

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Nine Mile Point Unit #1	DOCKET NUMBER (2) 0 5 0 0 0 2 2 0	PAGE (3) 1 OF 0 2
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TITLE (4)
SCRAM Resulting from spurious IRM trips on different channels of RPS

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)		
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)
0 5 0	9 8 4	8 4	8 4	0 0 5	0 0	0 6 0	8 8 4				0 5 0 0 0
											0 5 0 0 0

THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)

OPERATING MODE (9) N	20.402(b)	20.405(e)	<input checked="" type="checkbox"/>	80.73(a)(2)(iv)	73.71(b)
POWER LEVEL (10) 0 1 0 0	20.405(a)(1)(i)	80.38(a)(1)	<input type="checkbox"/>	80.73(a)(2)(v)	73.71(c)
	20.405(a)(1)(ii)	80.38(a)(2)	<input type="checkbox"/>	80.73(a)(2)(vi)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)
	20.405(a)(1)(iii)	80.73(a)(2)(i)	<input type="checkbox"/>	80.73(a)(2)(vii)(A)	
	20.405(a)(1)(iv)	80.73(a)(2)(ii)	<input type="checkbox"/>	80.73(a)(2)(vii)(B)	
	20.405(a)(1)(v)	80.73(a)(2)(iii)	<input type="checkbox"/>	80.73(a)(2)(ix)	

LICENSEE CONTACT FOR THIS LER (12)

NAME Robert G. Randall - Tech. Services Supervisor	TELEPHONE NUMBER AREA CODE 3 1 5 3 4 9 - 2 4 4 5
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COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS
X	J	E J I	G 0 8 0	N					

SUPPLEMENTAL REPORT EXPECTED (14)

<input type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE) <input checked="" type="checkbox"/> NO	EXPECTED SUBMISSION DATE (15) MONTH DAY YEAR
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ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

ABSTRACT

While in a refueling outage, spurious trips of the Intermediate Range Monitors (IRM), one on each channel of the Reactor Protection System (RPS), occurred. As a direct result of this, a full SCRAM signal was produced. The cause of these trips has been attributed to electromagnetic interference caused by welding in the drywell at the time of this event. No corrective actions were taken for this event, since the conditions which caused this event to occur would not be present during normal reactor operation.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

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		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER		

TEXT (If more space is required, use additional NRC Form 366A's) (17)

TEXT

At 8:43 a.m. on May 9, 1984, while in a refueling outage, a spurious trip of two Intermediate Range Monitors (IRM) occurred on monitors #11 and #16. Since each monitor was on a different channel of the Reactor Protection System, a full SCRAM signal was produced. The cause of these trips has been attributed to electromagnetic interference, which was caused by welding being done in the drywell at the time of the event. Prior and subsequent to this incident, weekly surveillance testing was satisfactorily performed on the IRM monitors.

ASSESSMENT OF SAFETY CONSEQUENCES

There are no potential safety consequences resulting from this event because: 1) The reactor was defueled at the time of the event; 2) Work would not be performed in the drywell while the plant is in operation; and 3) The IRM is not used in normal operation since its range is not high enough, and a spike from welding would not be as great in magnitude on the Average Power Range Monitor (APRM) so as to cause an APRM to trip due to welding interference.

CORRECTIVE ACTION

No corrective action was taken, since the conditions which caused this event to occur would not be present during normal reactor operation.

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NIAGARA MOHAWK POWER CORPORATION

NIAGARA  MOHAWK

NMP-8569

300 ERIE BOULEVARD, WEST
SYRACUSE, N. Y. 13202

June 8, 1984

United States Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

RE: Docket No. 50-220
LER 84-05

Gentlemen:

In accordance with 10 CFR 50.73, we hereby submit the following Licensee Event Report:

LER 84-05 Which is being submitted in accordance with 10 CFR 50.73 (a) (2) (iv), "Any event or condition that resulted in manual or automatic activation of any Engineered Safety Feature (ESF), including the Reactor Protection System (RPS)."

A 10 CFR 50.72 report was made at 0920 hrs on 5-9-84. This report was completed in the format designated in NUREG-1022, dated September 1983.

Very truly yours,



Thomas E. Lempges
Vice President
Nuclear Generation

TEL/ta
Attachments (3 copies)

cc: Dr. Thomas E. Murley
Regional Administrator

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