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VIRGINIA ELECTRIC AND POWER COMPANY
NORTH ANNA POWER STATION
P. O. BOX 402
MINERAL, VIRGINIA 25117

10 CFR 50.73

February 5, 1992

U. S. Nuclear Regulatory Commission
Attention: Document Control Desk
Washington, D.C. 20555

Serial No. N-92-002
NAPS:JHL
Docket No. 50-338
License No. NPF-4

Dear Sirs:

The Virginia Electric and Power Company hereby submits the following Licensee Event Report applicable to North Anna Unit No. 1.

Report No. 50-338/92-002-00

This Report has been reviewed by the Station Nuclear Safety and Operating Committee and will be forwarded to the Corporate Management Safety Review Committee for its review.

Very Truly Yours,


G. E. Kane
Station Manager

Enclosure:

cc: U.S. Nuclear Regulatory Commission
101 Marietta Street, N.W.
Suite 2900
Atlanta, Georgia 30323

Mr. M. S. Lesser
NRC Senior Resident Inspector
North Anna Power Station

9202100278 920205
PDR ADOCK 05000338
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JEZ

LICENSEE EVENT REPORT (LER)

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS AND REPORTS MANAGEMENT BRANCH (P-620), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1) NORTH ANNA POWER STATION UNIT 1		DOCKET NUMBER (2) 05000338	PAGE (3) 1 OF 4
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TITLE (4)
PRESSURIZER SAFETY VALVE SETPOINT OUT OF TOLERANCE DUE TO SETPOINT DRIFT

EVENT DATE (5)			LER NUMBER (6)		REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES
01	10	92	92	002	00	02	05	92	
									DOCKET NUMBER(S) 05000000
									DOCKET NUMBER(S) 05000000

OPERATING MODE (9) 5

POWER LEVEL (10) 000

THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 50. (Check one or more of the following) (11)

20.402(b)	20.409(d)	50.73(a)(2)(iv)	73.71(b)
20.425(a)(1)(i)	50.36(c)(1)	50.73(a)(2)(v)	73.71(c)
20.409(a)(1)(b)	50.36(c)(2)	50.73(a)(2)(vi)	OTHER (Specify in column below and in Tab. NRC Form 896)
20.409(a)(1)(c)	<input checked="" type="checkbox"/> 50.73(a)(2)(i)	50.73(a)(2)(vii)(A)	
20.409(a)(1)(d)	50.73(a)(2)(ii)	50.73(a)(2)(vii)(B)	
20.409(a)(1)(e)	50.73(a)(2)(iii)	50.73(a)(2)(x)	

LICENSEE CONTACT FOR THIS LER (12)

NAME: G. E. [unclear], Station Manager

TELEPHONE NUMBER: 703 894-2101

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC
X	A	B	R	V					
			D	2	4	3			Y

SUPPLEMENTAL REPORT EXPECTED (14)

YES (if you complete EXPECTED SUBMISSION DATE) NO

EXPECTED SUBMISSION DATE (15)

MONTH	DAY	YEAR

ABSTRACT (Limit to 1400 spaces, i.e., approximately three single-space typewriter lines) (16)

At 1415 hours on January 10, 1992, with Unit 1 in Mode 5 (Cold Shutdown), the "as found" set pressure for the "A" Pressurizer Safety Valve was found to be outside the setpoint tolerance allowed by Technical Specifications 3.4.2 and 3.4.3. This event is reportable pursuant to 10CFR50.73(a)(2)(i)(B) for a condition prohibited by the Technical Specifications.

The three pressurizer safety valves were sent to Wyle Labs for testing. The "as found" set pressure for the "A" Pressurizer Safety Valve was found to be outside the allowable Technical Specification tolerance of 2485 psig +/- 1 percent. In addition, leakage from the the three safety valves was noted following "as found" testing.

The safety valves will be repaired and readjusted, as necessary to be within the correct setpoint tolerance allowed by Technical Specifications.

This event posed no significant safety implications because the safety valve would have performed its intended safety function in the event of an overpressure condition. The health and safety of the public were not affected at any time during this event.

LICENSEE EVENT REPORT (LER)
TEXT CONTINUATION

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS AND REPORTS MANAGEMENT BRANCH (P-530), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20585, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503

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		YEAR	SEQUENCE NUMBER	REVISION NUMBER			
		9 2	0 0 2	0 0	0 2	OF	0 4

TEXT (if more space is required, use additional NRC Form 305A's) (17)

1.0 Description of the Event

At 1415 hours on January 10, 1992, with Unit 1 in Mode 5 (Cold Shutdown), the "as found" set pressure for the "A" Pressurizer Code Safety Valve (EIS System Identifier AB, Component Identifier RV, Vendor Identifier D243) (1-RC-SV-1551A) was found to be out of tolerance. The "as found" set pressure was not within the lift set pressure of 2485 psig +/- 1 percent allowed by Technical Specification 3.4.2 and 3.4.3.

