

August 18, 1995

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U. S. Nuclear Regulatory Commission ATTN: Document Control Desk Mail Station P1-137 Washington, D. C. 20555

Gentlemen:

ULNRC-3255 TAC No. M90859

CALLAWAY PLANT

DOCKET NUMBER 50-483

SECOND TEN-YEAR INTERVAL

INSERVICE INSPECTION PROGRAM PLAN

References: 1) ULNRC-3086 dated October 12, 1994
2) L. R. Wharton (NRC) Letter to
D. F. Schnell dated May 23, 1995

The attachme. to this letter provides the information requested in Reference 2 in support of the Callaway Plant Inservice Inspection Program Plan for the second inspection interval.

If you have any questions concerning this information, please contact us.

Very truly yours.

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WEK/sld

Attachment

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SECOND TEN-YEAR INTERVAL INSERVICE INSPECTION PLAN FOR THE CALLAWAY NUCLEAR POWER PLANT

UNION ELECTRIC COMPANY

SECOND INTERVAL INSERVICE INSPECTION PLAN

CALLAWAY NUCLEAR PLANT

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REVISION SUMMARY SHEET

REVSION NO.	DATE	PAGE(S)	DESCRIPTION
0	10/12/94	All	Initial Issue
1.	8/11/95	iv	Revised format of this Revision Summary Sheet.
		2-3	Corrected references to Bechtel Specifications MS-1 and MS-2.
		2-4 thru 6	Added the drawing revision column to Table 2.2.
		2-7 thru 10	Added the drawing revision column to Table 2.3.
		2-11	Added the drawing revision column to Table 2.4.
		2-12	Updated drawing references in Table 2.5.
		3-3	Added Item Number B5.70 to Table 3.1.
		3-7	Revised C3.20 number of components to incorporate CMP 90-1008 (BIT modification).
		3-8	Corrected C5.11 number of components to add an EN weld.
		3-11	1) Revised F1.20 number of components to add EM hangers and to incorporate CMP 94-1001 and MM 92-4015 (snubber reduction).
			2) Revised F1.30 number of components to incorporate CMP 92-1034 (GN modification).
			3) Revised F1.40 number of components to add CRDM platform restraints and to incorporate CMP 94-1001.
		4-1	Reworded paragraph 4.1.2 for clarification.
		4-2	Deleted N-498 and added N-498-1 to Table 4.1. Code Case N-498-1 was approved for Callaway per special permission from the NRC in accordance with FSAR, Appendix 3A (Reference paragraph 4.1.2 of this ISI Plan).

SECTION 1.0 INTRODUCTION AND PLAN DESCRIPTION

1.1 Overview

- 1.1.1 This Inservice Inspection Plan outlines the requirements for the inspection of Class 1, 2, and 3 pressure retaining components and their supports at the Callaway Nuclear Power Plant.
- 1.1.2 This Inservice Inspection Plan will be effective from August 1, 1995, through and including December 18, 2004, which represents the second ten-year interval for the Callaway Nuclear Power Plant.
- 1.1.3 The key features of this Plan are the Introduction and Plan Description, Relief Requests, and Summary Tables. The details of the Inservice Inspection Program are addressed in other documents that are available at the Callaway Plant. These documents include, but are not limited to, inservice inspection boundary drawings, piping isometric drawings, equipment detail drawings, a component database listing of each weld, valve, support, etc., and documents supporting implementation of the Inservice Inspection Program

1.2 Basis of Inservice Inspection Plan

- 1.2.1 This Inservice Inspection Plan was developed in accordance with the requirements delineated in the December 31, 1992, issue of 10 CFR 50.55a and the 1989 Edition of the American Society of Mechanical Engineers (ASME) Boiler and Pressure Vessel Code, Section XI, Subsections IWA, IWB, IWC, IWD, and IWF, for Inspection Program B. Accordingly, this Inservice Inspection Plan provides the details necessary for performing the inservice inspection of the Callaway Class 1, 2, and 3 pressure retaining components and supports.
- 1.2.2 The following ASME Section XI, 1989 Edition Subsections, Articles, or Paragraphs are not included or addressed in this Inservice Inspection Plan.
 - 1.2.2.1 The containment liner and concrete inspection and testing requirements of Subsections IWE and IWL are not included in this Inservice Inspection Plan. The rules of IWE and IWL are currently not required by 10 CFR 50.55a.
 - 1.2.2.2 The pump and valve testing requirements of Subsections IWP and IWV are not included in this Inservice Inspection Plan. The rules of IWP and IWV are addressed in a separate submittal to the NRC staff.

- 1.2.2.3 The snubber inservice inspection requirements of Paragraphs IWF-5200(a), IWF-5200(b), IWF-5300(a), and IWF-5300(b) are not addressed in this Inservice Inspection Plan. The extent, frequency, and acceptance standards for snubber assembly testing and inspection will be in accordance with Callaway Technical Specification 3/4.7.8.
- 1.2.2.4 The steam generator tubing examination requirements of Table IWB-2500-1, Examination Category B-Q and the acceptance standards of Paragraph IWB-3521.1 are not addressed in this Inservice Inspection Plan. The extent, frequency, and acceptance standards for steam generator tubing inspection and testing will be in accordance with Callaway Technical Specification 3/4.4.5.
- 1.2.3 Alternative requirements to ASME Section XI, 1989 Edition, are set forth in Section 4.0 of this Inservice Inspection Plan. Alternative requirements are in accordance with 10 CFR 50.55a and ASME Section XI.
- 1.2.4 With the exception of examinations that may be deferred until the end of the inspection interval as specified in Table IWB-2500-1, inservice inspections shall be performed in accordance with Inspection Program B as outlined in IWA-2432, IWB-2412, IWC-2412 and IWD-2412 of ASME Section XI. The inspection schedule for the second interval is divided into three periods such that approximately one third of the inspections will be completed every period. Successive inspections shall be in accordance with IWB-2420, IWC-2420, and Code Case N-491 -2420. Deviations to inspection schedules may occur provided compliance with Code requirements is maintained.
- 1.2.5 The commercial operating license date for the Callaway Nuclear Power Plant was December 19, 1984. As allowed by ASME Section XI, Paragraph IWA-2430, the first inspection interval was extended from December 18, 1994 to July 31, 1995.
- 1.2.6 The construction code for all ASME components is ASME Section III. The principal edition used for construction of the plant was the 1974 Edition thru Summer 1975 Addenda.

1.3 System Classification

1.3.1 The quality group classification system for radioactive water/steam-containing components important to the safety of water-cooled nuclear power plants is established by NRC Regulatory Guide 1.26, Revision 3, in conjunction with 10 CFR 50.55a. Regulatory Guide 1.26, "Quality Group Classification and Standards", defines the Quality Group Classification System consisting of four Quality Groups, A through D. The definition of Quality Group A is provided by 10 CFR 50.2 under "Reactor Coolant Pressure Boundary". The definitions of Groups B, C, and D are provided by Regulatory Guide 1.26.

1.3.2 Components subject to inservice inspection are shown on the Inservice Inspection Boundary Drawings listed in Section 2.2 of this Inservice Inspection Plan.

Pursuant to 10 CFR 50.55a, the inservice inspection requirements of ASME Section XI have been assigned to these components within the constraints of existing plant design.

1.4 Augmented Inservice Inspection Requirements

Augmented inservice inspection requirements are those examinations that are specified by documents other than the ASME Section XI Code. The additional examinations required by the following documents have been added to the Callaway Inservice Inspection Program:

- 1.4.1 U.S. Nuclear Regulatory Commission Standard Review Plan (SRP), Section 3.6.1, Rev. 1, "Plant Design for Protection Against Postulated Piping Failures in Fluid Systems Outside Containment" (NUREG-0800). Several piping systems require augmented inspections for protection against postulated piping failures as outlined in SRP Section 3.6.1. Piping systems subject to the augmented inspection criteria of SRP Section 3.6.1 are shown on the Inservice Inspection Boundary Drawings listed in Table 2.2 of this Inservice Inspection Plan. In addition, detailed high energy pipe break isometric drawings of applicable systems are provided in Section 3.6.1 of the Callaway Plant FSAR. A summary of the number of welds subject to the augmented inspection criteria of SRP Section 3.6.1, is provided in Table 3.2 of this Inservice Inspection Plan.
- 1.4.2 U.S. Nuclear Regulatory Commission Standard Review Plan, Section 6.6, Rev. 1, "Inservice Inspection of Class 2 & 3 Components" (NUREG-0800).
- 1.4.3 U.S. Nuclear Regulatory Commission Regulatory Guide 1.14, Rev. 1, "Reactor Coolant Pump Flywheel Integrity".
- 1.4.4 U.S. Nuclear Regulatory Commission Regulatory Guide 1.65, Rev. 0, "Materials and Inspections for Reactor Vessel Closure Studs".
- 1.4.5 U.S. Nuclear Regulatory Commission Regulatory Guide 1.137, Rev. 0, "Fuel-Oil Systems for Standby Diesel Generators".
- 1.4.6 U.S. Nuclear Regulatory Commission Regulatory Guide 1.150, Rev. 1, "Ultrasonic Testing of Reactor Vessel Welds During Preservice and Inservice Examinations", Regulatory Position C.8 (Alternative Method).
- 1.4.7 Code of Federal Regulations, Part 10, 50.55a(g)(6)(ii)(A), December 31, 1992, "Augmented Examination of Reactor Vessel".

SECTION 2.0 INSERVICE INSPECTION PROGRAM DRAWINGS

This section provides a listing of the various drawings applicable to the Callaway Inservice Inspection Program.

2.1 Drawing Nomenclature

2.1.1 System Designators

Table 2.1 below lists the two letter System Designator used for each piping system subject to inservice inspection at the Callaway Plant.

