIEW-DATA AND INFORMATION CAPABILITY			INITIATION 11/01/84
53726	TURKEY POINT 3 - ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS			11/01/84
53809	TURKEY PT 3- ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS			11/01/84
53892	TURKEY POINT 3 - ITEM 3.2.3 - POST-MAINTENANCE TESTING -CHANGES TO TECH. SPEC. - ALL SR COMPONENTS			11/01/84
53947	-ITEMS 4.2.3 & 4.2.4- PREVENTATIVE MAINT. PROGRAM FOR REACTOR TRIP BREAKERS-LIFE TESTING & REPLACEMENT			11/01/84
54036	TURKEY POINT 3 - ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS			11/01/84
54119	TURKEY POINT 3 - ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL TESTING - DIVERSE TRIP FEATURES			11/01/84

DATA AS OF - 04/30/84

CONDENSED MANAGEMENT REPORT

FACILITY: TURKEY POINT 4

PLANT LOCATION: 25 MI S OF MIAMI, FLA
 DOCKET NUMBER: 050-00251
 ARCH/ENGINEER: BECH
 IE INSPECTOR: R. VOGTLOWELL

LICENSED POWER: 2200 MWT
 DESIGN POWER: 0693 MWE
 NSSS VENDOR: WEST

PROJECT MANAGER: D. MCDONALD
 BRANCH CHIEF: S. VARGA
 LIC. ASSISTANT: C. PARRISH

TAC #	TAC DESCRIPTION	MULTI-PLANT	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				
08785	TURKEY POINT 4 - TECH SPEC SURVEILLANCE REQUIREMENTS FOR MECHANICAL SNUBBERS		2	<u>COMPLETE</u> 10/14/83
08089	TURKEY PT 4 - CONTROL OF HEAVY LOADS OVER SPENT FUEL POOL (NUREG 0612)		2	11/01/83
49612	TURKEY POINT 4 - PLANS FOR PREVENTING EXCEEDING PTS SCREENING CRITERION			11/15/83
12281	TURKEY POINT 4 - HELB AND CONSEQUENTIAL SYSTEM FAILURE TURKEY POINT 4		3	02/02/84
08783	TURKEY POINT 4 - REVIEW OF ASYMMETRIC LOCA LOADS SUBMITTAL		2	02/06/84
48553	TURKEY POINT 4 FIRE PROTECTION (APPEND R) EXEMPTION REQUEST			03/27/84
49416	TURKEY POINT 4 - BLOCKING SI SIGNAL DURING COOLDOWN			04/13/84
43850	TURKEY POINT 4 SAFETY SHUTDOWN		1	04/16/84
08128	TURKEY POINT 4 - APPENDIX I TECH SPEC IMPLEMENTATION REVIEW		3	04/23/84
<u>ACTIVE ACTIONS</u>				
52282	TURKEY POINT 4 - CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPA#C-10)			<u>TARGET</u>
52811	TURKEY POINT 4 - ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION AND PROCEDURES			
52892	TURKEY POINT 4 - ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RTS COMPONENTS			
52973	TURKEY POINT 4 - ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING PROCEDURES/VENDOR RECOMMENDATIONS - RTS COMPONENTS			
53108	TURKEY POINT 4 - REACTOR TRIP SYSTEM RELIABILITY - VENDOR RELATED MODIFICATIONS			
53161	TURKEY POINT 4 - ITEMS 4.2.1 & 4.2.2 - PREVENTATIVE MAINT PROGRAM FOR REACTOR TRIP BREAKERS - MAINTENANCE & TRENDING			
53203	TURKEY POINT 4 - ITEM 4.3 - AUTOAMTIC ACTUATION OF SHUNT TRIP ATTACHMENT FOR W AND B & W PLANTS			
42471	TURKEY POINT 4 - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21)		1	09/01/83
49767	TURKEY POINT 4 - "NUREG 0737 TECH SPECS (GENERIC LETTER 82-16 OR 83-02)"			12/16/83
42870	TURKEY POINT 4 - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11		2	02/17/84
10058	TURKEY POINT 4 - POTENTIAL EQUIPMENT FAILURES ASSOCIATED WITH GRID VOLTAGE		2	05/25/84
51364	TURKEY POINT 4 - INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)			06/01/84
49679	TURKEY POINT 4 - REACTOR COOLANT PUMP TRIP			07/01/84
TAC #	TAC DESCRIPTION	TMI ACTIONS	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				
44484	TURKEY POINT 4 - NUREG-0737 II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS		1	<u>COMPLETE</u> 11/08/83
46488	TURKEY POINT 4 - NUREG-0737 III.D.3.4, CONTROL ROOM HABITABILITY		1	11/25/83
47825	TURKEY POINT 4 - NUREG 0737 ITEM II K2.17 POTENTIAL FOR VOIDING IN RCS		1	01/06/84
<u>ACTIVE ACTIONS</u>				
44627	TURKEY POINT 4 - RV AND SV TESTING		1	<u>TARGET</u> 10/31/83
45177	TURKEY POINT 4 - NUREG-0737 II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION		1	03/16/84
45870	TURKEY POINT 4 - NUREG-0737 II.K.3.30, SB LOCA OUTLINE		1	03/16/84
48211	TURKEY POINT 4 -NUREG 0737 II.K.3.31 COMPLIANCE WITH 10 CFR 50.46		1	03/16/84
51214	TURKEY POINT 4 - I.D.1 CONTROL ROOM DESIGN REVIEW			06/01/84
51294	TURKEY POINT 4 - I.D.2. SAFETY PARAMETER DISPLAY SYSTEM			06/01/84
45989	TURKEY POINT 4 - NUREG-0737 III.A.1.2, TECHNICAL SUPPORT CENTER		1	09/30/84
46061	TURKEY POINT 4 - NUREG-0737 III.A.1.2, OPERATIONAL SUPPORT CENTER		1	09/30/84
46133	TURKEY POINT 4 - NUREG-0737 III.A.1.2, EMERGENCY OPERATIONS FACILITY		1	09/30/84
46348	TURKEY POINT 4 - NUREG-0737 III.A.2.2, METEOROLOGICAL DATA UPGRADE		1	09/30/84

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CONDENSED MANAGEMENT REPORT

(CONTINUATION)

TURKEY POINT 4

TAC #	TAC DESCRIPTION	TMI ACTIONS	PRIORITY	CRITICAL DATE
<u>ACTIVE ACTIONS (CONTINUATION)</u>				
46911	TURKEY PT 4 - II K2.13 THERMAL MECHANICAL REPORT		1	TARGET 09/30/84
44276	TURKEY POINT 4 - NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES		1	11/30/84
44346	TURKEY POINT 4 - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES		1	11/30/84
<u>PLANT SPECIFIC</u>				
<u>COMPLETED ACTIONS</u>				
52494	TURKEY POINT UNIT 4 - TECH SPEC AMEND - RESIDUAL HEAT REMOVAL PUMPS			COMPLETED 10/14/83
51769	TURKEY POINT 4 TECH SPEC AMEND REQUIRING MONTHLY SAFETY SYSTEM WALKDOWNS			10/26/83
52041	TURKEY POINT 4 SPENT FUEL STORAGE FACILITY EXPANSION			11/16/83
51794	TURKEY POINT 4 - FUEL DESIGN TECH SPEC AMENDMENT FOR OFA & WABA			12/09/83
52143	TURKEY POINT 4 - REQUEST TO INCREASE FH TECH SPEC LIMITS			12/23/83
51767	TURKEY POINT 4 - APPENDIX B TECH SPEC DELETIONS REQUEST - MONITORING PROGRAM			01/04/84
08812	TURKEY POINT 4 - SINGLE FAILURE IN BACKUP LINE (ECCS)		1	02/22/84
52501	TURKEY POINT UNIT 4 - ISI RELIEF REQUEST			03/01/84
54281	TURKEY POINT 4 - SCHEDULAR EXEMPTION APP R			03/21/84
54293	TURKEY POINT 4 - ISI EXEMPTION REQUEST			04/25/84
54242	TURKEY POINT 4 - REACTOR VESSEL MATERIAL DATA			04/26/84
<u>ACTIVE ACTIONS</u>				
54481	TURKEY POINT UNIT 4 SPENT FUEL POOL EXPANSION			TARGET
54827	TURKEY POINT 4 - ELECT DISTRIBUTION DESIGN REVIEW			
53054	TURKEY PT 4-ITEM 3.1.3 -POST-MAINTENANCE TESTING-CHANGES TO TECH SPECS-RTS COMPONENTS			
49919	TURKEY POINT 4 - CONTAINMENT VENT/PURGE VALVE OPERABILITY			12/30/83
49936	TURKEY POINT 4 - REVISED 1ST PROGRAM			12/30/83
53453	TURKEY POINT 4 - DELETION OF APP. B. ENVIRONMENTAL PROTECTION LIMITS			02/17/84
52385	TURKEY POINT 4 - DELETION REQUEST FOR APP. B G AND WATER MONITORING REQUIREMENTS			03/16/84
54957	TURKEY POINT UNIT 4 - PROPOSED TECHNICAL SPECIFICATION FOR CONTROL OF HEAVY LOADS			06/08/84
53410	TURKEY POINT 4 ISI RELIEF REQUEST			06/29/84
54852	TURKEY POINT 4 - PROPOSED TECH. SPED. - ELECTRICAL			06/29/84
54676	TURKEY POINT UNIT 4 - FUEL STORAGE LINEAR LOADING INCREASE			07/01/84
54678	TURKEY POINT UNIT 4 - ISI/IST PROPOSED TECH. SPEC.			08/15/84
54974	TURKEY POINT UNIT 4 - INSERVICE INSPECTION (ISI) SECOND 10-YEAR SUMMARY REPORT			09/14/84
54577	TURKEY POINT 4 - TECHNICAL SPECIFICATIONS COVERED BY GENERIC LETTERS 83-36, 83-37 FOR NUREG-0737			09/30/84
54423	TURKEY POINT 4- RCS VENTS TECHNICAL SPECIFICATIONS NUREG 073 ITEM II.B.1			09/30/84
<u>ANTICIPATED ACTIONS</u>				
53644	TURKEY POINT 4 - ITEM 1.2 - POST TRIP REVIEW-DATA AND INFORMATION CAPABILITY			INITIATION 11/01/84
53727	TURKEY POINT 4 - ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS			11/01/84
53810	TURKEY PT 4- ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS			11/01/84
53893	TURKEY POINT 4 - ITEM 3.2.3 - POST-MAINTENANCE TESTING -CHANGES TO TECH. SPEC. - ALL SR COMPONENTS			11/01/84
53948	-ITEMS 4.2.3 & 4.2.4-PREVENTATIVE MAINT. PROGRAM FOR REACTOR TRIP BREAKERS-LIFE TESTING & REPLACEMENT			11/01/84
54037	TURKEY POINT 4 - ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS			11/01/84
54120	TURKEY POINT 4 - ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL TESTING - DIVERSE TRIP FEATURES			11/01/84

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CONDENSED MANAGEMENT REPORT

FACILITY: VERMONT YANKEE 1

PLANT LOCATION: 5 MI S OF BRATTLEBORO, VT
 DOCKET NUMBER: 050-00271
 ARCH/ENGINEER: EBASCO
 IE INSPECTOR: W. RAYMOND

LICENSED POWER: 1593 MW
 DESIGN POWER: 0514 MWE
 NSSS VENDOR: GE

PROJECT MANAGER: V. ROONEY
 BRANCH CHIEF: D. VASSALLO
 LIC. ASSISTANT: S. SHEPPARD

TAC #	TAC DESCRIPTION	MULTI-PLANT	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				
12312	VERMONT YANKEE - HELB AND CONSEQUENTIAL SYSTEM FAILURE	VERMONT YANKEE	3	<u>COMPLETE</u> 01/31/84
<u>ACTIVE ACTIONS</u>				
51365	VERMONT YANKEE 1 - INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)			<u>TARGET</u>
52283	VERMONT YANKEE 1 - CONTROL OF HEAVY LOADS - PHASE II FOLLOWUP OF MPA#C-10			
52812	VERMONT YANKEE 1 - ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION AND PROCEDURES			
52893	VERMONT YANKEE 1 - ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RTS COMPONENTS			
52974	VERMONT YANKEE 1 - ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING PROCEDURES/VENDOR RECOMMENDATIONS - RTS COMPONENTS			
53055	VERMONT YANKEE 1-ITEM 3.1.3 -POST-MAINT TESTING-CHANGES TO TECH SPECS-RTS COMPONENTS			
42582	VERMONT YANKEE - LONG TERM REVIEW CONTAINMENT PURGE AND VENT (B-24)		1	10/30/83
07950	VERMONT YANKEE - MARK I CONTAINMENT LONG TERM PROGRAM IMPLEMENTATION		1	12/01/83
08129	VERMONT YANKEE - APPENDIX I TECH SPEC IMPLEMENTATION REVIEW		3	12/01/83
42888	VERMONT YANKEE - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11		2	12/30/83
43724	VERMONT YANKEE - SAFETY CONCERNS ASSOCIATED WITH PIPE BREAKS IN THE BWR SCRAM SYSTEM		1	12/30/83
43776	VERMONT YANKEE - FEEDWATER & NOZZLE CRACKING -NUREG 0619 IMPLEMENTATION		1	12/30/83
42480	VERMONT YANKEE - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21)		1	12/30/83
46674	VERMONT YANKEE - IMPLEMENTATION OF NUREG-0313, REV. 1		2	12/30/83
47132	VERMONT YANKEE CONTROL OF HEAVY LOADS AT NUCLEAR POWER PLANTS (NUREG 0612)		2	12/30/83
08939	VERMONT YANKEE - RPS POWER SUPPLY		1	12/31/83
51693	VERMONT YANKEE NUREG 0737 TECH SPECS			03/01/84
10428	VERMONT YANKEE - MECHANICAL SNUBBERS		2	12/30/84
10059	VERMONT YANKEE - POTENTIAL EQUIPMENT FAILURES ASSOCIATED DEGRADED GRID VOLTAGE		2	12/30/84
08820	VERMONT YANKEE - HYDRAULIC SNUBBERS		3	12/30/84
<u>TMI ACTIONS</u>				
<u>COMPLETED ACTIONS</u>				
45871	VERMONT YANKEE - NUREG-0737 II.K.3.30, SB LOCA OUTLINE		1	<u>COMPLETE</u> 01/04/84
48212	VERMONT YANKEE -NUREG 0737 II.K.3.31 COMPLIANCE WITH 10 CFR 50.46		1	01/04/84
46278	VERMONT YANKEE 1 - III.A.2.1 EMERGENCY PLAN UPGRADE TO MEET RULE		1	02/22/84
<u>ACTIVE ACTIONS</u>				
51215	VERMONT YANKEE 1 - I.D.1 CONTROL ROOM DESIGN REVIEW			<u>TARGET</u>
51295	VERMONT YANKEE 1 - I.D.2. SAFETY PARAMETER DISPLAY SYSTEM			
46349	VERMONT YANKEE - NUREG-0737 III.A.2.2, METEOROLOGICAL DATA UPGRADE		1	10/30/83
45178	VERMONT YANKEE - NUREG-0737 II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION		1	10/30/83
45606	VERMONT YANKEE - NUREG-0737 II.K.3.16, CHALLENGES AND FAILURES OF RELIEF VALVES		1	10/30/83
44628	VERMONT YANKEE 1 - II.D.1 RV AND SV TESTING		1	10/31/83
44277	VERMONT YANKEE - NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES		1	11/30/83
44347	VERMONT YANKEE - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES		1	11/30/83
48274	VERMONT YANKEE - NUREG 0737 II.K.3.28, VERIFY QUALIFICATION OF ACCUMULATORS ON ADS VALVES		1	12/30/83
45702	VERMONT YANKEE - NUREG-0737 II.K.3.18, ADS ACTUATION STUDY		1	06/30/84
45990	VERMONT YANKEE - NUREG-0737 III.A.1.2, TECHNICAL SUPPORT CENTER		1	09/30/84
46062	VERMONT YANKEE - NUREG-0737 III.A.1.2, OPERATIONAL SUPPORT CENTER		1	09/30/84

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CONDENSED MANAGEMENT REPORT

(CONTINUATION)