The three Pressurizer Code Safety Valves were sent to Wyle Laboratories for the performance of periodic test procedure 1-PT-50, "Pressurizer Code Safety Valve Setpoint Verification". Each valve was functionally tested for the "as found" set pressure and leak tightness.

The "as found" set pressure for the "A" Pressurizer Code Safety Valve was found to be outside the the setpoint tolerance allowed by Technical Specifications. Specifically, the safety valve was found to have a lift set pressure below the minimum allowed. In addition, the three safety valves leaked following the "as found" testing. A list of the "as found" set pressures is provided in Attachment 1.

2.0 Significant Safety Consequences and Implications

This event posed no significant safety implications because there is no impact on having a low "as found" set pressure. The safety valve would have performed its intended safety function in the event of an overpressure condition. The health and safety of the general public are not affected.

3.0 Cause of the Event

The cause of the event has been determined to be setpoint drift. The industry has experienced a history of setpoint drift for safety valves of this type and is not considered to be unusual.

4.0 Corrective Actions

The Pressurizer Code Safety Valves will be refurbished and retested, as necessary, at Wyle Laboratories to within the allowable limits of Technical Specification 3.4.2 and 3.4.3 and to ensure there is no leakage.

LICENSEE EVENT REPORT (LER)
TEXT CONTINUATION

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS AND REPORTS MANAGEMENT BRANCH (P-530), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20535, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

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TEXT (if more space is required, use additional NRC Form 300A's) (17)

5.0 Additional Corrective Actions

It has been determined that following a modification to the pressurizer loop seal, the Pressurizer Safety Valve setpoint tolerance may be increased from the current +/- 1 percent tolerance. This modification is scheduled for a future refueling outage. A Technical Specification change will be considered following the above mentioned modification.

6.0 Similar Events

Previous similar events where Pressurizer Code Safety Valves have been outside the requirements of Technical Specifications 3.4.2 and 3.4.3 have occurred at North Anna Power Station on Unit 1 on June 3, 1981 (LER 81-040/03L-0), May 28, 1987 (LER 87-008-00), February 15, 1991 (LER 91-002-00), and on Unit 2 on April 14, 1982 (LER 82-015/03L-0), October 6, 1987 (LER 87-000-00), April 12, 1989 (LER 89-006-00), and October 17, 1990 (LER 90-005-00).

7.0 Additional Information

North Anna Unit 2 was in Mode 1 throughout this event and was not affected.

LICENSEE EVENT REPORT (LER)
TEXT CONTINUATION

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FACILITY NAME (1)

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North Anna Power Station Unit 1

YEAR

SEQUENTIAL
NUMBER

REVISION
NUMBER

0 | 5 | 0 | 0 | 0 | 3 | 3 | 8 | 9 | 2 | -- | 0 | 0 | 2 | -- | 0 | 0 | 0 | 4 | OF | 0 | 4

TEXT (If more space is required, use additional NRC Form 305A's) (17)

ATTACHMENT 1

<u>VALVE</u>	<u>SET PRESSURE (PSIG)</u>	<u>AS FOUND (PSIG)</u>
1-RC-SV-1551A	2485 +/- 25	2397
1-RC-SV 4	2485 +/- 25	2510
1-RC-SV-1551C	2485 +/- 25	2472

LER N1/92-002-00

I. VERIFICATION OF ACCURACY

1. Deviation Report N-92-0058
2. Deviation Report N-92-0104
3. 1-PT-50
4. Technical Specification 3.4.2 and 3.4.3

II. ACTION PLAN

1. Following the future modification to the pressurizer loop seals, Engineering is to complete the analysis to determine if the setpoint tolerance of +/- 1 percent can be increased. A Technical Specification change will be initiated if it can be increased.
2. Maintenance will ensure the Pressurizer Code Safety Valves are refurbished and retested, as necessary, at Wyle Laboratories to within the allowable limits of Technical Specification 3.4.2 and 3.4.3 and to ensure there is no leakage.

III. COMMITMENTS (STATED OR IMPLIED)

1. An evaluation will be conducted to determine if the tolerance for the Pressurizer Safety Valve setpoints can be increased from +/- 1 percent. If additional tolerance can be justified, a Technical Specification change will be considered.
2. The Pressurizer Code Safety Valves will be refurbished and retested, as necessary, at Wyle Laboratories to within the allowable limits of Technical Specification 3.4.2 and 3.4.3 and to ensure there is no leakage.