Commonwealth Edison Company 1400 Opus Place Downers Grove, IL 60515

August 22, 1995



Office of Nuclear Reactor Regulation U.S. Nuclear Regulatory Commission Washington, D.C. 20555

Attn: Document Control Desk

Subject: Braidwood Station Units 1 and 2

Analysis of Capsule X from the Reactor Vessel Radiation

Surveillance Program and Evaluation of Pressurized

Thermal Shock

NRC Docket Numbers: 50-456 and 50-457

The enclosed technical reports: WCAP-14242, "Evaluation of Pressurized Thermal Shock for Braidwood Unit 1 and WCAP-14229, "Evaluation of Pressurized Thermal Shock for Braidwood Unit 2." This report is being submitted the Staff you pursuant to 10CFR50.61.

Updated pressurized thermal shock evaluations were performed to include material data obtained from the analysis of Braidwood Unit 1 surveillance Capsule X (WCAP-14241) and Braidwood Unit 2 surveillance Capsule X (WCAP-14228). The calculated RT_{PTS} will remain below the 10CFR50.61 screening values through the projected 32 EFPY end-of-license fluence. The calculations were performed using both 10CFR50.61 and Reg. Guide 1.99, Rev. 2, Position 2 methodologies. Girth weld WF562 is the limiting beltline material for both units.

Please address any further comments or questions regarding this matter to this office.

Sincerely,

Denise M. Saccomando

Nuclear Licensing Administrator

Enclosures: 1. WCAP-14242

2. WCAP-14229

cc: R. Assa, Braidwood Project Manager - NRR

S. Ray, Acting Senior Resident Inspector - Byron

H. Miller, Regional Administrator

Office of Nuclear Facility Safety -IDNS

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April 13, 1995

To: M.A. Gorski SEC Braidwood Station

References:

- 1. WCAP-14242, Evaluation of Pressurized Thermal Shock for Braidwood Unit 1, March 1995.
- 2. WCAP-14229, Evaluation of Pressurized Thermal Shock for Braidwood Unit 2, March 1995.

The referenced documents have been reviewed by SMAD-Metallurgy and NETS personnel. The document should be submitted to the NRC as required by 10CFR50.61.

Summary

Updated PTS evaluations were performed to include material data obtained from the analysis of Braidwood Unit 1 surveillance capsule X (WCAP-14241) and Braidwood Unit 2 surveillance capsule X (WCAP-14228). The calculated RT_{PTS} will remain below the 10CFR50.61 screening values through the projected 32 EFPY end-of-license fluence. The calculations were performed using both 10CFR50.61 and Reg. Guide 1.99, Rev. 2, Position 2 methodologies. Girth weld WF562 is the limiting beltline material for both units.

J.M. Chynoweth SMAD-Metallurgy

Thomas O Spr

T.D. Spry NETS-S/G & RV Projects

cc. Station Manager, Braidwood SEC Manager, Braidwood J.C. Blomgren, NETS