Mr. M. S. Tuckman Senior Vice President Nuclear Generation Duke Power Company P. O. Box 1006 Charlotte, NC 28201

SUBJECT: REQUEST FOR ADDITIONAL INFORMATION REGARDING GENERIC LETTER

(GL) 95-03 "CIRCUMFERENTIAL CRACKING OF STEAM GENERATOR TUBES"

CATAWBA NUCLEAR STATION, UNITS 1 AND 2

Dear Mr. Tuckman:

On April 28, 1995, the U.S. Nuclear Regulatory Commission issued Generic Letter (GL) 95-03 "Circumferential Cracking of Steam Generator Tubes" that requested addressees to evaluate recent operating experience related to circumferential cracking, justify continued operation until the next scheduled steam generator tube inspections, and to develop plans for the next steam generator tube inspections. The staff has reviewed your response for the Catawba Nuclear Station, Units 1 and 2. As a result of the review of your response, the staff has identified areas for which additional information and/or clarification is needed. The enclosure to this letter contains the information needed for the staff to complete its review of your response to GL 95-03. Please provide your response within 30 days of receipt of this letter.

The information requested by this letter is within the scope of the overall burden estimated in Generic Letter 95-03 of 350 hours. This request is covered by the Office of Management and Budget clearance number 3150-001, which expires July 31, 1997.

Sincerely,

Original signed by:

Robert E. Martin, Senior Project Manager Project Directorate II-2 Division of Reactor Projects - I/II Office of Nuclear Reactor Regulation

Docket Nos. 50-413 and 50-414

Enclosure:

Request for Additional Information

cc w/encl: See next page

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OFFICE	DRPE/PD22/14 6	DRPE/PD22/PM	DRPE/PD22/PD	
NAME	L.BERRY W	R.MARTIN	H. BERKOW AND	
DATE	6 31 195	8/3/ 195	8// 195	
COPY	YES NO	YES NO	YES NO	

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9509070277 950901 PDR ADOCK 05000413 PDR PDR Mr. M. S. Tuckman Senior Vice President Nuclear Generation Duke Power Company P. O. Box 1006 Charlotte, NC 28201

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NAME	L.BERRY WA	R.MARTIN	H. BERKON KANGO	
DATE	6 31 195	8/3/ 195	8/1 /95	
COPY	YES NO	YES NO	YES NO	



## UNITED STATES NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20565-0001

September 1, 1995

Mr. M. S. Tuckman Senior Vice President Nuclear Generation Duke Power Company P. O. Box 1006 Charlotte, NC 28201

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Project Directorate II-2

Division of Reactor Projects - I/II Office of Nuclear Reactor Regulation

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Duke Power Company

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Mr. W. R. McCollum Site Vice President Catawba Nuclear Station Duke Power Company 4800 Concord Road York, South Carolina 29745

## Request for Additional Information

## Catawba Units 1 and 2

- 1. It was indicated that Catawba Unit 1 is susceptible to circumferential cracking at dented locations; however, this region is not cited as a susceptible location for Catawba Unit 2. Please clarify. If susceptible, please submit the information requested in Generic Letter (GL) 95-03 per the guidance contained in the GL.
- Please clarify the outage during which the rotating pancake coil (RPC)
  examination of the U-bend portion of the tubes in Rows 1, 2, 3, and 4 at
  Catawba Unit 1 were performed.
- 3. Discuss the expansion criteria to be used during the Unit 2 inspections if circumferential indications are detected in the areas susceptible to circumferential cracking. Clarify the probe to be used for the inspection of the row 1 and 2 U-bend tubes in Unit 2.
- 4. Several plants with preheater model steam generators expanded tubes into the tube support plate in the preheater region to minimize the potential for vibration induced wear. Since these expansions contain similarities to other expanded regions which have experienced circumferential cracking, discuss whether or not this area is susceptible to circumferential cracking. If this area is susceptible to circumferential cracking, please submit the information requested in Generic Letter (GL) 95-03 per the guidance contained in the GL.
- 5. It is indicated that no tube inspections are planned for Unit 1 since the steam generators are to be replaced. The staff notes that the technical specifications contain requirements pertaining to preservice inspections which may apply to the replacement steam generators.