

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Washington Nuclear Plant - Unit 2	DOCKET NUMBER (2) 0 5 0 0 0 3 1 9 1 7	PAGE (3) 1 OF 0 2
--	--	----------------------

TITLE (4)
Reactor Trip

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)																																																										
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)																																																								
0 5	1 9	8 4	8 4	0 4	3 0	0 6	1 4	8 4			0 5 0 0 0																																																								
<table border="1" style="width:100%; border-collapse: collapse;"> <tr> <td style="width:15%;">OPERATING MODE (9)</td> <td style="width:15%;">1</td> <td colspan="10">THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 5: (Check one or more of the following) (11)</td> </tr> <tr> <td rowspan="6">POWER LEVEL (10) 0 1 7</td> <td>20.402(a)</td> <td></td> <td>20.406(e)</td> <td><input checked="" type="checkbox"/></td> <td>50.73(a)(2)(iv)</td> <td></td> <td>73.71(b)</td> <td></td> </tr> <tr> <td>20.406(a)(1)(ii)</td> <td></td> <td>50.38(e)(1)</td> <td></td> <td>50.73(a)(2)(v)</td> <td></td> <td>73.71(e)</td> <td></td> </tr> <tr> <td>20.406(a)(1)(iii)</td> <td></td> <td>50.38(e)(2)</td> <td></td> <td>50.73(a)(2)(vii)</td> <td></td> <td><input checked="" type="checkbox"/></td> <td>OTHER (Specify in Abstract below and in Text, NRC Form 366A)</td> </tr> <tr> <td>20.406(a)(1)(iii)</td> <td></td> <td>50.73(a)(2)(i)</td> <td></td> <td>50.73(a)(2)(viii)(A)</td> <td></td> <td rowspan="3">50.72(b)(2)(ii)</td> </tr> <tr> <td>20.406(a)(1)(iv)</td> <td></td> <td>50.73(a)(2)(ii)</td> <td></td> <td>50.73(a)(2)(viii)(B)</td> <td></td> </tr> <tr> <td>20.406(a)(1)(v)</td> <td></td> <td>50.73(a)(2)(iii)</td> <td></td> <td>50.73(a)(2)(ix)</td> <td></td> </tr> </table>												OPERATING MODE (9)	1	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 5: (Check one or more of the following) (11)										POWER LEVEL (10) 0 1 7	20.402(a)		20.406(e)	<input checked="" type="checkbox"/>	50.73(a)(2)(iv)		73.71(b)		20.406(a)(1)(ii)		50.38(e)(1)		50.73(a)(2)(v)		73.71(e)		20.406(a)(1)(iii)		50.38(e)(2)		50.73(a)(2)(vii)		<input checked="" type="checkbox"/>	OTHER (Specify in Abstract below and in Text, NRC Form 366A)	20.406(a)(1)(iii)		50.73(a)(2)(i)		50.73(a)(2)(viii)(A)		50.72(b)(2)(ii)	20.406(a)(1)(iv)		50.73(a)(2)(ii)		50.73(a)(2)(viii)(B)		20.406(a)(1)(v)		50.73(a)(2)(iii)		50.73(a)(2)(ix)	
OPERATING MODE (9)	1	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 5: (Check one or more of the following) (11)																																																																	
POWER LEVEL (10) 0 1 7	20.402(a)		20.406(e)	<input checked="" type="checkbox"/>	50.73(a)(2)(iv)		73.71(b)																																																												
	20.406(a)(1)(ii)		50.38(e)(1)		50.73(a)(2)(v)		73.71(e)																																																												
	20.406(a)(1)(iii)		50.38(e)(2)		50.73(a)(2)(vii)		<input checked="" type="checkbox"/>	OTHER (Specify in Abstract below and in Text, NRC Form 366A)																																																											
	20.406(a)(1)(iii)		50.73(a)(2)(i)		50.73(a)(2)(viii)(A)		50.72(b)(2)(ii)																																																												
	20.406(a)(1)(iv)		50.73(a)(2)(ii)		50.73(a)(2)(viii)(B)																																																														
	20.406(a)(1)(v)		50.73(a)(2)(iii)		50.73(a)(2)(ix)																																																														

LICENSEE CONTACT FOR THIS LER (12)

