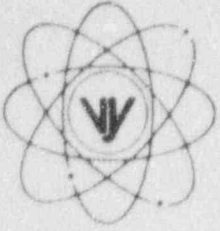


VERMONT YANKEE NUCLEAR POWER CORPORATION



P.O. Box 157, Governor Hunt Road
Vernon, Vermont 05354-0157
(802) 257-7711

August 25, 1995
BVY 95-93

U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

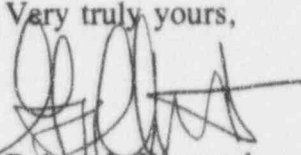
Attn: Document Control Desk

References: Operating License DPR-28
Docket No. 50-271
Reportable Occurrence No. LER 95-016

Dear Sir:

As defined by 10 CFR 50.73, we are reporting the attached Reportable Occurrence as LER 95-016.

Very truly yours,


for Robert J. Wanczyk
Plant Manager

RJW/dm

cc: Regional Administrator
USNRC
Region I
475 Allendale Road
King of Prussia, PA 19406

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PDR ADDCK 05000271
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LICENSEE EVENT REPORT (LER)

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS MANDATORY INFORMATION COLLECTION REQUEST: 50.0 HRS. REPORTED LESSONS LEARNED ARE INCORPORATED INTO THE LICENSING PROCESS AND FED BACK TO INDUSTRY. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE INFORMATION AND RECORDS MANAGEMENT BRANCH (T-6 F33), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555-0001, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1)	VERMONT YANKEE NUCLEAR POWER STATION	DOCKET NUMBER ()	PAGE (3)
		05000271	01 OF 04

TITLE (4) Stack Particulate Filter used in Composite Sample to detect Alpha and Strontium 89 and 90 was Misplaced due to Inadequate Tracking Methods

EVENT DATE (5)			LER NUMBER (6)				REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAME	DOCKET NO.(S)	
07	28	95	95	-- 016 --	0	08	28	95	N/A	05000	

OPERATING MODE (9)	N	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: CHECK ONE OR MORE (11)								
POWER LEVEL (10)	100	20.2201(b)	20.2203(a)(2)(v)	X	50.73(a)(2)(i)	50.73(a)(2)(viii)				
		20.2203(a)(1)	20.2203(a)(3)(i)		50.73(a)(2)(ii)	50.73(a)(2)(x)				
		20.2203(a)(2)(i)	20.2203(a)(3)(ii)		50.73(a)(2)(iii)	73.71				
		20.2203(a)(2)(ii)	20.2203(a)(4)		50.73(a)(2)(iv)	OTHER				
		20.2203(a)(2)(iii)	50.36(c)(1)		50.73(a)(2)(v)	(Specify in Abstract below or in NRC Form 366A)				
		20.2203(a)(2)(iv)	50.36(c)(2)		50.73(a)(2)(vii)					

LICENSEE CONTACT FOR THIS LER (12)

NAME	TELEPHONE NO. (Include Area Code)
ROBERT J. WANCZYK, PLANT MANAGER	802-257-7711

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS
NA					NA				
NA					NA				

SUPPLEMENTAL REPORT EXPECTED (14)				EXPECTED SUBMISSION DATE (15)	MO	DAY	YEAR
YES (If yes, complete EXPECTED SUBMISSION DATE)	X	NO					

ABSTRACT (Limit to 1400 spaces, i.e., approximately 15 single-spaced typewritten lines) (16)

On 7/28/95 at 1200 hours, with the plant at 100% power, a Chemistry Assistant discovered that one of the weekly stack gas particulate filters, stored for use as part of the monthly gross alpha and quarterly strontium 89-90 composite sample, was missing. The storage area for the filters, the Chemistry Laboratory and the Counting Room were subsequently searched but the filter could not be located.

Technical Specifications (TS) Table 4.8.2 requires stack samples be obtained to determine off-site dose rates contributed by various radionuclides. Contrary to this requirement, Vermont Yankee (VY) misplaced one of the filters that are used to monitor the plant stack for alpha emitters and strontium and therefore cannot determine the one week's contribution for these radionuclides, using actual measurements.

The root cause of this event is an inadequate tracking and storage program for the stack gas samples. Contributing causes include a non-specific sample control procedure and inadequate training.

Immediate corrective actions included a search of the lab and counting rooms to look for the missing sample and a revision to the method used to retain the filters.

Long term corrective actions include more secure storage, improved technician training and procedure revisions to retain samples for a longer period of time.

During this event, the plant was at steady state operations and a postulated activity level, based on levels found in prior and subsequent weeks, was used to determine the dose rates for the missing week.

