

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) SAN ONOFRE NUCLEAR GENERATING STATION, UNIT 3	DOCKET NUMBER (2) 0 5 0 0 0 3 6 2	PAGE (3) 1 OF 0 2
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TITLE (4)
CONTAINMENT ISOLATION VALVES SURVEILLANCE MISSED

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)																																																													
MONTH	DAY	YEAR	YEAR	SEQ. NUMBER	REV. NUMBER	MONTH	DAY	YEAR	FACILITY NAME(S)		DOCKET NUMBER(S)																																																											
0 5	0 5	8 4	8 4	0 1 9	0 0	0 5	3 1 1	8 4			0 5 0 0 0 0																																																											
<table border="1" style="width:100%"> <tr> <td>OPERATING MODE (9)</td> <td>1</td> <td colspan="10">THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)</td> </tr> <tr> <td rowspan="5">POWER LEVEL (10)</td> <td>0 6 0</td> <td>20.402(b)</td> <td></td> <td>20.405(c)</td> <td></td> <td>50.73(a)(2)(iv)</td> <td></td> <td>73.71(b)</td> </tr> <tr> <td></td> <td>20.405(a)(1)(i)</td> <td></td> <td>50.36(c)(1)</td> <td></td> <td>50.73(a)(2)(v)</td> <td></td> <td>73.71(c)</td> </tr> <tr> <td></td> <td>20.405(a)(1)(ii)</td> <td></td> <td>50.36(c)(2)</td> <td></td> <td>50.73(a)(2)(vii)</td> <td></td> <td rowspan="3">OTHER (Specify in Abstract below and in Text, NRC Form 366A)</td> </tr> <tr> <td></td> <td>20.405(a)(1)(iii)</td> <td>X</td> <td>50.73(a)(2)(i)</td> <td></td> <td>50.73(a)(2)(viii)(A)</td> <td></td> </tr> <tr> <td></td> <td>20.405(a)(1)(iv)</td> <td></td> <td>50.73(a)(2)(ii)</td> <td></td> <td>50.73(a)(2)(viii)(B)</td> <td></td> </tr> <tr> <td></td> <td>20.405(a)(1)(v)</td> <td></td> <td>50.73(a)(2)(iii)</td> <td></td> <td>50.73(a)(2)(x)</td> <td></td> <td></td> </tr> </table>												OPERATING MODE (9)	1	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)										POWER LEVEL (10)	0 6 0	20.402(b)		20.405(c)		50.73(a)(2)(iv)		73.71(b)		20.405(a)(1)(i)		50.36(c)(1)		50.73(a)(2)(v)		73.71(c)		20.405(a)(1)(ii)		50.36(c)(2)		50.73(a)(2)(vii)		OTHER (Specify in Abstract below and in Text, NRC Form 366A)		20.405(a)(1)(iii)	X	50.73(a)(2)(i)		50.73(a)(2)(viii)(A)			20.405(a)(1)(iv)		50.73(a)(2)(ii)		50.73(a)(2)(viii)(B)			20.405(a)(1)(v)		50.73(a)(2)(iii)		50.73(a)(2)(x)		
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LICENSEE CONTACT FOR THIS LER (12)

NAME J. G. HAYNES, STATION MANAGER	TELEPHONE NUMBER 7 1 4 4 9 2 - 7 7 0 0
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COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRRDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRRDS

SUPPLEMENTAL REPORT EXPECTED (14)

YES (If yes, complete EXPECTED SUBMISSION DATE) NO

EXPECTED SUBMISSION DATE (15)

MONTH	DAY	YEAR

Abstract (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On 5/6/84 and 5/7/84, it was determined that post maintenance testing was not performed on 5/5/84 following packing adjustment to reduce leakage on Containment Isolation Valves 3HV9205 and 3HV9218, respectively, as required by Technical Specification Surveillance Requirement 4.6.3.1. Post maintenance testing had not been specified when the packing adjustment had been requested based on the erroneous belief that packing adjustment could not affect operability of the valves. 3HV9205 and 3HV9218 were retested satisfactorily on 5/6/84 at 1015, and on 5/7/84 at 1555, respectively.

As corrective action, all Operations personnel will be briefed on the circumstances of this event. This briefing will emphasize the need for retesting following any maintenance on safety-related equipment.

Handwritten signature/initials

LICENSEE EVENT REPORT (LER)
TEXT CONTINUATION

FACILITY NAME (1) SAN ONOFRE NUCLEAR GENERATING STATION, UNIT 3	DOCKET NUMBER (2) 0 5 0 0 0 3 6 2	LER NUMBER (6)			PAGE (3)	
		YEAR	SEQ. NUMBER	REV. NUMBER		
		8 4	- 0 1 9	- 0 0	0 2	OF 0 2

TEXT (If more space is required, use additional NRC Form 366A's) (17)

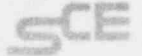
On May 6, and May 7, 1984, it was determined that post maintenance testing was not performed on May 5, 1984, following packing adjustment to reduce leakage on Containment Isolation Valves 3HV9205 and 3HV9218 (EIIIS Component Code ISV), respectively, as required by Technical Specification Surveillance Requirement 4.6.3.1. Post maintenance testing had not been specified when the packing adjustment had been requested based on the erroneous belief that packing adjustment could not affect operability of the valves. 3HV9205 and 3HV9218 were satisfactorily retested on May 6, 1984, at 1015, and on May 7, 1984, at 1555, respectively.

Since 3HV9205 and 3HV9218 were retested satisfactorily, the packing adjustments had not affected operability. Additionally, the penetrations isolated by 3HV9205 and 3HV9218 have redundant containment isolation valves that remained operable. Therefore, even if the packing adjustment had affected the operability of 3HV9205 and 3HV9218, the penetrations could have been isolated if required.

As corrective action, all Operations personnel will be briefed on the circumstances of this event. This briefing will emphasize the need for retesting following any maintenance on safety-related equipment.

There are no reasonable or credible circumstances under which this event would have been more severe.

Southern California Edison Company



SAN ONOFRE NUCLEAR GENERATING STATION
P.O. BOX 128
SAN CLEMENTE, CALIFORNIA 92672

J. G. HAYNES
STATION MANAGER

TELEPHONE
(714) 492-7700

May 31, 1984

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555

Subject: Docket No. 50-362
30-Day Report
Licensee Event Report No. 84-019
San Onofre Nuclear Generating Station, Unit 3

Pursuant to 10 CFR 50.73(a)(2)(i)(B), this submittal provides the required 30-day written Licensee Event Report (LER) for an occurrence involving Technical Specification Surveillance Requirement 4.6.3.1. The health and safety of plant personnel or the public were not affected by this event.

If you require any additional information, please so advise.

Sincerely,

JG Haynes / JG Haynes

Enclosure: LER 84-019

cc: A. E. Chaffee (USNRC Resident Inspector, Units 1, 2 and 3)
J. P. Stewart (USNRC Resident Inspector, Units 2 and 3)

J. B. Martin (Regional Administrator, NRC Region V)

Institute of Nuclear Power Operations (INPO)

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