NAME L.D. Kassakatis, Plant Compliance Engineer	TELEPHONE NUMBER 5 1 0 1 9 3 1 7 7 1 - 1 2 5 1 0 1
--	---

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13) Ext. 2201

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC
D	J C			N					

SUPPLEMENTAL REPORT EXPECTED (14)

YES (if yes, complete EXPECTED SUBMISSION DATE) NO

EXPECTED SUBMISSION DATE (15)

MONTH	DAY	YEAR

ABSTRACT (Limit to 1400 spaces - i.e. approximately fifteen single-space typewritten lines) (16)

During operation at 17% power normal surveillance test of primary containment pressure monitors 'A' and 'C' was being performed. Upon pressurization of monitor 'A' the isolation signal and load shedding logic was initiated. This resulted in loss of control power to the Reactor Feedwater Level Control System. This caused the operating Reactor feedpump turbine to run to minimum speed. The subsequent decrease in Reactor level caused a Reactor Protection System (RPS) trip. After investigation it was found that the half isolation signal received from the previously completed surveillance test of primary containment pressure monitors 'B' and 'D' had not been reset. The procedures have been revised to include the reset of the isolation signal after the testing of each monitor.

IEZ
11

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

FACILITY NAME (1) Washington Nuclear Plant - Unit 2	DOCKET NUMBER (2) 0 5 0 0 0 3 9 7 8 4	LER NUMBER (8)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
		84	043	010	0	2	OF

TEXT (If more space is required, use additional NRC Form 305A's) (17)

Plant Operating Conditions - Prior to the Event

Power Level - 17%
Plant Mode - 1

Event

Surveillance test procedure for the channel functional test of primary containment pressure monitors 'B' and 'D' was performed and completed at 1800 hrs. on May 19, 1984. This test causes a half trip of the Reactor Protection System, a half trip of the Primary and Secondary Containment Isolation Systems, and a half trip of the Load Shedding Logic. The procedure called for reset of the half trip from the Reactor Protection System but did not reset the isolation signal and load shedding logic.

Surveillance test procedure for the channel functional test of primary containment pressure monitors 'A' and 'C' was started. Upon pressurization of monitor 'A' the one out of two twice logic was completed to cause isolation and load shedding initiation of non-essential, non-class 1E loads. This caused loss of control power to the Reactor Feedwater Level Control System causing the Reactor feedwater startup valve to fail open and the operating Reactor feedwater pump turbine to go to minimum speed. The subsequent decrease in Reactor level caused a Reactor Protection System trip at 1910 hours on May 19, 1984. The NRC was notified at 2225 hours on May 19, 1984 pursuant to 10CFR50.72(b)(2)(ii).

Safety Significance

Personnel involved in the incident followed the procedures as written. Investigation following the event identified the procedural deficiency. There were no safety consequences associated with this event and all Plant systems performed as required during the event.

Corrective Actions

The procedures have been revised to include reset of the isolation signal after testing of each monitor.

Washington Public Power Supply System

P.O. Box 968 3000 George Washington Way Richland, Washington 99352 (509)372-5000

Docket No. 50-397
June 14, 1984

Document Control Desk
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

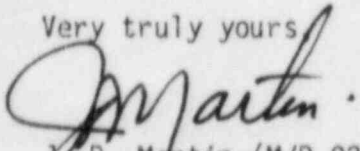
Subject: NUCLEAR PLANT NO. 2
LICENSEE EVENT REPORT NO. 84-043

Dear Sir:

Transmitted herewith is Licensee Event Report No. 84-043 for WNP-2 Plant. This report is submitted in response to the report requirements of Technical Specification Section 6.9.1.7 and discusses the item of reportability, corrective action taken, and action taken to preclude recurrence.

This is the follow-up report to the verbal notification given at 2225 hours on May 19, 1984.

Very truly yours



J. D. Martin (M/D 927M)
WNP-2 Plant Manager

JDM:mm

Enclosure:
Licensee Event Report No. 84-043

cc: Mr. John B. Martin, Administrator
Region V, Office of Inspection and Enforcement
U.S. Nuclear Regulatory Commission
1450 Maria Lane
Walnut Creek, California 94596
Mr. A. D. Toth, NRC Resident Inspector (901A)
Ms. Dottie Sherman
American Nuclear Insurers
The Exchange Suite 245
270 Farmington Ave.
Farmington, CT 06032

LE22
1/1