TABLE 2.1 SYSTEM DESIGNATORS

System Designator	System			
AB	Main Steam			
AE	Main Feedwater			
AL	Auxiliary Feedwater			
ВВ	Reactor Coolant System including: Reactor Pressure Vessel Pressurizer Steam Generators			
BG	Chemical and Volume Control			
BL	Reactor Make-up Water			
BM	Steam Generator Blowdown			
BN	Borated Refueling Water Storage			
EC	Fuel Pool Cooling and Cleanup			
EF	Essential Service Water			
EG	Component Cooling Water			
EJ	Residual Heat Removal			
EM	High Pressure Coolant Injection			
EN	Containment Spray			
EP	Accumulator Safety Injection			
FC	Auxiliary Feedwater Pump Turbine			
GF	Miscellaneous Building HVAC			
GG	Fuel Building			
GK	Control Building HVAC			
GL	Auxiliary Building HVAC			
GN	Containment Cooling			
GP	Containment Integrated Leak Rate Test			

System Designator	System			
GS	Containment Hydrogen Control			
GT	Containment Purge HVAC			
HB	Liquid Radwaste	1 -		
HD	Decontamination			
JЕ	Emergency Fuel Oil			
KA	Compressed Air			
KB	Breathing Air			
KC	Fire Protection			
KJ	Standby Diesel Generator			
LF	Auxiliary Building Equipment			
SJ	Nuclear Sampling			

2.1.2 Piping Classifications

2.1.2.1 Piping classifications are designated by a three-letter code. Listed below are the appropriate letter designations for piping subject to inservice inspection. The first letter indicates the Standard Rating Class; the second letter the type of material; and the third letter the Code or standard to which the piping is designed.

2.1.2.2 First Letter - Primary Rating Class

A - Specific pressure & specific temperature

B - Class 2500

C - Class 1500

D - Class 900

E - Class 600

F - Class 400

G - Class 300

H - Class 150

Second Letter - Material

B - Carbon Steel

C - Austenitic Stainless Steel

L - Carbon Steel - impact tested

Third Letter - Applicable Codes

A - ASME B & PV Code, Section III, Class 1

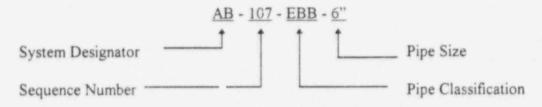
B - ASME B & PV Code, Section III, Class 2

C - ASME B & PV Code, Section III, Class 3

D - Power Piping Code, ANSI B31.1

2 1 3 Line Identification Numbers

The line numbers identified on the drawings listed herein provide useful information about the piping. The lines are identified using the following convention:



Each line, or portion of line, is assigned a system designator, line sequence number, pipe classification, and pipe size. The system designator and line sequence number can be cross referenced to Bechtel Specification MS-1 to determine the line description, design temperature and pressure, and service or normal operating temperature and pressure. The pipe classification can be cross referenced to Bechtel Specification MS-2 to determine the pipe schedule and material specification.

2.2 Inservice Inspection Boundary Drawings

Table 2.2 provides a listing of the Inservice Inspection Boundary Drawings applicable to the second interval at the Callaway Plant. These drawings are Callaway Plant Piping & Instrumentation Drawings that are color coded to identify the Quality Group A, B, and C piping and components subject to inservice inspection. The color code also identifies Non-Exempt, Exempt, and Augmented boundaries. Listed below is a brief description of each type boundary depicted on the boundary drawings.

2.2.1 Non-Exempt Boundaries

The non-exempt boundaries identify piping and components subject to the nondestructive examination, visual examination, and pressure test requirements of ASME Section XI, Subsections IWB, IWC, or IWD. Non-Exempt Quality Group A, B, and C boundaries are color coded red, blue, and green respectively.

2.2.2 Exempt Boundaries

The exempt boundaries identify piping and components that are subject only to the pressure test requirements of ASME Section XI, Articles IWB-5000, IWC-5000, and IWD-5000. As permitted by ASME Section XI, Paragraphs IWB-1220, IWC-1220, IWD-1220, and Code Case N-491, the piping, components, and supports within the exempt boundaries do not require nondestructive (i.e., volumetric or surface) examination or visual (i.e., VT-1 or VT-3) examination. Exempt Quality Group A, B, and C boundaries are color coded orange, light blue, and light green respectively.

2.2.3 Augmented Boundaries

The boundaries for the high energy lines that require augmented examination in accordance with NRC Standard Review Plan, Section 3.6.1 are identified by dashed colored lines on the applicable boundary drawings. In addition, the Diesel Generator Fuel Oil piping that requires augmented pressure testing in accordance with NRC Regulatory Guide 1.137 is identified on the applicable boundary drawings.

TABLE 2.2
INSERVICE INSPECTION BOUNDARY DRAWINGS

Drawing Number	Rev.	System
ISI-M-22AB01(Q)	A	Main Steam
ISI-M-22AB02(Q)	A	Main Steam
ISI-M-22AE02(Q)	A	Main Feedwater
ISI-M-22AL01(Q)	A	Auxiliary Feedwater
ISI-M-22BB01(Q)	A	Reactor Coolant
ISI-M-22BB02(Q)	A	Reactor Coolant
ISI-M-22BB03(Q)	A	Reactor Coolant
ISI-M-22BB04(Q)	A	Reactor Coolant
ISI-M-22BG01(Q)	A	Chemical and Volume Control
ISI-M-22BG02(Q)	A	Chemical and Volume Control
ISI-M-22BG03(Q)	В	Chemical and Volume Control
ISI-M-22BG04(Q)	A	Chemical and Volume Control
ISI-M-22BG05(Q)	A	Chemical and Volume Control
ISI-M-22BL01(Q)	A	Reactor Make-up Water
ISI-M-22BM01(Q)	A	Steam Generator Blowdown
ISI-M-22BN01(Q)	A	Borated Refueling Water Storage
ISI-M-22EC01(Q)	A	Fuel Pool Cooling and Clean-up
ISI-M-22EC02(Q)	A	Fuel Pool Cooling and Clean-up
ISI-M-2UEF01(Q)	A	Essential Service Water
ISI-M-22EF01(Q)	A	Essential Service Water

TABLE 2.2
INSERVICE INSPECTION BOUNDARY DRAWINGS (cont.)

Drawing Number	Rev.	System
ISI-M-22EF02(Q)	A	Essential Service Water
ISI-M-22EG01(Q)	A	Component Cooling Water
ISI-M-22EG02(Q)	A	Component Cooling Water
ISI-M-22EG03(Q)	A	Component Cooling Water
ISI-M-22EJ01(Q)	A	Residual Heat Removal
ISI-M-22EM01(Q)	A	High Pressure Coolant Injection
ISI-M-22EM02(Q)	В	High Pressure Coolant Injection
ISI-M-22EN01(Q)	A	Containment Spray
ISI-M-22EP01(Q)	A	Accumulator Safety Injection
ISI-M-22FC02(Q)	A	Auxiliary Feedwater Pump Turbine
ISI-M-22GF01(Q)	A	Miscellaneous Building HVAC
ISI-M-22GG02(Q)	A	Fuel Building HVAC
ISI-M-22GK01(Q)	A	Control Building HVAC
ISI-M-22GK03(Q)	A	Control Building HVAC
ISI-M-22GL01(Q)	A	Auxiliary Building HVAC
ISI-M-22GL02(Q)	A	Auxiliary Building HVAC
ISI-M-22GL03(Q)	A	Auxiliary Building HVAC
ISI-M-22GN01(Q)	В	Containment Cooling
ISI-M-22GP01(Q)	A	Containment Integrated Leak Rate Test
ISI-M-22GS01(Q)	A	Containment Hydrogen Control
ISI-M-22GT01(Q)	A	Containment Purge HVAC
ISI-M-22HB01(Q)	A	Liquid Radwaste
ISI-M-22HD01(Q)	A	Decontamination
ISI-M-22JE01(Q)	A	Emergency Fuel Oil
ISI-M-22KA01(Q)	A	Compressed Air
ISI-M-22KA02(Q)	A	Compressed Air (Service Air)
ISI-M-22KA05(Q)	A	Compressed Air
ISI-M-22KB01(Q)	A	Breathing Air
ISI-M-22KC02(Q)	A	Fire Protection
ISI-M-22KJ01(Q)	A	Standby Diesel Generator "A" Cooling Water
ISI-M-22KJ02(Q)	A	Standby Diesel Generator "A" Intake Exhaust Fuel Oil and Starting Air
ISI-M-22KJ03(Q)	A	Standby Diesel Generator "A" Lube Oil
ISI-M-22KJ04(Q)	A	Standby Diesel Generator "B" Cooling Water
ISI-M-22KJ05(Q)	A	Standby Diesel Generator "B" Intake Exhaust Fuel Oil and Starting Air
ISI-M-22KJ06(Q)	A	Standby Diesel Generator "B" Lube Oil

TABLE 2.2 INSERVICE INSPECTION BOUNDARY DRAWINGS (cont.)

Drawing Number	Rev.	System
!SI-M-22LF03(Q)	A	Auxiliary Building Floor and Equipment Drain
ISI-M-22LF09(Q)	A	Reactor Bldg. & Hot Machine Shop Floor and Equipment Drain
ISI-M-22SJ01(Q)	A	Nuclear Sampling
ISI-M-22SJ04(Q)	A	Nuclear Sampling

2.3 Piping Isometric Drawings

Table 2.3 provides a listing of the Piping Isometric Drawings for systems subject to inservice inspection. These drawings identify pipe welds, flange and valve bolted connections, pump and valve internal surfaces, integral attachments, and pipe supports that are within the non-exempt piping boundaries. In addition, system identification, location, room numbers, pipe classification, pipe size, and configuration are identified. Piping and components that are exempt from nondestructive and visual examination in accordance with ASME Section XI, Paragraphs IWB-1220, IWC-1220, and IWD-1220 are not normally depicted on these drawings. If exempt piping or components are shown, it is for information only.