VERMONT YANKEE 1

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>TMI ACTIONS</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>ACTIVE ACTIONS</u> (CONTINUATION)				
46134	VERMONT YANKEE - NUREG-0737 III.A.1.2, EMERGENCY OPERATIONS FACILITY		1	TARGET 09/30/84
44485	VERMONT YANKEE - NUREG-0737 II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS		1	09/30/84

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
52014	VERMONT YANKEE - DELETE TECH SPEC PLAN REQUIREMENT FOR ANNUAL EMERGENCY DRILL			COMPLETE 11/10/83
53340	VERMONT YANKEE EMERGENCY PLAN DRILL			11/10/83
52090	VERMONT YANKEE - WITHDRAWAL OF I.A.1.3 COMMITMENT			12/12/83
48897	VERMONT YANKEE - STA AS FIRE BRIGADE LEADER			01/30/84
54290	VERMONT YANKEE - REACTOR VESSEL NEO PRESSURE TEMPERATURE CURVES			03/13/84
51993	VERMONT YANKEE REACTOR VESSEL PRESSURE TEMPERATURE CURVES			03/13/84
52638	VERMONT YANKEE - 10/25/83 2.206 PETITION			04/16/84

<u>ACTIVE ACTIONS</u>			<u>TARGET</u>
51700	VERMONT YANKEE - DEGRADED GRID VOLTAGE PROCEDURES		
48671	VERMONT YANKEE - SUPPRESSION POOL TEMPERATURE		10/30/83
52026	VERMONT YANKEE - HCU SEISMIC QUALIFICATION		12/30/83
43078	VERMONT YANKEE - CURRENT EVENTS	3	12/30/83
43308	VERMONT YANKEE - RELOAD ANALYSIS METHODS	1	12/30/83
52639	VERMONT YANKEE - FIRE PROTECTION EXEMPTIONS DUE TO AUDIT		12/30/83
52102	VERMONT YANKEE - FOLLOWON REQUIREMENTS RELATED TO RECOMBINERS		12/31/83
52109	VERMONT YANKEE - SNUBBER CHANGES		12/31/83
53436	VERMONT YANKEE - NEW ISI PROGRAM		05/01/84
53437	VERMONT YANKEE - NEW IST PROGRAM		05/01/84
54288	VERMONT YANKEE - SHOCK SUPPRESSORS - CHANGE 117		05/05/84
54289	VERMONT YANKEE - HPCI AUTOMATIC SUCTION TRANSFER		05/05/84
54182	VERMONT YANKEE - MAIN STEAM LINE LO PRESSURE ISOLATION SETPOINT		06/01/84
54287	VERMONT YANKEE - REMOVE AUTOMATIC AIR DUMP		06/05/84
54496	VERMONT YANKEE - FIRE PROTECTION DEFICIENCIES		06/30/84
54747	VERMONT YANKEE - MSIV HIGH FLOW SETPOINT		06/30/84
54486	VERMONT YANKEE - WELD INSPECTION AND REPAIR		08/03/84
54578	VERMONT YANKEE 1 - TECHNICAL SPECIFICATIONS COVERED BY GENERIC LETTERS 83-36, 83-37 FOR NUREG-0737		09/30/84

<u>ANTICIPATED ACTIONS</u>		<u>INITIATION</u>
53645	VERMONT YANKEE 1 - ITEM 1.2 - POST TRIP REVIEW-DATA AND INFORMATION CAPABILITY	11/01/84
53728	VERMONT YANKEE 1 - ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS	11/01/84
53811	VERMONT YANKEE 1- ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS	11/01/84
53894	VERMONT YANKEE 1 - ITEM 3.2.3 - POST-MAINTENANCE TESTING -CHANGES TO TECH. SPEC. - ALL SR COMPONENTS	11/01/84
54038	VERMONT YANKEE 1 - ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS	11/01/84
54121	VERMONT YANKEE 1 - ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL TESTING - DIVERSE TRIP FEATURES	11/01/84

DATA AS OF - 04/30/84

CONDENSED MANAGEMENT REPORT

FACILITY: WASHINGTON NUCLEAR 2

PLANT LOCATION: 30 MI NW OF RICHLAND, WASH
 DOCKET NUMBER: 050-00397
 ARCH/ENGINEER: B&R
 IE INSPECTOR:

LICENSED POWER: 000 MWT
 DESIGN POWER: 1103 MWE
 NSSS VENDOR: GE

PROJECT MANAGER: R. AULUCK
 BRANCH CHIEF: A. SCHWENCER
 LIC. ASSISTANT: E. HYLTON

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>IMI ACTIONS</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
				<u>TARGET</u>
<u>ACTIVE ACTIONS</u>				
54586	WNP-2 - II.D.1 - RV & SV TESTING			

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
				<u>TARGET</u>
<u>ACTIVE ACTIONS</u>				

54186	WNP-2 - TECH SPEC AUDIT			03/31/84
54838	WNP-2, POST ACCIDENT SAMPLING			05/01/84
54442	WNP-2, REPORTING REQUIREMENTS OF 10 CFR 50			06/01/84
54620	WNP-2 - D.C. SOURCES SURVEILLANCE REQUIREMENTS			06/04/84
54443	WNP-2, MRSC. TECH. SPEC. CHANGES			06/08/84
54643	WNP-2 - RADIOACTIVE LIQUID EFFLUENT MONITORING SYSTEM			07/08/84
54644	WNP-2 - ECCS			07/08/84
54645	WNP-2, DRYWELL & SUPPRESSION CHAMBER H2 RECOMBINER SYSTEM			07/08/84
54646	WNP-2, ELECTRICAL EQUIPMENT PROTECTIVE DEVICES			07/08/84
54647	WNP-2, M-O VALVES THERMAL OVERLOAD PROTECTION			07/08/84
54642	WNP-2 - FIRE DETECTION INSTRUMENTATION			07/08/84
54648	WNP-2, ADMINISTRATIVE CONTROLS			07/08/84
54425	WPPS-2- RCS VENTS TECHNICAL SPECIFICATIONS NUREG 073 ITEM II.B.1			09/30/84
54640	WNP-2 - DETAIL CONTROL ROOM DESIGN REVIEW (LC-16)			12/30/85

<u>ANTICIPATED ACTIONS</u>				<u>INITIATION</u>
53647	WPPS-2 - ITEM 1.2 - POST TRIP REVIEW-DATA AND INFORMATION CAPABILITY			11/01/84
53730	WPPS-2 - ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS			11/01/84
53813	WPPS-2- ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS			11/01/84
53896	WPPS-2 - ITEM 3.2.3 - POST-MAINTENANCE TESTING -CHANGES TO TECH. SPEC. - ALL SR COMPONENTS			11/01/84
54040	WPPS-2 - ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS			11/01/84
54123	WPPS-2 - ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL CAPABILITY			11/01/84
54641	WNP-2, INITIAL TEST PROGRAM (LC-3)			12/09/84

DATA AS OF - 04/30/84

CONDENSED MANAGEMENT REPORT

FACILITY: YANKEE-ROWE 1

PLANT LOCATION: 25 MI NE OF PITTSFIELD, MASS
 DOCKET NUMBER: 050-00029
 ARCH/ENGINEER: S&W
 IE INSPECTOR: J. COLLINS

LICENSED POWER: 0600 MW
 DESIGN POWER: 0175 MWE
 NSSS VENDOR: WEST

PROJECT MANAGER: P. ERICKSON
 BRANCH CHIEF: D. CRUTCHFIELD
 LIC. ASSISTANT: H. SMITH

TAC #	TAC DESCRIPTION	MULTI-PLANT	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				
47145	YANKEE ROWE 1 NATURAL CIRCULATION COOLDOWN (GENERIC LTR 81-21)		2	<u>COMPLETE</u> 10/25/83
07259	YANKEE ROWE - INSERVICE INSPECTION PROGRAM TO MEET 10 CFR 50.55A		1	01/03/84
10147	YANKEE ROWE - REVIEW OF ASYMMETRIC LOCA LOADS SUBMITTAL		2	02/01/84
43672	YANKEE ROWE - SEISMIC QUALIFICATION OF THE AUXILIARY FEEDWATER SYSTEM		1	02/23/84
53204	YANKEE ROWE 1 - ITEM 4.3 - AUTOAMTIC ACTUATION OF SHUNT TRIP ATTACHMENT FOR W AND B & W PLANTS			04/13/84
<u>ACTIVE ACTIONS</u>				
49768	YANKEE ROWE 1 - "NUREG 0737 TECH SPECS (GENERIC LETTER 82-16 OR 83-02)"		3	<u>TARGET</u> 06/30/84
49655	YANKEE ROWE 1 - REACTOR COOLANT PUMP TRIP		3	07/01/84
43626	YANKEE ROWE - APPENDIX R-ALTERNATE SAFE SHUTDOWN REVIEW		1	07/01/84
42526	YANKEE ROWE - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21)		1	07/31/84
42917	YANKEE ROWE - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11		2	07/31/84
08091	YANKEE ROWE 1 - CONTROL OF HEAVY LOADS OVER SPENT FUEL POOL (NUREG 0612)		2	08/30/84
10060	YANKEE ROWE - POTENTIAL EQUIPMENT FAILURES ASSOCIATED WITH GRID VOLTAGE		2	09/30/84
52813	YANKEE ROWE 1 - ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION AND PROCEDURES			02/01/85
52894	YANKEE ROWE 1 - ITEM 3.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RTS COMPONENTS			02/01/85
52975	YANKEE ROWE 1 - ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING PROCEDURES/VENDOR RECOMMENDATIONS - RTS COMPONENTS			02/01/85
53056	YANKEE ROWE 1-ITEM 3.1.3-POST-MAINTENANCE TESTING-CHNGS TO TECH SPECS-RTS COMPONENTS			02/01/85
53109	YANKEE ROWE 1 - REACTOR TRIP SYSTEM RELIABILITY - VENDOR RELATED MODIFICATIONS			02/01/85
52284	YANKEE ROWE 1 - CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPA#C-10)			09/30/85
51366	YANKEE ROWE 1 - INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)		3	12/31/85
53162	YANKEE ROWE 1 - ITEMS 4.2.1 & 4.2.2 - PREVENTATIVE MAINT PROGRAM FOR REACTOR TRIP BREAKERS - MAINTENANCE & TRENDING			02/01/86
TAC #	TAC DESCRIPTION	TMI ACTIONS	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				
45179	YANKEE ROWE - NUREG-0737 II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION		1	<u>COMPLETE</u> 10/24/83
45256	YANKEE ROWE - NUREG-0737 II.K.3.1, AUTO PORV ISOLATION		1	12/13/83
45302	YANKEE ROWE - NUREG-0737 II.K.3.2, REPORT ON PORV FAILURES		1	12/13/83
47826	YANKEE ROWE - NUREG 0737 ITEM II K2.17 POTENTIAL FOR VOIDING IN RCS		1	12/22/83
<u>ACTIVE ACTIONS</u>				
45991	YANKEE ROWE - NUREG-0737 III.A.1.2, TECHNICAL SUPPORT CENTER		1	<u>TARGET</u> 06/01/84
46350	YANKEE ROWE - NUREG-0737 III.A.2.2, METEOROLOGICAL DATA UPGRADE		1	06/01/84
45872	YANKEE ROWE - NUREG-0737 II.K.3.30, SB LOCA OUTLINE		1	06/30/84
51296	YANKEE ROWE 1 - I.D.2. SAFETY PARAMETER DISPLAY SYSTEM		1	08/01/84
48213	YANKEE ROWE -NUREG 0737 II.K.3.31 COMPLIANCE WITH 10 CFR 50.46		1	09/30/84
46912	YANKEE ROWE 1 - II K2.13 THERMAL MECHANICAL REPORT		1	09/30/84
46063	YANKEE ROWE - NUREG-0737 III.A.1.2, OPERATIONAL SUPPORT CENTER		1	09/30/84
46135	YANKEE ROWE - NUREG-0737 III.A.1.2, EMERGENCY OPERATIONS FACILITY		1	09/30/84
44629	YANKEE ROWE 1 - RV AND SV TESTING		1	10/31/84
44278	YANKEE ROWE - NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES		1	11/30/84
44348	YANKEE ROWE - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES		1	11/30/84

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CONDENSED MANAGEMENT REPORT

(CONTINUATION)

YANKEE-ROWE 1

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>IMI ACTIONS</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>ACTIVE ACTIONS</u> (CONTINUATION)				
51216	YANKEE ROWE 1 - I.D.1 CONTROL ROOM DESIGN REVIEW		1	<u>TARGET</u> 12/31/84
44486	YANKEE ROWE - NUREG-0737 II.B.3.2. POST ACCIDENT SAMPLING MODIFICATIONS		1	08/15/86

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
49043	YANKEE: RECOMBINER EXEMPTION REQUEST			<u>COMPLETE</u> 10/05/83
49264	YANKEE - LOW POWER PHYSICS TEST PROBLEMS		3	10/05/83
52522	YANKEE - PROCESS CONTROL PROGRAM			11/02/83
51004	YANKEE - ISI RELIEF: PERSONNEL QUALIFICATION REQMTS		3	01/17/84
52114	YANKEE - TECH SPEC CHANGE RELATING TO CYCLE DEPENDENT VARIABLES			02/09/84
51890	YANKEE - REACTOR VESSEL ISI			02/28/84
51850	YANKEE - SPENT FUEL POOL RACK SEISMIC DESIGN			03/30/84

<u>ACTIVE ACTIONS</u>				<u>TARGET</u>
54447	YANKEE ROWE - REQUEST FOR EXEMPTION - HYROGEN RECOMBINER TAPS			04/30/84
54842	YANKEE - CHANGES TO TECH. SPEC. DUE TO REFUELING MODIFICATIONS			05/20/84
54216	YANKEE ROWE - CODE 17 REFUELING			05/30/84
51812	YANKEE - VENTILATION ISSUES			05/30/84
51810	YANKEE - ELECTRICAL AND INTERLOCK ISSUES			05/30/84
52113	YANKEE - CONTAINMENT SYSTEMS SURVEILLANCE & VALVE LISTING CHANGES			06/01/84
51889	YANKEE - REANALYSIS OF MAIN STEAM LINE RUPTURE EVENT-CYCLE XVI			06/30/84
51705	YANKEE ROWE - DEGRADED GRID VOLTAGE PROCEDURES			06/30/84
48784	YANKEE ROWE - ISI/IST REVIEW		3	06/30/84
51804	YANKEE - APPENDIX R HOT SAFE SHUTDOWN SYSTEM			07/01/84
54655	YANKEE - FUEL POOL TECH. SPEC. 4.9.7.2			07/15/84
46544	YANKEE-ROWE: T.S. CHANGES - CHANGE 139, SUPPLEMENT 4			08/01/84
51806	YANKEE - CLASSIFICATION OF STRUCTURES, COMPONENTS AND SYSTEMS			08/01/84
51808	YANKEE - PIPE BREAK EFFECTS ISSUES			08/01/84
54215	YANKEE ROWE - T.S. CHANGES - CHANGE 139 - SUPPLEMENT 5			08/01/84
51358	YANKEE - LTOP - APP. G CURVE SHIFT		3	08/31/84
54424	YANKEE ROWE 1- RCS VENTS TECHNICAL SPECIFICATIONS NUREG 073 ITEM II.B.1			09/30/84
51903	YANKEE - INTEGRATED ASSESSMENT REPORT SUPPLEMENT			09/30/84
54579	YANKEE ROWE 1 - TECHNICAL SPECIFICATIONS COVERED BY GENERIC LETTERS 83-36, 83-37 FOR NUREG-0737			09/30/84
51809	YANKEE - WATER CHEMISTRY ISSUES			10/01/84
54466	YANKEE - APPENDIX K EXEMPTION REQUEST			10/15/84
51805	YANKEE HYDROLOGIC ISSUES			10/30/84
51433	YANKEE ROWE - PROBABILISTIC SAFETY STUDY		3	10/31/84
51811	YANKEE - CONTAINMENT ISSUES			12/31/84
52437	YANKEE - COORDINATION OF HAKRIMAN DAM REVIEW WITH FERC			12/31/84
52515	YANKEE - CONFIRMATION OF SEP ACTION ITEMS			01/31/85
51807	YANKEE - INTEGRATED STRUCTURAL ANALYSIS			06/06/85

