NRC Form 366
(9-83)

U.S. NUCLEAR REGULATORY COMMISSION
APPROVED OMB NO. 3150-0104
EXPIRES: 8/31/85

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)	DOCKET NUMBER (2)	PAGE (3)
Palisades Nuclear Plant	0 5 0 0 0 2 5 5	1 OF 0 2

TITLE (4)
Engineered Safety Feature Actuation

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES	DOCKET NUMBER(S)
									NA	0 5 0 0 0
0 5	1 2	8 4	8 4	— 0 0 5	— 0 0	0 6	1 1	8 4	NA	0 5 0 0 0

OPERATING MODE (9)	N	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)			
POWER LEVEL (10)	0 0 0	20.402(b)	20.405(c)	X 50.73(a)(2)(iv)	73.71(b)
		20.405(a)(1)(i)	50.36(c)(1)	50.73(a)(2)(v)	73.71(c)
		20.405(a)(1)(ii)	50.36(c)(2)	50.73(a)(2)(vii)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)
		20.405(a)(1)(iii)	50.73(a)(2)(i)	50.73(a)(2)(viii)(A)	
		20.405(a)(1)(iv)	50.73(a)(2)(ii)	50.73(a)(2)(viii)(B)	
		20.405(a)(1)(v)	50.73(a)(2)(iii)	50.73(a)(2)(x)	

LICENSEE CONTACT FOR THIS LER (12)

NAME	TELEPHONE NUMBER
David W Rogers; Technical Engineer; Palisades	AREA CODE 6 1 6 — 7 6 4 - 8 9 1 4 3

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	

SUPPLEMENTAL REPORT EXPECTED (14)		EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
YES (If yes, complete EXPECTED SUBMISSION DATE)	X NO				

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On May 12, 1984, while operations personnel were replacing fuses in safety related electrical circuitry, the use of an inadequate procedure resulted in the spurious actuation of a Safety Injection Signal (SIS), Containment Isolation Signal and a Containment Spray Signal. The Plant was shutdown at the time of the occurrence. No threat to public health or safety resulted. The applicable procedures have been revised as necessary to preclude inadvertent signal actuation and equipment operation.

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		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	
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TEXT (If more space is required, use additional NRC Form 366A's) (17)

While shutdown on May 12, 1984, at 1735, electrical work associated with the replacement of General Electric (GE) HFA relays (RLY;JE] in safety related circuitry resulted in an Engineered Safety Feature Actuation. The Engineered Safety Feature Actuation consisted of a left channel Safety Injection Signal (SIS) Actuation, Containment Isolation Actuation and a Containment Spray Signal.

The incident occurred while non-licensed operations personnel were reinstalling fuses [FU;JE] in the circuitry. The procedure in use did not specify a sequence for reinstallation of the fuses. As fuses were reinstalled, the arbitrary order of installation caused the Containment High Pressure (CHP) relays [RLY;JE] to become energized through the CHP pressure switch auxiliary relays [RLY;JE]. The resulting spurious CHP signal initiated the Engineered Safety Feature Actuation.

The SIS initiation started High Pressure Safety Injection (HPSI) Pump P-66B [P;BQ], and opened the appropriate loop motor operated valves (MOVs) [20;BQ]. Level in the reactor vessel [RCT;AB] increased 1% before P-66B was manually tripped. The Containment Spray Signal opened control valve CV-3001 [FCV;BE], but did not result in spray actuation, because breakers [BKR;BE] were open, preventing the Containment Spray Pumps [P;BE] from starting.

The procedures for GE HFA relay replacement were reviewed and revised as necessary to preclude inadvertent equipment operation when removing/restoring fuses, links and jumpers. No threat to public health or safety resulted. The work activity which resulted in the Engineered Safety Feature actuation would only be performed with the Plant shutdown, thereby precluding an occurrence under alternative conditions, ie, power operation.

Consumers Power Company

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June 11, 1984

US Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

DOCKET 50-255 - LICENSE DPR-20 -
PALISADES PLANT - LICENSEE EVENT REPORT 84-005
(ENGINEERED SAFETY FEATURE ACTUATION)

Attached please find Licensee Event Report 84-005 (Engineered Safety Feature Actuation) which is reportable to the NRC per 10 CFR 50.73(a)(2)(iv).

Brian D Johnson

Brian D Johnson
Staff Licensing Engineer

CC Administrator, Region III, USNRC
Director, Office of Nuclear Reactor Regulation
NRC Resident Inspector - Palisades

Attachment

OC0684-0007A-NL02

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