TABLE 2.3
PIPING ISOMETRIC DRAWINGS

Drawing Number	Rev.	Title
		REFERENCE DRAWINGS
ISI-Ref., Sht. 1	0	Inservice Inspection Drawing Cross Reference
ISI-Ref., Sht. 2	0	Inservice Inspection Drawing Nomenclature
ISI-Ref., Sht. 3	1	Inservice Inspection Drawing Symbols
		MAIN STEAM
AB-01-01, Sht. 1	2	Loop 1
AB-02-01, Sht. 1	2	Loop 2
AB-03-01, Sht. 1	2	Loop 3
AB-04-01, Sht. 1	2	Loop 4
		MAIN FEEDWATER
AE-01-04, Sht. 1	2	Loop 1
AE-02-04, Sht. 1	2	Loop 2
AE-03-05, Sht. 1	2	Loop 3
AE-04-05, Sht. 1	2	Loop 4
		AUXILIARY FEEDWATER
AL-01-02, Sht. 1	1	Motor Driven Auxiliary Feedwater Pump "A" Discharge Piping
AL-02-03, Sht. 1	1	Motor Driven Auxiliary Feedwater Pump "B" Discharge Piping
AL-03-04, Sht. 1	1	Turbine Driven Auxiliary Feedwater Pump Discharge Piping
		REACTOR COOLANT
BB-00-01, Sht. 1	1	Primary Loop-General Layout
BB-01-01, Sht. 1	1	Loop 1
BB-01-02, Sht. 1	1	Pressurizer Relief Header
BB-01-04, Sht. 1	1	Pressurizer Spray Line
BB-01-04, Sht. 2	1	Pressurizer Spray Line
BB-01-08, Sht. 1	1	Pump "A" Seal Water Injection Line
BB-02-01, Sht. 1	1	Loop 2
BB-02-02, Sht. 1	1	Pressurizer Safety Valve Lines
BB-02-11, Sht. 1	1	Pump "B" Seal Water Injection Line
BB-03-01, Sht. 1	1	Loop 3
BB-03-09, Sht. 1	1	Pump "C" Seal Water Injection Line

TABLE 2.3 PIPING ISOMETRIC DRAWINGS (cont.)

Drawing Number	Rev.	Title
THE RESERVE OF THE PROPERTY OF	THE RESERVE THE PERSON NAMED IN	REACTOR COOLANT (cont.)
BB-04-01, Sht. 1	1	Loop 4
BB-04-07, Sht. 1	1	Pump "D" Seal Water Injection Line
BB-05-01, Sht. 1	1	Pressurizer Surge Line
BB-06-01, Sht. 1	1	Loop Drain to Reactor Coolant Drain Tank
		CHEMICAL AND VOLUME CONTROL
BG-01-01, Sht. 1	0	Normal Charging Line Containment Penetration
BG-01-21, Sht. 1	1	Normal & Alternate Charging Lines
BG-02-22, Sht. 1	1	Letdown Line
BG-02-22, Sht. 2	0	Letdown Line Containment Penetration
BG-03-23, Sht. 1	1	Excess Letdown Line
BG-04-09, Sht. 1	0	RCP Seal Water Injection Containment Penetration
BG-05-24, Sht. 1	1	Auxiliary Spray Line
BG-06-02, Sht. 1	1	CCP Suction
BG-06-02, Sht. 2	1	CCP Discharge (HPSI Discharge)
BG-06-10, Sht. 1	1	CCP Discharge to Seal Water Injection Filters
		STEAM GENERATOR BLOWDOWN
BM-01-01, Sht. 1	0	Loops "A" & "D"
BM-01-02, Sht. 1	0	Loops "B" & "C"
		ESSENTIAL SERVICE WATER
EF-01-01, Sht. 1	1	"A" Train Supply
EF-01-01, Sht. 2	0	"A" Train Supply
EF-01-02, Sht. 1	1	"A" Train Supply
EF-01-02, Sht. 2	1	"A" Train Supply
EF-01-03, Sht. 1	1	"A" Train Return
EF-01-03, Sht. 2	1	"A" Train Return
EF-01-07, Sht. 1	1	"A" Train Return
EF-01-08, Sht. 1	1	"A" Train Supply
EF-01-11, Sht. 1	0	"A" and "B" Train Return
EF-02-01, Sht. 1	1	"B" Train Supply
EF-02-01, Sht. 2	1	"B" Train Supply
EF-02-01, Sht. 3	1	"B" Train Return
EF-02-04, Sht. 1	1	"B" Train Supply
EF-02-05, Sht. 1	1	"B" Train Return
EF-02-05, Sht. 2	1	"B" Train Return
EF-02-06, Sht. 1	1	"B" Train Supply
EF-02-08, Sht. 1	1	"B" Train Supply
EF-02-08, Sht. 2	1	"B" Train Return
***************************************		COMPONENT COOLING WATER
EG-01-01, Sht. 1	1	"A" Train CCW Heat Exchanger
EG-01-02, Sht. 1	1	"A" Train Supply
EG-01-02, Sht. 2	1	"A" Train Return
EG-02-03, Sht. 1	1	"B" Train CCW Heat Exchanger

TABLE 2.3 PIPING ISOMETRIC DRAWINGS (cont.)

Drawing Number	Rev.	Title	
		COMPONENT COOLING WATER (Cont.)	
EG-02-03, Sht. 2	1	"B" Train	
EG-02-05, Sht. 1	1	"B" Train Return	
EG-02-05, Sht. 2	0	"B" Train Supply	
EG-03-06, Sht. 1	1	Common Header	
EG-03-07, Sht. 1	1	Common Header	
EG-04-01, Sht. 1	1	RHR Heat Exchanger Supply	
EG-04-01, Sht. 2	1	RHR Heat Exchanger Return	
EG-05-06, Sht. 1	1	Letdown Heat Exchanger Supply	
EG-05-06, Sht. 2	1	Letdown Heat Exchanger Return	
EG-06-09, Sht. 1	1	RCP "B" & "C" Supply	
EG-06-09, Sht. 2	1	RCP Return	
		RESIDUAL HEAT REMOVAL	
EJ-01-01, Sht. 1	1	"A" Train RHR Pump Suction	
EJ-01-01, Sht. 2	1	"A" Train RHR Pump Discharge	
EJ-01-01, Sht. 3	1	"A" Train RHR Pump Discharge	
EJ-01-01, Sht. 4	1	"A" Train Pump Discharge to Safety Injection	
EJ-01-01, Sht. 5	0	Return to Refueling Water Storage Tank	
EJ-01-04, Sht. 1	1	"A" Train RHR Pump Suction	
EJ-02-02, Sht. 1	1	"B" Train RHR Pump Suction	
EJ-02-02, Sht. 2	1	"B" Train RHR Pump Discharge	
EJ-02-02, Sht. 3	1	"B" Train RHR Pump Discharge	
EJ-02-02, Sht. 4	1	"B" Train RHR Pump Discharge to Safety Injection	
EJ-02-02, Sht. 5	1	"B" Train RHR Pump Discharge to Safety Injection	
EJ-02-04, Sht. 1	1	"B" Train RHR Pump Suction	
EJ-02-04, Sht. 2	1	"B" Train RHR Discharge to Accumulator Injection	
EJ-02-04, Sht. 3	1	"B" Train RHR Pump Discharge to Safety Injection Loops 2 & 3	
		HIGH PRESSURE COOLANT INJECTION	
EM-01-01, Sht. 1	1	Safety Injection Pump "A" Suction	
EM-02-01, Sht. 2	1	Safety Injection Pump "B" Suction	
EM-03-05, Sht. 1	2	Safety Injection Pumps to RHR	
E' i-04-03, Sht. 1	1	Safety Injection Pumps to RCS	
EM-05-01, Sht. 1	1	Safety Injection Pump Suction Cross Tie	
EM-05-01, Sht. 2	1	Safety Injection Pump Suction Cross Tie to CVCS	
EM-06-02, Sht. 1	2	HPSI Discharge to RCS	
EM-06-02, Sht. 2	2	HPSI Discharge to RCS	
EM-06-03, Sht. 1	1	HPSI Discharge to RCS	
		CONTAINMENT SPRAY	
EN-01-01, Sht. 1	1	"A" Train Pump Suction	
EN-01-01, Sht. 2	1	"A" Train Pump Discharge	
EN-02-02, Sht. 1	1	"B" Train Pump Suction	
EN-02-02, Sht. 2		"B" Train Pump Discharge	

TABLE 2.3 PIPING ISOMETRIC DRAWINGS (cont.)

Drawing Number	Rev.	Title
		ACCUMULATOR SAFETY INJECTION
EP-01-01, Sht. 1	1	Loop 1
EP-02-02, Sht. 1	1	Loop 2
EP-03-02, Sht. 1	1	Loop 3
EP-04-01, Sht. 1	1	Loop 4
		AUXILIARY TURBINE
FC-01-01, Sht. 1	0	Aux. Feedwater Pump Turbine Steam Supply Piping
		CONTAINMENT COOLING
GN-01-01, Sht. 1	1	"A" Train Supply
GN-01-01, Sht. 2	0	"A" Train Return
GN-01-01, Sht. 3	1	"A" Train Cooler "A" Supply and Return Headers
GN-01-01, Sht. 4	0	"A" Train Cooler "C" Supply and Return Headers
GN-01-02, Sht. 1	1	"A" Train Supply
GN-01-03, Sht. 1	1	"A" Train Return
GN-02-02, Sht. 1	1	"B" Train Supply
GN-02-02, Sht. 2	1	"B" Train Return
GN-02-02, Sht. 3	0	"B" Train Cooler "B" Supply and Return Headers
GN-02-02, Sht. 4	0	"B" Train Cooler "D" Supply and Return Headers
GN-02-04, Sht. 1	0	"B" Train Supply
GN-02-05, Sht. 1	1	"B" Train Return
		STANDBY DIESEL GENERATOR
KJ-01-01, Sht. 1	0	Diesel Generator "A" Cooling Water Piping
KJ-02-04, Sht. 1	0	Diesel Generator "B" Cooling Water Piping

2.4 Equipment Detail Drawings

Table 2.4 provides a listing of Equipment Detail Drawings for equipment welds or components subject to inservice inspection. These drawings display unique identification numbers for equipment welds, integral attachments, or bolted connections that require examination. In addition, weld joint details, component configuration details, weld locations, etc., are depicted.

