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OPERATING DATA REPORT

DOCKET NO.	50-416
DATE	6/15/84
COMPLETED BY	J. G. Cesare
TELEPHONE	601-969-2585

OPERATING STATUS

1 Unit Name Grand Gulf N	uclear Station	Notes
2. Reporting Period: May 1984		
3. Licensed Thermal Power (MWt):	191	
4. Nameplate Rating (Gross MWe):	1372.5	
5. Design Electrical Rating (Net MW	(e): <u>1250</u>	
6. Maximum Dependable Capacity (Gross MWe): NA	
7. Maximum Dependable Capacity (Net MWet: NA	

8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report. Give Reasons:

NA

9. Power Level To Whicl. Restricted, If Any (Net MWe): ____NA__

10. Reasons For Restrictions. If Any: _____as restricted by O.L. NPF-13 as listed in 3.0 above

This Month	Yrto-Date	Cumulative
744	3,648	6,576
ritical 554.8	748.8	1744.1
0	0	0
0	0	0
0	0	0
(MWH) 77,280	97,951	175,339
d (MWH) 0	0	0
(MWH) 0	0	0
NA	NA	NA
NA	NA	NA
(Net) NA	NA	NA
(Net) NA	NA	NA
NA	NA	NA
	This Month 744 ritical 554.8 0 0 0 0 0 0 0 0 0 0 0 0 0 0 0 0 0 0 0 0 (MWH) 0 0 NA NA NA NA NA NA NA	This Month Yrto-Date 744 3,648 554.8 748.8 0 0 0 NA

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):

NA

25. If Shut Down At End Of Report Period, Estimated Date of Startup:26. Units In Test Status (Prior to Commercial Operation):

NA Forecast

NITIAL CRITICALITY		8/18/8
SITIAL ELECTRICITY	8/84	
OMMERCIAL OPERATION	2/85	

8406200239 84053 PDR ADOCK

11 II

(9/77)

2

Achieved

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INSTRUCTIONS FOR COMPLETING OPERATING DATA REPORT

This report should be furnished each month by licensees. The name and telephone number of the preparer should be provided in the designated spaces. The instructions below are provided to assist licensees in reporting the data consistently. The number of the instruction corresponds to the item number of the report format.

- 1. UNIT NAME. Self-explanatory.
- 2. REPORTING PERIOD. Designate the month for which the data are presented.
- LICENSED THERMAL POWER (MWt) is the maximum thermal power, expressed in megawatts, currently authorized by the Nuclear Regulatory Commission.
- NAMEPLATE RATING (GROSS MW_e). The nameplate power designation of the turbine-generator in megavolt amperes (MVA) times the nameplate power factor of the turbine generator.
- DESIGN ELECTRICAL RATING (NET MW_e) is the nominal net electrical output of the unit specified by the utility and used for the purpose of plant design.
- MAXIMUM DEPENDABLE CAPACITY (GROSS MW_e) is the gross electrical output as measured at the output terminals of the turbine-generator during the most restrictive seasonal conditions.
- MAXIMUM DEPENDABLE CAPACITY (NET MW_e). Maximum dependable capacity (gross) less the normal station service loads.
- 8. Self-explanatory.
- 9. POWER LEVEL TO WHICH RESTRICTED, IF ANY (NET MW_e). Note that this item is applicable only if restrictions on the power level are in effect. Short-term (less than one month) limitations on power level need not be presented in this item.

Since this information is used to develop figures on capacity lost due to restrictions and because most users of the "Operating Plant Status Report" are primarily interested in energy actually fed to the distribution system, it is requested that this figure be expressed in MWe-Net in spite of the fact that the figure must be derived from MWt or percent power.

