Mr. John F. Opeka Executive Vice President - Nuclear Northeast Nuclear Energy Company P. O. Box 270 Hartford, CT 06141-0270

SUBJECT: MILLSTONE 2 CORE RELOAD INSPECTION 95-27

Dear Mr. Opeka:

This refers to the inspection of your reload design and engineering activities, conducted by Mr. Daniel Moy and Ms. Brenda Whitacre of this office from July 10 to 14, 1995, at Millstone Unit 2 in Waterford, Connecticut. The inspection was focused on issues important to public health and safety, specifically evaluating reload calculations and safety analyses. At the conclusion of the inspection, the preliminary findings were discussed with Mr. John Riley, Jr., and other members of your staff.

During this inspection of your reload engineering activities, the inspectors found no documentation supporting the performance of independent reviews of reload safety analyses for Cycles 10, 11, 12 and current Cycle 13, by your engineering staff. The documentation of these formal reviews is apparently required by your engineering procedures governing design review, although these procedures are somewhat ambiguous. The lack of documentation has limited safety consequences because other reviews had been accomplished which included safety evaluations and past quality audits. However, this finding has regulatory significance because of the importance in adhering to established procedures and because of its similarity to the problems with the control of purchased services and the acceptance of vendor calculations identified in NRC Inspection Report 50-336/95-08. We believe that the corrective actions discussed in your June 27, 1995 letter should also apply to the finding in this report. Accordingly, we have left this issue as unresolved pending your implementation of short-term actions to ensure that appropriate independent design review was accomplished for Cycle 13 calculations, and your review of the corrective actions documented in your June 27, 1995 letter to assess their applicability to reviews of core reload analyses. This understanding was confirmed in a telephone conversation with engineering representatives on August 30, 1995, and with Mr. R. Kacich on September 5, 1995.

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Your cooperation in these matters is appreciated. No reply to this letter is requested, and we would be pleased to discuss any questions you have concerning this inspection.

Sincerely,

Eugene M. Kelly, Chief Systems Section Division of Reactor Safety

Docket No. 50-336

Enclosure: NRC Inspection Report No. 50-336/95-27

cc w/encl:

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