



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

U.S. GEOLOGICAL SURVEY, DEPARTMENT OF THE INTERIOR (USGS)

DOCKET NO. 50-274

AMENDMENT TO FACILITY LICENSE

Amendment No. 7
License No. R-113

1. The U.S. Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment filed by the United States Geological Survey (USGS or the licensee), dated October 16, 1991, as supplemented on November 20, 1991, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations as set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public;
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied; and
 - F. Prior notice of the amendment was not required by 10 CFR 2.105(a)(4) and publication of the amendment is not required by 10 CFR 2.106(a)(2).

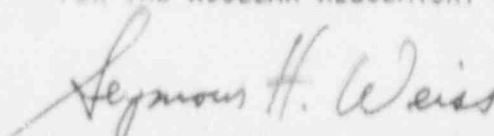
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the enclosure to this license amendment, and paragraph 3.B. of License No. R-113 is hereby amended to read as follows:

B. Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 7, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance

FOR THE NUCLEAR REGULATORY COMMISSION



Seymour H. Weiss, Director
Non-Power Reactors, Decommissioning and
Environmental Project Directorate
Division of Advanced Reactors
and Special Projects
Office of Nuclear Reactor Regulation

Enclosure:
Appendix A Technical
Specifications Changes

Date of Issuance: January 30, 1992

ENCLOSURE TO LICENSE AMENDMENT NO. 7

FACILITY LICENSE NO. R-113

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Replace the following page of the Appendix A Technical Specifications with the enclosed page. The revised page is identified by amendment number and contains a vertical line indicating the area of change.

Remove Page

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Insert Page

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3. Written instructions shall be in effect and followed for:
 - a. Testing and calibration of reactor operating instrumentation and control systems, control rod drives, area radiation monitors and air particulate monitors.
 - b. Reactor startup, routine operation and reactor shutdown.
 - c. Emergency and abnormal conditions, including evacuation, reentry and recovery.
 - d. Fuel loading or unloading.
 - e. Control rod removal and replacement.
 - f. Maintenance operations which may affect reactor safety.
 4. Any additions, modifications, or maintenance to the core and its associated support structure, the pool structure, and rod drive mechanisms, or the reactor safety system, shall be made and tested in accordance with the specifications to which the systems or components were originally designed and fabricated, or to specifications approved by the Reactor Operations Committee as suitable and not involving an unreviewed safety question. The reactor shall not be placed in operation until the affected system has been verified to be operable.
 5. The reactor facility emergency plan, emergency procedures and physical security plan shall be audited by the Reactor Operations Committee at least biennially, with the interval not to exceed 30 months.
- I. Experiments
1. Prior to performing any new reactor experiment, the proposed experiment shall be evaluated by a person or persons appointed by the Reactor Administrator to be responsible for reactor safety. He shall consider the experiment in terms of its effect on reactor operation