TABLE 2.4 EQUIPMENT DETAIL DRAWINGS

Drawing Number	Rev.	Title
ISI-EBB01A, Sht. 1	1	Steam Generator "A" EBB01A ISI Equipment Welds
ISI-EBB01A, Sht. 2	0	Steam Generator "A" EBB01A ISI Equipment Welds
ISI-EBB01B, Sht. 1	1	Steam Generator "B" EBB01B ISI Equipment Welds
ISI-EBB01B, Sht. 2	0	Steam Generator "B" EBB01B ISI Equipment Welds
ISI-EBB01C, Sht. 1	1	Steam Generator "C" EBB01C ISI Equipment Welds
ISI-EBB01C, Sht. 2	0	Steam Generator "C" EBB01C ISI Equipment Welds
ISI-EBB01D, Sht. 1	1	Steam Generator "D" EBB01D ISI Equipment Welds
ISI-EBB01D, Sht. 2	0	Steam Generator "D" EBB01D ISI Equipment Welds
ISI-EEJ01A, Sht. 1	0	RHR Heat Exchanger "A" EEJ01A ISI Equipment Welds
ISI-EEJ01B, Sht. 1	0	RHR Heat Exchanger "B" EEJ01B ISI Equipment Welds
ISI-RBB01, Sht. 1	1	Reactor Vessel RBB01 ISI Equipment Welds
ISI-RBB01, Sht. 2	0	Reactor Vessel RBB01 ISI Equipment Welds
ISI-RBB01, Sht. 3	0	Reactor Vessel RBB01 ISI Equipment Welds
ISI-RBB01, Sht. 4	0	Reactor Vessel RBB01 ISI Equipment Welds
ISI-TBB03, Sht. 1	1	Pressurizer Vessel TBB03 ISI Equipment Welds
ISI-TBB03, Sht. 2	0	Pressurizer Vessel TBB03 ISI Equipment Welds
ISI-TBB03, Sht. 3	0	Pressurizer Vessel TBB03 ISI Equipment Welds
ISI-PUMPS, Sht. 1	1	Miscellaneous Pumps ISI Equipment Welds

2.5 Component/Equipment Support Drawings

Table 2.5 provides a listing of the Component/Equipment Support Drawings for component or equipment supports subject to inservice inspection. These drawings display the configuration of the supports and provide a unique location number for each support assembly that requires visual examination.

TABLE 2.5
COMPONENT/EQUIPMENT SUPPORT DRAWINGS

Drawing Number	Component Description
OP-1459F01	Reactor Vessel (RBB01) Supports
OP-1459F02, OP-1459F04, OP-1459F08, OP-1459F09	Steam Generator "A", "B", "C" & "D" (E8801A, B, C & D) Supports
OP-1459F07, OP-1459F02	Reactor Coolant Pump "A", "B", "C", & "D" (PBB01A, B, C & D) Supports
OP-1459F10	Pressurizer (TBB03) Supports
5736	RHR Heat Exchanger "A" & "B" (EEJ01A & B) Supports
C-2S1904 C-2S1908	RHR Pump "A" & "B" (PEJ01A & B) Supports
C-2S1904	Containment Spray Pump "A" & "B" (PEN01A & B) Supports
M-721-040	Centrifugal Charging Pump "A" & "B" (PBG05A & B) Supports
OP-300-J49728	Safety Injection Pump "A" & "B" (PEM01A & B) Supports
OP-D-75-569	Chemical & Volume Control Letdown Heat Exchanger (EBG01) Supports
M-072-0001	Component Cooling Water Heat Exchanger "A" & "B" (EEG01A & B) Supports
M-082-012	Component Cooling Water Pump "A", "B", "C", & "D" (PEG01A, B, C & D) Supports
M-089-U0012	Essential Service Water Pump "A" & "B" (PEF01A & B) Supports
M-154-U0018	Essential Service Water Self Cleaning Strainer "A" & "B" (FEF02A & B) Supports
M-71-00001	Fuel Pool Cooling Heat Exchanger "A" & "B" (EEC01A & B) Supports
M-021-00045	Motor Driven Auxiliary Feed Water Pump "A" & "B" (PAL01A & B) Supports
M-021-005	Turbine Driven Auxiliary Feed Water Pump (PAL02) Supports
11908662	Diesel Generator Jacket Water Heat Exchanger "A" & "B" (EKJ04A & B) Supports
M-018-00091	Diesel Generator Intercooler Heat Exchanger "A" & "B" (EKJ03A & B) Supports
M-018-00724	Diesel Generator Lube Oil Heat Exchanger "A" & "B" (EKJ06A & B) Supports

SECTION 3.0 INSERVICE INSPECTION SUMMARY TABLES

This section provides a summary listing of all items subject to inservice inspection. Section 3.1 addresses the inservice inspections required by ASME Section XI while Section 3.2 covers augmented inspections.

3.1 ASME Section XI Inservice Inspections

The ASME Section XI Inservice Inspection Summary Table 3.1 provides the following information:

3.1.1 Examination Category

This column lists the examination category as identified in ASME Section XI, Tables IWB-2500-1, IWC-2500-1, IWD-2500-1, and Code Case N-491 -2500-1. Only those examination categories applicable to the Callaway Plant are identified.

3.1.2 Item Number and Description of Components Examined

These columns list the item number and description as defined in ASME Section XI, Tables IWB-2500-1, IWC-2500-1, IWD-2500-1, and Code Case N-491-2500-1. Only those item numbers applicable to the Callaway Plant are identified.

3.1.3 Number of Components

This column lists the population of components potentially subject to examination. The number of components actually examined during the inspection interval will be based upon the Code requirements for the subject item number (e.g., 25% of Examination Category B-J, Item Number B9.11 components will be examined during the inspection interval).

3.1.4 Examination Method

The column lists the examination method(s) required by ASME Section XI, Tables IWB-2500-1, IWC-2500-1, IWD-2500-1, and Code Case N-491 -2500-1.

3.1.5 Relief Request Number

This column provides a listing of applicable relief requests. If a relief request number is identified, see the corresponding relief request in Section 4.4.

Examination Category	Item Number	Description of Components Examined	Number of Components	Examination Method	Relief Request Number
B-A	B1.11	Circumferential Shell Welds	2	Volumetric	
Pressure	B1.12	Longitudinal Shell Welds	9	Volumetric	
Retaining	B1.21	Circumferential Head Welds	3	Volumetric	
Welds in	B1.22	Meridional Head Welds	8	Volumetric	
Reactor	B1.30	Shell-to-Flange Weld	1	Volumetric	
Vessel B1 40	Head-to-Flange Weld	1	Volumetric & Surface		
В-В	B2.11	Pressurizer Circumferential Shell-to-Head Welds	2	Volumetric	
Pressure	B2.12	Pressurizer Longitudinal Shell-to-Head Welds	2	Volumetric	
Retaining B2.40 Welds in Vessels Other Than Reactor Vessels	B2.40	Steam Generator Tube Sheet-to-Head Weld	4	Volumetric	
B-D	B3.90	Reactor Vessel Nozzle-to-Vessel Welds	8	Volumetric	ISI-05
Full	B3.100	Reactor Vessel Nozzle Inside Radius Section	8	Volumetric	ISI-05
Penetration	B3.110	Pressurizer Nozzle-to-Vessel Welds	6	Volumetric	
Welds of	B3.120	Pressurizer Nozzle Inside Radius Section	6	Volumetric	
Nozzles in Vessels	B3.140	Steam Generator (Primary Side) Inside Radius Section	8	Volumetric	

Examination Category	Item Number	Description of Components Examined	Number of Components	Examination Method	Relief Request Number
B-E Pressure	B4.11	Partial Penetration Vessel Nozzle Welds	1	Visual, VT-2	
Retaining Partial	B4.12	Partial Penetration Control Rod Drive Nozzle Welds	78	Visual, VT-2	
Penetration Welds in	B4.13	Partial Penetration Instrumentation Nozzle Welds	58	Visual, VT-2	
Vessels	B4.20	Partial Penetration Pressurizer Heater Penetration Welds	78	Visual, VT-2	
B-F Pressure Retaining	B5.10	Reactor Vessel Dissimilar Metal Nozzle-to-Safe End Butt Welds NPS 4 or Larger	8	Volumetric & Surface	ISI-05
Dissimilar Metal Welds	B5.40	Pressurizer Dissimilar Metal Nozzle-to-Safe End Butt Welds NPS 4 or Larger	6	Volumetric & Surface	
	B5.70	Steam Generator Dissimilar Metal Nozzle-to-Elbow Butt Welds NPS 4 or Larger	8	Volumetric & Surface	
B-G-1	B6.10	Reactor Vessel Closure Head Nuts	54	Visual, VT-1	
Pressure	B6.20	Reactor Vessel Closure Studs, in Place	54	Volumetric	
Retaining Bolting Greater	B6.30	Reactor Vessel Closure Studs, when Removed	54	Volumetric & Surface	
Than 2 in. in	B6.40	Threads in Reactor Vessel Flange	54	Volumetric	
Diameter	B6.50	Reactor Vessel Closure Washers, Bushings	54	Visual, VT-1	
	B6.180	Bolts & Studs in Pumps	4	Volumetric	
	B6.190	Flange Surface, When Connection Disassembled, in Pumps	4	Visual, VT-1	
	B6.200	Nuts, Bushings, & Washers in Pumps	4	Visual, VT-1	

Examination Category	Item Number	Description of Components Examined	Number of Components	Examination Method	Relief Request Number
B-G-2	B7.20	Bolts, Studs, & Nuts in the Pressurizer	1	Visual, VT-1	
Pressure	B7.30	Bolts, Studs, & Nuts in Steam Generators	8	Visual, VT-1	
Retaining	B7.50	Bolts, Studs, & Nuts in Piping	11	Visual, VT-1	
Bolting, 2 in. & Less in Diameter	B7.70	Bolts, Studs, & Nuts in Valves	25	Visual, VT-1	
B-J Pressure	B9.11	Circumferential Welds in Piping NPS 4 or Larger	315	Volumetric & Surface	
Retaining Welds in	B9.12	Longitudinal Welds in Piping NPS 4 or Larger	N/A	Volumetric & Surface	ISI-04
Piping	B9.21	Circumferential Welds in Piping Less than NPS 4	309	Surface	
	B9.31	Branch Pipe Connection Welds NPS 4 or Larger	13	Volumetric & Surface	
	B9.32	Branch Pipe Connection Welds Less than NPS 4	37	Surface	
	B9.40	Socket Welds	11	Surface	
B-K ¹ Integral Attachments for Class 1 Vessels, Piping, Pumps & Valves	B10.10 ¹	Integrally Welded Attachments to Vessels	5	Surface	ISI-06

