Mr. Oliver D. Kingsley, Jr.
President, TVA Nuclear and
Chief Nuclear Officer
Tennessee Valley Authority
6A Lookout Place
1101 Market Street
Chattanooga, TN 37402-2801

September 6, 1995

SUBJECT: REQUEST FOR ADDITIONAL INFORMATION - GENERIC LETTER 95-03, CIRCUMFERENTIAL CRACKING OF STEAM GENERATOR TUBES - SEQUOYAH NUCLEAR PLANT UNITS 1 AND 2 (TAC NOS. M92274 AND M92275)

Dear Mr. Kingsley:

On April 28, 1995, the U.S. Nuclear Regulatory Commission issued Generic Letter (GL) 95-03 "Circumferential Cracking of Steam Generator Tubes" which requested addressees to evaluate recent operating experience related to circumferential cracking, justify continued operation until the next scheduled steam generator tube inspections, and to develop plans for the next steam generator tube inspections.

The staff has reviewed the response provided by the Tennessee Valley Authority for Sequoyah Nuclear Plant Units 1 and 2 dated June 27, 1995. As a result of this review, the staff has identified areas for which additional information and/or clarification is needed. The enclosure to this letter contains the information needed for the staff to complete its review of your response to GL 95-03.

Please provide this information within 60 days from the receipt of this request. This request is within the original reporting burden for information collection of 350 hours covered by the Office of Management and Budget clearance number 3150-0011, which expires July 31, 1997.

Sincerely,

Original signed by

David E. LaBarge, Sr. Project Manager
Project Directorate II-3
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket Nos. 50-327 and 50-328

Enclosure: Request for Additional Information

cc w/enclosure: See next page

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GENERIC LETTER 95-03, CIRCUMFERENTIAL CRACKING OF STEAM GENERATOR TUBES

REQUEST FOR ADDITIONAL INFORMATION

SEQUOYAH NUCLEAR PLANT UNITS 1 AND 2

1. TVA reported that 16 expansion transition circumferential indications were detected in Unit 1 during the last steam generator tube inspection outage and that over 50 percent of the tubes were inspected with a rotating pancake coil probe. Please discuss the expansion criteria used during these examinations.

   TVA indicated that the expansion criteria contained in the Westinghouse Owners' Group WEXTEX Guidelines would be followed for indications detected at the expansion transition. Provide a summary of these expansion criteria.

2. TVA indicated that dents greater than 5.0 volts were inspected with a rotating pancake coil probe. Provide the procedures used for sizing the dents. If the procedure is identical to the procedure for the voltage-based repair criteria, a detailed description is not necessary.

   Future inspection plans for dented (> 5V) intersections concentrate at the lowest hot-leg tube support plates. A large dent at an upper tube support plate may be more significant in terms of corrosion susceptibility as a result of higher stresses than a small dent at a lower tube support plate even though the temperature is lower at the upper tube support plate. Given this, discuss the basis for the proposed sample strategy given that cracking depends on many factors including temperature and stress levels.

3. During the Unit 1 Cycle 6 outage, TVA indicated that one tube was conservatively plugged for a circumferential indication in a tube with a small radius U-bend and that the degradation mechanism is not active. Discuss the basis for these statements, particularly with respect to being conservatively plugged. For Units 1 and 2, clarify the extent of the rotating pancake coil examinations performed in the U-bend region of Rows 1 and 2 (i.e., percentage of tubes inspected). Provide the expansion criteria implemented during the previous outage, if applicable.

4. Please provide the month and year for the completion of the last two steam generator tube inspections at both Sequoyah Units 1 and 2.

ENCLOSURE

cc:
Mr. Oliver D. Kingsley, Jr.
Tennessee Valley Authority
President, TVA Nuclear and
Chief Nuclear Officer
Tennessee Valley Authority
6A Lookout Place
1101 Market Street
Chattanooga, TN 37402-2801

Mr. O. J. Zeringue, Sr. Vice President
Nuclear Operations
Tennessee Valley Authority
3B Lookout Place
1101 Market Street
Chattanooga, TN 37402-2801

Dr. Mark O. Medford, Vice President
Engineering & Technical Services
Tennessee Valley Authority
3B Lookout Place
1101 Market Street
Chattanooga, TN 37402-2801

Mr. D. E. Nunn, Vice President
New Plant Completion
Tennessee Valley Authority
3B Lookout Place
1101 Market Street
Chattanooga, TN 37402-2801

Site Vice President
Sequoyah Nuclear Plant
Tennessee Valley Authority
P.O. Box 2000
Soddy Daisy, TN 37379

General Counsel
Tennessee Valley Authority
ET 11H
400 West Summit Hill Drive
Knoxville, TN 37902

Mr. P. P. Carier, Manager
Corporate Licensing
Tennessee Valley Authority
4G Blue Ridge
1101 Market Street
Chattanooga, TN 37402-2801

Mr. Ralph H. Shell
Site Licensing Manager
Sequoyah Nuclear Plant
Tennessee Valley Authority
P.O. Box 2000
Soddy Daisy, TN 37379

TVA Representative
Tennessee Valley Authority
11921 Rockville Pike
Suite 402
Rockville, MD 20852

Regional Administrator
U.S. Nuclear Regulatory Commission
Region II
101 Marietta Street, NW., Suite 2900
Atlanta, GA 30323

Mr. William E. Holland
Senior Resident Inspector
Sequoyah Nuclear Plant
U.S. Nuclear Regulatory Commission
2600 Igou Ferry Road
Soddy Daisy, TN 37379

Mr. Michael H. Mobley, Director
Division of Radiological Health
3rd Floor, L and C Annex
401 Church Street
Nashville, TN 37243-1532

County Judge
Hamilton County Courthouse
Chattanooga, TN 37402-2801