



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

ENERGY OPERATIONS, INC.

DOCKET NO. 50-382

WATERFORD STEAM ELECTRIC STATION, UNIT 3

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 111  
License No. NPF-38

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Entergy Operations, Inc. (the licensee) dated December 9, 1994, as supplemented by letter dated July 25, 1995, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C(2) of Facility Operating License No. NPF-38 is hereby amended to read as follows:

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 111, and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance to be implemented within 60 days.

FOR THE NUCLEAR REGULATORY COMMISSION

*Chandu P. Patel*

Chandu P. Patel, Project Manager  
Project Directorate IV-1  
Division of Reactor Projects III/IV  
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical  
Specifications

Date of Issuance: September 1, 1995

ATTACHMENT TO LICENSE AMENDMENT NO. 111

TO FACILITY OPERATING LICENSE NO. NPF-38

DOCKET NO. 50-382

Replace the following pages of the Appendix A Technical Specifications with the attached pages. The revised pages are identified by Amendment number and contain vertical lines indicating the areas of change. The corresponding overleaf pages are also provided to maintain document completeness.\*

REMOVE PAGES

3/4 4-7  
3/4 4-8  
3/4 7-1  
3/4 7-2  
3/4 7-3  
3/4 7-4\*

INSERT PAGES

3/4 4-7  
3/4 4-8  
3/4 7-1  
3/4 7-2  
3/4 7-3  
3/4 7-4\*

## REACTOR COOLANT SYSTEM

### 3/4.4.2 SAFETY VALVES

#### SHUTDOWN

#### LIMITING CONDITION FOR OPERATION

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3.4.2.1 A minimum of one pressurizer code safety valve shall be OPERABLE with a lift setting of 2500 psia  $\pm$  3%.\*

APPLICABILITY: MODE 4.

#### ACTION:

With no pressurizer code safety valve OPERABLE, immediately suspend all operations involving positive reactivity changes (except cooldown in shutdown cooling) and place an OPERABLE shutdown cooling loop into operation.

#### SURVEILLANCE REQUIREMENTS

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4.4.2.1 Verify each required pressurizer code safety valve is OPERABLE in accordance with Specification 4.0.5. Following testing, lift settings shall be within  $\pm$  1%.

\*The lift setting pressure shall correspond to ambient conditions of the valve at nominal operating temperature and pressure.

## REACTOR COOLANT SYSTEM

### OPERATING

#### LIMITING CONDITION FOR OPERATION

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3.4.2.2 All pressurizer code safety valves shall be OPERABLE with a lift setting of 2500 psia  $\pm$  3%.\*

APPLICABILITY: MODES 1, 2, and 3.

#### ACTION:

With one pressurizer code safety valve inoperable, either restore the inoperable valve to OPERABLE status within 15 minutes or be in at least HOT STANDBY within 6 hours and in HOT SHUTDOWN within the following 6 hours.

#### SURVEILLANCE REQUIREMENTS

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4.4.2.2 Verify each required pressurizer code safety valve is OPERABLE in accordance with Specification 4.0.5. Following testing, lift settings shall be within  $\pm$  1%.

\*The lift setting pressure shall correspond to ambient conditions of the valve at nominal operating temperature and pressure.

### 3/4.7 PLANT SYSTEMS

#### 3/4.7.1 TURBINE CYCLE

##### SAFETY VALVES

##### LIMITING CONDITION FOR OPERATION

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3.7.1.1 All main steam line code safety valves shall be OPERABLE with lift settings as specified in Table 3.7-1.

APPLICABILITY: MODES 1, 2, and 3.

ACTION:

- a. With both reactor coolant loops and associated steam generators in operation and with one or more main steam line code safety valves inoperable, operation in MODES 1, 2 and 3 may proceed provided, that within 4 hours, either the inoperable valve is restored to OPERABLE status or the Linear Power Level-High trip setpoint is reduced per Table 3.7-2; otherwise, be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.
- b. The provisions of Specification 3.0.4 are not applicable.

