

NRC FORM 366
(12-81)
10 CFR 50

U.S. NUCLEAR REGULATORY COMMISSION
LICENSEE EVENT REPORT

APPROVED BY OMB
3150-0011

CONTROL BLOCK: [] [] [] [] [] [] [] [] (1) (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

0 1 | A | L | B | R | F | 3 | 2 | 0 | 0 | - | 0 | 0 | 0 | 0 | 0 | - | 0 | 0 | 3 | 4 | 1 | 1 | 1 | 1 | 4 | | | 5
7 8 9 14 15 20 26 30 37 CAT 38

CONT
0 1 | REPORT SOURCE | L | 6 | 0 | 5 | 0 | 0 | 0 | 2 | 9 | 6 | 7 | 0 | 5 | 0 | 6 | 7 | 9 | 8 | 0 | 4 | 2 | 7 | 8 | 4 | 9
7 8 60 61 68 69 74 75 80

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)
0 2 | During normal unit 3 operation, the RHR service water supply to the 2B RHR
0 3 | heat exchanger was isolated for maintenance. The isolation of the water
0 4 | supply also isolated the standby coolant supply to unit 3. (T.S. 3.5.C.3 and
0 5 | 3.5.C.5). There was no hazard to the public health or safety. Previous
0 6 | occurrences: None.
0 7 |
0 8 |

0 9 | SYSTEM CODE | C | F | 11 | CAUSE CODE | D | 12 | CAUSE SUBCODE | A | 13 | COMPONENT CODE | H | T | E | X | C | H | 14 | COMP SUBCODE | C | 15 | VALVE SUBCODE | Z | 16 |
7 8 9 10 11 12 13 14 15 16 17 18 19 20
17 | LER/RO REPORT NUMBER | 7 | 9 | 21 22 | SEQUENTIAL REPORT NO. | 0 | 0 | 5 | 23 24 26 | OCCURRENCE CODE | / | 0 | 1 | 27 28 29 | REPORT TYPE | T | 30 | REVISION NO. | 2 | 32 |
ACTION TAKEN | X | 18 | FUTURE ACTION | G | 19 | EFFECT ON PLANT | Z | 20 | SHUTDOWN METHOD | Z | 21 | HOURS | 0 | 0 | 0 | 0 | 22 | ATTACHMENT SUBMITTED | Y | 23 | NFRD-4 FORM SUB. | N | 24 | PRIME COMP. SUPPLIER | L | 25 | COMPONENT MANUFACTURER | Z | 9 | 9 | 9 | 26
33 34 35 36 37 38 39 40 41 42 43 44 45

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)
1 0 | Isolation of the unit 3 water supply during unit 2 maintenance was caused by a lack
1 1 | of administrative control over system cross-connect valves. Upon error discovery,
1 2 | RHR heat exchanger 2E was returned to service within four hours and water supply
1 3 | restored. Administrative controls have been improved. Changes were made to Tech.
1 4 | Specs. to allow a 10-day standby coolant supply outage.

1 5 | FACILITY STATUS | E | 28 | % POWER | 0 | 7 | 5 | 29 | OTHER STATUS | NA | 30 | METHOD OF DISCOVERY | B | 31 | DISCOVERY DESCRIPTION | Operator Inspection Tour | 32
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32
1 6 | ACTIVITY CONTENT RELEASED OF RELEASE | Z | 33 | AMOUNT OF ACTIVITY | Z | 34 | LOCATION OF RELEASE | NA | 36
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32
1 7 | PERSONNEL EXPOSURES NUMBER | 0 | 0 | 0 | 37 | TYPE | Z | 38 | DESCRIPTION | NA | 39
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32
1 8 | PERSONNEL INJURIES NUMBER | 0 | 0 | 0 | 40 | DESCRIPTION | NA | 41
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32
1 9 | LOSS OF OR DAMAGE TO FACILITY TYPE | Z | 42 | DESCRIPTION | NA | 43
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32
2 0 | PUBLICITY ISSUED | N | 44 | DESCRIPTION | NA | 45
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32

IE22

NAME OF PREPARER | E. T. Holder | PHONE: 205 729-0885
840 190207 840427
PDR ADOCK 05000296
S PDR

LER SUPPLEMENTAL INFORMATION

BFRO-50-296/79005 R2 Technical Specification Involved 3.5.C.3 and 3.5.C.5

Reported Under Technical Specification 6.7.2.A.2

Date of Occurrence 5/6/79 Time of Occurrence 1300 Unit 3

Identificaiton and Description of Occurrence

The RHR service water supply to the 2B RHR heat exchanger was isolated for maintenance. The isolation of the water supply also isolated the standby coolant supply to unit 3.

Conditions Prior to Occurrence:

Unit 2 - refueling outage in progress.
Unit 3 - operating normally at 75% power.

Action specified in the Technical Specification Surveillance Requirements met due to inoperable equipment. Describe

Place the unit in cold shutdown within 24 hours.

Apparent Cause of Occurrence:

The isolation of the water supply for maintenance was the direct cause; however, lack of administrative controls over cross-connect valves was the proximate cause of the accident.

Analysis of Occurrence:

There was no hazard to the public health or safety.

Corrective Action

Upon discovery of the standby coolant supply isolation, action was immediately taken to restore the heat exchanger and the standby coolant was returned to service in four hours. The standby coolant supply valve was painted a distinctive color and strict administrative controls imposed on its operation. A revision was made to the technical specification for standby coolant supply to allow removal of the standby coolant supply for up to 10 days. This allows ample time for performing maintenance during future outages.

TENNESSEE VALLEY AUTHORITY

Browns Ferry Nuclear Plant

P. O. Box 2000

Decatur, Alabama 35602

April 27, 1984

MAY 3 12:44

Mr. James P. O'Reilly, Regional Administrator
U. S. Nuclear Regulatory Commission
Region II
101 Marietta Street, Suite 3100
Atlanta, Georgia 30303

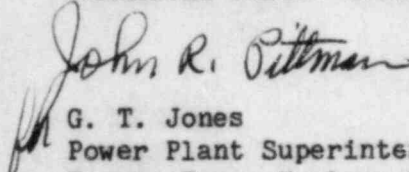
Dear Mr. O'Reilly:

TENNESSEE VALLEY AUTHORITY - BROWNS FERRY NUCLEAR PLANT UNIT 3 - DOCKET
NO. 50-296 - FACILITY OPERATING LICENSE DPR-68 - REPORTABLE OCCURRENCE
REPORT BFRO-50-296/79005 R2

The enclosed report provides followup information concerning isolation of
RHR service water supply to the 2B RHR heat exchanger for maintenance.
This report is submitted in accordance with Browns Ferry Unit 3 Technical
Specification 6.7.2.A.2.

Very truly yours,

TENNESSEE VALLEY AUTHORITY


G. T. Jones
Power Plant Superintendent
Browns Ferry Nuclear Plant

Enclosure

cc (Enclosure):
U. S. Nuclear Regulatory Commission
Document Control Desk
Washington D.C. 20555

NRC Inspector, Browns Ferry Nuclear Plant