<u>ANTICIPATED ACTIONS</u>				<u>INITIATION</u>
53646	YANKEE ROWE 1 - ITEM 1.2 - POST TRIP REVIEW-DATA AND INFORMATION CAPABILITY			11/01/84
53729	YANKEE ROWE 1 - ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS			11/01/84
53812	YANKEE ROWE 1- ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS			11/01/84
53895	YANKEE ROWE 1 - ITEM 3.2.3 - POST-MAINTENANCE TESTING -CHANGES TO TECH. SPEC. - ALL SR COMPONENTS			11/01/84

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CONDENSED MANAGEMENT REPORT

(CONTINUATION)

YANKEE-ROWE 1

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>	<u>INITIATION</u>
<u>ANTICIPATED ACTIONS (CONTINUATION)</u>					
53949	-ITEMS 4.2.3 & 4.2.4-PREVENTATIVE MAINT. PROGRAM FOR REACTOR TRIP BREAKERS-LIFE TESTING & REPLACEMENT				11/01/84
54039	YANKEE ROWE 1 - ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS				11/01/84
54122	YANKEE ROWE 1 - ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL TESTING - DIVERSE TRIP FEATURES				11/01/84

DATA AS OF - 04/30/84

CONDENSED MANAGEMENT REPORT

FACILITY: ZION 1

PLANT LOCATION: 40 MI N OF CHICAGO, ILL
 DOCKET NUMBER: 050-00295
 ARCH/ENGINEER: S&L
 IE INSPECTOR: J. WATERS

LICENSED POWER: 3250 MWT
 DESIGN POWER: 1040 MWE
 NSSS VENDOR: WEST

PROJECT MANAGER: J. NORRIS
 BRANCH CHIEF: S. VARGA
 LIC. ASSISTANT: C. PARRISH

TAC #	TAC DESCRIPTION	MULTI-PLANT	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				<u>COMPLETE</u>
49769	ZION 1 - "NUREG 0737 TECH SPECS (GENERIC LETTER 82-16 82-16 OR 83-02)"			11/01/83
08572	ZION 1 - REVIEW OF ASYMMETRIC LOCA LOADS SUBMITTAL		2	02/01/84
08092	ZION 1 - CONTROL OF HEAVY LOADS OVER SPENT FUEL POOL (NUREG 0612)		2	03/22/84

ACTIVE ACTIONS

TAC #	TAC DESCRIPTION	MULTI-PLANT	PRIORITY	CRITICAL DATE
<u>ACTIVE ACTIONS</u>				<u>TARGET</u>
53110	ZION 1 - REACTOR TRIP SYSTEM RELIABILITY - VENDOR RELATED MODIFICATIONS			
53163	ZION 1 - ITEMS 4.2.1 & 4.2.2 - PREVENTATIVE MAINTENANCE PROGRAM FOR REACTOR TRIP BREAKERS - MAINTENANCE & TRENDING			
53205	ZION 1 - ITEM 4.3 - AUTOAMTIC ACTUATION OF SHUNT TRIP ATTACHMENT FOR W AND B & W PLANTS			
51367	ZION 1 - INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)			
52285	ZION 1 - CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPA#C-10)			
52814	ZION 1 - ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION AND PROCEDURES			
52895	ZION 1 - ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RTS COMPONENTS			
52976	ZION 1 - ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING PROCEDURES/VENDOR RECOMMENDATIONS - RTS COMPONENTS			
53057	ZION 1 - ITEM 3.1.3 - POST-MAINTENANCE TESTING-CHANGES TO TECH SPECS-RTS COMPONENTS			
12282	ZION 1 - HELB AND CONSEQUENTIAL SYSTEM FAILURE ZION 1		3	06/30/83
42871	ZION 1 - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11		2	06/30/83
08130	ZION 1 - APPENDIX I TECH SPEC IMPLEMENTATION REVIEW		3	09/30/83
06431	ZION 1 - N-1 LOOP OPERATION		3	01/01/84
08864	ZION 1 - MECHANICAL SNUBBERS		2	03/30/84
11228	ZION 1 - HYDRAULIC SNUBBERS UPGRADE TECH SPECS		3	03/30/84
49683	ZION 1 - REACTOR COOLANT PUMP TRIP			07/01/84
42472	ZION 1 - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21)		1	03/30/85

TAC #	TAC DESCRIPTION	TMI ACTIONS	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				<u>COMPLETE</u>
46913	ZION 1 - II K2.13 THERMAL MECHANICAL REPORT		1	10/26/83
47827	ZION 1 - NUREG 0737 ITEM II K2.17 POTENTIAL FOR VOIDING IN RCS		1	01/16/84

ACTIVE ACTIONS

TAC #	TAC DESCRIPTION	TMI ACTIONS	PRIORITY	CRITICAL DATE
<u>ACTIVE ACTIONS</u>				<u>TARGET</u>
51217	ZION 1 - I.D.1 CONTROL ROOM DESIGN REVIEW			
51297	ZION 1 - I.D.2. SAFETY PARAMETER DISPLAY SYSTEM			
46351	ZION 1 - NUREG-0737 III.A.2.2, METEOROLOGICAL DATA UPGRADE		1	06/01/83
48214	ZION 1 - NUREG 0737 II.K.3.31 COMPLIANCE WITH 10 CFR 50.46		1	09/30/83
45180	ZION 1 - NUREG-0737 II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION		1	11/30/83
44487	ZION 1 - NUREG-0737 II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS		1	11/30/83
45873	ZION 1 - NUREG-0737 II.K.3.30, SB LOCA OUTLINE		1	03/30/84
45992	ZION 1 - NUREG-0737 III.A.1.2, TECHNICAL SUPPORT CENTER		1	09/30/84
46064	ZION 1 - NUREG-0737 III.A.1.2, OPERATIONAL SUPPORT CENTER		1	09/30/84
46136	ZION 1 - NUREG-0737 III.A.1.2, EMERGENCY OPERATIONS FACILITY		1	09/30/84
44630	ZION 1 - RV AND SV TESTING		1	10/31/84
44349	ZION 1 - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES		1	11/30/84
44279	ZION 1 - NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES		1	11/30/84

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CONDENSED MANAGEMENT REPORT

(CONTINUATION)

ZION 1

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				<u>COMPLETE</u>
42306	ZION 1 - CONTROL ROOM - HUMAN FACTORS ENGINEERING (ORDER 2/29/80 ITEM E.2)		1	11/21/83
52116	ZION 1 - REVIEW AND APPROVE OR DISAPPROVE SECT.2.5 OF TOPICAL REPORT: "BENCHMARK OF PWR NUCLEAR DESIGN METHODS"			12/02/83
52620	ZION 1 - ROD SWAP ANALYSIS METHOD			12/07/83
52288	ZION 1 - T.S. CHANGE FOR OPTIMIZED FUEL ASSEMBLY			12/23/83
52525	ZION 1 - T.S. CHANGE RE PREVENTIVE MAINTENANCE ON SWING DIESEL GENERATOR			03/12/84
53464	ZION 1 TS CHANGE REPVENTIVE MAINTENANCE ON SWING GENERATE (REVISION)			03/12/84
51306	ZION 1 - OPERABILITY OF VENT/PURGE VALVES			04/03/84
54306	ZION 1 - CONTROL RAD. CONT. MATERIALS - DEMINIMUS LEVELS			04/13/84
52118	ZION 1 - T.S. CHANGE TO CONT. LEAKAGE TESTS AND ADD T.S FOR AIRLOCK LEAKAGE TESTS			04/18/84
52380	ZION 1 - TS CHANGE RE COOLANT CHEMISTRY AND SPECIFIC ACTIVITY			04/19/84
54365	ZION - 1 ONE TIME EXEMPTION FROM STATE AND COUNTY PANTIC. IN ANNUAL EMERG. EXERCISE			04/19/84
<u>ACTIVE ACTIONS</u>				<u>TARGET</u>
54176	ZION 1 - RECONSIDERATION OF FEB. 29, 1980, ORDER			
54203	ZION 1 - SCHEDULAR EXEMPTION FROM 10CFR 50.48(C)4			
54160	ZION 1 - ITEM B.6 OF 1980 ORDER			
54901	ZION 1 - RECONSIDERATION OF FEB. 29, 1980 ORDER ITEM A-1			
54903	ZION 1 - RECONSIDERATION OF FEB. 29, 1980 ORDER ITEM A-2			
54905	ZION 1 - RECONSIDERATION OF FEB. 29, 1980 ORDER ITEM A-3			
54907	ZION 1 - RECONSIDERATION OF FEB. 29, 1980 ORDER ITEM A-4			
54909	ZION 1 - RECONSIDERATION OF FEB. 29, 1980 ORDER ITEM A-5			
54911	ZION 1 - RECONSIDERATION OF FEB. 29, 1980 ORDER ITEM A-6			
54913	ZION 1 - RECONSIDERATION OF FEB. 29, 1980 ORDER ITEM A-7			
54915	ZION 1 - RECONSIDERATION OF FEB. 29, 1980 ORDER ITEM B-1			
52185	ZION 1 - T.S. CHANGES RE AUXILIARY ELECTRIC POWER AND ECCS			
54935	ZION 1 - RECONSIDERATION OF FEB. 29, 1980 ORDER ITEM E-1			
54937	ZION 1 - RECONSIDERATION OF FEB. 29, 1980 ORDER ITEM E-2			
54939	ZION 1 - RECONSIDERATION OF FEB. 29, 1980 ORDER ITEM B-3			
54941	ZION 1 - RECONSIDERATION OF FEB. 29, 1980 ORDER ITEM F-4			
54943	ZION 1 - RECONSIDERATION OF FEB. 29, 1980 ORDER ITEM F-5			
54917	ZION 1 - RECONSIDERATION OF FEB. 29, 1980 ORDER ITEM B-3			
54919	ZION 1 - RECONSIDERATION OF FEB. 29, 1980 ORDER ITEM B-4			
54921	ZION 1 - RECONSIDERATION OF FEB. 29, 1980 ORDER ITEM B-5			
54923	ZION 1 - RECONSIDERATION OF FEB. 29, 1980 ORDER ITEM B-6			
54925	ZION 1 - RECONSIDERATION OF FEB. 29, 1980 ORDER ITEM C-3			
54927	ZION 1 - RECONSIDERATION OF FEB. 29, 1980 ORDER ITEM C-4			
54929	ZION 1 - RECONSIDERATION OF FEB. 29, 1980 ORDER ITEM C-5			
54931	ZION 1 - RECONSIDERATION OF FEB. 29, 1980 ORDER ITEM D-1			
54933	ZION 1 - RECONSIDERATION OF FEB. 29, 1980 ORDER ITEM D-2			
48095	ZION 1 - OPERATION WITH ROD URGENT FAILURE ALARM		3	05/30/83
49601	ZION 1 - MOD TO 1980 ORDER - AFWS			05/30/83
48103	ZION 1 - BORATION FLOW PATHS HEAT TRACING		3	06/30/83
42146	ZION 1 - ATWS INSTRUMENTATION MODIFICATIONS (ORDER 2/29/80 ITEM C.4)		1	06/30/83
42836	ZION 1 - FAILURE MODE EFFECTS ANALYSIS (2/29/80 ORDER ITEM F.4)		1	09/01/83
51901	ZION 1 - 6/27/83 ISI REQUEST			09/01/83
48692	ZION 1 - LICENSE CONDITION FOR TECH SPECS IN EMERG			09/30/83
41044	ZION 1 - RESIDUAL RISKS		1	02/28/84
13021	ZION 1 - STANDARD TECH SPEC LCD'S FOR TASK ACTION PLAN ITEM F.1 (F)(3)		1	03/15/84
54162	ZION 1 - REVIEW OF REVISED LICENSED OPERATOR REQUALIFICATION TOPICAL REPORT			03/30/84

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CONDENSED MANAGEMENT REPORT

(CONTINUATION)

ZION 1

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>ACTIVE ACTIONS</u> (CONTINUATION)				<u>TARGET</u>
54426	ZION 1- RCS VENTS TECHNICAL SPECIFICATIONS NUREG 073 ITEM II.B.1			09/30/84
54580	ZION 1 - TECHNICAL SPECIFICATIONS COVERED BY GENERIC LETTERS 83-36, 83-37 FOR NUREG-0737			09/30/84
52100	ZION 1 - ISI AND IST PROGRAM, SECOND INTERVAL			12/31/84
<u>ANTICIPATED ACTIONS</u>				<u>INITIATION</u>
53648	ZION ITEM 1.2 - POST TRIP REVIEW-DATA AND INFORMATION CAPABILITY			11/01/84
53731	ZION 1 - ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS			11/01/84
53814	ZION 1- ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS			11/01/84
53897	ZION 1 - ITEM 3.2.3 - POST-MAINTENANCE TESTING -CHANGES TO TECH. SPEC. - ALL SR COMPONENTS			11/01/84
53950	-ITEMS 4.2.3 & 4.2.4-PREVENTATIVE MAINT. PROGRAM FOR REACTOR TRIP BREAKERS-LIFE TESTING & REPLACEMENT			11/01/84
54041	ZION 1 - ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS			11/01/84
54124	ZION ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL CAPABILITY			11/01/84

DATA AS OF - 04/30/84

CONDENSED MANAGEMENT REPORT

FACILITY: ZION 2

PLANT LOCATION: 40 MI N OF CHICAGO, ILL
 DOCKET NUMBER: 050-00304
 ARCH/ENGINEER: S&L
 IE INSPECTOR: J. WATERS

LICENSED POWER: 3250 MWT
 DESIGN POWER: 1040 MWE
 NSSS VENDOR: WEST

PROJECT MANAGER: J. NORRIS
 BRANCH CHIEF: S. VARGA
 LIC. ASSISTANT: C. PARRISH

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>MULTI-PLANT</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
49770	ZION 2 - "NUREG 0737 TECH SPECS (GENERIC LETTER 82-16 82-16 OR 83-02)"			<u>COMPLETE</u> 11/01/83
08873	ZION 2 - REVIEW OF ASYMMETRIC LOCA LOADS SUBMITTAL		2	02/01/84
08093	ZION 2 - CONTROL OF HEAVY LOADS OVER SPENT FUEL POOL (NUREG 0612)		2	03/22/84
<u>ACTIVE ACTIONS</u>				
51368	ZION 2 - INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)			<u>TARGET</u>
52286	ZION 2 - CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPA#C-10)			
52815	ZION 2 - ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION AND PROCEDURES			
52896	ZION 2 - ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RTS COMPONENTS			
52977	ZION 2 - ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING PROCEDURES/VENDOR RECOMMENDATIONS - RTS COMPONENTS			
53058	ZION 2-ITEM 3.1.3 -POST-MAINTENANCE TESTING-CHANGES TO TECH SPECS-RTS COMPONENTS			
53111	ZION 2 - REACTOR TRIP SYSTEM RELIABILITY - VENDOR RELATED MODIFICATIONS			
53164	ZION 2 - ITEMS 4.2.1 & 4.2.2 - PREVENTATIVE MAINTENANCE PROGRAM FOR REACTOR TRIP BREAKERS - MAINTENANCE & TRENDING			
53206	ZION 2 - ITEM 4.3 - AUTOAMTIC ACTUATION OF SHUNT TRIP ATTACHMENT FOR W AND B & W PLANTS			
12283	ZION 2 - HELB AND CONSEQUENTIAL SYSTEM FAILURE ZION 2		3	06/30/83
42872	ZION 2 - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11		2	06/30/83
08131	ZION 2 - APPENDIX I TECH SPEC IMPLEMENTATION REVIEW		3	09/30/83
08844	ZION 2 - N-1 LOOP OPERATION		3	01/01/84
08865	ZION 2 - MECHANICAL SNUBBERS		2	03/30/84
11229	ZION 2 - HYDRAULIC SNUBBERS UPGRADE TECH SPECS		3	03/30/84
49684	ZION 2 - REACTOR COOLANT PUMP TRIP			07/01/84
42473	ZION 2 - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21)		1	03/30/85
<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>TMI ACTIONS</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
46914	ZION 2 - II K2.13 THERMAL MECHANICAL REPORT		1	<u>COMPLETE</u> 10/26/83
47828	ZION 2 - NUREG 0737 ITEM II K2.17 POTENTIAL FOR VOIDING IN RCS		1	01/16/84
<u>ACTIVE ACTIONS</u>				
51218	ZION 2 - I.D.1 CONTROL ROOM DESIGN REVIEW			<u>TARGET</u>
51298	ZION 2 - I.D.2. SAFETY PARAMETER DISPLAY SYSTEM			
46352	ZION 2 - NUREG-0737 III.A.2.2, METEOROLOGICAL DATA UPGRADE		1	06/01/83
44631	ZION 2 - RV AND SV TESTING		1	10/31/83
45181	ZION 2 - NUREG-0737 II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION		1	11/30/83
44488	ZION 2 - NUREG-0737 II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS		1	11/30/83
45874	ZION 2 - NUREG-0737 II.K.3.30, SB LOCA OUTLINE		1	03/30/84
45993	ZION 2 - NUREG-0737 III.A.1.2, TECHNICAL SUPPORT CENTER		1	09/30/84
46065	ZION 2 - NUREG-0737 III.A.1.2, OPERATIONAL SUPPORT CENTER		1	09/30/84
46137	ZION 2 - NUREG-0737 III.A.1.2, EMERGENCY OPERATIONS FACILITY		1	09/30/84
44280	ZION 2 - NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES		1	11/30/84
44350	ZION 2 - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES		1	11/30/84

