

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Kewaunee Nuclear Power Plant	DOCKET NUMBER (2) 0 5 0 0 0 3 0 5	PAGE (3) 1 OF 0 2
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TITLE (4)
Reactor Trip on SB 1B Lo-Lo Level

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)		
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)
									NA		0 5 0 0 0
0 5	0 7	8 4	8 4	0 1 0	0 0	0 6	0 6	8 4			0 5 0 0 0

THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR § (Check one or more of the following) (11)

OPERATING MODE (9) N	20.402(b)	20.408(e)	<input checked="" type="checkbox"/>	80.73(a)(2)(iv)	73.71(b)
POWER LEVEL (10) 0 1 0	20.405(a)(1)(ii)	80.36(e)(1)	<input type="checkbox"/>	80.73(a)(2)(v)	73.71(c)
	20.405(a)(1)(iii)	80.36(e)(2)	<input type="checkbox"/>	80.73(a)(2)(vi)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)
	20.405(a)(1)(iii)	80.73(a)(2)(ii)	<input type="checkbox"/>	80.73(a)(2)(vii)(A)	
	20.405(a)(1)(iv)	80.73(a)(2)(iii)	<input type="checkbox"/>	80.73(a)(2)(vii)(B)	
	20.405(a)(1)(iv)	80.73(a)(2)(iii)	<input type="checkbox"/>	80.73(a)(2)(ix)	

LICENSEE CONTACT FOR THIS LER (12)

NAME Edwin Hiltunen - Plant Nuclear Engineer	TELEPHONE NUMBER AREA CODE: 4 1 4 3 8 8 - 2 5 6 0
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COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC
A									

SUPPLEMENTAL REPORT EXPECTED (14)

<input type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE)	<input checked="" type="checkbox"/> NO	EXPECTED SUBMISSION DATE (15) NA	MONTH	DAY	YEAR
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ABSTRACT (Limit to 1400 spaces; i.e., approximately fifteen single space typewritten lines) (16)

On May 7, 1984, a plant escalation from 10% to 25% of full power was underway. Main Feedwater controls were in manual using the feedwater bypass control valves. Following the 2300 shift turnover, the control operator was controlling feedwater flow to the steam generators (SG) to match the increasing steam demand and maintain SG narrow range (NR) levels between 35% and 40%. While the operator attempted to stabilize a SG level oscillation, the lo-lo level setpoint (17% NR level) was reached in SG 1B and initiated a reactor trip. Plant operating procedures were followed to place the plant in the hot shutdown operating mode. No equipment or system failures contributed to this event.

Although no additional corrective actions are necessary at this time, the sensitivity of steam generator level control in manual is being addressed as part of the human factors associated with the control room design review program.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

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		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER		
			0 1 0	0 0	0 2	OF

TEXT (If more space is required, use additional NRC Form 388A's) (17)

Following the scheduled annual refueling outage, a power escalation from 10% to 25% of full power was being made on May 7, 1984. This escalation had been started prior to the 2300 shift turnover and was being continued by the relieving operations crew.

Power was being increased at 1/2% per minute. Control of Main Feedwater (SJ) was in manual, using the feedwater bypass control valves (V), FW10A and FW10B. Using valve demand and the Steam Generator Narrow Range level indications, (LI) the operator attempted to match the increasing steam demand while maintaining an indicated narrow range level between 35% and 40%. The lo-lo level setpoint (17% narrow range level) was reached in steam generator 1B, while the operator adjusted the feedwater bypass control valves in an attempt to stabilize a level oscillation. Per design, the reactor tripped on the lo-lo level signal.

The reactor/turbine trip was followed by the immediate operator actions of procedure E-0-04, "Turbine and Reactor Trip", and the plant was placed in the hot shutdown operating mode. No equipment or system failures contributed to this event.

Although no additional corrective actions are necessary at this time, the sensitivity of steam generator level control in manual is being addressed as part of the human factors associated with the control room design review program.

WISCONSIN PUBLIC SERVICE CORPORATION



P.O. Box 1200, Green Bay, Wisconsin 54305

June 6, 1984

U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555

Gentlemen:

Docket 50-305
Operating License DPR-43
Kewaunee Nuclear Power Plant
Reportable Occurrence 84-010-00

In accordance with the requirements of 10 CFR 50.73 "Licensee Event Report System", the attached Licensee Event Report for reportable occurrence 84-010-00 is being submitted.

Very truly yours,

A handwritten signature in cursive script, appearing to read "C. W. Giesler".

C. W. Giesler
Vice President - Power Production

DJM/jks

Attach.

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