NRC Form 366 (4-95) U.S. NUCLEAR REGULATORY COMMISSION		APPROVED BY OMB NO. 3150-0104 EXPIRES 04/30/98			
LICENSEE EVENT REPORT (LER)		ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS MANDATORY INFORMATION COLLECTION REQUEST: 50.0 HRS. REPORTED LESSONS LEARNED ARE INCORPORATED INTO THE LICENSING PROCESS AND FED BACK TO INDUSTRY. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE INFORMATION AND RECORDS MANAGEMENT BRANCH (T-6 F33), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555-0001, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.			
FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)		PAGE (3)	
VERMONT YANKEE NUCLEAR POWER CORPORATION	05000271	YEAR	SEQUENTIAL NUMBER	REV #	02 OF 04
		95	-- 016 --	0	

TEXT (If more space is required, use additional copies of NRC Form 366A) (17)

DESCRIPTION OF EVENT

On 7/28/95 at 1200 hours, with the plant at 100% power, a Chemistry Assistant discovered that one of the weekly stack particulate filters (EHS = FLT), used as part of the monthly gross alpha and quarterly strontium 89-90 composite sample, was missing.

Normally, two stack particulate filters are collected each week and analyzed by gamma spectroscopy. The filter with the highest activity is identified and both filters are brought to the Chemistry Laboratory storage room where the filters are stored in separate drawers of a designated filter storage cabinet. The filter with the lower activity is retained for possible analysis by the State of Vermont, if they desire an independent analysis. The filter with higher activity is retained for use as a monthly and quarterly composite sample to be analyzed for gross alpha and strontium 89 & 90 respectively. Each week the previous lower-activity filter is discarded since it is no longer needed and replaced by the current low activity filter. During investigation into this event it was discovered that on July 4, 1995, a Chemistry Technician found that three of the drawers of the storage cabinet, used to store the filters, had been removed and placed next to the cabinet. The drawers were replaced, possibly out of order. It is believed that this event contributed to the loss of the filter cartridge.

Based on the investigation performed it is concluded that the missing filter cartridge was disposed of as radioactive waste.

CAUSE OF EVENT

The root cause of this event is an inadequate method to control the storage of the stack particulate filters.

Contributing causes are a non-specific sample control procedure and inadequate training regarding the consequences of lost or misplaced samples.

ANALYSIS OF EVENT

The filters that are used to determine gross alpha releases are collected each week and composited at the end of the month (a total of four filters for the month of July, 1995). Strontium 89 & 90 releases are assessed at the end of each quarter using the composited filters from each of the three months in the quarter for a total of thirteen filters.

NRC Form 366 (4-95)		U.S. NUCLEAR REGULATORY COMMISSION		APPROVED BY OMB NO. 3150-0104 EXPIRES 04/30/98		
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FACILITY NAME (1)		DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)
VERMONT YANKEE NUCLEAR POWER CORPORATION		05000271	YEAR	SEQUENTIAL NUMBER	REV #	03 OF 04
			95	-- 016 --	0	

TEXT (If more space is required, use additional copies of NRC Form 366A) (17)

ANALYSIS OF EVENT (CONT.)

Due to the missing particulate filter, only three of the four filters could be analyzed to determine dose rate and dose contribution from gross alpha emitters. Similarly, of the thirteen samples expected from the current quarter, only twelve filters will be available for analysis of Strontium 89 & 90 (analysis to be performed at the end of the 3rd quarter, 1995). During the week in question, the plant operated in a steady state condition with no fuel problems or evidence of unusual releases; therefore it can be concluded that the production and release of these radionuclides remained constant during the period. This hypothesis is further supported by the gamma isotopic analysis performed on the missing filter prior to its disappearance which showed no abnormal or unexpected radionuclides. Given the information available, the dose rate and total dose for gross alpha and Strontium 89 & 90 can be closely approximated for the week corresponding to the missing filter by utilizing information obtained from samples collected prior to and immediately following the period in question.

Based on the above, the dose rate was calculated to be within applicable limits, therefore, there was no threat to the health and safety of the public due to this event.

CORRECTIVE ACTIONS

The immediate corrective actions included:

- 1) A search of the Chemistry Laboratory and the Counting Room to attempt to find the missing filter, and
- 2) A change in the method used to retain the filters.

Long term corrective actions include:

- 1) A revision to the sample procedure to detail the identification, storage and disposal of filters. This is expected to be completed by 9/30/95.
- 2) Replacement of the present storage cabinet with one that has non-removable drawers and that can be locked. This is expected to be completed by 9/15/95.
- 3) Additional training for all Chemistry Department Personnel dealing with the immediate reporting and disposition of any laboratory equipment, samples, etc, that are out of place and the consequences of mis-placed or lost samples. This is expected to be completed by 9/15/95.

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VERMONT YANKEE NUCLEAR POWER CORPORATION	05000271	YEAR	SEQUENTIAL NUMBER	REV #	04 OF 04
		95	-- 016 --	0	

TEXT (If more space is required, use additional copies of NRC Form 366A) (17)

CORRECTIVE ACTIONS (CONT.)

- 4) A review of all Chemistry sampling procedures to determine the security of all samples. This is expected to be completed by 12/31/95.

ADDITIONAL INFORMATION

No similar events reported to the Commission within the past five years.