Examination Category	Item Number	Description of Components Examined	Number of Components	Examination Method	Relief Request Number
B-L-2 Pump Casings	B12.20	Pump Casings	4	Visual, VT-3	
B-M-2 Valve Bodies	B12.50	Valve Bodies, Exceeding NPS 4	25	Visual, VT-3	
B-N-1 Interior of Reactor Vessel	B13.10	Vessel Interior	1	Visual, VT-3	
B-N-2 Integrally Welded Core Support Structures & Interior Attachments to Reactor Vessels	B13.60	Interior Attachments beyond Beltline Region in Reactor Vessel	6	Visual, VT-3	
B-N-3 Removable Core Support Structures	B13.70	Core Support Structure in Reactor Vessel	1	Visual, VT-3	

Examination Category	Item Number	Description of Components Examined	Number of Components	Examination Method	Relief Request Number
B-O Pressure Retaining Welds in Control Rod Housings	B14.10	Welds in CRD Housing	32	Volumetric or Surface	
B-P	B15.10	RPV - System Leakage Test	1	Visual, VT-2	ISI-07
All Pressure	B15.11	RPV - System Hydrostatic Test	1	Visual, VT-2	ISI-07
Retaining	B15.20	Pressurizer - System Leakage Test	-1	Visual, VT-2	ISI-07
Components	B15.21	Pressurizer - System Hydrostatic Test	1	Visual, VT-2	ISI-07
(Class 1)	B15.30	Steam Generator - System Leakage Test	4	Visual, VT-2	ISI-07
	B15.31	Steam Generator - System Hydrostatic Test	4	Visual, VT-2	ISI-07
	B15.50	Piping - System Leakage Test	See Note 6	Visual, VT-2	ISI-07
	B15.51	Piping - System Hydrostatic Test	See Note 6	Visual, VT-2	ISI-07
	B15.60	Pumps - System Leakage Test	4	Visual, VT-2	ISI-07
	B15.61	Pumps - System Hydrostatic Test	4	Visual, VT-2	ISI-07
	B15.70	Valves - System Leakage Test	See Note 6	Visual, VT-2	ISI-07
	B15.71	Valves - System Hydrostatic Test	See Note 6	Visual, VT-2	ISI-07
B-Q Steam Generator Tubing	B16.20	Steam Generator Tubing in U-Tube Design	4	Volumetric ²	

Examination Category	Item Number	Description of Components Examined	Number of Components	Examination Method	Relief Request Number
C-A	C1.10	Shell Circumferential Welds	16	Volumetric	
Pressure	C1.20	Head Circumferential Welds	4	Volumetric	
Retaining Welds in Pressure Vessels	C1.30	Tubesheet-to-Shell Welds	4	Volumetric	
C-B Pressure	C2.21	Nozzle-to-Shell (or Head) Weld without Reinforcing Plate in Vessels > 1/2" Nominal Thickness	8	Volumetric & Surface	
Retaining	C2.22	Nozzle Inside Radius Section	8	Volumetric	
	C2.33	Nozzle-to-Shell (or Head) Welds when Inside of Vessel is Inaccessible, for Vessels > 1/2" Nominal Thickness with Reinforcing Plates	4	Visual, VT-2	
C-C1	C3.10 ¹	Integrally Welded Attachments to Pressure Vessels	2	Surface	ISI-06
Integral	C3.201	Integrally Welded Attachments to Piping	100	Surface	ISI-06
Attachments for Class 2 Vessels, Piping, Pumps and Valves	C3.30 ¹	Integrally Welded Attachments to Pumps	20	Surface	ISI-06

Examination Category	Item Number	Description of Components Examined	Number of Components	Examination Method	Relief Requests Number
C-D Pressure Retaining Bolting Greater Than 2 in. in Diameter	C4.40	Bolts and Studs in Valves	4	Volumetric	
C-F-1 Pressure Retaining	C5.11	Circumferential Welds in Austenitic Stainless Steel or High Alloy Piping ≥ 3/8" Nominal Wall Thickness for Piping > NPS 4	718	Volumetric & Surface	
Welds in Austenitic Stainless Steel	C5.12	Longitudinal Welds in Austenitic Stainless Steel or High Allow Piping ≥ 3/8" Nominal Wall Thickness for Piping > NPS 4	N/A	Volumetric & Surface	ISI-04
or High Alloy Piping	C5.21	Circumferential Welds in Austenitic Stainless Steel or High Alloy Piping ≥ 1/5" in. Nominal W. II Thickness for Piping ≥ NPS 2 and ≤ NPS 4	177	Volumetric & Surface	
	C5.30	Socket Welds	20	Surface	
	C5.41	Circumferential Welds in Pipe Branch Connections of Branch Piping ≥ NPS 2	14	Surface	

Examination Category	Item Number	Description of Components Examined	Number of Components	Examination Method	Relief Request Number
C-F-2 Pressure Retaining	C5.51	Circumferential Welds in Carbon or Low Alloy Steel Piping ≥ 3/8" Nominal Wall Thickness for Piping > NPS 4	303	Volumetric & Surface	
Welds in Carbon or Low Alloy Steel	C5.52	Longitudinal Welds in Carbon or Low Alloy Steel Piping ≥ 3/8" Nominal Wall Thickness for Piping > NPS 4	N/A	Volumetric & Surface	ISI-04
Piping	C5.81	Circumferential Welds in Carbon or Low Alloy Steel Pipe Branch Connections of Branch Piping ≥ NPS 2	2	Surface	
C-G Pressure Retaining Welds in Pumps and Valves	C6.10	Pump Casing Welds	8	Surface	
С-Н	C7.10	Pressure Vessels - System Pressure Test	See Note 6	Visual, VT-2	ISI-07
All Pressure	C7.20	Pressure Vessels - System Hydrostatic Test ⁵	See Note 6	Visual, VT-2	ISI-07
Retaining	C7.30	Piping - System Pressure Test	See Note 6	Visual, VT-2	ISI-07, 09
Components	C7.40	Piping - System Hydrostatic Test ⁵	See Note 6	Visual, VT-2	ISI-07, 09
(Class 2)	C7.50	Pumps - System Pressure Test	See Note 6	Visual, VT-2	ISI-07
	C7.60	Pumps - System Hydrostatic Test ⁵	See Note 6	Visual, VT-2	ISI-07
	C7.70	Valves - System Pressure Test	See Note 6	Visual, VT-2	ISI-07
	C7.80	Valves - System Hydrostatic Test ⁵	See Note 6	Visual, VT-2	ISI-07

Examination Category	Item Number	Description of Components Examined	Number of Components	Examination Method	Relief Request Number
D-A ¹	D1.10 ¹	Integrally Welded Attachments to Pressure Vessels	42	Visual, VT-1	ISI-06
Integral	D1.201	Integrally Welded Attachments to Piping	165	Visual, VT-1	ISI-06
Attachments for Class 3 Vessels, Piping, Pumps & Valves	D1.30 ¹	Integrally Welded Attachments to Pumps	4	Visual, VT-1	ISI-06
D-A Systems in Support of Reactor Shutdown	D1.10	Class 3 Pressure Retaining Components - System Pressure Test - System Hydrostatic Test	See Note 6	Visual, VT-2 Visual, VT-2	
Function D-B	D2.10	Class 3 Pressure Retaining Components			
Systems in Support of Emergency Core Cooling, Containment Heat Removal, Atmosphere Cleanup, and Reactor Residual Heat Removal		- System Pressure Test - System Hydrostatic Test	See Note 6 See Note 6	Visual, VT-2 Visual, VT-2	

Examination Category	Item Number	Description of Components Examined	Number of Components	Examination Method	Relief Request Number
D-C Systems in Support of Residual Heat Removal from Spent Fuel Storage Pool	D3.10	Class 3 Pressure Retaining Components - System Pressure Test - System Hydrostatic Test	See Note 6	Visual, VT-2 Visual, VT-2	
F-A ^{3,4}	F1.10 ³	Class 1 Piping Supports	297	Visual, VT-3	
Supports	F1.20 ³	Class 2 Piping Supports	378	Visual, VT-3	
	F1.30 ³	Class 3 Piping Supports	656	Visual, VT 3	
F	F1.40 ³	Supports Other Than Piping Supports (Class 1, 2, and 3)	169	Visual, VT-3	ISI-03

Notes:

- Reference Callaway Relief Request ISI-06 for a description of the Examination Categories and Item Numbers used for the inservice inspection of integrally welded attachments.
- The extent, frequency and acceptance standards for the examination of Steam Generator tubing will be in accordance with Callaway Technical Specification 3/4.4.5.
- Reference Code Case N-491 for a description of the Examination Category and Item Numbers used for the inservice inspection of supports.
- 4. Snubber assemblies will be tested and inspected in accordance with Callaway Technical Specification 3/4.7.8. Integral and non-integral attachments for snubbers, including lugs, bolting, pins, and clamps, shall be examined in accordance with the requirements of Code Case N-491.
- The system hydrostatic pressure test will be performed to the alternate rules for 10-year hydrostatic pressure testing delineated in Code Case N-498.
- Pressure retaining components (e.g., pressure vessels, pumps, valves, piping, etc.) that are subject to a system
 pressure or hydrostatic test are identified on Inservice Inspection Boundary Drawings. Reference Section 2.2 of
 this Plan for details pertaining to these drawings.

3.2 Augmented Inservice Inspections

The Augmented Inservice Inspection Summary Table 3.2 provides the following information:

3.2.1 Implementing Document

This column lists the basis document for the augmented inservice examination.

3.2.2 Description of Components Examined

This column provides a description of the components subject to augmented inservice inspection.

3.2.3 Number of Components

This column lists the population of components potentially subject to augmented examination. The number of components actually examined during the inspection interval will be in accordance with the requirements of the implementing document.