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CONDENSED MANAGEMENT REPORT

(CONTINUATION)

ZION 2

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
42307	ZION 2 - CONTROL ROOM - HUMAN FACTORS ENGINEERING (ORDER 2/29/80 ITEM E.2)		1	<u>COMPLETE</u> 11/21/83
52117	ZION 2 - REVIEW AND APPROVE OR DISAPPROVE SECT.2.5 OF TOPICAL REPORT: "BENCHMARK OF PWR NUCLEAR DESIGN METHODS"			12/02/83
52621	ZION 2 ROD SWAP ANALYSIS METHOD			12/07/83
52289	ZION 2 - T.S. CHANGE FOR OPTIMIZED FUEL ASSEMBLY			12/23/83
52526	ZION 2 - T.S. CHANGE RE PREVENTIVE MAINTENANCE ON SWING DIESEL GENERATOR			03/12/84
53465	ZION 2 - TS CHANGE RE PREVENTIVE MAINTENANCE ON GIVING GENERATOR (REVISION)			03/12/84
51307	ZION 2 - OPERABILITY OF VENT/PURGE VALVES			04/03/84
54307	ZION 2 - CONTROL RAD. CONT. MATERIALS - DEMINIMUS LEVELS			04/13/84
54366	ZION-2 - ONE TIME EXEMPTION FROM STATE AND COUNTY PENTIC. IN ANNUAL ENERG. EXERCISE			04/16/84
52119	ZION 2 - T.S. CHANGE TO CONT. LEAKAGE TESTS AND ADD T.S. FOR AIRLOCK LEAKAGE TESTS			04/18/84
52381	ZION 2 - TS CHANGE RE COOLANT CHEMISTRY AND SPECIFIC ACTIVITY			04/19/84
<u>ACTIVE ACTIONS</u>				
52186	ZION 2 - T.S. CHANGE RE AUXILIARY ELECTRIC POWER AND ECCS			<u>TARGET</u>
54161	ZION 2 - ITEM B.6 OF 1980 ORDER			
54177	ZION 2 - RECONSIDERATION OF FEB. 29, 1980 ORDER			
54204	ZION 2 - SCHEDULAR EXEMPTION FROM 10CFR 50.48(C)4			
54902	ZION 1 - RECONSIDERATION OF FEB. 29, 1980 ORDER ITEM A-1			
54904	ZION 2 - RECONSIDERATION OF FEB. 29, 1980 ORDER ITEM A-2			
54906	ZION 2 - RECONSIDERATION OF FEB. 29, 1980 ORDER ITEM A-3			
54908	ZION 2 - RECONSIDERATION OF FEB. 29, 1980 ORDER ITEM A-4			
54910	ZION 2 - RECONSIDERATION OF FEB. 29, 1980 ORDER ITEM A-5			
54912	ZION 2 - RECONSIDERATION OF FEB. 29, 1980 ORDER ITEM A-6			
54914	ZION 2 - RECONSIDERATION OF FEB. 29, 1980 ORDER ITEM A-7			
54916	ZION 2 - RECONSIDERATION OF FEB. 29, 1980 ORDER ITEM B-1			
54918	ZION 2 - RECONSIDERATION OF FEB. 29, 1980 ORDER ITEM B-3			
54920	ZION 2 - RECONSIDERATION OF FEB. 29, 1980 ORDER ITEM B-4			
54922	ZION 2 - RECONSIDERATION OF FEB. 29, 1980 ORDER ITEM B-5			
54924	ZION 2 - RECONSIDERATION OF FEB. 29, 1980 ORDER ITEM B-6			
54926	ZION 2 - RECONSIDERATION OF FEB. 29, 1980 ORDER ITEM C-3			
54928	ZION 2 - RECONSIDERATION OF FEB. 29, 1980 ORDER ITEM C-4			
54930	ZION 2 - RECONSIDERATION OF FEB. 29, 1980 ORDER ITEM C-5			
54932	ZION 2 - RECONSIDERATION OF FEB. 29, 1980 ORDER ITEM D-1			
54934	ZION 2 - RECONSIDERATION OF FEB. 29, 1980 ORDER ITEM D-2			
54936	ZION 2 - RECONSIDERATION OF FEB. 29, 1980 ORDER ITEM E-1			
54938	ZION 2 - RECONSIDERATION OF FEB. 29, 1980 ORDER ITEM E-2			
54940	ZION 2 - RECONSIDERATION OF FEB. 29, 1980 ORDER ITEM F-2			
54942	ZION 2 - RECONSIDERATION OF FEB. 29, 1980 ORDER ITEM F-4			
54944	ZION 2 - RECONSIDERATION OF FEB. 29, 1980 ORDER ITEM F-5			
49602	ZION 2 - MOD TO 1980 ORDER - AFWS			05/30/83
42147	ZION 2 - ATWS INSTRUMENTATION MODIFICATIONS (ORDER 2/29/80 ITEM C.4)		1	06/30/83
48104	ZION 2 - BORATION FLOW PATHS HEAT TRACING		3	06/30/83
42837	ZION 2 - FAILURE MODE EFFECTS ANALYSIS (2/29/80 ORDER ITEM F.4)		1	09/01/83
51902	ZION 2 - 6/27/83 ISI REQUEST			09/01/83
48693	ZION 2 - LICENSE CONDITION FOR TECH SPECS IN EMERG			09/30/83
41045	ZION 2 - RESIDUAL RISKS		1	02/28/84
13022	ZION 2 - STANDARD TECH SPEC LCO'S FOR TASK ACTION PLAN ITEM F.1 (F)(3)		1	03/15/84
54163	ZION 2 - REVIEW OF REVISED LICENSED OPERATOR REQUALIFICATION TOPICAL REPORT			03/30/84
54427	ZION 2 - RCS VENTS TECHNICAL SPECIFICATIONS NUREG 073 ITEM II.B.1			09/30/84

DATA AS OF - 04/30/84

CONDENSED MANAGEMENT REPORT

(CONTINUATION)

ZION 2

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>ACTIVE ACTIONS</u> (CONTINUATION)				<u>TARGET</u>
54581	ZION 2 - TECHNICAL SPECIFICATIONS COVERED BY GENERIC LETTERS 83-36, 83-37 FOR NUREG-0737			09/30/84
52101	ZION 2 - ISI AND IST PROGRAM, SECOND INTERVAL			12/31/84
<u>ANTICIPATED ACTIONS</u>				<u>INITIATION</u>
53649	ZION 2 - ITEM 1.2 - POST TRIP REVIEW-DATA AND INFORMATION CAPABILITY			11/01/84
53732	ZION 2 - ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS			11/01/84
53815	ZION 2- ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS			11/01/84
53898	ZION 2 - ITEM 3.2.3 - POST-MAINTENANCE TESTING -CHANGES TO TECH. SPEC. - ALL SR COMPONENTS			11/01/84
53951	-ITEMS 4.2.3 & 4.2.4-PREVENTATIVE MAINT. PROGRAM FOR REACTOR TRIP BREAKERS-LIFE TESTING & REPLACEMENT			11/01/84
54042	ZION 2 - ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS			11/01/84
54125	ZION 2 - ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL CAPABILITY			11/01/84

TECHNICAL ASSIGNMENT CONTROL SYSTEM
CONDENSED MANAGEMENT SUMMARY REPORT

DATA AS OF - 04/30/84

FACILITY	COMPLETE	ACTIVE	ANTICIPATED
ARKANSAS 1	17	53	8
ARKANSAS 2	23	44	7
BEAVER VALLEY 1	22	47	8
BIG ROCK POINT 1	18	42	6
BROWNS FERRY 1	13	76	6
BROWNS FERRY 2	12	69	6
BROWNS FERRY 3	12	81	6
BRUNSWICK 1	28	45	8
BRUNSWICK 2	35	51	9
CALVERT CLIFFS 1	25	45	8
CALVERT CLIFFS 2	22	38	8
COOK 1	14	34	7
COOK 2	15	37	7
COOPER STATION	12	29	7
CRYSTAL RIVER 3	10	57	8
DAVIS-BESSE 1	13	70	8
DIABLO CANYON 1	0	40	7
DRESDEN 1	1	3	0
DRESDEN 2	19	52	6
DRESDEN 3	22	44	6
DUANE ARNOLD	19	47	7
FARLEY 1	28	43	7
FARLEY 2	19	55	7
FITZPATRICK	12	50	6
FORT CALHOUN 1	11	41	7
FORT ST VRAIN	18	37	6
GINNA	22	75	7
GRAND GULF 1	16	81	6
HADDAM NECK	10	65	8
HATCH 1	18	43	6
HATCH 2	21	42	6
HUMBOLDT BAY 3	0	3	0
INDIAN POINT 2	10	56	7
INDIAN POINT 3	8	55	7
KEWAUNEE	13	44	7
LA CROSSE	18	53	7
LASALLE 1	6	16	7
LASALLE 2	5	2	6
MAINE YANKEE	19	54	10
MCGUIRE 1	9	27	7
MCGUIRE 2	10	25	7
MILLSTONE 1	19	63	7
MILLSTONE 2	16	47	8
MONTICELLO	17	50	6
NINE MILE POINT 1	18	47	6
NORTH ANNA 1	25	50	7
NORTH ANNA 2	25	49	8
OCONEE 1	14	49	8
OCONEE 2	15	50	8
OCONEE 3	14	50	8

DATA AS OF - 04/30/84

CONDENSED MANAGEMENT SUMMARY REPORT

FACILITY	COMPLETE	ACTIVE	ANTICIPATED
OYSTER CREEK 1	12	63	6
PALISADES	17	54	8
PEACH BOTTOM 2	24	51	6
PEACH BOTTOM 3	20	49	7
PILGRIM 1	16	47	6
POINT BEACH 1	21	44	8
POINT BEACH 2	20	41	8
PRAIRIE ISLAND 1	10	29	7
PRAIRIE ISLAND 2	9	29	7
QUAD CITIES 1	9	40	7
QUAD CITIES 2	12	37	7
RANCHO SECO 1	19	66	8
ROBINSON 2	13	64	8
SALEM 1	9	45	7
SALEM 2	8	46	8
SAN ONOFRE 1	17	48	9
SAN ONOFRE 2	28	85	10
SAN ONOFRE 3	17	77	10
SEQUOYAH 1	13	55	7
SEQUOYAH 2	13	47	7
ST LUCIE 1	12	42	7
ST LUCIE 2	6	33	7
SUMMER 1	15	31	8
SURRY 1	19	43	7
SURRY 2	17	45	7
SUSQUEHANNA 1	12	33	9
SUSQUEHANNA 2	0	7	3
THREE MILE ISLAND 1	32	65	8
TROJAN	13	48	7
TURKEY POINT 3	23	41	7
TURKEY POINT 4	23	41	7
VERMONT YANKEE 1	11	52	6
WASHINGTON NUCLEAR 2	0	15	7
YANKEE-ROWE 1	16	55	7
ZION 1	16	69	7
ZION 2	16	67	7
TOTAL	1,326	4,030	602

SECTION VI
MULTIPLANT ACTION DESCRIPTION
FOR
SELECTED ACTIVE ISSUES

Multi-Plant Action No. A-02

Title: Appendix A - ALARA

Priority: 1

Cognizant Assistant Director: G. C. Lainas

Cognizant Branch Chief: J. F. Stolz

Lead Project Manager: G. Gears

Lead Technical Review Branch: Effluent Treatment Systems Branch

Problem:

Implementation of Technical Specifications (TSs) called for in 50.36a and Appendix I to 10 CFR 50.

Background:

Appendix I was published in 1975 and Model TSs were sent to all licensees on 7/78, 11/78 and 7/79.

Current Status:

Action has been completed on 18 operating plants: Prairie Island (2), Cook (2), Monticello (1), Yankee Rowe (1), Beaver Valley (1), North Anna (2), Ginna (1), Brunswick (2), Fort St. Vrain (1), Oconee (3), Rancho Seco (1), and St. Lucie (1).

Thirty-eight TERs and SERs have been received from FRC and EG&G to date. Forthcoming actions will primarily involve completion of TERs, updated SERs, negotiated TSs and DL issuance of license amendments. DL is proceeding to expedite issuance of 18 TS packages it has already received in FY83 and FY84. Forty-three licensing actions have been scheduled for completion in FY84.

Actions Remaining:

1. Licensee to submit final TS packages, OOCM and PCP.
2. Contractor provide NOR with updated SERs and negotiated TS.
3. DL issue license amendment.

Status Updated: April 23, 1984.

Multi-Plant Action No. A-04

Title: Containment Leak Testing - Appendix J

Priority: I

Cognizant Assistant Director: G. C. Lainas

Cognizant Branch Chief: S. A. Varga

Lead Project Manager: Philip J. Polk

Lead Technical Review Branch: CSB

Problem:

Some operating plants may not be in full compliance with Appendix J to 10 CFR Part 50.

Background:

Appendix J to 10 CFR Part 50 was published after many nuclear power plants had either received their operating license or their containments had reached an advanced state of design of construction. Accordingly, some plants may not be in full compliance with Appendix J. Licensee submittals, including exemption requests and proposed Technical Specifications need to be reviewed.

Current Status:

Franklin Research Center performed the review of the 47 remaining plants in this program under contract. All 46 TERS have been received from Franklin Research Institute. No TER is expected for Dresden 1 which is on hold. All 46 SERs have been forwarded to DL by CSB. Thirty-one licensing actions have been completed.

The licensee for Nine Mile Point I has submitted a revised package. An SER is expected from CSB in 3rd Q FY84. CSB reviews may be required for revised submittals for Peach Bottom 2/3, Pilgrim 1, and Hatch 1.

MPA A-04 (continued)

- 2 -

Actions Remaining:

ORPMs are working on the remaining SERs in preparation for issuance.

Status Updated: April 4, 1984.

Multi-Plant Action No. A-14

Title: 10 CFR 50.55a(g) Inservice Testing

Priority: 1

Cognizant Assistant Director: G. C. Lainas

Cognizant Branch Chief: S. Varga

Lead Project Manager: P. Tam

Lead Technical Review Branch: MEB, R-I, R-II, R-III, R-IV, R-V

Problem:

Licenses for boiling or pressurized water-cooled nuclear power facilities are required by 10 CFR 50.55a(g) to have an inservice testing program that meets the requirements of Section XI of the ASME Boiler and Pressure Vessel Code and Addenda (the Code). If the licensee determines that it is impractical to meet certain requirements of the Code, justification must be provided to the Commission, which can grant relief from specific Code requirements or impose alternate requirements.

Background:

Staff review is currently several years behind originally planned schedules. The staff has previously given interim implementation guidance to the licensees by letter. In general, this guidance authorized the licensees to implement their IST plans as proposed pending completion of review by the staff. In some cases the period for which the proposed plans and interim implementation authorization would apply has expired and the licensees have made subsequent submittals before the staff completed review of the original plan.

Current Status:

The backlog of IST relief requests is being reviewed by the above named organizations. After the current backlog is cleared, reviews of requests for relief from Code requirements will be conducted on a plant-specific basis with a schedule keyed to testing program update requirements.

Actions Remaining:

SERs to be written - 6; licensing actions to be completed - 14.

