

NOTICE OF VIOLATION

Commonwealth Edison Company
Quad Cities Units 1 and 2

Docket Nos. 50-254; 50-265
License No. DPR-29; DPR-30

During an NRC inspection conducted on November 24, 1991, through January 4, 1992, violations of NRC requirements were identified. In accordance with the "General Statement of Policy and Procedure for NPC Enforcement Actions," 10 CFR Part 2, Appendix C (1991), the violations are listed below:

1. 10 CFR 50.73(a)(2)(v) requires that any event or condition that alone could have prevented the fulfillment of the safety function of structures or systems that are needed to remove residual heat, regardless of the plant mode or power level, and regardless of the significance of the structure, system, or component that initiated the event be reported to the NRC.

Contrary to the above:

- a. On April 24, 1991, during performance of a surveillance activity the reactor core isolation cooling system (RCIC) pump discharge valve (1-1301-49) failed to open. This event was not reported to the NRC.
- b. On December 1, 1991, during valve stroking activities the RCIC pump discharge valve (1-1301-49) failed to open. This failure was not reported to the NRC.

This is a Severity Level IV violation (Supplement I).

2. Quad Cities Technical Specification 6.2.B, Plant Operating Procedures, requires that radiation control procedures be maintained, made available to all station personnel, and adhered to.

Contrary to the above, on November 20 and 26, 1991, electronic dosimeters assigned to personnel working in the radiological control area were found unattended, and not worn as required by QRP 1001-1, Rev. 10.

This is a Severity Level V violation (Supplement I).

3. 10 CFR Part 50, Appendix B, Criterion V states, in part, that activities affecting quality shall be prescribed and accomplished in accordance with instructions of a type appropriate to the circumstances, which shall include acceptance criteria for determining that the activities have been satisfactorily accomplished.

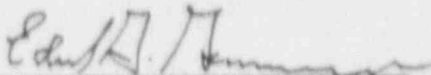
Contrary to the above:

- a. On September 22, 1991, the return to service instructions utilized to return the Unit 2 reactor recirculation system motor-generator sets deluge fire protection header isolation valves (2-4199-174, 2-4199-175) to service did not appropriately prescribe repressurizing the fire headers after they had been isolated and drained.
- b. On December 9, 1991, the out of service instructions utilized to secure the 1A and 2A primary containment purge fan isolation dampers did not appropriately prescribe the orientation the dampers were to be secured in, nor did the instructions contain acceptance criteria for determining or verifying the activity had been satisfactorily accomplished.

This is a Severity Level IV violation (Supplement I).

Pursuant to the provisions of 10 CFR 2.201, Commonwealth Edison Company is hereby required to submit a written statement or explanation to the U.S. Nuclear Regulatory Commission, ATTN: Document Control Desk, Washington D.C. 20555 with a copy to the U.S. Nuclear Regulatory Commission, Region III, 799 Roosevelt Road, Glen Ellyn, Illinois, 60137, and a copy to the NRC Resident Inspector at the facility that is the subject of this Notice, within 30 days of the date of the letter transmitting this Notice of Violation. This reply should be clearly marked as a "Reply to a Notice of Violation" and should include for each violation: (1) the reason for the violation, or, if contested, the basis for disputing the violation, (2) the corrective steps that have been taken and the results achieved, (3) the corrective steps that will be taken to avoid further violations, and (4) the date when full compliance will be achieved. If an adequate reply is not received within the time specified in this Notice, an order may be issued to show cause why the license should not be modified, suspended, or revoked, or why such other action as may be proper should not be taken. Where good cause is shown, consideration will be given to extending the response time.

Dated at Glen Ellyn, Illinois
this 24 day of January 1992


Edward G. Greenman, Director
Division of Reactor Projects