3.2.4 Examination Method

This column lists the examination method required by the implementing document.

3.2.5 Relief Request Number

This column provides a listing of applicable relief requests. If a relief request number is identified, see the corresponding relief request in Section 4.4.

TABLE 3.2 AUGMENTED INSERVICE INSPECTION SUMMARY TABLE

Implementing Document	Description of Components Examined	Number of Components	Examination Method	Relief Request Number
NRC Standard Review Plan	Circumferential, Longitudinal and Branch Welds in Piping 2" NPS and Greater	376	Volumetric	
Sections 3.6.1 and 6.6	Circumferential Welds in Piping 1 1/2" NPS	16	Surface	ISI-02
	Socket Welds in Piping 1 1/2" NPS and Greater	105	Surface	
Regulatory Guide 1.14	RCP Flywheel in Areas of Higher Stress Concentration at the Bore and Keyway	4_	Volumetric	
	RCP Flywheel at Exposed Surfaces	4	Surface	
	RCP Flywheel, Entire Volume	4	Volumetric	
Regulatory Guide 1.65	Reactor Vessel Closure Studs, When Removed	54	Surface	
Regulatory Guide 1.137	Fuel System for Standby Diesel Generators	2	Visual, VT-2	
Regulatory Guide 1.150	Reactor Vessel Welds	32	Volumetric	

SECTION 4.0 ALTERNATIVE REQUIREMENTS TO ASME SECTION XI, 1989 EDITION

This section lists the alternative requirements to ASME Section XI, 1989 Edition, being adopted for the Second Interval Inservice Inspection Program at the Callaway Plant. The alternative requirements presented are in accordance with ASME Section XI and 10 CFR 50.55a, as applicable.

4.1 Adoption of Code Cases

This Section addresses the adoption of Code Cases during the Second Inservice Inspection Interval at the Callaway Plant. Code Cases adopted for Inservice Inspection use during the Second Interval will be listed in Table 4.1 of this Inservice Inspection Plan. Code Cases for Repair/Replacement activities are not addressed in this Inservice Inspection Plan. In all cases, the use and adoption of Code Cases will be in accordance with ASME Section XI, IWA-2440 and 10 CFR 50.55a. The methodology for adopting Code Cases is divided into the four categories clarified below.

4.1.1 Adoption of Code Cases Listed for Generic Use in Regulatory Guide 1.147

Code Cases that are listed for generic use in Regulatory Guide 1.147, Revision 10, and later, will be adopted for use during the Second Inservice Inspection Interval by listing them in Table 4.1 of this Inservice Inspection Plan. All conditions or limitations delineated in Regulatory Guide 1.147 for a particular Code Case will apply.

4.1.2 Adoption of Code Cases Not Listed for Generic Use in Regulatory Guide 1.147

Code Cases that have been approved by the Board of Nuclear Codes and Standards, but that have not been listed for generic use in Regulatory Guide 1.147, may be submitted in the form of a Relief Request in accordance with 10 CFR 50.55a(a)(3). Alternatively, Code Cases may be adopted for use by special permission from the NRC as specified in Callaway FSAR, Appendix 3A.

4.1.3 Adoption of Code Cases Listed for Generic Use in Regulatory Guide 1.147 But Subsequently Annulled by ASME Section XI

Under certain circumstances, it may be necessary to adopt a Code Case that has been listed for generic use in Regulatory Guide 1.147, but subsequently annulled by ASME Section XI. Therefore, Union Electric Company endorses all revisions of Regulatory Guide 1.147 from Revision 10 up to and including the most recent revision. Endorsement of these revisions of Regulatory Guide 1.147 does not commit the Callaway Plant to all Code Cases listed therein, but rather allows for selection of a previously accepted Code Case. The purpose of this endorsement is to identify all Code Cases that could potentially be incorporated into the Inservice Inspection Plan in accordance with IWA-2441.

4.1.4 Adoption of Code Cases Issued Subsequent to Filing this Inservice Inspection Plan

Code Cases issued by ASME Section XI subsequent to filing this Inservice Inspection Plan will be proposed for use in amendments to this Plan in accordance with ASME Section XI, IWA-2441(d).

TABLE 4.1 LIST OF ADOPTED CODE CASES

CODE CASE NO.	TITLE	REG. GUIDE 1.147 REVISION	DATE ADOPTED
N-460	Alternative Examination Coverage for Class 1 and Class 2 Welds	10 8/1/9	
N-461	Alternative Rules for Piping Calibration Block Thickness	10	8/1/95
N-489	Alternative Rules for Level III NDE Qualification Examinations	10	8/1/95
N-491 Alternative Rules for Examination of Class 1, 2, 3 and MC Component Supports of Light Water Cooled Power Plants		10	8/1/95
N-498-1	Alternative Rules for 10-Year Hydrostatic Testing for Class 1, 2 and 3 Systems	N/A	8/1/95

4.2 Use of Subsequent Editions of ASME Section XI

In accordance with 10 CFR 50.55a(g)(3)(v), components (including supports) may meet the requirements set forth in subsequent editions of Codes and Addenda, or portions thereof, which are incorporated by reference in 10 CFR 50.55a(b), subject to the limitations and modifications listed therein. This Section of the Inservice Inspection Plan provides for alternative requirements from approved subsequent Code editions that may be adopted during the Second Inservice Inspection Interval. This Inservice Inspection Plan will be amended for adoption of subsequent Code rules.

4.3 Inservice Inspection Relief Request Index

This section provides a summary listing and revision status of all Relief Requests related to inservice inspections at the Callaway Plant.

TABLE 4.3 INSERVICE INSPECTION RELIEF REQUEST INDEX

Relief Request	Page	Rev.	Date	Topic	
ISI-01	4.4-2	0	10/12/94	Exemption from Appendix VII Ultrasonic Examination Personnel Qualification Requirements	
ISI-02 4.4-4 0 10/12/94		Exemption of Circumferential Welds in 1 1/2" NPS Piping from Volumetric Examination Requirements of NRC Standard Review Plan, Section 3.6.1			
ISI-03	4.4-6	0	10/12/94	Limited Examination of Reactor Vessel Supports	
ISI-04	4.4-9	0	10/12/94	Alternate Examination Requirements for Longitudinal Welds in Class 1 and 2 Piping	
ISI-05	4.4-10	0	10/12/94	Alternate Rules for Deferral of Inspections on Nozzle-to-Vessel Welds, Inside Radius Sections and Nozzle-to- Safe End Welds of the Reactor Vessel	
ISI-06	4.4-15	0	10/12/94	Alternate Rules for the Selection and Examination of Class 1, 2 and 3 Integrally Welded Attachments	
ISI-07	4.4-21	0	10/12/94	Alternate Rules for Insulation Removal During IWB-5000 and IWC-5000 Pressure Tests at Bolted Connections in Systems Borated for the Purpose of Controlling Reactivity	

TABLE 4.3 INSERVICE INSPECTION RELIEF REQUEST INDEX

(cont.)

Relief Request	Page	Rev.	Date	Topic
ISI-08	4.4-24	0	10/12/94	Alternate Rules for Corrective Measures if Leakage Occurs at a Bolted Connection
ISI-09	4.4-26	0	10/12/94	Alternate Provisions for Pressure Testing Code Class 2 Piping and Valves at Containment Penetrations Where the Balance of the System is Outside the Scope of Section XI

4.4 Inservice Inspection Relief Requests

- 4.4.1 This section contains Relief Requests written in accordance with 10 CFR 50.55a(g)(5) when specific ASME Section XI requirements for inservice inspection are considered impractical. The enclosed Relief Requests are subject to change throughout the inspection interval. If examination requirements are determined to be impractical during the course of the interval, additional or modified relief requests shall be submitted in accordance with 10 CFR 50.55a(g)(5).
- 4.4.2 Exceptions to Code required examinations may also be authorized by NRR, as allowed by 10 CFR 50.55a (a)(3), provided that design, fabrication, installation, testing and inspection performed in compliance with Codes and Section XI requirements would result in hardship without a compensating increase in the level of quality and safety, or provided that the proposed alternative examination will assure an acceptable level of quality and safety. Specific exceptions may also be documented in the form of Relief Requests and included in this Section, as applicable.
- 4.4.3 Relief Requests for incomplete examinations shall be submitted in accordance with 10 CFR 50.55a(g)(5)(iv) throughout the interval as limitations are identified. Due to ongoing changes in nondestructive examination procedures, techniques and requirements, the Union Electric Company considers that submitting Relief Requests for incomplete examinations when they are evaluated will provide a more accurate representation of the limitations.

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COMPONENT IDENTIFICATION

Code Classes: 1 and 2

References: IWA-2311(b) and Appendix VII

Examination Categories: B-A, B-B, B-D, B-F, B-G-1, B-J, C-A, C-B, C-D, C-F-1, C-F-2

Item Numbers: B1.11, B1.12, B1.21, B1.22, B1.30, B1.40, B2.11, B2.12, B2.40,

B3.90, B3.100, B3.110, B3.120, B3.140, B5.10, B5.40, B5.70,

B6.20, B6.30, B6.40, B6.180, B9.11, B9.12, B9.31

C1.10, C1.20, C1.30, C2.21, C2.22, C4.40, C5.11, C5.12, C5.21,

C5.51, C5.52

Description: Exemption from Appendix VII Ultrasonic Examination Personnel

Qualification Requirements

Component Numbers: All Class 1 and 2 components requiring ultrasonic examination

CODE REQUIREMENT

Paragraph IWA-2311(b) specifies that the training, qualification, and certification of ultrasonic examination personnel shall also comply with the requirements of Appendix VII.

Appendix VII provides requirements for the employer's written practice, qualification of ultrasonic examiners, qualification records, and the minimum content of initial training courses for the ultrasonic examination method in addition to those required in SNT-TC-1A.

BASIS FOR RELIEF

Union Electric requests relief from implementation of Appendix VII until the performance demonstration requirements of Appendix VIII are fully implemented. Implementation of Appendix VII prior to full implementation of Appendix VIII is considered impractical and without a compensating increase in quality and safety.