Status Updated: April 13, 1984.

Multi-Plant Action No. B-04

Title: Reactor Vessel Overpressure Protection

Priority: 2

Cognizant Assistant Director: G. C. Lainas

Cognizant Branch Chief: J. F. Stolz

Lead Project Manager: A. W. DeAgazio

Lead Technical Review Branch: RSB*

Problem:

Prevent overpressurizing the reactor vessel when at low temperature. Applicable to PWRs only.

Background:

A number of events have demonstrated that administrative controls alone are not adequate to prevent low-temperature overpressure events at PWRs. All PWRs were requested to provide an overpressure protection system that is to be used when the plant is in a low-temperature shutdown condition.

Current Status:

All plants implemented their system with preliminary NRC approval followed by post implementation review of (a) electrical aspects; (b) reactor system aspects; and (c) Technical Specifications. Four plants remain for which licensing action is required. Technical reviews for all plants are complete however open issues remain to be resolved for one plant. Administrative action only is required to close the other three licensing actions.

Actions Remaining:

PM to resolve open issues on the one plant. PM to issue SER and close out remaining actions.

Status Updated: April 30, 1984.

Multi-Plant Action No. B-05 (USI A-42)

Title: Stress Corrosion Cracking - Implementation Of NUREG-0313, Rev. 1

Priority: High

Cognizant Assistant Director: G. C. Lainas

Cognizant Branch Chief: D. Vassallo

Lead Project Manager: R. J. Clark

Lead Technical Review Branch: ORAB, MTEB

Problem:

Leaks and cracks in the heat-affected zones (HAZs) of welds in BWR austenitic stainless steel piping and components have been observed since the mid-1960's. All the cracks have been attributed to intergranular stress corrosion cracking (IGSCC). The revised purpose of this task is to ensure that all BWRs have adequate programs - as defined in NUREG-0313, Rev. 1 - to: (1) design systems constituting the reactor coolant system pressure boundary (RCSPB) to minimize the potential for IGSCC, (2) select materials for the RCSPB resistant to IGSCC, (3) control the water chemistry environment so as to reduce the potential for intergranular corrosion, (4) detect potential cracking through augmented inspections before it develops into a through-wall leak, and (5) detect possible RCSPB leakage before it becomes significant by adopting more stringent limits on allowable unidentified leakage.

Background:

In 1975, NRC (AEC) formed a Pipe Cracking Study Group (PCSG) to investigate the cause of cracking in BWR piping. In July 1977 the staff issued an implementation document, NUREG-0313, based on recommendations of the PCSG. As a result of cracking in large diameter pipes and in nozzles which had not been observed prior to 1975, a new PCSG was activated in September 1978. In October 1979 Revision 1 to NUREG-0313 was issued for comment; this contained the latest recommendations of the Study Group. Based on industry comments, the document was modified and reissued. In view of the revised staff acceptance criteria, the previous reviews were terminated (TACs 08207 through 08230 plus 11263). New TACs were issued (TACs 46651-46674) to cover the revised review effort.

Current Status:

In February 1981, NUREG-0313, Rev. 1 was issued to all holders of BWR operating licenses or construction permits and to all applicants for operating licenses. By July 1, 1981 the applicants/licensees were to provide their program for replacement of service sensitive lines and welds, their program for augmented inservice inspection, their program for improving the water chemistry environment and incorporation of adequate leak detection capability. These submittals were reviewed by EG&G Idaho under a technical assistance contract. While these reviews were underway, extensive cracks were found in the summer of 1982 in the large diameter recirculation system piping at Nine Mile Point Unit No. 1. This led to issuance of IE Bulletin 82-03 and 83 02, as well as some orders, requiring extensive inspections of service-sensitive welds in RCSPB piping. These inspection requirements in NUREG-0313, Rev. 1. Also, licensees of BWRs that found indications of cracks were required by amendments or orders to adopt more stringent limits on unidentified leakage. In effect, the requirements in NUREG-0313, Rev. 1 have been superseded by more recent NRC requirements. To close out this multiplant action, Safety Evaluations are scheduled to be issued in the fourth quarter of FY84 documenting the reviews that were performed and how the requirements of NUREG-0313, Rev. 1 have been implemented for each plant.

Actions Remaining:

MTEB is to provide Safety Evaluations for the 24 BWRs covered by this MPA to DL in third quarter of FY84. DL is to issue letters with Safety Evaluations to utilities in fourth quarter FY84 closing out action.

Status Updated: April 30, 1984.

Multi-Plant Action No. B-17

Title: Tech Spec Surveillance for Hydraulic Snubbers

Priority: 1

Cognizant Assistant Director: G. C. Lainas

Cognizant Branch Chief: J. R. Miller

Lead Project Manager: Leon B. Engle

Lead Technical Review Branch: See Current Status Below

Problem:

Operability of snubbers for LWRs is required to provide assurance that the structural integrity of the reactor coolant system and all other related systems is maintained during and following a seismic or other event initiating dynamic loads. OPERABILITY is verified by an Inservice Inspection and Testing Surveillance program specified in plant Technical Specifications (TSs). Recent operating experience has indicated the need for changes, clarifications and improvements in the Inservice Surveillance Requirements for hydraulic snubbers.

Background:

By letter dated November 20, 1981, the NRC requested that all power reactor licensee's (except SEP) incorporate the model NRC TS in plant specific Appendix A TS. A similar request was sent to SEP operating licensees on March 23, 1981. The NRC TS require that the visual inspection frequency be based upon maintaining a constant level of snubber protection to systems. In order to provide assurance of hydraulic snubber functional reliability, a representative sampling of plant installed snubbers must be functionally tested during plant shutdowns at 18 month intervals. The required sampling provides a confidence level of 95% that 90% to 100% of plant specific snubbers will be OPERABLE within acceptance limits.

Current Status:

In October 1982, management decided to transfer the remaining B-17 actions to the Regions. Upon official transfer, review schedules will be established with the Regions (approximately 45 days) so that the remaining units (38) for B-17 will be scheduled for completion in the 2nd, 3rd, and 4th quarters, FY83.

The official transfer package to the Regions for B-17 was completed on March 1983. Region I has established completion dates for FY83. However, it is noted that Region I may contract B-17 to an outside consultant. The other regions have still to establish firm dates. A meeting is being set up at NRR headquarters with Region representatives for expediting B-17.

In the interim, the completion package; i.e., amendment to license was completed for D.C. Cook on April 13, 1983 for B-17 by NRR.

Status Updated: May 13, 1983.

Multi-Plant Action No. B-22

Title: Tech Spec Surveillance for Mechanical Snubbers

Priority: 1

Cognizant Assistant Director: G. C. Lainas

Cognizant Branch Chief: J. R. Miller

Lead Project Manager: Leon B. Engle

Lead Technical Review Branch: See Current Status Below

Problem:

OPERABILITY of snubbers for LWRs is required to provide assurance that the structural integrity of the reactor coolant system and all other related systems is maintained during and following a seismic or other event initiating dynamic loads. Operability is verified by an Inservice Inspection and Testing Surveillance program specified in plant Technical Specifications (TS). Prior to initiation of MB-22, mechanical snubbers had not been included in operating plant surveillance programs. Recent operating experience has indicated that mechanical snubbers require OPERABILITY requirements similar to those required for hydraulic snubbers.

Background:

By letter dated November 20, 1981, the NRC requested that all power reactor licensee's (except SEP) incorporate the model NRC TS for mechanical snubbers in plant specific Appendix A TS. A similar request was sent to SEP operating licensees on March 23, 1981. The NRC TS require that the visual inspection frequency be based upon maintaining a constant level of snubbers protection to systems. In order to provide assurance of mechanical snubber functional reliability, a representative sampling of plant installed snubbers must be functionally tested during plant shutdowns at 18 month intervals. The required sampling provides a confidence level of 95% that 90% to 100% of plant specific snubbers will be OPERABLE within acceptance limits.

Current Status:

In October 1982, management decided to transfer the remaining B-22 actions to the Regions. Upon official transfer, review schedules will be established with the Regions (approximately 45 days) so that the remaining units (32) for B-22 will be scheduled for completion in the 2nd, 3rd, and 4th quarters, FY-83.

The official transfer package to the Regions for B-22 was completed in March, 1983. Region I has established completion dates for FY83. However, it is noted that Region I may contract B-22 to an outside consultant. The other Regions have still to establish firm dates. A meeting is being set up at NRR headquarters with Region representatives for expediting B-22.

In the interim, the completion package, i.e., amendment to license was completed for D.C. Cook on April 13, 1983 for B-22 by NRR.

Status Updated: May 13, 1983

Multi-Plant Action No. B-23

Title: Potential Equipment Failures Associated with Degraded Grid Voltage

Priority: 1

Cognizant Assistant Director: F. Miraglia

Cognizant Branch Chief: D. Crutchfield

Lead Project Manager: R. Dudley

Lead Technical Review Branch: Power Systems Branch

Problem:

Potential exists for supplying safety and non-safety equipment with voltage outside the range for which it was designed for longer than transient periods. Such conditions could result in safety-related equipment being unavailable to mitigate the consequences of an accident.

Background:

On July 20, 1976, Northeast Nuclear Energy Company (NNECo) reported that, following a trip of Millstone Unit no. 2 on July 5, 1976, several motors powered from 480 volt (v) motor control centers failed to start as required. The failure of the 480 v motors to start was traced to blown control power fuses on the individual motor controllers. These controllers received control power through 480 v/120 v transformers within the controller.

NNECo's investigation disclosed that, as a result of the plant trip, the grid voltage dropped from 352 kv to 333 kv. This voltage drop, in conjunction with additional voltage drops associated with the transformers involved, reduced the control power and voltage within individual 480 v controllers to a voltage which was insufficient to actuate the main line controller contactors. As a result, when the motors were signalled to start, the control power fuses were blown. Subsequent testing by NNECo showed that the contractors required at least 410 v to function properly.

NNECo concluded that under similar low voltage conditions, the operability of 480 v Engineered Safety Feature equipment could not be assured.

NNECO's immediate corrective action was to raise the setpoint of the Engineered Safeguards Actuation System (ESAS) "loss of power" under-voltage relays to assure that the plant would be separated from the grid and that emergency power system (dual) operation would be initiated before the control voltage fell below that required for contactor operation. A trip of the under-voltage relays causes the emergency buses to be de-energized and a load shed signal to strip the emergency buses, the diesel generators to start and power the emergency buses, and required safety related loads to sequence start on the buses.

On July 21, 1976, NNECO reported that the earlier correction action taken was no longer considered appropriate because during starting of a circulating water pump, the voltage dropped below the new ESAS undervoltage relay setting. This de-energized the emergency buses, caused load shedding to occur, started the diesel generators and began sequencing loads onto the emergency buses in accordance with the design. However, during sequencing of the loads onto the buses the voltage again dropped below the undervoltage relay setting which caused the load shed signal to strip the buses. The result was energized emergency buses with no loads supplied.

Current Status:

All utilities operating plants (BWR, PWR, HTGR) licensed prior to 1976 must provide under-voltage relays which separate the plant from the grid (in case of degraded grid voltage) and activate the plant emergency power system. The current DL schedule requires this item to be completed in FY84. Technical review is provided by the Power Systems Branch, with contractor technical support from Lawrence Livermore Lab and EG&G.

67 - operating plants covered by B-23
67 - SERs completed (4 of which have open items)
61 - Licensing Actions completed

Actions Remaining:

Complete the 4 remaining SERs. Complete the 6 remaining licensing actions.

Status Updated:

April 16, 1984

Multi-Plant Action No. B-24*
*B-20 integrated into B-24 on 8/25/80

Title: Venting and Purging Containment While At Full Power and Effect of LOCA

Priority: High

Cognizant Assistant Director: G. C. Lainas

Cognizant Branch Chief: Steven A. Varga

Lead Project Manager: Edward A. Reeves

Lead Technical Review Branch: CSB/DSI, M. Fields

Problem:

Purging and/or venting of reactor containment buildings with reactor coolant temperature above 200°F may result in site boundary doses exceeding 10 CFR 50 Part 100 limits if purge or vent valves fail to shut during a LOCA.

Background:

Events at Millstone 1 and Salem 1 (where the automatic high radiation closure of purge valves was overridden by operators) resulted in a generic letter being sent to all licensees of operating reactors in November 1978 and a report to Congress (#78-05). NRC advised licensees to take action to minimize purge and vent operations and to review and respond to SRP 6.2.4 and BTP CSB 6-4. In October 1979 a Staff Interim Position was developed. Licensees were requested to conform to it pending completion of our long-term review. In August 1980 generic item B-20 Leakage of Resilient Seals for purge and vent valves was integrated into the B-24 issue. A staff position to require Technical Specifications (as a leakage surveillance at either 3-months or 6-months intervals depending on valve usage) was developed and is included in the guidance being provided to licensees as specific plant reviews programs.

Current Status:

1. Based on EQB/DE evaluation, requests for valve qualification data and 50.54(f) letters were sent to thirty licensees where insufficient qualification data had been provided. Ten of the licensees closed the valves; other provided data which EQB/DE is reviewing on a plant specific basis. EQB will provide a memorandum or SE to DL for each licensee response to the EQB evaluation as scheduled during the Quarterly Review Meeting.

2. Most of remaining action is in DL. Project Managers should expedite closeout actions and finalize Technical Specifications per lead PM memoranda of June 19, 1981, January 4, 1984, and MPA Quarterly Review memorandum dated April 20, 1984.

Status Updated: April 24, 1984.

Multi-Plant Action No. B-25

Title: Feedwater and CRD Nozzle Cracking

Priority: _____

Cognizant Assistant Director: F. Miraglia

Cognizant Branch Chief: D. Crutchfield

Lead Project Manager: R. Gilbert

Lead Technical Review Branch: _____

Problem:

BWR feedwater nozzles and control rod drive nozzles have developed cracks due to thermal fatigue.

Background:

The generic issue has been resolved by the 11/13/80 issuance of NUREG-0619 which provided licensees of BWRs with acceptable methods to resolve the issue on a plant-by-plant basis.

Current Status:

Sixteen plants have committed to follow the staff's guidance and have received letters confirming that their proposed action is acceptable. The licensing action is complete on these plants. Each will submit a post-implementation report following completion of their proposed modification.

SERs from MTEB have been received for the remaining seven units. However, some rewriting of the SERs has been necessary. It is anticipated that the issue will be closed out by the end of the third quarter of FY84.

Actions Remaining:

Complete the review on the remaining plants. Confirm commitments to follow NUREG-0619 guidance or accept the licensee's proposed deviations from the NUREG suggestions.

Status Updated: April 17, 1984.

Multi-Plant Action No. B-32

Title: Blocked SI Signal During Cooldown

Priority: 2

Cognizant Assistant Director: G. C. Lainas

Cognizant Branch Chief: S. A. Varga

Lead Project Manager: D. G. McDonald

Lead Technical Review Branch: ORAB (contractor) and ICSB

Problem:

The Division of Licensing was requested by IE to determine the acceptability of blocking the steam line differential pressure safety injection signal during cooldown simultaneously with the blocking of the coincident pressurizer low level and low pressure signal. All PWR's should be reviewed for the same problem.

Background:

An inspection at H. B. Robinson 2 determined that during cooldown the licensee blocks the safety injection initiation signal from the high differential pressure between steam lines and the steam line header. This block is hard-wired into the same switch that is used to block the pressurizer low pressure and low level safety injection signal when primary pressure is below 2000 psig. The technical specifications (Table 3.5-3) allow blocking the prssurizer signals but did not address the blocking of the steam line high differential pressure signal. We also determined that Surry 1 and 2 and Turkey Point 3 and 4 have the same instruments, signal blocking arrangements and technical specification requirements as H. B. Robinson 2.

Current Status:

A contract was initiated to resolve the concern on all PWRs and determine which PWRs have the problem. The contractor's report indicated that a total of 18 Westinghouse designed facilities appeared to have a similar problem. By memorandum dated January 6, 1983, the Instrumentation and Control Systems Branch (ICSB) indicated that plant specific comments provided in the memorandum should be sent to the 18 facilities and to one additional facility, identified during the ICSB review, for a total of 19 facilities. The Project Managers were then directed to closeout the items based on the responses to the plant specific concerns. By memorandum dated March 1, 1983, ICSB included McGuire 2 on the list of facilities requiring resolution bringing the total of plants to be reviewed under this MPA to 20.