- 10. REASONS FOR RESTRICTIONS. IF ANY. If item 9 is used, item 10 should explain why. Brief narrative is acceptable. Cite references as appropriate. Indicate whether restrictions are self-imposed or are regulatory requirements. Be as specific as possible within space limitations. Plants in startup and power ascension test phase should be identified here.
- HOURS IN REPORTING PERIOD. For units in power ascension at the end of the period, the gross hours from the beginning of the period or the first electrical production, whichever comes last, to the end of the period.

For units in commercial operation at the end of the period, the gross hours from the beginning of the period or of commercial operation, whichever comes last, to the end of the period or decommissioning, whichever comes first. Adjustments in clock hours should be made in which a change from standard to daylight-savings time (or vice versa) occurs.

- NUMBER OF HOURS REACTOR WAS CRITICAL. Show the total number of hours the reactor was critical during the gross hours of the reporting period.
- REACTOR RESERVE SHUTDOWN HOURS. The total number of hours during the gross hours of reporting period that the reactor was removed from service for administrative or other reasons but was available for operation.
- 14. HOURS GENERATOR ON-LINE. Also called Service Hours. The total number of hours expressed to the nearest tenth of an hour during the gross hours of the reporting period that the unit operated with breakers closed to the station bus. These hours, plus those listed in Unit Shutdowns for the generator outage hours, should equal the gross hours in the reporting period.
- 15. UNIT RESERVE SHUTDOWN HOURS. The total number of hours expressed to the nearest tenth of an hour during the gross hours of the reporting period that the unit was removed from service for economic or similar reasons but was available for operation.
- 16. GROSS THERMAL ENERGY GENERATED (MWH). The thermal output of the nuclear steam supply system during the gross hours of the reporting period, expressed in megawatt hours (no decimals).
- GROSS ELECTRICAL ENERGY GENERATED (MWH). The electrical output of the unit measured at the output terminals of the turbine-generator during the gross hours of the reporting period, expressed in megawatt hours (no decimals).
- 18. NET ELECTRICAL ENERGY GENERATED (MWH). The gross electrical output of the unit measured at the output terminals of the turbine-generator minus the normal station service loads during the gross hours of the reporting period, expressed in megawatt hours. Negative quantities should not be used. If there is no net positive value for the period, enter zero (no decimals).
- 19- For units still in the startup and power ascension test 23. phase, items 19-23 should not be computed. Instead, enter N/A in the current month column. These five factors should be computed starting at the time the unit is declared to be in commercial operation. The cumulative figures in the second and third columns should be based on commercial operation as a starting date.

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- 19. UNIT SERVICE FACTOR. Compute by dividing hours the generator was on line (item 14) by the gross hours in the reporting period (item 11). Express as percent to the nearest tenth of a percent. Do not include reserve shutdown hours in the calculation.
- 20. UNIT AVAILABILITY FACTOR. Compute by dividing the unit available hours (item 14 plus item 15) by the gross hours in the reporting period (item 11). Express as percent to the nearest tenth of a percent.
- 21. UNIT CAPACITY FACTOR (USING MDC NET). Compute by dividing net electrical energy generated (item 18) by the product of maximum dependable capacity (item 7) times the gross hours in the reporting period (item 11). Express as percent to the nearest tenth of a percent.
- 22. UNIT CAPACITY FACTOR (USING DER NET). Compute as in item 21. substituting design electrical rating (item 5) for maximum dependable capacity.
- 23. UNIT FORCED OUTAGE RATE. Compute by dividing the total forced outage hours (from the table in Unit Shutdowns and Power Reductions) by the sum of hours generator on line (item 14) plus total forced outage hours (from the table in Unit Shutdowns and Power Reductions). Express as percent to the nearest tenth of a percent.
- 24. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH). Include type (refueling, maintenance, other), proposed date of start of shutdown, and proposed length of shutdown. It is recognized that shutdowns may be scheduled between reports and that this item may not be all inclusive. Be as accurate as possible as of the date the report is prepared. This item is to be prepared each month and updated if appropriate until the actual shutdown occurs.
- 25. Self-explanatory.
- 26. Self-explanatory. Note, however, that this information is requested for all units in startup and power ascension test status and is not required for units already in commercial operation.

