

JAN 28 1992

Docket Nos. 50-445
50-446
License No. NPF-87
Construction Permit No. CPPR-127
EA No. 91-187

TU Electric
ATTN: W. J. Cahill, Jr., Executive
Vice President, Nuclear
Skyway Tower
400 North Olive Street, Lock Box 81
Dallas, Texas 75201

Gentlemen:

SUBJECT: NRC INSPECTION REPORT NOS. 50-445/91-66; 50-446/91-66

This refers to the inspection conducted by D. N. Graves, R. M. Latta, and R. J. Evans during the period November 20, 1991, through January 7, 1992, of activities authorized for the Comanche Peak Steam Electric Station, Unit 2. At the conclusion of the inspection, the findings were discussed with H. D. Bruner and those members of your staff identified in the enclosed report.

Areas examined during the inspection included followup on previously identified items and NRC Bulletins; followup on construction deficiencies; preoperational test program implementation verification, including plant tours, test witnessing, work observations and the system takeover process; and corrective actions.

Several strengths were identified during this reporting period, which are identified in the enclosed inspection report. Strong management support was evident in response to Significant Deficiency Analysis Report CP-91-04 (paragraph 4.5) in that significant engineering and construction resources were applied to correct the identified deficiency associated with rejectable weld indications in three containment electrical penetrations. The response to the falsified test data (paragraph 6.4) was prompt and extensive. Excellent oversight was demonstrated by the quality assurance organization in the early identification and resolution of the investment recovery yard issue (paragraph 8.1). One additional strength, not noted in the inspection report, was the high quality and completeness of the packages that the Unit 2 licensing organization presented to the NRC for closure of open and outstanding inspection items.

However, based on the results of this inspection, certain of your activities were in violation of NRC requirements. As discussed in paragraph 6.4 of the enclosed inspection report, NRC concluded that the falsification of prerequisite test data in November 1991 constituted a violation of 10 CFR 50.9, which stipulates that information required to be maintained by an applicant must be accurate in all material respects. While the NRC does not condone

*RIV:SRI:DRP/B	*RI:DRP/B	*PE:DRP/B	*RI:DRP/D	*C:DRP/B	EO
DNGraves:ww	RM.Latta	CEJohnson	RJEvans	LAYandel	GFCamborn
1/22/92	1/22/92	1/22/92	1/22/92	1/22/92	1/23/92

DE	DD:DRP	DD:DRP
JLieberman	TPGwynn	ABBeach
1/27/92	1/27/92	1/28/92

by memo on 12/11/91
*previously concurred

Coordinated
w/CI 1/23/92

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PDR ADDOCK 05000445
G PDR

IE/4
111

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violations of this nature and, in fact, considers the falsification of any required record a very serious matter, the NRC has considered the circumstances of this matter and has decided that no citation will be issued. This decision was made in accordance with the discretionary provisions of Section 2.6 of the NRC Enforcement Policy (10 CFR Part 2, Appendix C) and after consultation with the Director, Office of Enforcement.

In making this decision, NRC took into consideration TU Electric's discovery of this matter, its having promptly brought this matter to NRC's attention, the apparent involvement of a single individual at a relatively low level in the startup organization, and TU Electric's prompt and extensive actions upon discovery, including disciplinary action against the involved individual, a review of the individual's previous work for suspect data, and a discussion of the event with startup personnel. These actions are described in more detail in the inspection report.

In accordance with 10 CFR 2.790 of the NRC's "Rules of Practice," a copy of this letter and its enclosure will be placed in the NRC Public Document Room.

Should you have any questions concerning this inspection, we will be pleased to discuss them with you.

Sincerely,

Original Signed By:
Thomas P. Gwynn

A. Bill Beach,
Division of Reactors ts

Enclosure:
Appendix - NRC Inspection Report
50-445/91-66; 50-446/91-66

cc w/enclosure:
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