



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

February 5, 1992

Docket Nos. 50-259, 50-260  
and 50-296

Mr. Dan A. Nauman  
Senior Vice President, Nuclear Power  
Tennessee Valley Authority  
6N 38A Lookout Place  
1101 Market Street  
Chattanooga, Tennessee 37402-2801

Dear Mr. Nauman:

SUBJECT: SAFETY PARAMETER DISPLAY SYSTEM - BROWNS FERRY NUCLEAR PLANT,  
UNITS 1, 2, AND 3 (TAC NOS. M51223, M51224, M51225, M73634,  
M73635, M73636, M74601, M74602, M74606, M74607, M74611, AND 74612)

By letter dated October 22, 1990, the Tennessee Valley Authority (TVA) submitted for NRC staff review its detailed design description of the Safety Parameter Display System (SPDS) for the Browns Ferry Nuclear Plant (BFNP). On March 6, 1991, the staff issued a safety evaluation (SE) regarding TVA's submittal. This SE concluded that TVA's SPDS design was generally acceptable; however, TVA's design did not fully meet all of the requirements in NUREG-0737, Supplement 1. The staff's SE identified several open issues to be resolved and requested TVA to address each one. TVA responded to the staff's request by letter dated December 17, 1991.

The NRC staff has completed its safety evaluation of TVA's SPDS design for BFNP, as documented in the enclosed supplement to our previous SE. Based on the results from these safety evaluations, the staff can now conclude that TVA's SPDS design description for BFNP fully meets the requirements of NUREG-0737, Supplement 1.

With regard to Generic Letter (GL) 89-06, "Task Action Plan Item I.D.2 - Safety Parameter Display System," TVA has committed to certify its compliance with the requirements of NUREG-0737, Supplement 1 within two months after the SPDS for each unit is fully operational. Furthermore, TVA committed to make the SPDS operational following restart for BFNP, Units 1 and 3, and

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Mr. Dan A. Nauman

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after restart from the Cycle 6 outage for BFNP Unit 2. The staff considers TVA's commitments and schedules acceptable for implementing the SPDS at each BFNP unit in accordance with the requirements of NUREG-0737, Supplement 1 and the guidelines of GL 89-06.

Sincerely,



Thierry M. Ross, Senior Project Manager  
Project Directorate II-4  
Division of Reactor Projects I/II  
Office of Nuclear Reactor Regulation

Enclosure:  
Safety Evaluation

cc w/enclosure:  
See next page

OFC	PD II-4LA	PD II-4PM	PD II-4PM	PD II-4PD
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