TEST STATUS is defined as that period following initial criticality during which the unit is tested at successively higher outputs, culminating with operation at full power for a sustained period and completion of warranty runs. Following this phase, the unit is generally considered by the utility to be available for commercial operation.

Date of COMMERCIAL OPERATION is defined as the date that the unit was declared by the utility owner to be available for the regular production of electricity, usually related to the satisfactory completion of qualification tests as specified in the purchase contract and to the accounting policies and practices of the utility.

AVERAGE DAILY UNIT POWER LEVEL

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DOCKET NO.	50-416
UNIT	_1
DATE	6/15/84
COMPLETED BY	J. G. Cesare
TELEPHONE	601-969-2585

AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
NA	17	
	18	
	19	-
	20	
	21	
	22	
	23	
	24	
	25	
	26	
	27	
	28	
	29	
	30	
	31	

INSTRUCTIONS

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO.	50-416
UNIT NAME	1
DATE	6/15/84
COMPLETED BY	J. G. Cesare
TELEPHONE	601-969-2585

Attachment to AECM-84/0331 Page 5 of 8

REPORT MONTH May

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	Licensee Event Report #	System Cude ⁴	Component Code ⁵	Cause & Corrective Action to Prevent Recurrence
5	5/2/84	F	85.8	A	1	84-024	во	PSF PSP SPT	The plant was shutdown as required by Technical Specification 3.6.3.2 due to the inoperability of both Contain- ment Spray loops of the Residual Heat Removal (RHR) System. The RHR "B" loop was declared inoperable on April 30 due to 2 cracks found in a 3 inch diameter branch pipe off the main RHR "B" loop header. Pipe support deficiencies found on May 2 resulted in both RHR loops being declared inoperable. The pipe supports were reworked or redesigned. The 3 inch pipe was disconnected and the branch connection capped as a temporary solution to restore the "B" loop.
1 F: Fc S: Sc (9/77)	rced heduled	2 Reas A-Eq B-Ma C-Re D-Re E-Op F-Ad G-Op H-Ot	on: uipment Fa intenance o fueling gulatory Re erator Train ministrativo erational E her (Explai	nilure (E or Test estriction ning & L e rror (Ex n)	xplain) n .icense Exa .plain)	mination	Method I-Manu 2-Manu 3-Auto 4-Othe	d: ial ial Scram. imatic Scram. r (Explain)	4 Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG- 0161) 5 Exhibit I - Same Source

UNIT SHUTDOWNS AND POWER REDUCTIONS

COMPLETED BY TELEPHONE

DOCKET NO 50-416	
UNIT NAME 1	•
DATE _6/15/84	
MPLETED BY J. G. Cesar	e
TELEPHONE _601-969-258	5

REPORT MONTH May

No.	Date	Type ¹	Duration (Hours)	Reason 2	Method of Shutting Down Reactor ³	Licensee Event Report #	System Cude ⁴	Component Cude ⁵	Cause & Corrective Action to Prevent Recurrence	
6	5/8/84	F	77.6	A	1	84-028	AD	HCU	During adverse weather, a loss of 500 KV offsite power supply occurred which caused both Reactor Recirc pumps to trip. Recirc pump "A" was restarted. The "B" loop could not be restored due to problems with the flow control valves' hydraulic power unit (HPU). A shutdown was, there- fore, completed as required by Tech- nical Specification 3.4.1.1. One subloop of the HPU had become air bound; the other subloop had its operating pressure set incorrectly due to drift of the pressure gauge used in setting the pressure. The gauge was recalibrated and the operating pressure set correctly.	
1 • F: Fo S: Sct	rced heduled	Reass A-Eq B-Ma C-Re D-Re E-Op F-Ad G-Op H-Ot	on: uipment Fa iintenance o fueling gulatory Re erator Train ministrative erational E her (Explain	illure (E or Test estrictio ning & l ror (Ex n)	xplain) n License Exa (plain)	mination	3 Method 1-Manu 2-Manu 3-Auto 4-Othe	l: al sal Scram. matic Scram. r (Explain)	4 Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG- 0161) 5 Exhibit I - Same Source	Accondent to AECM-84/0331 Page 6 of 8

UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. _50-416 UNIT NAME DATE 6/15/84 COMPLETED BY _I. G. Cesare TELEPHONE _601-969-2585

REPORT MONTH May

No.	Date	Typel	Duration (Hours)	Reason?	Method of Shutting Down Reactor 3	Licensee Event Report #	System Code ⁴	Component Cude ⁵	Cause & Corrective Action to Prevent Recurrence
7	5/25/84	F	25.8	A	3	84-030	SD BN	FCV	The Reactor scrammed on low water level 3. The condensate booster pumps and feedwater pumps tripped when a broken position arm linkage on the condensate minimum flow control valve allowed the valve to open creating a recirculation path to the condenser, bypassing the booster pumps. Operators attempted to start RCIC manually to restore the level but the RCIC turbine tripped on over- speed. The scram occurred and RCIC was restarted to restore level. A mechanical restrictor was placed on the RCIC turbine governor valve to limit steam flow and prevent the resulting overspeed trip. The control valve positioner arm was replaced.
. F: Fc S: Sc	nced heduled	Reas A-Ec B-Mi C-Re D-Ri E-Oj F-Ac G-O H-O	son: quipment F aintenance efueling egulatory R perator Trai dministrativ perational I ther (Expla	ailure (I or Test testriction ining & re Error (E in)	Explain) on License Exa xplain)	imination	3 Metho 1-Man 2-Man 3-Auto 4-Oth	d: ual ual Scram. omatic Scram. er (Explain)	4 Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG- 0161) 5 Exhibit 1 - Same Source

INSTRUCTIONS

This report should describe all plant shutdowns during the report period. In addition, it should be the source of explanation of significant dips in average power levels. Each significant reduction in power level (greater than 20% reduction in average daily power level for the preceding 24 hours) should be noted, even though the unit may not have been shut down completely¹. For such reductions in power level, the duration should be listed as zero, the method of reduction should be listed as 4 (Other), and the Cause and Corrective Action to Prevent Recurrence column should explain. The Cause and Corrective Action to Prevent Recurrence column should be used to provide any needed explanation to fully describe the circumstances of the outage or power reduction.

NUMBER. This column should indicate the sequential number assigned to each shutdown or significant reduction in power for that calendar year. When a shutdown or significant power reduction begins in one report period and ends in another, an entry should be made for both report periods to be sure all shutdowns or significant power reductions are reported. Until a unit has achieved its first power generation, no number should be assigned to each entry.

DATE. This column should indicate the date of the start of each shutdown or significant power reduction. Report as year, month, and day. August 14, 1977 would be reported as 770814. When a shutdown or significant power reduction begins in one report period and ends in another, an entry should be made for both report periods to be sure all shutdowns or significant power reductions are reported.

TYPE. Use "F" or "S" to indicate either "Forced" or "Scheduled," respectively, for each shutdown or significant power reduction. Forced shutdowns include those required to be initiated by no later than the weekend following discovery of an off-normal condition. It is recognized that some judgment is required in categorizing shutdowns in this way. In general, a forced shutdown is one that would not have been completed in the absence of the condition for which corrective action was taken.

DURATION Self-explanatory. When a shutdown extends beyond the end of a report period, count only the time to the end of the report period and pick up the ensuing down time in the following report periods. Report duration of outages rounded to the nearest tenth of an hour to facilitate summation. The sum of the total outage hours plus the hours the generator was on line should equal the gross hours in the reporting period.