##### SURVEILLANCE REQUIREMENTS

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4.7.1.1 Verify each required main steam line code safety valve lift setpoint per Table 3.7-1 in accordance with Specification 4.0.5. Following testing, lift settings shall be within  $\pm 1\%$ .

TABLE 3.7-1  
STEAM LINE SAFETY VALVES PER LOOP

	<u>VALVE NUMBER</u>		<u>LIFT SETTING (<math>\pm</math> 3%)*</u>	<u>ORIFICE SIZE</u>
	<u>Line No. 1</u>	<u>Line No. 2</u>		
a.	2MS-R613A (MS-106A)	2MS-R619B (MS-106B)	1070 psig	26 in <sup>2</sup>
b.	2MS-R614A (MS-108A)	2MS-R620B (MS-108B)	1085 psig	26 in <sup>2</sup>
c.	2MS-R615A (MS-110A)	2MS-R621B (MS-110B)	1100 psig	26 in <sup>2</sup>
d.	2MS-R616A (MS-112A)	2MS-R622B (MS-112B)	1115 psig	26 in <sup>2</sup>
e.	2MS-R617A (MS-113A)	2MS-R623B (MS-113B)	1125 psig	26 in <sup>2</sup>
f.	2MS-R618A (MS-114A)	2MS-R624B (MS-114B)	1135 psig	26 in <sup>2</sup>

\*The lift setting pressure shall correspond to ambient conditions of the valve at nominal operating temperature and pressure.

TABLE 3.7-2

MAXIMUM ALLOWABLE LINEAR POWER LEVEL-HIGH TRIP SETPOINT WITH INOPERABLE  
STEAM LINE SAFETY VALVES DURING OPERATION WITH BOTH STEAM GENERATORS

MAXIMUM NUMBER OF INOPERABLE SAFETY  
VALVES ON ANY OPERATING STEAM GENERATOR

MAXIMUM ALLOWABLE LINEAR POWER  
LEVEL-HIGH TRIP SETPOINT  
(PERCENT OF RATED THERMAL POWER)

1	86.8
2	69.4
3	52.1



## PLANT SYSTEMS

### EMERGENCY FEEDWATER SYSTEM

#### LIMITING CONDITION FOR OPERATION

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3.7.1.2 At least three independent steam generator emergency feedwater pumps and associated flow paths shall be OPERABLE with:

- a. Two feedwater pumps, each capable of being powered from separate OPERABLE emergency busses, and
- b. One feedwater pump capable of being powered from an OPERABLE steam supply system.

APPLICABILITY: MODES 1, 2, and 3.

#### ACTION:

- a. With one emergency feedwater pump inoperable, restore the required emergency feedwater pumps to OPERABLE status within 72 hours or be in at least HOT STANDBY within the next 6 hours and in HOT SHUTDOWN within the following 6 hours.
- b. With two emergency feedwater pumps inoperable be in at least HOT STANDBY within 6 hours and in HOT SHUTDOWN within the following 6 hours.
- c. With three emergency feedwater pumps inoperable, immediately initiate corrective action to restore at least one emergency feedwater pump to OPERABLE status as soon as possible.

#### SURVEILLANCE REQUIREMENTS

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4.7.1.2 The emergency feedwater system shall be demonstrated OPERABLE:

- a. At least once per 31 days by:
  1. Verifying that each valve (manual, power-operated, or automatic) in the flow path that is not locked, sealed, or otherwise secured in position, is in its correct position.
- b. At least once per 92 days on a STAGGERED TEST BASIS by:
  1. Verifying that each motor-driven pump develops a discharge pressure of greater than or equal to 1298 psig on recirculation flow.
  2. Verifying that the turbine-driven pump develops a discharge pressure of greater than or equal to 1342 psig on recirculation flow when the steam generator pressure is greater than 750 psig. The provisions of Specification 4.0.4 are not applicable for entry into MODE 3.