MPA B-32 (continued)

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McGuire and Diablo Canyon 1 have been completed as part of the Operating License review and the remaining 18 actions necessary to closeout this MPA have been completed.

Actions Remaining:

None. All actions complete as of April 27, 1984.

Multi-Plant Action No. B-41

Title: Fire Protection - Appendix R, 10 CFR 50 Compliance

Priority: High

Cognizant Assistant Director: F. Miraglia

Cognizant Branch Chief: D. Crutchfield

Lead Project Manager: T. Wambach

Lead Technical Review Branch: CEB, ASB, PSB, ICSB, RSB

Problem:

Protection of safe, shutdown capability from the adverse effects of fire in any area of the plant. During the OL review of plants licensed prior to July 1876, no in-depth review was performed in this area.

Background:

Fire at Browns Ferry Nuclear Plant in March 1975 disabled redundant safety divisions and normal safe shutdown equipment. NUREG-0050.

Current Status:

10 CFR 50.48 and Appendix R to 10 CFR 50 became effective February 17, 1981. The licensees for 69 units licensed prior to January 1, 1979 have made submittals describing (1) plans and schedules to meet Appendix R including requests for exemption from the schedules of 10 CFR 50.48, (2) design descriptions of alternative, safe shutdown modifications, where those have been proposed, and (3) exemption requests from the technical requirements of Appendix R. We have completed the review and licensing actions for the exemption requests on 51 of the units. There were no exemption requests from 5 of the units. Technical review for 10 units is ongoing for second round submittals resulting from the denial of the original exemption requests. The technical review of exemption requests for 8 units have been completed and are awaiting licensing action in DL. Design descriptions of modifications for alternative, safe shutdown were submitted for all but 5 units. The modifications for 50 units have been approved. The technical reviews of another 8 units have been completed. The technical review of 6 units is ongoing.

Actions Remaining:

CMEB must complete the review of exemption requests for the 10 units which are not yet completed. DL must complete the licensing action for the 8 units for which the technical review has been completed and for the 10 units above when the technical review is completed. ASB must complete the review of the 6 units for which submittals were recently received and of the 5 units that are still to make submittals. DL must issue the approvals for 8 units for which acceptable SERs have been received.

Status Updated: Janaury 24, 1984

Multi-Plant Action No. B-48

Title: Adequacy of Station Electric Distribution Voltage

Priority: 2

Cognizant Assistant Director: F. Miraglia

Cognizant Branch Chief: D. Crutchfield

Lead Project Manager: R. Dudley

Lead Technical Review Branch: Power Systems Branch

Problem:

This review is intended to determine if the onsite distribution system in conjunction with the offsite power sources has sufficient capacity and capability to automatically start and operate all required safety loads within the equipment voltage rating.

Background:

At Arkansas Nuclear 1, loss of offsite power indicated that the electric distribution systems were inadequate for two unit operation. All utilities were asked to review their distribution systems to see if voltages were adequate.

Current Status:

69 - operating plants covered by B-48
68 - SERs completed (7 of which have open items)
61 - Licensing Actions completed

Actions Remaining:

Complete the remaining 8 SERs. Complete the remaining 8 licensing actions

Status Updated: April 16, 1984

Multi-Plant Action No. B-57

Title: Decay Heat Removal Capability

Priority: 2

Cognizant Assistant Director: G. C. Lainas

Cognizant Branch Chief: J. F. Stolz

Lead Project Manager: A. DeAgazio

Lead Technical Review Branch: RSB for five plants; ORPMs for one plant

Problem:

To assure redundancy in decay heat removal capability by incorporation of requirements into the Technical Specification.

Background:

Various events at PWRs that have occurred during shutdown conditions have indicated that administrative controls to assure decay heat removal capability have been inadequate. All PWR licensees were requested (in a letter dated June 11, 1980) to incorporate requirements for redundancy into their TSs.

Current Status:

Acceptance criteria are: (1) redundancy in decay heat removal capability must be provided for all modes or temperature ranges of operation, particularly in Modes 4, 5 and 6, and (2) adequate surveillance requirements must be provided. A total of 11 plants remain to be reviewed. Indian Point 2 and 3 have not responded to our original request.

INEL has reviewed the submittals to identify where the proposed TSs, in conjunction with existing TSs, do not conform with out Standard TSs.

Actions Remaining:

Review TERs and determine whether deficiencies are serious enough to warrant additional requests to licensees. If not, issue the license amendment as requested.

Status Updated: April 30, 1984.

Multi-Plant Action No. B-59

Title: Masonry Wall Design

Priority: 2

Cognizant Assistant Director: G. C. Lainas

Cognizant Branch Chief: J. R. Miller

Lead Project Manager: C. Trammell

Lead Technical Review Branch: SGEB

Problem:

Inadequate design criteria for masonry walls. Applies to all plants.

Background:

Problem was discovered at Trojan during reviews associated with IEB 79-02 (anchor bolts) and IEB 79-14 (as-built verification). A masonry wall was found which supported numerous safety-related pipes which was not adequate to resist seismic forces. This resulted in the issuance of IE Bulletin 80-11 on May 8, 1980. NRR (SEB) has been assigned the responsibility to review licensee's responses to this Bulletin.

Current Status:

Franklin Research Center has been contracted to review most licensee submittals. Trojan is being reviewed by SEB. Dresden 1 and Humboldt Bay are currently in a shutdown status, and will be reviewed by SEB if/when needed. Review schedule and acceptance criteria for review has been established.

Actions Remaining:

44 plants remain to be completed. 26 finished to date. FRC review is active. Q-1s issued to all licensees.

Status Updated: April 12, 1984.

Multi-Plant Action No. B-60

Title: Environmental Qualification of Electric Equipment for Nuclear Power Plants

Priority: 1

Cognizant Assistant Director: F. J. Miraglia

Cognizant Branch Chief: G. Holahan

Lead Project Manager: Rudy O. Karsch

Lead Technical Review Branch: Equipment Qualification Branch

Problem:

All plants are required to meet 10 CFR 50.49 concerning 10 CFR 50, Appendix A, GDC 4.

Background:

The issue originated from a 1977 petition from the Union of Concerned Scientists on fire protection and equipment qualification. The multiplant action arose from the Commission Memorandum and Order CLI 80-21. Technical resolution of acceptance criteria were produced from Unresolved Safety Issue A-24.

Current Status:

Technical Evaluation Reports (TERs) for all of the seventy-one (71) operating plants have been issued by Franklin Research Center.

The Equipment Qualification Branch has issued all seventy-one (71) Safety Evaluation Reports. The Division of Licensing has issued all of the seventy-one (71) attached with TERs to the licensees.

The equipment qualification rule, 10 CFR 50.49, was issued on January 21, 1983, Federal Register Volume 48, No. 15, pp 2729-2734, and has been effective since February 22, 1983. The rule requires that each holder of an operating license issued prior to February 22, 1983, shall, by May 20, 1983, identify the electrical equipment important to safety within the scope of the rule already qualified and submit a schedule for establishing a goal of completing final equipment qualification for the remaining electrical equipment important to safety within the scope of the rule by the end of the second refueling outage after March 31, 1983, or by March 31, 1985, whichever is earlier.

The staff is currently holding meetings with the licensee to discuss their responses to the SERs including the information for items in various categories for which justification for continued operation was not previously submitted to the NRC.

Actions Remaining:

Future actions required to complete the MPA will be predicated on the responses by the licensees to the SERs. The licensees are being requested (1) to provide, within ten (10) days of the receipt of the SER from the NRC, the resolution for the unqualified equipment if any, (2) to reaffirm the justification for continued operation, and within thirty (30) days of the receipt of the SER, submit information for equipment items neither considered qualified nor unqualified for which justification for continued operation was not previously submitted to the NRC, and (3) to provide the schedule for accomplishing proposed corrective action by May 20, 1983 as required by Paragraph (g) of 10 CFR 50.59.

An implementation plan needs to be developed which will establish the ground rules for evaluating the licensee's justification for continued operation.

Status Updated: April 24, 1984.

Multi-Plant Action No. B-65

Title: Safety Concerns Associated With Pipe Breaks in BWR Scram System

Priority: High

Cognizant Assistant Director: G. C. Lainas

Cognizant Branch Chief: D. Vassallo

Lead Project Manager: V. Rooney

Lead Technical Review Branch: ASB

Problem:

Assess the overall risk to long-term core cooling from postulated pipe break in Scram Discharge Volume; investigate the mitigation capability for the postulated event; establish generic recommendations for correcting any deficiencies identified.

Background:

Draft NUREG-0785 issued April 3, 1981 by AEOD described certain safety concerns associated with postulated pipe breaks in the BWR Scram System. NRC letter dated April 10, 1981 to all BWR licensees requested a generic evaluation of the concerns within 45 days and a plant specific review within 120 days. The generic evaluation was submitted by GE on behalf of the BWR owners by letter dated April 30, 1981.

Current Status:

NUREG-0803 was issued to BWR licensees and applicants on 08/31/81. Plant specific licensee responses to concerns identified in this NUREG were evaluated together with a BWROG submittal. In response to a July 25, 1983 letter, the BWROG provided additional information by letter dated November 18, 1983. On February 23, 1984 the staff met with the BWROG and identified information needed to close out this issue generically.

Actions Remaining:

Review BWROG report (expected to be received May 1984) and take appropriate closeout action.

Status Updated: April 13, 1984.

Multi-Plant Action No. B-66

Title: Natural Circulation Cooldown

Priority: Medium

Cognizant Assistant Director: G. Lainas

Cognizant Branch Chief: J. Miller

Lead Project Manager: K. Heitner

Lead Technical Review Branch: RSB (R. Jones)

Problem:

All PWRs must review plant cooldown on natural circulation to determine if voiding will occur in the reactor vessel.

Background:

St. Lucie Unit 1, while cooling down on natural circulation due to a component cooling water malfunction, developed a steam space in the upper head of the reactor vessel. Core cooling was not lost, however, complex situation was deemed an undesirable challenge to the operator.

Current Status:

Generic Letter 81-21, issued May 5, 1981, requested:

1. demonstration that natural circulation cooldown by procedure should not result in reactor vessel voiding,
2. verification that supplies of condensate grade aux feedwater are sufficient, and
3. aspects of training and procedures dealing with this subject.

All licensees have responded.

Westinghouse Owners Group Generic Report (OG-57) dated 04-20-81.

Plant specific review for leadcase (St. Lucie) is complete.

MPA B-66 (continued)

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Safety Evaluations have been completed for all Westinghouse and Combustion Engineering plants. Licensing action is completed for all but Diablo Canyon.

Actions Remaining:

Complete review for remaining 8 B&W plants. A new round of questions is being prepared to send to the licensees.

Status Updated: April 12, 1984

Multi-Plant Action No. B-69

Title: PWR Main Steam Line Break with Continued Feedwater Addition

Priority: High (IE Bulletin 80-04)

Cognizant Assistant Director: G. Lainas

Cognizant Branch Chief: J. Miller

Lead Project Manager: E. Tourigny

Lead Technical Review Branch: RSB, ICSB/DSI

Problem:

A licensee submitted a report to NRC identifying a deficiency in the original main steam line break (MSLB) analysis for their plant. They determined that, if the auxiliary feedwater system continued to supply feedwater at runout conditions to the affected steam generator, containment design pressure would be exceeded in about 10 minutes. The long term effect of the water supplied by the auxiliary feedwater system also had not been considered in prior analyses.

Background:

Virginia Electric and Power Company (VEPCO) submitted a report to the Nuclear Regulatory Commission (NRC) dated September 7, 1979, identifying a deficiency in the original main steam line break (MSLB) analysis for North Anna Power Station, Units 3 and 4. Stone and Webster Engineering Corporation re-analyzed the containment pressure response following a MSLB and determined that, if the auxiliary feedwater system continued to supply feedwater at runout conditions to the affected steam generator, containment design pressure would be exceeded in about 10 minutes. The long term effect of the water supplied by the auxiliary feedwater system had not been considered in prior analyses.

On October 1, 1971, IE Information Notice No. 79-24 was sent to all holders of operating licenses and construction permits, informing them of VEPCO's finding. The Consumer Power Company of Michigan (Palisades, Unit 1 facility) conducted an accident analysis review pursuant to the Information Notice and found that with offsite power available, the condensate pumps would feed the affected steam generator at an excessive rate. This was not considered in their analysis of the MSLB accident.

Because of the deficiencies that were identified in the original MSLB analyses, IE Bulletin No. 80-04 was issued on February 8, 1980. The IE Bulletin requested licensees and certain applicants of PWR facilities to assess the effect of continued feedwater or condensate addition on the containment response and reactor core reactivity response to a MSLB accident. The IE Bulletin was sent to all licensed PWR facilities and to those PWR facilities nearing licensing (Diablo Canyon, McGuire, North Anna 2, Salem 2, and Sequoyah).

Current Status:

All reviews are complete except for Indian Point 2 and Rancho Seco. These reviews should be completed by Jun 30, 1984.

Status Updated: April 9, 1984

Multi-Plant Action No. B-72

Title: NUREG-0737 Tech. Specs. (Generic Letter 82-16 and 83-02)

Priority: 2

Cognizant Assistant Director: F. J. Miraglia

Cognizant Branch Chief: D. Crutchfield

Lead Project Manager: Peter B. Erickson

Lead Technical Review Branch: Regions for 82-16 responses, ORAB for most 83-02 responses

Problem:

To establish Technical Specifications for TMI (NUREG-0737) issues that were originally scheduled for completion by December 31, 1981.

Background:

Generic Letters 82-16 and 83-02 requested that each PWR and BWR licensee review their facilities Technical Specifications (TSs) with respect to the guidance provided. Licensees were requested to propose TSs for the 0737 issues identified in 82-16 and 83-02. Responses have been received from all licensees, except San Onofre 1. San Onofre 1 application for TS changes depends on decision to restart plant.

Current Status:

Regional review of PWR responses to 82-16 is complete except for Arkansas 1 and 2, Crystal River, Fort Calhoun, Indian Point 1 and 2, San Onofre 1, 2 and 3, Summer 1, Turkey Point 3 and 4, and Yankee Rowe. All reviews by regions are scheduled for completion by August 30, 1984 or earlier. The regions have also completed reviews of two BWR responses to 83-02 and will also complete the review of the Cooper response to 83-02 by May 15, 1984.

ORAB is reviewing all but the above three BWR responses to 83-02. ORAB has completed their review of 83-02 responses for all but five plants. Reviews for the five remaining BWRs are scheduled for April 20, 1984.

MPA B-72 (continued)

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Project Managers have closed out B-72 for 34 plants with the balance scheduled for close out by September 30, 1984 or earlier.

Status Updated: April 30, 1984.

Multi-Plant Action No. B-75

Title: Cracking of Core Barrel Bolts In B&W Reactors

Priority: 1

Cognizant Assistant Director: G. Lainas

Cognizant Branch Chief: J. Stolz

Lead Project Manager: N. Prasad Kadambi

Lead Technical Review Branch: MTEB, MEB, RSB

Problem:

Ultrasonic testing and laboratory results indicate that the A-286 core barrel bolts may crack prematurely in the head-to-shank transition area. Justification for continued operation is required for reactors which cannot demonstrate integrity in a sufficient number of core barrel bolts.

Background:

The core barrel bolts (upper and lower) support the full weight of the core in the B&W reactor design. The main problem appears to exist in the upper core barrel bolts, although a significantly smaller number of lower core barrel bolts also show indications of failure. On May 6, 1983, the B&W Owners Group reported to NRC staff on the results of their efforts to determine the nature, extent, cause and consequences of the bolt cracking problem. The objective of the presentations was to provide justification for continued operation in the cases of the units ANO-1, Oconee 1, 2 and 3, and Davis Besse 1. Also, they sought to provide assurance that a sufficient understanding has been achieved of the problem to assure public health and safety for all B&W units with the introduction of proposed new elements into their ISI and surveillance programs.

Current Status:

The B&W Owners Group has submitted to NRC a report based on the May 6, 1983 meeting (BAW 1784, May 1983). All the reactors which had a refueling outage in 1983 has inspected the core barrel bolts. The broken bolts which have been found beyond those reported in BAW 1784 are two upper core barrel bolts in Oconee 3 reported in March 1984.

