October 31, 1991

Docket No. 50-482

8. 4

Mr. Bart D. Withers President and Chief Executive Officer Wolf Creek Nuclear Operating Corporation P. O. Box 411 Burlington, Kansas 66839

Dear Mr. Withers:

SUBJECT: RELIEF FROM AN ASME SECTION XI CODE REQUIREMENT (TAC NO. 81286)

By letter dated August 15, 1991, Wolf Creek Nuclear Operating Corporation submitted a request for relief from a requirement of Section XI of the ASME Boiler and Pressure Vessel Code concerning the procurement of a replacement disc for a Residual Heat Removal System check valve. The NRC staff has reviewed the relief request and determined that although relief pursuant to 10 CFR 50.55a(g)(6)(1) is not appropriate due to the lack of demonstration of hardship, an acceptable alternative to the ASME Code was provided by the procurement of this valve disc and therefore the requirements of 10 CFR 50.55a(a)(3)(i) have been satisfied and the disc can remain in service until replacement for maintenance or performance reasons is warranted. This letter grants the relief pursuant to 10 CFR 50.55a(a)(3)(i).

The staff's detailed findings are provided in the enclosed Safety Evaluation.

Sincerely,

Original Signed By George Dick for

Suzanne C. Black, Director Project Directorate IV-2 Division of Reactor Projects - III/IV/V Office of Nuclear Reactor Regulation

Enclosure: Safety Evaluation

cc w/enclosure: See next page DISTRIBUTION: Docket NRC PDR Local PDR PDIV-2 Reading BBoger MVirgilio WReckley EPeyton GHill (4)

ACRS (10) GPA/PA OC/LFMB PDIV-2 Plant File SShankman AHowell, Region IV OGC EJordan CR6551

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## Mr. Bart D. Withers

cc w/enclosure: Jay Silberg, Esq. Shaw, Pittman, Potts & Trowbridge 2300 N Street, NW Washington, D.C. 20037

Mr. Chris R. Rogers, P.E. Manager, Electric Department Public Service Commission P. O. Box 360 Jefferson City, Missouri 65102

Regional Administrator, Region III U.S. Nuclear Regulatory Commission 799 Roosevelt Road Glen Ellyn, Illinois 60137

Senior Resident Inspector U. S. Nuclear Regulatory Commission P. O. Box 311 Burlington, Kansas 66839

Mr. Robert Elliot, Chief Engineer Utilities Division Kansas Corporation Commission 1500 SW Arrowhead Road Topeka, Kansas 66604-4027

Office of the Governor State of Kansas Topeka, Kansas 66612

Attorney General 1st Floor - The Statehouse Topeka, Kansas 66612

Chairman, Coffey County Commission Coffey County Courthouse Burlington, Kansas 66839

Mr. Gerald Allen Public Health Physicist Bureau of Air Quality & Radiation Control Division of Environment Kansas Department of Health and Environment Forbes Field Building 321 Topeka, Kansas 66620 October 31, 1991

Mr. Gary D. Boyer Director Plant Operations Wolf Creek Nuclear Operating Corporation P. O. Box 411 Burlington, Kansas 66839

Regional Administrator, Region IV U.S. Nuclear Regulatory Commission 611 Ryan Plaza Drive, Suite 1000 Arlington, Texas 76011

Mr. Harold K. Chernoff Supervisor Licensing Wolf Creek Nuclear Operating Corporation P. O. Box 411 Burlington, Kansas 66839

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