NRC Form 386 (9-83)  LICENSEE EVENT REPORT (LER)													U	A	NUCLEAR FEGULATORY COMMISSION APPROVED OMB NO. 3150-0104 EXPIRES: 8/31/85														
FACILIT	Y NA	ME (1	1)	-	_		-											DOC	CKET	ET NUMBER (2)						PAGE (S)			
R.E. Ginna Nuclear Power Plant, Un									Init	nit No. 1						0  5   0   0					0121414			FO	12				
TITLE (	4)							-										_	_	_	_				_				
In	ope	ra	b16	e 1	Res	sid	ua.	1 He	eat	nam.	ova1	Syst	em																
EVENT DATE (8)						LER NUMBER (6)					REPORT DATE (7)				OTHER FACILITIES INV						INVO	DLVED (8)							
MONTH DAY YEAR			YEAR SEQUENTIAL NUMBER					REVISION		ONTH DAY YEAR			FACILITY NAME					DO				CKET NUMBER(S)							
							T															0	15	01	0	01	11		
	١.						_			_				. [		1-1-1													
0 5	1	4	8	4	8	4		0	0 5		0 0	0 6	1 3	8 4								0	15	0 1	0 1	01	1	1	
	ERAT				TH	IS RE	POR	T IS 8U	BMITTE	EU PU	RSUANT	TO THE R	EQUIREM	ENTS OF 10	CFR 9: /	Check on	or mor	e of t	he fo	/lowi	ng) (11	1)							
MODE (9)			N		20.402(b)					20.406	20.405(c)			50.73						73,71(b)									
POWE						20	406	(a)(1)(i)	»)(1)(i)			80.36(e)(1)				50.73(a)(2)(v)								73.71(e)					
(10) 0 1 010				20.406(a)(1)(ii)					50.36(a)(2)				50.73(a)(2)(vii)							OTHER (Specify in Abstract below and in Text, NRC Form					et				
						20	406	(a)(1)(iii	1			50.73(e	(2)(1)			60.73	a)(2)(viii	(A)(					36	6A)	in i	##1, 141			
						20.406(a)(1)(iv)					60.73(a)(2)(ii)				50.73(a)(2)(viii)(B)														
						20.408(a)(1)(v)						50.73(a)(2)(iii)				50.73(a)(2)(x)													
											7 - 1	LICENSEE	CONTACT	FOR THIS	LER (12)														
NAME																				_		TEL	EPHO	NE NU	MBE	A			
m ·	D	Cal	h 1			Ma				11-		The state of							AR	EAC	ODE								
1.	к.	oc.	nuı	lei	. ,	Ma.	LII	tena	ince	ma	nage			5 10	March 2				3	11	15	5	12	141	4	414	14	16	
							_	CO	MPLETE	ONE	LINE FO	EACH CO	MPONEN	FAILURE	DESCRIBE	D IN TH	IS REP	ORT	13)			_			•				
CAUSE	USE SYSTEM COMP			ONE	NT	MANUFAC- TURER			NPROS			CAUSE	SYSTEM	STEM COMPO			MANUFAC- TURER		REPORTABLE TO NPRDS										
В	В	P	-	I	IS	V	v	101	815		Y								1	1	ı								
X	В	P	-	-	12	10	L	1 21	010		Y				1	1	1.1		1	1	1								
SUPPLEMENTAL REPORT EXPECTED (14)														EXPECT						MONT	гн	DAY	Y	EAR					
YES (If yes, complete EXPECTED SUBMISSION DATE)										X	X NO						SUBMISSION DATE (16)					0 10		0.10	0	b			
ABSTRA												ng dot		e Read	ctor (	Coola	ant	Sys	ste	m	(RC	s)	to	o th	ie		10		

On May 14, 1984 while cooling down the Reactor Coolant System (RCS) to the cold shutdown condition for sludge lancing and crevice cleaning, MOV-700 (RCS Loop-A Residual Heat Removal Suction Valve) failed to stroke to the open position when acutated from the Control Room. Following manual unseating of the valve, maintenance personnel performed an inspection of the valve exterior. This inspection revealed that the packing gland flange had shifted out of the vertical position to a point where the flange was in contact with the valve stem. This could have caused a mechanical binding in the stem and torque-out of valve operator.

The valve was then stroked manually to verify no more chanical binding. The valve was then stroked twice electrically. The valve functioned satisfactorily with proper motor current readings, and acceptable opening and closing times, indicating no mechanical binding.

8406190244 840613 PDR ADOCK 05000244 LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO. 3150-0104
EXPIRES. 8/31/85

FACILITY NAME (1)

R.E. Ginna Nuclear Power Plant,
Unit No. 1

O 5 0 0 0 2 4 4 8 4 - 0 0 5 - 0 0 0 2 0 2

A visual inspection of the valve stem and stem threads verified adequate cleanliness and lubrication. Torque switch settings were verified within the

manufacturers design settings.

TEXT (If more space is required, use additional NRC Form 366A's) (17)

On 5/22/84 when the RCS was heating up to hot shutdown, the valve was again stroked to verify proper operation. Again, the valve functioned properly with proper motor current readings, and acceptable opening and closing times. Operation of this valve will continue to be monitored during the next cooldown of the RCS.





ROCHESTER GAS AND ELECTRIC CORPORATION . 89 EAST AVENUE, ROCHESTER, N.Y. 14649-0001

ROGER W. KOBER
VK.E PRESIDENT
ELECTRIC & STEAM PRODUCTION

TELEPHONE AREA CODE 716 546-2700

June 13, 1984

U.S. Nuclear Regulatory Commission Document Control Desk Washington, D.C. 20555

Subject: LER 84-005, Inoperable Residual Heat Removal (RHR) System

R. E. Ginna Nuclear Power Plant, Unit No. 1 Docket No. 50-244

Document Control Desk:

In accordance with 10 CFR 50.73, Licensee Event Report System, item (a) (2) (vii), "any event where a single cause or condition caused at least one independent train or channel to become inoperable in multiple systems or two independent trains or channels to become inoperable in a single system designed to:

- (a) Shutdown the Reactor and maintain it in a safe shutdown condition;
- (b) Remove Residual Heat;
- (c) Mitigate the consequences of an accident."

The attached Licenses Event Report LER 84-005 is hereby submitted.

Truly Yours,

Roger W. Kober

xc: Dr. Thomas E. Murley

TEXT