REASON. Categorize by letter designation in accordance with the table appearing on the report form. If category H must be used, supply brief comments.

METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER. Categorize by number designation Attachment to AECM-84/0331 Page 8 of 8

in accordance with the table appearing on the report form. If category 4 must be used, supply brief comments.

LICENSEE EVENT REPORT #. Reference the applicable reportable occurrence pertaining to the outage or power reduction. Enter the first four parts (event year, sequential report number, occurrence code and report type) of the five part designation as described in Item 17 of Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161). This information may not be immediately evident for all such shutdowns, of course, since further investigation may be required to ascertain whether or not a reportable occurrence was involved.) If the outage or power reduction will not result in a reportable occurrence. the positive indication of this lack of correlation should be noted as not applicable (N/A).

SYSTEM CODE. The system in which the outage or power reduction originated should be noted by the two digit code of Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161).

Systems that do not fit any existing code should be designated XX. The code ZZ should be used for those events where a system is not applicable.

COMPONE. TCODE. Select the most appropriate component from Exhibit I - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161). using the following critieria:

- A. If a component failed, use the component directly involved.
- B. If not a component failure, use the related component: e.g., wrong valve operated through error: list valve as component.
- C. If a chain of failures occurs, the first component to malfunction should be listed. The sequence of events, including the other components which fail, should be described under the Cause and Corrective Action to Prevent Recurrence column.

Components that do not fit any existing code should be designated XXXXXX. The code ZZZZZZ should be used for events where a component designation is not applicable.

CAUSE & CORRECTIVE ACTION TO PREVENT RECUR-RENCE. Use the column in a narrative fashion to amplify or explain the circumstances of the shutdown or power reduction. The column should include the specific cause for each shutdown or significant power reduction and the immediate and contemplated long term corrective action taken, if appropriate. This column should also be used for a description of the major safety-related corrective maintenance performed during the outage or power reduction including an identification of the critical path activity and a report of any single release of radioactivity or single radiation exposure specifically associated with the outage which accounts for more than 10 percent of the allowable annual values.

For long textual reports continue narrative on separate paper and reference the shutdown or power reduction for this narrative

¹Note that this differs from the Edison Electric Institute (EEI) definitions of "Forced Partial Outage" and "Scheduled Partial Outage." For these terms, EEI uses a change of 30 MW as the break point. For larger power reactors, 30 MW is too small a change to warrant explanation.



MISSISSIPPI POWER & LIGHT COMPANY Helping Build Mississippi P. D. BOX 1640, JACKSON, MISSISSIPPI 39205

June 15, 1984

NUCLEAR LICENSING & SAFETY DEPARTMENT

Mr. Richard C. DeYoung, Director Office of Inspection and Enforcement U.S. Nuclear Regulatory Commission Washington, D.C. 20555

Attention: Document Control Desk

Dear Mr. DeYoung:

SUBJECT: Grand Gulf Nuclear Station Unit 1 Docket No. 50-416 License No. NPF-13 File 0260/L-835.0 Monthly Operating Report AECM-84/0331

In accordance with 10CFR50.36, Mississippi Power & Light Company (MP&L) is providing twelve copies of the Monthly Operating Report for Grand Gulf Nuclear Station Unit 1 for May, 1984 (attachment).

If you have any questions or require additional information, please contact this office.

Yours truly,

F. Dale

Director of Nuclear Licensing & Safety

GWD/JGC:1m Attachment

cc: Mr. J. B. Richard (w/a)
Mr. R. B. McGehee (w/o)
Mr. N. S. Reynolds (w/o)
Mr. G. B. Taylor (w/o)

Mr. J. P. O'Reilly, Regional Administrator (w/a)
U.S. Nuclear Regulatory Commission
Region II
101 Marietta St., N.W., Suite 2900
Atlanta, Georgia 30323

Chief (w/2) Office of Resource Management U.S. Nuclear Regulatory Commission Washington, D.C. 20555



AJ1