

UNITED STATES NUCLEAR REGULATORY COMMISSION
GEORGIA POWER COMPANY, ET AL.
DOCKET NOS. 50-424 AND 50-425
VOGTLE ELECTRIC GENERATING PLANT, UNITS 1 AND 2
NOTICE OF CONSIDERATION OF ISSUANCE OF AMENDMENTS TO
FACILITY OPERATING LICENSES AND OPPORTUNITY FOR A HEARING

The U.S. Nuclear Regulatory Commission (the Commission) is considering issuance of amendments to Facility Operating License Nos. NPF-68 and NPF-84, issued to the Georgia Power Company, et al. (the licensee), for operation of the Vogtle Electric Generating Plant (VEGP, Vogtle), Units 1 and 2, located at the licensee's site in Burke County, Georgia.

The proposed amendments, requested by the licensee in a letter dated May 1, 1995, would represent a full conversion from the current Technical Specifications (TS) to a set of TS based on NUREG-1431, "Standard Technical Specifications, Westinghouse Plants," Revision 1, dated April, 1995. NUREG-1431 was developed through working groups composed of NRC staff members and industry representatives and has been endorsed by the staff as part of an industry-wide initiative to standardize and improve the TS. As part of this submittal, the licensee has applied the criteria contained in the Commission's Final Policy Statement on Technical Specification Improvements for Nuclear Power Reactors of July 22, 1993, to the current Vogtle TS, and, using NUREG-1431 as a basis, developed a proposed set of improved TS for Vogtle. The criteria in the Final Policy Statement were subsequently added to 10 CFR 50.36, "Technical Specifications," in a rule change which became effective on August 18, 1995 (60 FR 36953).

The licensee has categorized the proposed changes to the existing TS into four general groupings. These groupings are characterized as administrative changes, relocated changes, more restrictive changes, and less restrictive changes.

Non-technical, administrative changes were intended to incorporate human-factors principles into the form and structure of the improved plant TS so that they would be easier to use for plant operations personnel. Administrative changes are editorial in nature or involve the reorganization or reformatting of requirements without affecting technical content or operational requirements. The proposed changes include: (a) providing the appropriate numbers, etc., for NUREG-1431 bracketed information (information which must be supplied on a plant-specific basis, and which may change from plant to plant), (b) identifying plant-specific wording for system names, etc., and (c) changing NUREG-1431 section wording to conform to existing licensee practices.

Relocated changes, those current TS requirements which do not satisfy or fall within any of the four criteria specified in the Commission's policy statement, may be relocated to appropriate licensee-controlled documents. The licensee's application states that such requirements will be relocated from the TS to administratively controlled documents such as the Final Safety Evaluation Report. Changes made to these documents will be made pursuant to 10 CFR 50.59 or other appropriate control mechanisms. These changes reduce the number of current TS requirements but the actual commitment to continue to perform the requirement will be unchanged upon implementation of the improved TS.

The licensee's proposed improved TS include certain more restrictive requirements than are contained in the current TS, which are either more conservative than corresponding requirements in the current TS, or are additional restrictions that are contained in NUREG-1431 but are not contained in the current TS. Examples of more restrictive requirements include: placing a limiting condition for operation (LCO) on plant equipment that is not required by the present TS to be operable; more restrictive requirements to restore inoperable equipment; and more restrictive surveillance requirements.

Less restrictive changes are those where current requirements are relaxed or eliminated, or new flexibility is provided. The more significant "less restrictive" requirements are justified on a case-by-case basis. When requirements have been shown to provide little or no safety benefit, their removal from the TS may be appropriate. In most cases, relaxations previously granted to individual plants on a plant-specific basis were the result of (a) generic NRC actions, (b) new NRC staff positions that have evolved from technological advancements and operating experience, or (c) resolution of the Owners Groups' comments on the improved Standard Technical Specifications. Generic relaxations contained in NUREG-1431 were reviewed by the staff and found to be acceptable because they are consistent with current licensing practices and NRC regulations. The licensee's design was reviewed to determine if the specific design basis and licensing basis are consistent with the technical basis for the model requirements in NUREG-1431 and thus provides a basis for these revised TS.

These administrative, relocated, more restrictive and less restrictive changes to the requirements of the current TS do not result in operations that

will alter assumptions relative to mitigation of an analyzed accident or transient event.

In addition to the changes described above, the licensee proposed certain changes to the current TS that are both less restrictive and are not within the scope of application for conversion to the guidance of NUREG-1431. All of the differences will be reviewed by the NRC staff and a determination will be made regarding the approval or disapproval of each item as a part of this licensing action. Specifically, the licensee identified the following instances where their submittal varied from the provisions of NUREG-1431.

