

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Sequoyah, Unit 2	DOCKET NUMBER (2) 0 5 0 0 0 3 2 8	PAGE (3) 1 OF 0 2
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TITLE (4)
Reactor Trip

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)																																																																																			
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)																																																																																	
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LICENSEE CONTACT FOR THIS LER (12)

NAME Glenn B. Kirk, Compliance Section Engineer	TELEPHONE NUMBER 6 1 5 8 7 0 - 6 1 4 6
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COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS

SUPPLEMENTAL REPORT EXPECTED (14)

YES (If yes, complete EXPECTED SUBMISSION DATE) NO

EXPECTED SUBMISSION DATE (15)

MONTH	DAY	YEAR

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single space typewritten lines) (16)

On 05/19/84, unit 2 was in mode 1 (556 degrees F, 2235 psig) at 30% reactor power with various maintenance activities underway while the unit was at reduced power. A reactor trip occurred at 1157 CST when the balance of plant reactor operator intended to stop the 2B main feedwater pump oil pumps but inadvertently stopped the 2A main feedwater pump oil pumps. This event resulted in a reactor trip on low-low steam generator level.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

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		YEAR 8 4	SEQUENTIAL NUMBER - 0 0 8	REVISION NUMBER - 0 0		
					0 2	OF 0 2

TEXT (If more space is required, use additional NRC Form 366A's) (17)

Unit 2 had been reduced to 30% reactor power for addition of oil to reactor coolant pumps. During the reduced power level condition, various maintenance activities were being performed which included repairing oil leaks on both the 2A and 2B main feedwater pumps. The 2A main feedwater pump was removed from service, all of its oil pumps stopped, and the leaks repaired. The 2A main feedwater pump was returned to service and the 2B main feedwater pump removed from service. When it came time to stop the 2B main feedwater pump oil pumps, the balance of plant reactor operator inadvertently stopped the 2A main feedwater pump oil pumps. This action caused the 2A main feedwater pump to trip resulting in a subsequent reactor trip on low-low level in the number 2 steam generator. Unit 2 was in mode 1 (556 degrees F, 2235 psig) at 30% reactor power just prior to the reactor trip which occurred at 1157 CST on 05/19/84.

With exception of an excessive packing leak on the loop 3 feedwater isolation valve, all other personnel and equipment performed as expected during the event. The event was discussed thoroughly with the operator involved and the cause of the event was confirmed to be a personnel error.

There was no effect on public health or safety. Although there have been several reactor trips on unit 1 during 1984, this was the first reactor trip (both units) initiated by inadvertent tripping of a main feedwater pump due to a personnel error.

TENNESSEE VALLEY AUTHORITY

Sequoyah Nuclear Plant
Post Office Box 2000
Soddy Daisy, Tennessee 37379

June 15, 1984

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

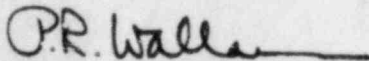
Gentlemen:

TENNESSEE VALLEY AUTHORITY - SEQUOYAH NUCLEAR PLANT UNIT 2 - DOCKET NO.
50-328 - FACILITY OPERATING LICENSE DPR-79 - REPORTABLE OCCURRENCE REPORT
SQRO-50-328/84008

The enclosed licensee event report provides details concerning a reactor trip from 30% reactor power. This event is reported in accordance with 10 CFR 50.73, paragraph a.2.iv.

Very truly yours,

TENNESSEE VALLEY AUTHORITY



P. R. Wallace
Plant Manager

Enclosure
cc (Enclosure):

James P. O'Reilly, Director
U.S. Nuclear Regulatory Commission
Suite 2900
101 Marietta Street, NW
Atlanta, Georgia 30303

Records Center
Institute of Nuclear Power Operations
Suite 1500
1100 Circle 75 Parkway
Atlanta, Georgia 30339

NRC Inspector, NUC PR, Sequoyah

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