

UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D. C. 20555

THE TOLEDO EDISON COMPANY

AND

THE CLEVELAND ELECTRIC ILLUMINATING COMPANY

DOCKET NO. 50-346

DAVIS-BESSE NUCLEAR POWER STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No.68 License No. NPF-3

- 1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by The Toledo Edison Company and The Cleveland Electric Illuminating Company (the licensees) dated November 21, 1983, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this license amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
- Accordingly, Facility Operating License No. NPF-3 is hereby amended as indicated below and by changes to the Technical Specifications as indicated in the attachment to this license amendment:

Revise paragraph 2.C.(2) to read as follows:

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Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 68, are hereby incorporated in the license. The Toledo Edison Company shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

George W Ruenbach

George W. Rivenbark, Acting Chief Operating Reactors Branch #4 Division of Licensing

Attachment: Changes to the Technical Specifications

Date of Issuance: May 30, 1984

ATTACHMENT TO LICENSE AMENDMENT NO. 68

FACILITY OPERATING LICENSE NO. NPF-3

DOCKET NO. 50-346

Replace the following page of the Appendix "A" Technical Specifications with the enclosed page as indicated. The revised page is identified by Amendment number and contains a vertical line indicating the area of change. The corresponding overleaf page is also provided to maintain document completeness.

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TABLE 3.3-10

POST-ACCIDENT NONITORING INSTRUMENTATION

INSTRUMENT		MINIMUM CHANNELS OPERABLE
١.	SG Outlet Steam Pressure	1/Steam Generator
2.	RC Loop Outlet Temperature	2/Loop
3.	RC Loop Pressure	2/Loop
4.	Pressurizer Level	2
5.	SG Startup Range Level	2/Steam Generator
6.	Auxiliary Feedwater Status	1/AFW System
7.	Containment Vessel Hydrogen	2
8.	Containment Vessel Post-Accident Radiation	2
9.	Containment Vessel Isolation Status	1/Valve
10.	SFAS Status	1/Channel
11.	Safety Features Equipment Status	1/System
12.	RPS Status	
13.	SFRCS Status	1/Channe1
		1/Channe1
14.	High Pressure Injection Flow	1/Channel

DAVIS-BESSE, UNIT 1

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TABLE 3.3-10 (Continued)

POST-ACCIDENT MONITORING INSTRUMENTATION

INSTRUMENT		CHANNELS
		OPI RABLE
	i. Low Pressure Injection (DHR) Flow	1/Channel
16	. HPI System Pump and Valve Status	
17.	. LPI System Pump and Valve Status	1/System
18.		1/System
19.		1/System
20.		1/System
		1/System
21.	. Containment Emergency Sump Valve Status	1/Valve
22.	. Containment Air Recirculation Fan Status	
23.	. Containment Air Cooling Fan Status	1/Fan
24.		1/Fan
25.		1/System
	reconduct flow Rate	2/Stoom Comment
26.	. RC System Subcooling Margin Monitor	2/Steam Generator
27.	. PORV Position Indicator	1
28.	PORV Block Valve Position Indicator	1
29.		1
30.		1/Valve
		3