Shutdown margin requirements for Mode 2 with $K_{eff} < 1.0$ are deleted. The applicability of the TS requirements for shutdown bank insertion limits, the requirement to verify a nonindicating rod position immediately after movement of more than 24 steps, and the required actions and surveillance requirements for quadrant power tilt ratio are revised.

With regard to reactor trip system (RTS) instrumentation, revisions are made to: (1) three NUREG-1431 surveillances based on Vogtle operating practices and vendor recommendations; (2) notes 1 and 2 to NUREG-1431 LCO 3.3.1 required actions for Condition T (one RTS channel inoperable); (3) the NUREG-1431 surveillance requirements for the P-7 interlock; (4) the completion time for required actions with one intermediate range neutron flux channel inoperable (LCO 3.3.1 Condition F); and, (5) the Applicable Modes or other specified conditions for the RTS Interlocks P-7, P-8, P-9, P-10, and P-13.

For the Engineered Safety Features Actuation System instrumentation, the surveillance intervals for the Channel Operational Test of the refueling water storage tank level Low-Low signal for Semi-Automatic Switchover to Containment

Emergency Sump function and the surveillance interval for the Channel Operational Test of the low Reactor Coolant System (RCS) T_{avg} function are increased from monthly to quarterly.

For the RCS, the surveillance interval for the pressurizer heater capacity is revised from 92 days to 18 months. Also, the completion time for depressurizing the RCS in the event of an inoperable cold overpressure protection system is revised from 8 hours to 12 hours, and the requirements for performing RCS water inventory balances are revised. The lift setpoints for the residual heat removal suction relief valves, the RCS vent capacity for cold overpressure protection, and the pressurizer safety valve lift settings are moved to the Bases of the improved TS.

For the Emergency Core Cooling System, the requirements for seal water injection flow is revised to locate the limits for seal injection flow to the Bases. Also, the Mode 4 requirements are revised.

For containment systems, the current allowed outage time for the containment spray and cooling systems is revised from 72 hours to 14 days, and the air lock door interlock mechanism surveillance frequency is revised from the current frequency of 6 months to 18 months. An allowance to open the 14-inch purge valves for maintenance testing is added.

For other plant systems, the Condensate Storage Tank LCO is revised consistent with a planned design modification that will result in two 100% capacity tanks. The surveillance requirement to operate the Piping Penetration Area Filtration and Exhaust System (PPAFES) monthly for ≥ 10 continuous hours is revised to operate for ≥ 15 minutes and the heater capacity verification is deleted. The Completion Time to reduce the Power Range Neutron Flux High Trip setpoints is increased from 4 hours to 12 hours.

The currently licensed footnote in the Control Room Emergency Filtration System (CREFS) LCO that requires Train B CREFS to be started whenever a CREFS train must be placed in service to comply with Actions is deleted. A Note is added to the current LCO for the engineered safety feature room coolers and the safety-related chiller system providing an exception to the LCO for surveillance testing.

For electrical systems, several revisions to the LCO for AC Sources - Operating were proposed to support the addition of a new Standby Auxiliary Transformer as a Unit 1 and Unit 2 common offsite circuit. Also, the diesel generator accelerated test frequency requirements in the AC Sources - Operating LCO are relocated outside of the TS.

Regarding administrative controls, the Ventilation Filter Testing Program is revised consistent with the proposed change in the Plant Systems Chapter for the deletion of the heater capacity test for the PPAFES. The reference to the ASTM standard in Paragraph 5.5.13.c is deleted.

Before issuance of the proposed license amendments, the Commission will have made findings required by the Atomic Energy Act of 1954, as amended (the Act) and the Commission's regulations.

By October 10, 1995, the licensee may file a request for a hearing with respect to issuance of the amendments to the subject facility operating licenses and any person whose interest may be affected by this proceeding and who wishes to participate as a party in the proceeding must file a written request for a hearing and a petition for leave to intervene. Requests for a hearing and a petition for leave to intervene shall be filed in accordance with the Commission's "Rules of Practice for Domestic Licensing Proceedings" in 10 CFR Part 2. Interested persons should consult a current

copy of 10 CFR 2.714 which is available at the Commission's Public Document Room, the Gelman Building, 2120 L Street, NW., Washington, DC, and at the local public document room located at the Burke County Library, 412 Fourth Street, Waynesboro, Georgia. If a request for a hearing or petition for leave to intervene is filed by the above date, the Commission or an Atomic Safety and Licensing Board, designated by the Commission or by the Chairman of the Atomic Safety and Licensing Board Panel, will rule on the request and/or petition; and the Secretary or the designated Atomic Safety and Licensing Board will issue a notice of hearing or an appropriate order.

