



UNITED STATES
 NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

August 28, 1995

Mr. Roger O. Anderson, Director
 Licensing and Management Issues
 Northern States Power Company
 414 Nicollet Mall
 Minneapolis, Minnesota 55401

SUBJECT: REQUEST FOR ADDITIONAL INFORMATION ON THE MONTICELLO NUCLEAR
 GENERATING PLANT (TAC NO. M90979)

Dear Mr. Anderson:

By letter dated October 19, 1994, Northern States Power Company submitted a Licensee Event Report (LER 94-10) concerning the main steam isolation valves exceeding the technical specifications local leak rate limits. LER 94-10 indicated that an engineering analysis determined that the increased leakages would not cause the 10 CFR Part 100 limits to be exceeded for the design-basis accidents; however, the General Design Criterion 19 criteria would be exceeded using conservative assumptions. The staff routinely reviews licensee submitted LERs for, among other things, abnormal occurrence reportability to the Congress in accordance with Section 208 of the Energy Reorganization Act of 1974.

The Commission had established the abnormal occurrence criteria implementing Section 208 of the Energy Reorganization Act of 1974 in an NRC Policy Statement that was published in the Federal Register (42 FR 10950) on February 24, 1977. In this policy statement, an abnormal occurrence is defined as an event involving a major reduction in the degree of protection of the public health or safety. Such an event would involve a moderate or more severe impact on the public health or safety and could include but need not be limited to: (1) major degradation of essential safety-related equipment; (2) a major deficiency in design, construction, or operation having safety implications requiring immediate remedial action; or (3) loss of plant capability to perform essential safety functions such that a potential release of radioactivity in excess of 10 CFR Part 100 guidelines could result from a postulated transient or accident.

The NRC staff is performing an independent review of your engineering evaluation to determine if 10 CFR Part 100 guidelines could have been exceeded. During the review of your analysis, the staff has determined the need for additional information. Enclosed is our request for additional information. The staff requests that you submit your response to the enclosed request for additional information within 30 days in order for the staff to complete its review.

If you have any questions regarding this request, please call me at 301-415-1392.

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 P PDR

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Mr. Roger O. Anderson, Director
Northern States Power Company

Monticello Nuclear Generating Plant

cc:

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St. Paul, Minnesota 55119

Commissioner
Minnesota Pollution Control Agency
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St. Paul, Minnesota 55119

Regional Administrator, Region III
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801 Warrenville Road
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Commissioner of Health
Minnesota Department of Health
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Minneapolis, Minnesota 55440

Darla Groshens, Auditor/Treasurer
Wright County Government Center
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Buffalo, Minnesota 55313

Kris Sanda, Commissioner
Department of Public Service
121 Seventh Place East
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St. Paul, Minnesota 55101-2145

January 1995

REQUEST FOR ADDITIONAL INFORMATION
MONTICELLO NUCLEAR GENERATING PLANT
REGARDING LEAKAGE OF MAIN STEAM ISOLATION VALVES

1. Provide the following information and references which will allow the staff to calculate off-site doses from increased MSIV leakage:
(a drawing would be helpful)
 - a) Main steam line (MSL) inside and outside pipe diameters
 - b) MSL pipe length between the MSIV and the drain line
 - c) MSL pipe material
 - d) MSL initial (operating) and ambient temperatures
 - e) Drain line inside and outside pipe diameters
 - f) Drain line pipe length to the drain header
 - f) Drain line pipe material
 - g) Drain line header inside and outside diameters
 - i) Drain line header length to common drain line to condenser
 - j) Drain line header material
 - k) Drain line initial (operating) and ambient temperatures
 - l) Common drain line length to the condenser
 - m) Condenser total volume
 - n) Condenser air space volume
 - o) Condenser hotwell (liquid) volume
 - p) Condenser temperature
 - q) Type of material used to insulate the main steam and drain lines
 - r) Thickness of insulating material used on the main steam and drain lines
 - s) The MSIV leakage rate use in your calculations and the justification for that value.

2. Provide the following information and references which will allow the staff to calculate control room operator doses from increased MSIV leakage.
 - a) Iodine protection factor
 - b) Control room geometry factor
 - c) Recirculation flow rate (cfm)
 - d) Charcoal adsorber thickness in inches
 - e) Adsorber efficiency (%)

R. Anderson

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This requirement affects nine or fewer respondents and, therefore, is not subject to the Office of Management and Budget review under P.L. 96-511.

Sincerely,

Original Signed by

Tae Kim, Project Manager
Project Directorate III-1
Division of Reactor Projects - III/IV
Office of Nuclear Reactor Regulation

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Docket No. 50-236
Enclosure: As stated
cc w/encl: See next page

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