Actions Remaining:

The B&W Owners Group's Internal Bolting Task Force is scheduled to submit a comprehensive report on the bolting problem in July 1984. The report is expected to propose long term solutions; including consideration of materials selection, assembly procedures, ISI, surveillance, etc. The report will use the data from all the examinations performed so far. The action required to complete this MPA is for NRC staff to review the report when submitted and determine if the recommendations constitute a generic solution to the problem.

Status Updated: April 26, 1984

Multi-Plant Action No. B-76

Title: Item 1.1 - Post-Trip Review - Program Description and Procedure

Priority: High

Cognizant Assistant Director: T. Novak

Cognizant Branch Chief: _____

Lead Project Manager: _____

Lead Technical Review Branch: LQB

Problem:

Licensees shall describe their program for ensuring that unscheduled reactor shutdowns are analyzed and that a determination is made that the plant can be restarted safely.

Background:

Generic Letter 83-28 was issued on July 8, 1983. The letter identified staff positions stemming from the review of the Salem ATWS Events. These positions are related to reactor trip system reliability and general management capability. The letter requested licensees and applicants to furnish, no later than November 5, 1983 (120 days from July 8, 1983), the status of current conformance with the positions, and the licensees' plans and schedules for any needed improvements for conformance with the positions. Item 1.1 Generic Letter 83-28 concerns post-trip review procedures.

Current Status:

Sixty-one percent of the November 5, 1983 responses are complete and ready for technical review by the staff.

Status Updated: April 30, 1984.

Multi-Plant Action No. B-77

Title: Item 2.1 - Equipment Clarification and Vendor Interface - RTS Components

Priority: High

Cognizant Assistant Director: T. Novak

Cognizant Branch Chief: _____

Lead Project Manager: _____

Lead Technical Review Branch: ICSB

Problem:

Licensees shall confirm that all components whose functioning is required to trip the reactor are identified as safety-related on documents, procedures, and information handling systems used in the plant to control safety-related activities, including maintenance, work orders, and parts replacement. In addition, for these components, licensees and applicants shall establish, implement and maintain a continuing program to ensure that vendor information is complete, current and controlled throughout the life of the plant.

Background:

Generic Letter 83-28 was issued on July 8, 1983. The letter identified staff positions stemming from the review of the Salem ATWS Events. These positions are related to reactor trip system reliability and general management capability. The letter requested licensees and applicants to furnish, no later than November 5, 1983 (120 days from July 8, 1983), the status of current conformance with the positions, and the licensees' plans and schedules for any needed improvements for conformance with the positions. Item 2.1 of Generic Letter 83-28 concerns equipment classification and vendor interface for reactor trip system components.

Current Status:

Thirty-two percent of the November 5, 1983 responses are complete and ready for technical review by the staff.

Status Updated: April 30, 1984.

Multi-Plant Action No. B-78

Title: Items 3.1.1 and 3.1.2 - Post Maintenance Testing Procedures and Vendor Recommendations - RTS Components

Priority: High

Cognizant Assistant Director: T. Novak

Cognizant Branch Chief: _____

Lead Project Manager: _____

Lead Technical Review Branch: Regions

Problem:

Licensees shall submit the results of their review of test and maintenance procedures and Technical Specifications to assure that post-maintenance operability testing of safety-related components in the reactor trip system is required to be conducted and that the testing demonstrates that the equipment is capable of performing its safety functions before being returned to service. Licensees shall submit the results of their check of vendor and engineering recommendations to ensure that any appropriate test guidance is included in the test and maintenance procedures or the Technical Specifications, where required.

Background:

Generic Letter 83-28 was issued on July 8, 1983. The letter identified staff positions stemming from the review of the Salem ATWS Events. These positions are related to reactor trip system reliability and general management capability. The letter requested licensees and applicants to furnish, no later than November 5, 1983 (120 days from July 8, 1983), the status of current conformance with the positions, and the licensees' plans and schedules for any needed improvements for conformance with the positions. Items 3.1.1 and 3.1.2 of Generic Letter 83-28 concern post-maintenance testing procedures and vendor recommendations for reactor trip system components.

Current Status:

Fifty percent of the November 5, 1983 responses are complete and ready for technical review by the staff.

Status Updated: April 30, 1984.

Multi-Plant Action No. B-79

Title: Item 3.1.3 - Post-Maintenance Testing - Changes to Tech Specs - RTS Components

Priority: High

Cognizant Assistant Director: T. Novak

Cognizant Branch Chief: E. Adensam

Lead Project Manager: R. Birkel

Lead Technical Review Branch: ICSB (J. Calvo)

Problem:

Licensees shall identify, if applicable, any post-maintenance test requirements in existing Technical Specifications which can be demonstrated to degrade rather than enhance safety. Appropriate changes to these test requirements, with supporting justification, shall be submitted for staff approval.

Background:

Generic Letter 83-28 was issued on July 8, 1983. The letter identified staff positions stemming from the review of the Salem ATWS Events. These positions are related to reactor trip system reliability and general management capability. The letter requested licensees and applicants to furnish, no later than November 5, 1983 (120 days from July 8, 1983), the status of current conformance with the positions, and the licensees' plans and schedules for any needed improvements for conformance with the positions. Item 3.1.3 of Generic Letter 83-28 concerns changes to the Technical Specifications regarding post-maintenance testing for reactor trip system components.

Current Status:

Seventy percent of the November 5, 1983 responses are complete and ready for technical review by the staff.

Status Updated: April 30, 1984.

Multi-Plant Action No. B-80

Title: Item 4.1 - Reactor Trip System Reliability - Vendor Related
Modifications

Priority: High

Cognizant Assistant Director: T. Novak

Cognizant Branch Chief: E. Adensam

Lead Project Manager: R. Birkel

Lead Technical Review Branch: Regions

Problem:

Licensees shall review all vendor-recommended reactor trip breaker modifications to verify that either: (1) each modification has, in fact, been implemented; or (2) a written evaluation of the technical reasons for not implementing a modification exists.

Background:

Generic Letter 83-28 was issued on July 8, 1983. The letter identified staff positions stemming from the review of the Salem ATWS Events. These positions are related to reactor trip system reliability and general management capability. The letter requested licensees and applicants to furnish, no later than November 5, 1983 (120 days from July 8, 1983), the status of current conformance with the positions, and the licensees' plans and schedules for any needed improvements for conformance with the positions. Item 4.1 of Generic Letter 83-28 concerns vendor related modifications for reactor trip breakers.

Current Status:

Eighty-one percent of the November 5, 1983 responses are complete and ready for technical review by the staff.

Status Updated: April 30, 1984.

Multi-Plant Action No. B-81

Title: Items 4.2.1 and 4.2.2 - Preventative Maintenance and Trending Program for Reactor Trip Breakers

Priority: High

Cognizant Assistant Director: T. Novak

Cognizant Branch Chief: _____

Lead Project Manager: _____

Lead Technical Review Branch: EQB (R. Wright)

Problem:

Licensees shall describe their preventative maintenance and surveillance program to ensure reliable reactor trip breaker operation. The program shall include a planned program of periodic maintenance, including lubrication, housekeeping, and other items recommended by the equipment supplier, and trending of parameters affecting operation and measured during testing to forecast degradation of operability.

Background:

Generic Letter 83-28 was issued on July 8, 1983. The letter identified staff positions stemming from the review of the Salem ATWS Events. These positions are related to reactor trip system reliability and general management capability. The letter requested licensees and applicants to furnish, no later than November 5, 1983 (120 days from July 8, 1983), the status of current conformance with the positions, and the licensees' plans and schedules for any needed improvements for conformance with the positions. Items 4.2.1 and 4.2.2 of Generic Letter 83-28 concern a preventative maintenance and trending program for reactor trip breakers.

Current Status:

Twenty-three percent of the November 5, 1983 responses are complete and ready for technical review by the staff.

Status Updated: April 30, 1984.

Multi-Plant Action No. B-82

Title: Item 4.3 - Automatic Actuation of Shunt Trip Attachment for W and B&W Plants

Priority: High

Cognizant Assistant Director: T. Novak

Cognizant Branch Chief: _____

Lead Project Manager: T. Kenyon

Lead Technical Review Branch: ICSB (J. Calvo)

Problem:

Licenses of Westinghouse and B&W plants shall modify their reactor trip systems to provide automatic actuation of the breaker shunt trip attachments. The shunt trip attachment shall be considered safety related.

Background:

Generic Letter 83-28 was issued on July 8, 1983. The letter identified staff positions stemming from the review of the Salem ATWS Events. These positions are related to reactor trip system reliability and general management capability. The letter requested licensees and applicants to furnish, no later than November 5, 1983 (120 days from July 8, 1983), the status of current conformance with the positions, and the licensees' plans and schedules for any needed improvements for conformance with the positions. Item 4.3 of Generic Letter 83-28 concerns automatic actuation of the shunt trip attachment for Westinghouse and B&W plants.

Current Status:

Twenty-four percent of the November 5, 1983 responses are complete and ready for technical review by the staff.

Status Updated: April 30, 1984.

Multi-Plant Action No. C-04

Title: ESF Filter Tech. Specs.

Priority: 3

Cognizant Assistant Director: F. Miraglia

Cognizant Branch Chief: D. Crutchfield

Lead Project Manager: J. Lombardo

Lead Technical Review Branch: ETSB

Problem:

Some licensees have not submitted T.S. regarding ESF filters which comply with the position of R.G. 1.52.

Background:

Letters sent out in 1976-1977 to all licensees. Most have been completed.

Current Status:

Three plants are outstanding.

Actions Remaining:

Project Managers are pursuing.

Status Updated: January 10, 1984

Multi-Plant Action No. C-07

Title: Fuel Handling Accident Inside Containment

Priority: 2

Cognizant Assistant Director: F. Miraglia

Cognizant Branch Chief: D. Crutchfield

Lead Project Manager: J. Lombardo

Lead Technical Review Branch: _____

Problem:

By letter dated January 3, 1977 to Commissioner Gilinsky, Mr. Robert Pollard recommended that NRR review the potential consequences of a postulated fuel handling accident at each operating plant because this accident had not been considered during licensing of the plants.

Background:

Letters were sent to licensees of 39 operating plants in January 1977 requesting that they provide a detailed evaluation of the potential consequences of this accident. Letters were not sent to licensees of 24 operating plants because this accident had been evaluated in their SER.

Current Status:

There are four plants which are not complete.

Actions Remaining:

The Project Managers are pursuing resolution of the remaining issues.

Status Updated: January 10, 1984

Multi-Plant Action No. C-10

Title: Control of Heavy Loads At Nuclear Power Plants

Priority: Phase II, High

Cognizant Assistant Director: G. C. Lainas

Cognizant Branch Chief: S. Varga

Lead Project Manager: D. Neighbors

Lead Technical Review Branch: ASB

Problem:

Dropped Heavy Loads may impact spent fuel, fuel in the core, or equipment that may be required to achieve safe shutdown and continued decay heat removal for PWRs and BWRs. If sufficient spent fuel or fuel in the core were damaged and if the fuel is highly radioactive, the potential releases of material could result in off-site doses that exceed 10 CFR Part 100 limits.

Background:

USI - Task A-36 was established to examine staff licensing criteria, the adequacy of measures in effect at operating plants, and to recommend necessary changes to assure the safe handling of Heavy Loads. NUREG-0612 provides the results of the review and includes the task groups recommendation.

Current Status:

1. NUREG-0612 was issued to all licensees by letter dated December 22, 1980. The letter requested: interim actions to be completed in 90 days, a Phase I Action (report, confirmation and justification) in six months and Phase II (specific requirements) in nine months.
2. The criteria contained in NUREG-0612 is used for the acceptance criteria. The contractor is Franklin Research Center (FRC). FRC provides the TER's to the lead technical group ASB.
3. Thirty-three actions have been closed out at this time.

Actions Remaining:

Project Managers expedite licensees' responses necessary to close out open items as discussed in draft TER issued.

Status Updated: April 4, 1984

Multi-Plant Action No. C-11

Title: Reactor Protection System (RPS) Power Supplies

Priority: _____

Cognizant Assistant Director: G. Lainas

Cognizant Branch Chief: D. Vassallo

Lead Project Manager: F. Apicella

Lead Technical Review Branch: PSB

Problem:

Design deficiencies have been identified in the BWR RPS MG set regulators that could adversely affect the operability of the RPS if (1) a sequence of undetected single component failures were to occur, or (2) a seismic event occurred that resulted in the postulated component malfunctions.

Background:

The design deficiency was first identified during the Hatch 2 licensing process. By letter dated August 7, 1978, the other BWRs were directed to review their designs and submit a report. That letter also directed that a surveillance program be implemented. By letter dated September 24, 1980 the licensees were requested to: (1) commit to installing a Class 1E system, (2) provide a schedule for completion of the modification, and (3) provide a schedule for submitting design information and proposed Technical Specifications. All licensees responded satisfactorily.

Current Status:

LLNL is writing TERs based on the licensee's submittals. PSB is writing SERs. Seven reviews are complete. Four reviews are scheduled to be completed and SERs issued by the end of the third quarter and four more by the end of the fourth quarter of FY83. The reviews of the remaining five plants (Browns Ferry 1,2 and 3, Millstone 1, and Nine Mile Point) have been deferred until FY84. Eight plants have installed the modifications. Three plants (Nine Mile Point, Pilgrim, Oyster Creek) have revised the modification installation date from the end of FY83 to FY84 and two plants (Peach Bottom 2 and 3) from FY83 to FY85. No unresolved technical issues exist at this time.

Actions Remaining:

1. Review licensee's designs (post-implementation)
2. Review licensee's proposed TS changes
3. Issue license amendment that revises TS

Status Updated: May 13, 1983

Multi-Plant Action No. C-14

Title: Seismic Qualification of Auxiliary Feedwater Systems

Priority: 1

Cognizant Assistant Director: F. Miraglia

Cognizant Branch Chief: G. Holahan

Lead Project Manager: J. T. Beard

Lead Technical Review Branch: ASB

Problem:

Since TMI-2, great attention has been focused on the ability of PWRs to reliably provide for shutdown decay heat removal at normal system pressure. The steam generator is the preferred and proven choice for accomplishing a safe shutdown of the plant. Therefore, the auxiliary feedwater systems (AFW) should satisfy the seismic design criteria for the Safe Shutdown Earthquake (SSE), consistent with other safety-related systems in the plant. However, only those facilities licensed recently have been reviewed against the NRC Standard Review Plan which treats the AFW as an Engineered Safety Feature.

Background:

In August 1980, the NRC completed a preliminary study of the risk of a seismically induced loss of decay heat removal for plants without a seismically qualified AFW. One of the conclusions was that the increase in risk during the next three years is sufficiently small to allow plant operations to continued. Another conclusion was that plants should be surveyed to confirm the extent of thw BWR plants without seismically qualified AFW.

In February 1981, the NRC developed and embarked on Multi-Plant Action Plan C-14, "Seismic Qualification of Auxiliary Feedwater Systems". An objective to this plan is to increase, to the extent practicable, the capability of those plants without seismically qualified AFW to withstand earthquakes up to the SSE level. This Multi-Plant plan is described in an NRC memorandum to H. Denton dated February 20, 1981. Generic Letter 81-14 requested licensees: (1) to identify, via a walkdown of the AFW system, deficiencies ammenable to simple remedial actions, and (2) to review the plants to determine the extent and areas where significant improvements may be needed. The NRC will then take action to require licensees to make improvements such as re-analysis and/or modifications, for those plants which cannot demonstrate reasonable assurance of the functioning of the AFW system following an earthquake of SSE magnitude.

Current Status:

Under technical assistance contract FIN A-G429, Lawrence Livermore National Laboratory (LLNL) has reviewed the licensees' responses to Generic Letter 81-14 for the 47 PWRs involved. It was then necessary to generate Requests for Additional Information for almost every plant, in order to rectify deficiencies in the responses or to obtain technical clarification of the responses. LLNL documented the seismic capabilities of the AFW's in plant-specific Technical Evaluation Reports (TERs), which serve as the technical basis for NRR licensing actions.

During FY83, LLNL completed all the TERs. In some cases, additional communications with the licensees have been necessary to complete a Safety Evaluation Report (SER). In FY81, SERs for two plants were completed; in FY82, for 20 plants; in FY83 for 16 plants.