Appendix VII was first introduced in the 1988 Addenda to Section XI. This Appendix represents a dramatic change from previous Code editions and current industry practices in the requirements for qualification of ultrasonic examination personnel. New training programs must be developed and taught by trained instructors, employer's written practices must be completely rewritten, examination question banks must be developed, flaw specimens containing actual or simulated flaws must be acquired, and performance demonstrations (practical examinations) must be completed.

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Implementation of Appendix VII will require a substantial industry effort. Although work is progressing towards compliance with Appendix VII, full implementation has not yet been achieved. Since Appendix VII provides for use of specimens prepared for ultrasonic performance demonstrations per Appendix VIII, many NDE vendors are developing these two programs concurrently in order to avoid duplicated effort. Though currently not required, the nuclear industry anticipates that the Appendix VIII performance demonstration requirements will be mandated by a backfit ruling in the Federal Register. In anticipation of this ruling, the Performance Demonstration Initiative (PDI) Committee is currently leading an industry wide effort to implement Appendix VIII. The tentative completion dates for pipe weld performance demonstrations and reactor vessel performance demonstrations are January of 1996, and January of 1997, respectively.

The Union Electric Company intends to fully implement Appendix VII when the performance demonstrations of Appendix VIII are mandated by a backfit ruling in the Federal Register.

PROPOSED ALTERNATE PROVISIONS

The Callaway Plant shall utilize ultrasonic examination personnel qualified in accordance with the requirements of IWA-2300, except for IWA-2311(b). The additional Appendix VII training, qualification, and certification requirements referenced in IWA-2311(b) shall be fully implemented when the performance demonstrations of Appendix VIII are mandated by a ruling in the Federal Register.

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COMPONENT IDENTIFICATION

Code Class:

2

References:

NRC Standard Review Plan, Section 3.6.1 (NUREG-0800)

Examination Category:

N/A

Item Number:

N/A

Description:

Exemption of Circumferential Welds in 1 1/2" NPS Piping from the Volumetric Examination Requirements of NRC Standard

Review Plan. Section 3.6.1

Component Numbers:

RCP "A" Seal Water Injection Line Welds

2" x 1 1/2" Reducer to 1 1/2" Pipe 2-BG-09-FW387

2-BG-09-FW386

1 1/2" Pipe to Valve

2-BG-09-FW385

1 1/2" Pipe to Valve

2-BG-09-FW384

2" x 1 1/2" Reducer to 1 1/2" Pipe

RCP "B" Seal Water Injection Line Welds

2-BG-09-FW432

2" x 1 1/2" Reducer to 1 1/2" Pipe

2-BG-09-FW431

1 1/2" Pipe to Valve

2-BG-09-FW430

1 1/2" Pipe to Valve

2-BG-09-FW429

2" x 1 1/2" Reducer to 1 1/2" Pipe

RCP "C" Seal Water Injection Line Welds

2-BG-09-FW417

2" x 1 1/2" Reducer to 1 1/2" Pipe

2-BG-09-FW416

1 1/2" Pipe to Valve

2-BG-09-FW415

1 1/2" Pipe to Valve

2-BG-09-FW414

2" x 1 1/2" Reducer to 1 1/2" Pipe

RCP "D" Seal Water Injection Line Welds

2-BG-09-FW402

2" x 1 1/2" Reducer to 1 1/2" Pipe

2-BG-09-FW401

1 1/2" Pipe to Valve

2-BG-09-FW400

1 1/2" Pipe to Valve

2-BG-09-FW399

2" x 1 1/2" Reducer to 1 1/2" Pipe

AUGMENTED REQUIREMENT

Standard Review Plan, Section 3.6.1 requires that circumferential welds in piping exceeding 1" NPS be subject to volumetric examination.

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BASIS FOR RELIEF

The sixteen welds listed above are all within portions of the Reactor Coolant Pump Seal Water Injection Lines which are schedule 160, 1 1/2" NPS. A combination of the small pipe diameter and pipe thickness cause the volumetric examinations to yield meaningless results.

PROPOSED ALTERNATE EXAMINATIONS

As an alternative to the requirements of NRC Standard Review Plan, Section 3.6.1, liquid penetrant examinations shall be performed on all sixteen of the subject welds. In addition, a visual VT-2 examination shall be performed on these welds as specified in ASME Section XI.

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COMPONENT IDENTIFICATION

Code Class:

References: Code Case N-491, Table -2500-1

Examination Category: F-A Item Number: F1.40

Description: Limited Examination of Reactor Vessel Supports

Component Numbers: 2-RBB01-01, 2-RBB01-02, 2-RBB01-03, and 2-RBB01-04

CODE REQUIREMENT

ASME Section XI, Code Case N-491, Table -2500-1 requires that 100% of Class 1 supports, other than piping, be subject to a visual VT-3 examination once every inspection interval.

BASIS FOR RELIEF

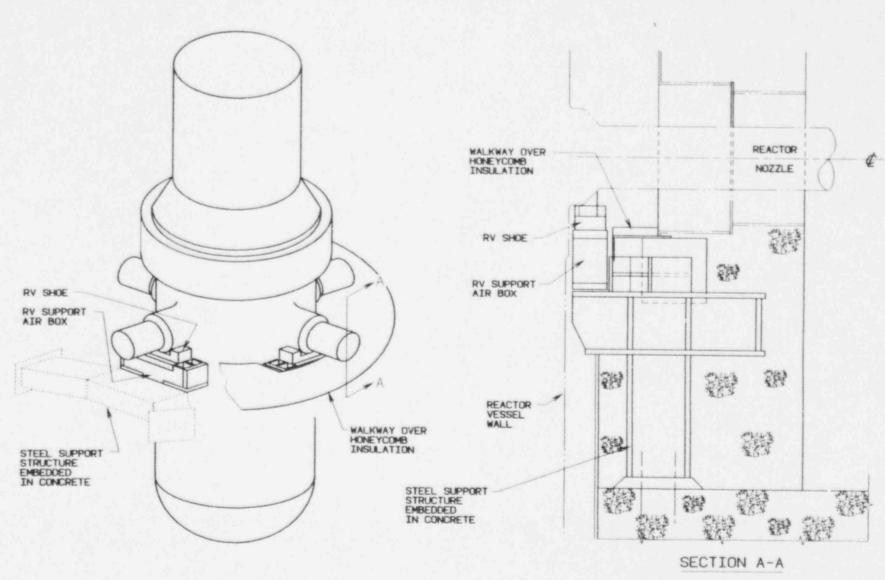
There is a support assembly at each of these nozzles that consists of a nozzle weld build up, shoe plate, air cooled box, and steel support structure embedded in the primary shield wall. Figure ISI-03 depicts these support assemblies. As shown in the figure, only the nozzle weld build up and shoe plate are completely accessible for a visual VT-3 examination. The majority of the air cooled box and the entire steel support structure are located beneath a steel walk plate and only the top of the air cooled box is directly accessible. An additional 20 to 30 percent of the air cooled box and a very small percentage of the steel support structure would be made accessible if the steel walk plate and insulation were removed.

The Reactor Vessel supports are located in a confined space below the refueling pool permanent seal ring. The area can only be accessed through four seal ring hatches. In addition to difficult access, the radiation in the area is between 1.5 to 2.0 man-rem per hour. It is estimated that the removal and re-installation of the walk plate and insulation in this confined space, combined with the visual VT-3 examination, would result in an exposure of approximately 36 man-rem. Removal of the walk plate and insulation under these conditions to increase the examination of the air cooled box by approximately 20 to 30 percent and a very small percentage of the steel support structure is considered impractical without a commensurate increase in quality or safety. Based on this, relief is requested from the visual VT-3 examination of the air cooled box and steel support structure that is obstructed by the walk plate and insulation.

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PROPOSED ALTERNATE EXAMINATION

A limited visual VT-3 examination, with the walk plate and insulation installed, shall be performed on the accessible NF portions of the Reactor Vessel support assemblies to satisfy the requirements of Code Case N-491, Table -2500-1, Item No. F1.40. If conditions are discovered during this limited VT-3 examination that do not meet the acceptance standards of N-491, -3400, the walk plate or insulation will, if necessary, be removed to meet the evaluation requirements of N-491, -3112.2 or -3112.3.



RELIEF REQUEST NUMBER: ISI-03

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COMPONENT IDENTIFICATION

Code Classes:

1 and 2

References:

Table IWB-2500-1, Table IWC-2500-1

Examination Categories:

B-J, C-F-1, C-F-2

Item Numbers:

B9.12, B9.22, C5.12, C5.22, C5.42, C5.52, C5.62, and C5.82 Alternate Examination Requirements for Longitudinal Welds in

Description:

Class 1 and 2 Piping

Component Numbers:

All Class 1 and 2 Longitudinal Piping Welds Subject to Surface or

Volumetric Examination

CODE REQUIREMENTS

Class 1 Piping:

Table IWB-2500-1 requires the performance of surface and volumetric examinations on Item No. B9.12 longitudinal welds, and a surface examination on Item No. B9.22 longitudinal welds. The examination includes at least a pipe diameter length, but not more than 12 in., of each longitudinal weld intersecting the circumferential welds required to be examined by Examination Categories B-F and B-J.

Class 2 Piping:

Table IWC-2500-1 requires the performance of surface and volumetric examination on Item Nos. C5.12, C5.22, C5.52 and C5.62, and a surface examination on Item Nos. C5.42 and C5.82. The examination includes a length of 2.5t at the intersecting circumferential welds required to be examined by Examination Categories C-F-1 and C-F-2.

BASIS FOR RELIEF

Based on the reasons stated below, the performance of surface and volumetric examination on longitudinal piping welds has a negligible compensating effect on the quality or safety of Class 1 and 2 piping. In addition, there is little, if any, technical benefit associated with the performance of these examinations, but they result in a substantial man-rem exposure and cost.

- Throughout the nuclear industry, there has been no evidence of rejectable service induced flaws being attributed to longitudinal piping welds.
- During the first inservice inspection interval at the Callaway Plant, no inservice flaws have been detected in longitudinal piping welds.

