

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Susquehanna Steam Electric Station-Unit 2	DOCKET NUMBER (2) 0 5 0 0 0 3 8 8	PAGE (3) 1 OF 0 2
--	--------------------------------------	----------------------

TITLE (4)
Inadvertent ESF Actuation (RwCU valve)

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)																	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)															
0	5	1	5	8	4	8	4	0	0	0	5	0	0	0	6	1	2	8	4			0	5	0	0	0

OPERATING MODE (9) 2	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)									
POWER LEVEL (10) 0 0 4	20.402(b)	20.406(c)	<input checked="" type="checkbox"/>	50.73(a)(2)(iv)	73.71(b)					
	20.406(a)(1)(i)	50.36(e)(1)	<input type="checkbox"/>	50.73(a)(2)(v)	73.71(c)					
	20.406(a)(1)(ii)	50.36(e)(2)	<input type="checkbox"/>	50.73(a)(2)(vii)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)					
	20.406(a)(1)(iii)	50.73(a)(2)(i)	<input type="checkbox"/>	50.73(a)(2)(viii)(A)						
	20.406(a)(1)(iv)	50.73(a)(2)(ii)	<input type="checkbox"/>	50.73(a)(2)(viii)(B)						
20.406(a)(1)(v)	50.73(a)(2)(iii)	<input type="checkbox"/>	50.73(a)(2)(ix)							

LICENSEE CONTACT FOR THIS LER (12)		TELEPHONE NUMBER	
NAME L.A. Kuczynski - Nuclear Plant Specialist III	AREA CODE 7 1 7	5 4 2	- 3 7 5 1 9

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)											
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS		
A	CIE	*	*	N							

SUPPLEMENTAL REPORT EXPECTED (14)	EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
<input type="checkbox"/> YES (if yes, complete EXPECTED SUBMISSION DATE)	<input checked="" type="checkbox"/> NO			

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

During the performance of the monthly functional test of the Reactor Water Cleanup System Area Ventilation Differential Temperature Channels, an error by the Instrumentation and Controls (I&C) personnel performing the test caused the inadvertent closure of a valve that is part of the Primary Containment Isolation System (an Engineered Safety Feature). The valve was restored to its normal position within ten minutes. The I&C personnel received formal instruction stressing verbatim compliance with procedures.

B406150230 B40612
PDR ADOCK 05000388
S PDR

* Not Applicable

IE22

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

FACILITY NAME (1) Susquehanna Steam Electric Station-Unit 2	DOCKET NUMBER (2) 0 5 0 0 0 3 8 8 8 4 - 0 0 5 - 0 0 0 0 2 OF 0 2	LER NUMBER (6)			PAGE (3)	
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER		

TEXT (If more space is required, use additional NRC Form 366A's) (17)

The six Reactor Water Cleanup (RWCU) System Area Ventilation differential temperature channels undergo a channel functional surveillance test on a monthly basis. Temperature channels A, C and E (Division I) provide high area differential temperature isolation signals to the RWCU containment inboard isolation valve; channels B, D and F provide signals to the RWCU containment outboard isolation valve. Both the RWCU valves specified above are considered part of the Primary Containment Isolation System, which is an Engineered Safety Feature.

On May 15, 1984, with the unit at less than 1% power, Instrumentation and Controls (I&C) technicians commenced the monthly functional test of the differential temperature channels. In preparation for testing the Division II channels, the I&C technicians mistakenly bypassed the Division I channels. (This was a cognitive error. The test procedure is written correctly.) Thus, when the Division II channels were activated per the test procedure, the RWCU containment outboard isolation valve closed on the simulated high temperature signal. (An inadvertent ESF activation.)

Both RWCU pumps had been in operation, with the system lined up to reject excess reactor coolant to the main condenser. (This is a normal configuration during heatup of the reactor pressure vessel.) The pumps tripped when the valve closed. Operations personnel reacted promptly to assess the situation and restored the RWCU system to operation within ten minutes.

The I&C personnel involved (non-licensed, utility) received formal instruction by their supervisor stressing the importance of verbatim compliance with procedures. If this event had occurred at 100% power, its consequences would still have remained minimal. The error was correctly identified and mitigated by the personnel involved with no safety impact to the plant. Due to the short duration of the event, none of the sampling required by Surveillance Requirement 4.4.4.c was or needed to be performed.



Pennsylvania Power & Light Company

Two North Ninth Street • Allentown, PA 18101 • 215 / 770-5151

June 12, 1984

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

SUSQUEHANNA STEAM ELECTRIC STATION
LICENSEE EVENT REPORT 84-005-00
ER 100450 FILE 841-23
PLA-2235

Docket No. 50-388
License No. NPF-22

Attached is Licensee Event Report 84-005-00. This event was determined reportable per 10CFR50.73(a)(2)(iv) in that during a surveillance test, the unit experienced an unanticipated Engineered Safety Feature actuation limited to the closure of a valve that is part of the Primary Containment Isolation System.

H.W. Keiser
Superintendent of Plant-Susquehanna

LAK/pjg

cc: Dr. Thomas E. Murley
Regional Administrator, Region I
U.S. Nuclear Regulatory Commission
631 Park Avenue
King of Prussia, PA 19406

Mr. R.H. Jacobs
Senior Resident Inspector
U.S. Nuclear Regulatory Commission
P.O. Box 52
Shickshinny, PA 18655

IE22
//