As required by 10 CFR 2.714, a petition for leave to intervene shall set forth with particularity the interest of the petitioner in the proceeding, and how that interest may be affected by the results of the proceeding. The petition should specifically explain the reasons why intervention should be permitted with particular reference to the following factors: (1) the nature of the petitioner's right under the Act to be made a party to the proceeding; (2) the nature and extent of the petitioner's property, financial, or other interest in the proceeding; and (3) the possible effect of any order which may be entered in the proceeding on the petitioner's interest. The petition should also identify the specific aspect(s) of the subject matter of the proceeding as to which petitioner wishes to intervene. Any person who has filed a petition for leave to intervene or who has been admitted as a party may amend the petition without requesting leave of the Board up to 15 days prior to the first prehearing conference scheduled in the proceeding, but such an amended petition must satisfy the specificity requirements described above.

Not later than 15 days prior to the first prehearing conference scheduled in the proceeding, a petitioner shall file a supplement to the

petition to intervene which must include a list of the contentions which are sought to be litigated in the matter. Each contention must consist of a specific statement of the issue of law or fact to be raised or controverted. In addition, the petitioner shall provide a brief explanation of the bases of the contention and a concise statement of the alleged facts or expert opinion which support the contention and on which the petitioner intends to rely in proving the contention at the hearing. The petitioner must also provide references to those specific sources and documents of which the petitioner is aware and on which the petitioner intends to rely to establish those facts or expert opinion. Petitioner must provide sufficient information to show that a genuine dispute exists with the applicant on a material issue of law or fact. Contentions shall be limited to matters within the scope of the amendment under consideration. The contention must be one which, if proven, would entitle the petitioner to relief. A petitioner who fails to file such a supplement which satisfies these requirements with respect to at least one contention will not be permitted to participate as a party.

Those permitted to intervene become parties to the proceeding, subject to any limitations in the order granting leave to intervene, and have the opportunity to participate fully in the conduct of the hearing, including the opportunity to present evidence and cross-examine witnesses.

A request for a hearing or a petition for leave to intervene must be filed with the Secretary of the Commission, U.S. Nuclear Regulatory Commission, Washington, DC 20555, Attention: Docketing and Services Branch, or may be delivered to the Commission's Public Document Room, the Gelman Building, 2120 L Street, NW., Washington, DC, by the above date. Where petitions are filed during the last 10 days of the notice period, it is

requested that the petitioner promptly so inform the Commission by a toll-free telephone call to Western Union at 1-(800) 248-5100 (in Missouri 1-(800) 342-6700). The Western Union operator should be given Datagram Identification Number N1023 and the following message addressed to Herbert N. Berkow, Director, Project Directorate II-2: petitioner's name and telephone number; date petition was mailed; plant name; and publication date and page number of this FEDERAL REGISTER notice. A copy of the petition should also be sent to the Office of the General Counsel, U.S. Nuclear Regulatory Commission, Washington, DC 20555, and to Ernest L. Blake, Jr, Esquire, Shaw, Pittman, Potts, and Trowbridge, 2300 N Street, NW., Washington, DC 20037, attorney for the licensee.

Nontimely filings of petitions for leave to intervene, amended petitions, supplemental petitions and/or requests for hearing will not be entertained absent a determination by the Commission, the presiding officer or the presiding Atomic Safety and Licensing Board that the petition and/or request should be granted based upon a balancing of the factors specified in 10 CFR 2.714(a)(1)(i)-(v) and 2.714(d).

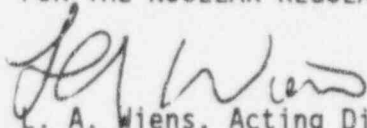
If a request for a hearing is received, the Commission's staff may issue the amendments after it completes its technical review and prior to the completion of any required hearing if it publishes a further notice for public comment of its proposed finding of no significant hazards consideration in accordance with 10 CFR 50.91 and 50.92.

For further details with respect to this action, see the application for amendments dated May 1, 1995, which is available for public inspection at the Commission's Public Document Room, the Gelman Building, 2120 L Street, NW.,

Washington, DC, and at the local public document room located at the Burke County Library, 412 Fourth Street, Waynesboro, Georgia.

Dated at Rockville, Maryland, this 31st day of August 1995.

FOR THE NUCLEAR REGULATORY COMMISSION



L. A. Wiens, Acting Director
Project Directorate II-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation