

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Sequoyah, Unit 1	DOCKET NUMBER (2) 0 5 0 0 0 3 2 7	PAGE (3) 1 OF 0 2
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TITLE (4)
Reactor Trip

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)			
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES			DOCKET NUMBER(S)
0 5	1 1	8 4	8 4	0 3 3	0 0	0 6	0 5	8 4				0 5 0 0 0
												0 5 0 0 0

OPERATING MODE (9) 1	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR § (Check one or more of the following) (11)									
POWER LEVEL (10) 0 1 2 1 1	20.402(b)	20.405(c)	<input checked="" type="checkbox"/>	50.73(a)(2)(iv)	73.71(b)					
	20.405(a)(1)(i)	50.38(c)(1)		50.73(a)(2)(v)	73.71(c)					
	20.405(a)(1)(iii)	50.38(c)(2)		50.73(a)(2)(vii)	OTHER (Specify in Abstract below and in Text, NRC Form 365A)					
	20.405(a)(1)(iii)	50.73(a)(2)(i)		50.73(a)(2)(viii)(A)						
	20.405(a)(1)(iv)	50.73(a)(2)(ii)		50.73(a)(2)(viii)(B)						
20.405(a)(1)(v)	50.73(a)(2)(iii)		50.73(a)(2)(x)							

LICENSEE CONTACT FOR THIS LER (12)					
NAME Glenn B. Kirk, Compliance Section Engineer				TELEPHONE NUMBER 6 1 5 8 7 0 - 6 1 4 6	

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)										
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	
X	S	B F C V	C 6 3 5	Y						

SUPPLEMENTAL REPORT EXPECTED (14)			EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
<input type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE) <input checked="" type="checkbox"/> NO						

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

During a normal startup on 05/11/84 at 1400 CST, unit 1 experienced an automatic reactor trip on low-low level in the number 2 steam generator following a turbine trip. Just prior to the event, unit 1 was in mode 1 (554 degrees F, 2235 psig) at 21% reactor power.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

FACILITY NAME (1) Sequoyah, Unit 1	DOCKET NUMBER (2) 0 5 0 0 0 3 2 7 8 4	LER NUMBER (6)			PAGE (3)	
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER		
		8 4	- 0 3 3	- 0 0	0 2	OF 0 2

TEXT (If more space is required, use additional NRC Form 366A's) (17)

During a normal reactor startup following a reactor trip on 05/10/84 (see SQRO-50-327/84032), unit 1 experienced another reactor trip. Just prior to the event which occurred at 1400 CST on 05/11/84, unit 1 was in mode 1 (554 degrees F, 2235 psig) at 21% reactor power with steam generator level controls in manual.

Steam dump valve 1-FCV-1-107 spuriously opened resulting in swell of steam generator levels. This action resulted in a high-high level in the number 3 steam generator which caused a turbine trip and main feedwater isolation to all four steam generators. The transient then resulted in shrink of steam generator levels causing a reactor trip on low-low level in steam generator number 2. All equipment and personnel responded as expected and the unit stabilized at 547 degrees F.

Inspection revealed a broken repositioning linkage on steam dump valve 1-FCV-1-107 allowing the valve to cycle erratically. The repositioning linkage was replaced and the valve was returned to service.

There was no effect on public health or safety. For 1984, this was the sixth automatic reactor trip on low-low steam generator level and the second event initiated by the opening of a steam dump valve.

TENNESSEE VALLEY AUTHORITY

Sequoyah Nuclear Plant
Post Office Box 2000
Soddy Daisy, Tennessee 37379

June 5, 1984

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

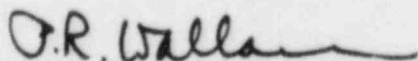
Gentlemen:

TENNESSEE VALLEY AUTHORITY - SEQUOYAH NUCLEAR PLANT UNIT 1 - DOCKET NO.
50-327 - FACILITY OPERATING LICENSE DPR-77 - REPORTABLE OCCURRENCE REPORT
SQRO-50-327/84033

The enclosed licensee event report provides details concerning a reactor
trip from 21% reactor power. This event is reported in accordance with
10 CFR 50.73, paragraph a.2.iv.

Very truly yours,

TENNESSEE VALLEY AUTHORITY



for C. C. Mason
Power Plant Superintendent

Enclosure
cc (Enclosure):

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Atlanta, Georgia 30303

Records Center
Institute of Nuclear Power Operations
Suite 1500
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Atlanta, Georgia 30339

NRC Inspector, NUC PR, Sequoyah

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