

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Washington Nuclear Plant - Unit 2	DOCKET NUMBER (2) 0 5 0 0 0 3 9 1 7	PAGE (3) 1 OF 0 2
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TITLE (4)
Reactor Scram

EVENT DATE (5)			LER NUMBER (6)		REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES
0 5	1 7	8 4	8 4	0 4	2	0 6	0 8	8 4	
									DOCKET NUMBER(S) 0 5 0 0 0
									0 5 0 0 0

OPERATING MODE (9) 1

POWER LEVEL (10) 0.2 0

THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)

<input type="checkbox"/> 20.402(b)	<input type="checkbox"/> 20.406(c)	<input type="checkbox"/> 50.73(a)(2)(iv)	<input type="checkbox"/> 73.71(b)
<input type="checkbox"/> 20.406(a)(1)(i)	<input type="checkbox"/> 50.36(c)(1)	<input type="checkbox"/> 50.73(a)(2)(v)	<input type="checkbox"/> 73.71(c)
<input type="checkbox"/> 20.406(a)(1)(ii)	<input type="checkbox"/> 50.36(c)(2)	<input type="checkbox"/> 50.73(a)(2)(vii)	<input checked="" type="checkbox"/> OTHER (Specify in Abstract below and in Text, NRC Form 366A)
<input type="checkbox"/> 20.406(a)(1)(iii)	<input type="checkbox"/> 50.73(a)(2)(i)	<input type="checkbox"/> 50.73(a)(2)(viii)(A)	50.72(c)(2)(ii)
<input type="checkbox"/> 20.406(a)(1)(iv)	<input type="checkbox"/> 50.73(a)(2)(ii)	<input type="checkbox"/> 50.73(a)(2)(viii)(B)	
<input type="checkbox"/> 20.406(a)(1)(v)	<input type="checkbox"/> 50.73(a)(2)(iii)	<input type="checkbox"/> 50.73(a)(2)(ix)	

LICENSEE CONTACT FOR THIS LER (12)

NAME L.D. Kassakatis, Plant Compliance Engineer	TELEPHONE NUMBER 510 931 771-1 25101 1
AREA CODE 510 931 771-1 25101 1	

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13) Ext. 2201

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRRDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRRDS
BC	CHA	Turbine	D 0 5 5	N					

SUPPLEMENTAL REPORT EXPECTED (14)

YES (if yes, complete EXPECTED SUBMISSION DATE) NO

EXPECTED SUBMISSION DATE (15)

MONTH	DAY	YEAR

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On May 17, 1984 at 1016 the Reactor was manually shutdown due to loss of control of vessel water level as a result of a malfunction with mechanical linkage from a servo control motor designed to control Reactor feedpump turbine speed.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

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		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
		0 4 2	0 0 0	2	OF	0 2	

TEXT (If more space is required, use additional NRC Form 388A's) (17)

Plant Conditions

- (a) Reactor Power - 17%
- (b) Vessel Pressure - 935 Psig
- (c) 40% Bypass Valve Position
- (d) Power Ascension Testing
- (e) Reactor - Mode 1

Event

On May 17, 1984 at approximately 1010 a Control Room Operator received a "Feed Pump on Minimum Speed" annunciator. The Operator observed feed pump turbine speed had decreased from its demanded speed of 4000 RPM to 3300 RPM (idle speed). At that speed pump discharge pressure was insufficient to supply feedwater to the Reactor resulting in a slowly decreasing vessel level.

Attempts to gain control of the turbine proved futile prompting the Operator to reduce Reactor pressure to 875 Psig. This reduction in pressure was successful in restoring vessel water level to normal.

However, at approximately 1016 feed pump turbine speed dropped to 300 RPM resulting in a rapidly decreasing Reactor water level. The Reactor was manually shutdown prior to reaching the Reactor low water level Reactor Protection System (RPS) actuation setpoint.

Immediate Corrective Action

Investigation subsequent to the event disclosed two socket head hex screws designed to retain the set position of the feed pump turbine servo control feedback mechanical linkage had backed out, allowing a loss of speed control due to the linkage slipping on its shaft. The malfunctioning linkage was repositioned, its retaining screws were re-tightened and staked in position to preclude reoccurrence.

Further Corrective Action

Inspection of the other feedpump turbine unit was performed. The retaining screws on the servo control feedback mechanical linkage were similarly staked to preclude loosening.

Safety Significance

Any loss of level control is a potentially significant event; however, in this failure Operator action mitigated any potential hazard and precluded ESF activation. This event posed no danger to Plant personnel or to the public.

Washington Public Power Supply System

P.O. Box 968 3000 George Washington Way Richland, Washington 99352 (509) 372-5000

Docket No. 50-397
June 8, 1984

Document Control Desk
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

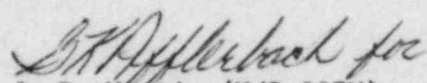
Subject: NUCLEAR PLANT NO. 2
LICENSEE EVENT REPORT NO. 84-042

Dear Sir:

Transmitted herewith is Licensee Event Report No. 84-042 for WNP-2 Plant. This report is submitted in response to the report requirements of Technical Specification Section 6.9.1.7 and discusses the item of noncompliance, corrective action taken, and action taken to preclude recurrence.

This is the follow-up report to the verbal notification given at 1105 hours on May 17, 1984.

Very truly yours,


J. D. Martin (M/D 927M)
WNP-2 Plant Manager

JDM:mm

Enclosure:
Licensee Event Report No. 84-042

cc: Mr. John B. Martin, Administrator
Region V, Office of Inspection and Enforcement
U.S. Nuclear Regulatory Commission
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Walnut Creek, California 94596
Mr. A. D. Toth, NRC Resident Inspector (901A)
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