LLNL has produced a preliminary draft report for this project which provides a summary and overall evaluation of the seismic capabilities of AFW systems. The project report (NUREG/CR-3271) has been reviewed by the NRC and a final draft developed. LLNL has delivered a camera-ready copy of the overall project report. This action fulfilled the last of the contractual requirements.

Actions Remaining:

The NRC has to continue discussions with licensees for the last nine plants and complete these SERs. These actions are presently scheduled to be completed in the last quarter of FY84.

Status Updated: April 20, 1984

Multi-Plant Action No. C-15

Title: Control of Heavy Loads At Nuclear Power Plants

Priority: Phase II, Medium

Cognizant Assistant Director: G. C. Lainas

Cognizant Branch Chief: S. Varga

Lead Project Manager: D. Neighbors

Lead Technical Review Branch: ASB

Problem:

Dropped Heavy Loads may impact spent fuel, fuel in the core, or equipment that may be required to achieve safe shutdown and continued decay heat removal for PWRs and BWRs. If sufficient spent fuel or fuel in the core were damaged and if the fuel is highly radioactive, the potential releases of radioactive material could result in off-site doses that exceed 10 CFR Part 100 limits.

Background:

USI - Task A-36 was established to examine staff licensing criteria, the adequacy of measures in effect at operating plants, and to recommend necessary changes to assure the safe handling of Heavy Loads. NUREG-0612 provides the results of the review and includes the task groups recommendation.

Current Status:

(See C-10 also)

Twelve sites (20 plants) have been selected as a pilot program to determine if significant problems exist for Phase II. As a result of this program, a determination will be made whether or not to review the remainder.

Actions Remaining:

1. Project Managers expedite licensees' responses necessary to close out open items as discussed in draft TER for 12 sites as they are issued.
2. Two TERs are being issued at this time.

Status Updated: April 4, 1984

Multi-Plant Action No. D-01

Title: Mark I Containment Program

Priority: 1

Cognizant Assistant Director: G. Lainas

Cognizant Branch Chief: D. Vassallo

Lead Project Manager: B. Siegel

Lead Technical Review Branch: CSB - Load Audit Review
MEB - Structural Audit Review

Problem:

Since the original design of the Mark I containment system additional loading conditions resulting from the dynamic effects of drywell air and steam being rapidly forced into the suppression pool during a postulated LOCA and from suppression pool response to SRV operation generally associated plant transient operating conditions have been identified, the objective of this program is to restore the originally intended design safety margins for each Mark I containment system.

Background:

A Mark I Owners Group (OG) with GE the lead technical organization was formed to quantify the hydrodynamic loads and assess the effects of these loads on the Mark I containment structure. The OG divided this task into a short term (STP) and long term program (LTP).

The objectives of the STP were to verify that each Mark I containment system would maintain its integrity and functional capability when subjected to the most probable loads induced by a postulated design-basis LOCA, and to verify that licensed Mark I BWR facilities would continue to operate safely, without endangering the health and safety of the public, while a methodical, comprehensive LTP was being conducted. The staff in the "Mark I Containment Short-Term Program Safety Evaluation Report", NUREG-0408 concluded that sufficient margin of safety had been demonstrated to assure the functional performance of the containment system and, therefore, any undue risk to the health and safety of the public was precluded.

The objectives of the LTP were to establish design-basis loads that are appropriate for the anticipated life of each Mark I BWR facility (40 years), and to restore the originally intended design-safety margins for each Mark I containment system. The principle thrust of this program has been the development of generic methods for the definition of

suppression pool hydrodynamic loading events and the associated structural assessment techniques for the Mark I configuration. The generic analysis techniques are intended to be used to perform a plant-unique analysis (PUA) for each Mark I facility. The staff in the "Mark I Containment Long-Term Program Safety Evaluation Report", NUREG-0661 concluded that the load definition procedures utilized by the BWR Owners Group, as modified by the staff's requirements provide conservative estimates of these loading conditions and that the structural acceptance criteria are consistent with the requirements of the applicable codes and standards.

Current Status:

The licensees have designed and are in the process of installing modifications to meet the Commission's Order date for each operating plant. More than half of the 22 plants affected have completed these modifications. The licensees have prepared and submitted plant-unique analysis reports which provide the bases for these modifications for staff review. Technical assistance contracts have been awarded to Franklin Research Center (FRC) and Brookhaven National Laboratory (BNL) to perform an audit review of the licensee's PUA. CSB has lead technical review responsibility for the BNL load audit review and MEB for the FRC structure audit review. Technical Evaluation Reports from the contractors have been received for nine of the 22 plants. Some licensees have submitted Technical Specifications which are being reviewed by the staff.

Actions Remaining:

1. Post implementation audit of PUA by Franklin Institute and BNL on Technical Assistance Contracts.
2. Issuance of SER to each licensee on acceptability of Mark I LTP modifications and analyses for their plant.
3. Technical Specification changes resulting from Mark I modifications.

Status Updated: April 13, 1984

Multi-Plant Action No. D-05

Title: Plant UPI Model Problem

Priority: 3

Cognizant Assistant Director: G. Lainas

Cognizant Branch Chief: S. Varga

Lead Project Manager: S. MacKay

Lead Technical Review Branch: RSB

Problem:

Only two-loop PWRs are affected. All Westinghouse (W) two-loop plants have a Low Pressure Safety Injection System (LPSI) which injects water directly into the Upper Plenum of the Reactor Vessel (UPI). This is used to counter the consequences of a single failure for two-loop units.

However, the currently approved ECCS evaluation models for two-loop plants are based on an ECCS system in which HPSI is directly to the cold leg of the Reactor Coolant System and because of this may be considered as not meeting the requirements of 10 CFR Appendix K Section II.5.

Background:

This has been an outstanding issue for two-loop plants since the implementation of Regulations for an approved ECCS system. The issue became a MPA during late 1977 and 1978 when an "interim" model was developed to review the potential effect of UPI against the conventional model.

Current Status:

Six facilities are affected. W two-loop UPI plants are presently operating on the bases of a number of approaches being taken to determine allowable peaking coefficients in an interim manner.

A letter has been sent to each of the four utilities informing them that a Safety Evaluation will be sent out which documents our efforts and concerns. The letter recommends that the utilities provide a plan of action or respond to our concerns in 90 days. The peaking factor penalty will be reevaluated based on the 90 day response.

Actions Remaining:

A Safety Evaluation will be prepared by RSB by June 30, 1984 for transmittal to the licensees.

Status Updated: April 18, 1984

Multi-Plant Action No. D-10

Title: Asymmetric LOCA Loads On Primary Reactor Coolant System

Priority: Medium

Cognizant Assistant Director: F. Miraglia

Cognizant Branch Chief: D. Crutchfield

Lead Project Manager: J. Shea

Lead Technical Review Branch: MEB, MTEB, CPB, CSB

Problem:

Asymmetric loads resulting from rupture of the primary piping could potentially damage reactor vessel support internals and other primary system components. Damage to the reactor internals could impair core coolability.

Background:

In January 1978, all pressurized water reactor (PWR) licensees were required by the NRC to provide an assessment of the adequacy of their reactor vessel supports and other affected structures to withstand combinations of responses to asymmetric loss-of-coolant accident (LOCA) loads and the Safe Shutdown Earthquake (SSE). This potential safety problem was subsequently identified as an unresolved safety issue Task Action Plan A-2.

Current Status:

All PWR plant assessments for asymmetric loads have been received and are currently being evaluated by staff and contractor EG&G. As a basis for this evaluation, staff is implementing criteria developed as a part of the Task Action Plan A-2. These criteria are included in NUREG-0609. Concurrently some PWR licensees have performed a study of "leak before break". The "leak before break" concept was reviewed by CRGR on September 28, 1983 and received written CRGR approval several days later. Transmissions of the staff findings have been delayed pending determination as to the best way to send the conclusions to the licensees to assure that the concept is only applied to the asymmetric loads issue.

Actions Remaining:

Transmit SER to Westinghouse Owners Group and the affected licensees. Complete the remaining 4 B&W SERs (4 already issued) (2nd qtr. FY84). Complete the 4 CE SERs (4th qtr. FY84). Resume evaluation of 8 remaining non owner group submittals.

Status Updated: December 22, 1983

Multi-Plant Action No. D-15

Title: High Energy Line Break and Consequential System Failure

Priority: 3

Cognizant Assistant Director: G. C. Lainas

Cognizant Branch Chief: S. Varga

Lead Project Manager: M. Grotenhuis

Lead Technical Review Branch: RRAB

Problem:

Certain non-safety grade equipment, if subjected to an adverse environment, could impact the safety analyses and the adequacy of the protection functions performed by safety grade equipment. The NRC is concerned that a potential may exist at operating light water reactor facilities, for an unreviewed safety matter related to the effects of the environment on control systems resulting from high energy line breaks inside or outside containment and the results of these effects on the required safety systems.

Background:

On September 17, 1979, the NRC requested that all LWR licensees review their plants to determine whether control system failures caused by high energy line breaks (HELB) could result in consequences more severe than previously analyzed for the HELB. All licensees responded in a timely manner (20 days). A status report from D. G. Eisenhower to H. Denton dated October 15, 1979 provided a basis for continuing operation. A meeting with representatives of the nuclear industry was held on November 9, 1979. A status report P. S. Check to D. G. Eisenhower dated December 19, 1979 stated that the NRC task group had found no specific identified safety problems. There were remaining concerns which fall within the scope of NUREG-0660 II.C.3

Current Status:

A memo has been prepared which includes instructions to PM's to close the issue.

Actions Required:

The above memo has been concurred in by DSI, DST, & DE and was approved by

D. G. Eisenhut January 31, 1984.

Status Updated: April 19, 1984

Multi-Plant Action No. D-17

Title: Definition of Operable

Priority: Medium

Cognizant Assistant Director: G. C. Lainas

Cognizant Branch Chief: J. F. Stolz

Lead Project Manager: Sydney Miner

Lead Technical Review Branch: None

Problem:

Clarification of the term "OPERABLE" as it applies to support system outages or multiple outages or redundant components.

Background:

In a letter dated April 10, 1980, an explanation of the problem plus accompanying illustrate examples of recommended changes contained in the associated bases, were delineated to all licensees. The licensees were required to submit proposed changes to tech specs applicable to their plant and to implement procedures designed to assure compliance with the specifications containing new requirements.

Current Status:

1. Acceptance Criteria

The acceptance criteria that LCO should be incorporated in the Technical Specification which requires that the plants be in cold shutdown within 36 to 39 hours if redundant ECCS loops become inoperable either due to failures in redundant ECCS loops or the diesel generators.

2. Important Schedule Considerations

The remaining 12 licensing actions are scheduled to be completed this year.

3. Review Responsibilities

This MPA is being handled completely within the Project Management Organization. Ten reviews were assigned to a contractor. All of the reviews have been completed. One final TER remains to be issued.

Actions Required:

Remaining Milestones

- (a) Approval of remaining changes to the Technical Specifications of the remaining plants.
- (b) Issuance of the SER and the License Amendments.

Multi-Plant Action No. E-04

Title: BWR Single Loop Operation

Priority: Low

Cognizant Assistant Director: G. Lainas

Cognizant Branch Chief: D. Vassallo

Lead Project Manager: _____

Lead Technical Review Branch: RSB, CPB

Problem:

The standard BWR Technical Specifications do not permit a plant with two recirculation loops to operate either with one loop or in the natural circulation mode. Fifteen operating BWRs have requested approval to operate at reduced power levels with a single loop if problems develop with a jet pump, valve, seals, flow controller, etc., in the other loop. Most of the potential safety reviews can more appropriately be resolved on a generic basis.

Background:

Single loop operation at limited power levels had previously been authorized for 6 BWRs for short periods of time (one week to two months). During the fall of 1978 while Browns Ferry Unit 1 was operating on a single loop, an increase in instrument noise was noted above 60% power, which could be indicative of flow and flux variations. Based on this operating experience, the staff has concluded that BWRs can safely operate at at least 50% power by natural circulation (i.e., without forced flow from any recirculation loop). Accordingly, as an interim measure, six plants have been authorized to operate, if necessary, at power levels of 50% with a single recirculation loop while the generic concern of flow stability at higher power levels during single loop operation are being resolved.

Current Status:

With respect to operation at power levels above 50% power, a meeting was held October 22, 1981, with all BWR licensees and GE to discuss GE's evaluation of Browns Ferry's and other BWR operating experience with single loop operation. RSB and CPB have prepared a draft list of questions which we discussed in a meeting with GE on January 21, 1982. The issues regarding operation at power levels above 50% are not likely to be resolved in FY83. In the interim, the effort is being directed approving operation at 50% power.

Actions Remaining:

Complete SE inputs by CPB for operation at 50% power and issue revised Technical Specifications.

Status Updated: December 8, 1982

Multi-Plant Action No. E-05

Title: Westinghouse N-1 Loop Operation

Priority: Low

Cognizant Assistant Director: G. C. Lainas

Cognizant Branch Chief: S. Varga

Lead Project Manager: D. Wigginton

Lead Technical Review Branch: RSB

Problem:

All Westinghouse plants have a provision in their license that prohibits operation with less than all loops (N) in service.

Background:

It may be desirable to operate plants with one loop out of service. Present restrictions in the license do not permit this type of operation.

Current Status:

Only three Westinghouse plants, Beaver Valley and Zion (1 and 2), have submitted proposed amendments for N-1 loop operation. Beaver Valley is lead plant and most of the review has been completed. Trojan, a plant without loop isolation valves, has expressed an interest in submitting an application.

Actions Remaining:

NRC to review contractor (Franklin Inst., Lawrence Livermore) inputs, review for core thermodynamic, procedures, initial tests, and write SER on Beaver Valley. Zion review to follow. Other Westinghouse plants will probably submit similar applications if Beaver Valley is approved.

Status Updated: April 30, 1984.

Multi-Plant Action No. G-1

Title: Reactor Coolant Pump Trip

Priority: 1

Cognizant Assistant Director: G. C. Lainas

Cognizant Branch Chief: J. R. Miller

Lead Project Manager: D. H. Jaffe

Lead Technical Review Branch: Reactor Systems Branch

Problem:

The implementation of MPA G-1 involves the selection of a post LOCA RCP operation strategy (auto-trip, manual trip, or no trip) based upon calculations using approved analytic models. This strategy is to be implemented via operator training and procedures.

Background:

The NRC, its licensees, and the PWR vendors have been evaluating the reactor coolant pump (RCP) trip issue since the accident at TMI. The technical understanding of the industry and the requirements of NRC on this issue have changed twice in that period. As a result, there have been extensive studies to better understand the dynamic response of all classes of PWRs to small break LOCAs. Although some confirmatory information is still to be received concerning some models, we conclude that the analytical models are sufficiently reliable to be used by licensees to choose their own best method for RCP operation upon indication that a LOCA has occurred.

Current Status:

Generic Letters 83-10a through 83-10f were issued on February 8, 1983 to licensees and applicants (PWR vendor-specific) providing the NRC's position on reactor coolant pump trip. A 60-day response was requested (50.54f). All responses to GL 83-10 have been received. The members of the CE, Westinghouse and B&W Owners Groups have submitted generic analyses to satisfy the requirements of GL 83-10. We are presently awaiting endorsements of these reports by the Owners Groups together with implementation plans. We are also awaiting plant specific submittals from Yankee Rowe, Maine Yankee, Haddam Neck, and San Onofre Unit 1 facilities.

Actions Remaining:

1. Review generic and plant specific RCP trip calculations.
2. Review licensee's endorsements of Owners Groups' generic reports, including commitments on training and implementation.
3. Issue generic or plant specific SERs with letter indicating approval of training and implementation schedule.

Status Updated: April 13, 1984.

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12b. PERIOD COVERED (Inclusive dates)

April 1 - 30, 1984

13. SUPPLEMENTARY NOTES

14. ABSTRACT (200 words or less)

The Operating Reactors Licensing Actions Summary is designed to provide the Management of the Nuclear Regulatory Commission (NRC) with an overview of licensing actions dealing the operating power and nonpower reactors.

15a. KEY WORDS AND DOCUMENT ANALYSIS

15b. DESCRIPTORS

16. AVAILABILITY STATEMENT

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17. SECURITY CLASSIFICATION (This report)

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