

NOTICE OF VIOLATION

Consumers Power Company
Palisades Nuclear Power Plant

Docket No. 50-255
License No. DPR-20

During an NRC inspection conducted November 4 through December 13, 1991, violations of NRC requirements were identified. In accordance with the "General Statement of Policy and Procedure for NRC Enforcement Actions," 10 CFR Part 2, Appendix C (1991), the violations are listed below:

1. 10 CFR 50, Appendix B, Criterion III, requires that applicable regulatory requirements and the design basis be correctly translated into drawings, procedures and instructions. Criterion III also requires that design changes be subject to design control measures commensurate with the original design.

Contrary to the above, from March 1986 until December 13, 1991, the licensee failed to translate a design change, which re-sized 17 thermal overloads associated with safety related motors, into field instructions for implementing the changes.

This is a Severity Level IV violation (Supplement 1).

2. 10 CFR 50, Appendix B, Criterion XI, requires that a test program be established to assure that all testing required to demonstrate that systems and components will perform satisfactorily in service is identified and performed in accordance with written test procedures.

Contrary to the above:

- a. Prior to December 13, 1991, the licensee failed to establish a program to periodically test the ability of each safety related battery charger to deliver rated output current and supply all required loads following a design basis accident (DBA).
- b. Prior to December 13, 1991, the licensee failed to satisfactorily demonstrate in post modification testing, the proper operation of the control functions associated with contacts 3/3C, 4/4C, 5/5C, and 11/11C located on handswitch HS-152-106RLTS.
- c. Although required by modification FC-839, completed in 1989, the licensee failed to test whether the low suction pressure and low lube oil pressure trips were blocked when charging pump "B" was fed from the alternate power supply.
- d. Prior to December 13, 1991, the monthly emergency diesel generator test method failed to demonstrate the ability of the diesel generators to accept loads within the required time.

This is a Severity Level IV violation (Supplement 1).

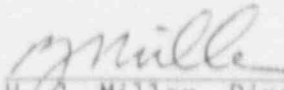
3. 10 CFR 50, Appendix B, Criterion XVI, requires that measures be established to assure that conditions adverse to quality are promptly identified and corrected.

Contrary to the above, as of December 13, 1991, the licensee failed to identify and implement adequate corrective action in each of the three instances on where the Technical Specification limit of 750 amps was exceeded during diesel generator surveillance testing.

This is a Severity Level IV violation (Supplement 1).

Pursuant to the provisions of 10 CFR 2.201, Consumers Power Company is hereby required to submit a written statement or explanation to the U. S. Regulatory Commission, ATTN: Document Control Desk, Washington, D.C. 20555, with a copy to the U. S. Nuclear Regulatory Commission, Region III, 799 Roosevelt Road, Glen Ellyn, Illinois, 60137, and a copy to the NRC Resident Inspector at the facility that is the subject of this Notice, within 30 days of the date of the letter transmitting this Notice of Violation (Notice). This reply should be clearly marked as a "Reply to a Notice of Violation" and should include for each violation: (1) the reason for the violation, or, if contested, the basis for disputing the violation, (2) the corrective steps that have been taken and the results achieved, (3) the corrective steps that will be taken to avoid further violations, and (4) the date when full compliance will be achieved. If an adequate reply is not received within the time specified in this Notice, an order may be issued to show cause why the license should not be modified, suspended, or revoked, or why such other action as may be proper should not be taken. Where good cause is shown, consideration will be given to extending the response time.

Dated at Glen Ellyn, Illinois
this 31st day of Jan, 1992


H. O. Miller, Director
Division of Reactor Safety