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- There are distinct differences between the processes used in the manufacturing of longitudinal and circumferential welds which enhance the integrity of longitudinal welds. First, longitudinal welds are typically manufactured under controlled shop conditions whereas circumferential welds are produced in the field under less ideal conditions. Secondly, longitudinal welds usually undergo heat treatment in the shop which improves their material properties and relieves the residual stresses created by welding. Finally, shop manufacturing inspections can be performed under more favorable conditions which further increase the confidence level of the longitudinal weld quality.
- During field installation of piping, the ends of the longitudinal welds may be affected during welding of the intersecting circumferential field welds. This small area falls within the circumferential weld inspection boundaries. Therefore, the ends of the longitudinal welds will still be subject to examination.
- 5) From an industry-wide standpoint, there has been no evidence of longitudinal weld defects compromising safety at nuclear generating facilities.
- 6) No significant loading conditions or known material degradation mechanisms have become evident to date which specifically relate to longitudinal seam welds in nuclear plant piping.
- 7) There is a significant accumulation of man-rem exposure and cost associated with the inspection of Class 1 and 2 longitudinal piping welds.
- 8) The alternative examinations proposed below provide an acceptable level of quality and safety without causing undue hardship or difficulties.

PROPOSED ALTERNATE EXAMINATION

Surface and volumetric examinations shall be performed, as applicable, on the length of the longitudinal weld that is normally examined during inspection for the intersecting circumferential weld(s). The volumetric examination at the intersection of circumferential and longitudinal welds will include both transverse and parallel scans within the length of the longitudinal weld that falls within the circumferential weld examination boundary.

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COMPONENT IDENTIFICATION

Code Class:

References: IWB-2500, Table IWB-2500-1

Examination Categories: B-D, B-F

Item Numbers: B3.90, B3.100, B5.10

Description: Alternate Rules for the Deferral of Inspections on Nozzle-to-

Vessel Welds, Inside Radius Sections and Nozzle-to-Safe End

Welds of the Reactor Vessel

Component Numbers: Nozzie-to-Vessel Welds (Item No. B3.90)

2-RV-105-121-A 2-RV-105-121-B 2-RV-105-121-C 2-RV-105-121-D 2-RV-107-121-A 2-RV-107-121-B

2-RV-107-121-C 2-RV-107-121-D

Inner Radius Sections (Item No. B3 100)

2-RV-105-121-A-IR 2-RV-105-121-B-IR 2-RV-105-121-C-IR 2-RV-105-121-D-IR 2-RV-107-121-A-IR 2-RV-107-121-B-IR 2-RV-107-121-C-IR

2-RV-107-121-D-IR

Nozzle-to-Sase End Welds (Item No. B5.10)

2-RV-301-121-A 2-RV-301-121-B 2-RV-301-121-C 2-RV-301-121-D 2-RV-302-121-A 2-RV-302-121-B

2-RV-302-121-C 2-RV-302-121-D

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CODE REQUIREMENT

ASME Section XI, 1989 Edition, Table IWB-2500-1, Examination Category B-D requires volumetric examination of reactor vessel nozzle-to-vessel welds and their inside radius sections once each ten-year inspection interval. Note (2) of Examination Category B-D states that at least 25 percent, but not more than 50 percent (created), of the reactor vessel nozzles shall be examined by the end of the first inspection period, and the remainder by the end of the inspection interval.

Table IWB-2500-1, Examination Category B-F, Note (1) states that the reactor vessel nozzle-to-safe end weld examinations may be performed coincident with the vessel nozzle examinations required by Examination Category B-D.

BASIS FOR RELIEF

Relief is requested to defer 100 percent of the reactor vessel nozzle-to-vessel weld examinations, the nozzle inside radius section examinations, and the nozzle-to-safe end weld examinations to the end of Callaway's second ten-year inspection interval.

Union Electric believes that performing 25 percent to 50 percent of the reactor vessel nozzle examinations in the first period of the second inspection interval is impractical for the following reasons:

- 1) The vendor cost alone (not including site training, plant support, or potential critical path time) to perform these examinations with automated tooling in the first inspection period is currently estimated at \$250,000. The cost to perform these same examinations at the end of the second inspection interval concurrent with the reactor vessel ten-year examination is estimated at only \$25,000. The major expense associated with the first inspection period examinations is the added equipment and personnel mobilization costs and equipment assembly and disassembly costs.
- 2) Approximately three to four man-rem exposure is currently expended for automated equipment assembly and disassembly in the reactor cavity area. In addition to exposure, there are approximately two to three cubic feet of solid radwaste generated during performance of automated examinations in the reactor vessel. Under current Code rules, this personnel exposure and radwaste generation would be incurred twice, once for the nozzle first inspection period examinations and again for the reactor vessel examinations at the end of the inspection interval. Performing the nozzle examinations concurrent with the reactor vessel ten-year examinations will save approximately three to four man-rem exposure and two to three cubic feet of solid radwaste.

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For reasons listed below, Union Electric believes that deferral of 100 percent of the reactor vessel nozzle examinations to the end of the second inspection interval will provide an acceptable level of safety and quality.

- 1) All four of Callaway's Reactor Vessel hot leg nozzle-to-vessel welds, hot leg nozzle inside radius sections, and hot leg nozzle-to-safe end welds were examined in 1987 during the first period of the first ten-year inspection interval. No indications or relevant conditions were discovered that required successive inspections in accordance with Paragraph IWB-2420(b). Furthermore, no inservice repairs or replacements by welding have ever been performed on any of the nozzle-to-vessel welds, nozzle inside radius sections, or nozzle-to-safe end welds at Callaway.
- 2) From an industry perspective, there are two reasons why deferral of Callaway's nozzle examinations to the end of the second inspection interval will not decrease the level of quality and safety. First, PWR reactor vessels similar to Callaway's have been operating for over 20 years with no recorded inservice induced flaws or potential degradation mechanisms. Since each PWR reactor vessel in operation is representative of the operating conditions throughout the industry, continued inspection of these vessels ensures that any potential degradation mechanism will be detected. Second, given the present large population of PWR reactor vessels in operation, the examination of nozzles within the industry during any ten-year interval is evenly distributed. This distribution is essentially equivalent, regardless of whether or not a percentage of the nozzle examinations are performed in the first inspection period or performed concurrent with the reactor vessel ten-year examinations at the end of the inspection interval.
- 3) The pressurizer and primary steam generator nozzle-to-vessel welds, inside radius sections, and nozzle-to-safe end welds are similar in configuration, material properties, weld process parameters, and operate in the same reactor coolant system environment as the reactor vessel nozzles. Due to this similarity, distribution of the pressurizer and steam generator nozzle examinations in accordance with Examination Category B-D and Examination Category B-F will further substantiate the integrity of the reactor vessel nozzles until they are examined at or near the end of the second inservice inspection interval.
- 4) Performing all the automated reactor vessel examinations during a single refueling outage improves consistency of the examinations by utilizing the same equipment, personnel, and procedures. Moreover, this improves the reliability and reproducibility of the examinations.

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PROPOSED ALTERNATE EXAMINATION

Union Electric shall complete the required nozzle-to-vessel weld examinations, the nozzle inside radius section examinations, and the nozzle-to-safe end weld examinations concurrent with the reactor vessel ten-year examinations at or near the end of the second ten-year inservice inspection interval. In addition, the reactor vessel hot leg nozzle inside surfaces, including the inside radius sections and nozzle-to-safe end weld areas, that are made accessible with the upper internals removed and lower internals (core barrel) installed, shall be visually VT-3 examined once each inspection period of the second ten-year inservice inspection interval.

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COMPONENT IDENTIFICATION

Code Classes: 1, 2 and 3

References: Table IWB-2500-1, Table IWC-2500-1, Table IWD-2500-1

Examination Categories: B-H, B-K-1, C-C, D-A, D-B, D-C

Item Numbers: B8.10, B8.20, B8.30, B8.40, B10.10, B10.20, B10.30, C3.10,

C3.20, C3.30, C3.40, D1.20, D1.30, D1.40, D1.50, D1.60, D2.20,

D2.30, D2.40, D2.50, D2.60

Description: Alternate Rules for the Selection and Examination of Class 1, 2

and 3 Integrally Welded Attachments

Component Numbers: All Class 1, 2 and 3 Integral Attachments Subject to Inservice

Inspection

CODE REQUIREMENTS

Class 1 Attachments:

Table IWB-2500-1, Examination Categories B-H and B-K-1 require the performance of surface or volumetric examinations, as applicable, on integral attachments with a design thickness of 5/8" or greater.

Class 2 Attachments:

Table IWC-2500-1, Examination Category C-C requires the performance of a surface examination on integral attachments with a design thickness of 3/4" or greater.

Class 3 Attachments:

Table IWD-2500-1, Examination Categories D-A, D-B and D-C require the performance of a visual VT-3 inspection on integral attachments.

For complete details on ASME Section XI Code examination requirements, see Tables IWB-2500-1, IWC-2500-1 and IWD-2500-1.

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BASIS FOR RELIEF

Relief is being requested to allow the use of alternate requirements for the examination of Class 1, 2 and 3 integral attachments. The basis for this request is as follows:

- During the first inservice inspection interval at the Callaway Plant, no inservice flaws were detected in integrally welded attachments which would affect safety or compromise the integrity of the plant.
- Within the nuclear industry, failures in integral attachments have been very rare and have not affected plant safety. When failures or inservice defects are found in integral attachments, they are usually associated with a support which has been damaged during operation. Therefore, flawed or broken integral attachments are typically detected during the investigation of damaged supports rather than during Code scheduled inservice inspections. One purpose of the alternative examination proposed below is to focus the inspection of integral attachments on those instances where the associated supports show signs of damage. This will increase the likelihood of locating damaged integral attachments.
- 3) There is a significant amount of man-rem exposure and cost associated with the scheduled inspection of Class 1, 2, and 3 integral attachments.
- 4) Unlike ASME Section XI, the alternate examinations proposed below do not impose a minimum thickness requirement for the inspection of an integral attachment. Therefore, a greater population of integral attachments will be available for inspection because inspections will not be limited to thick attachments. This provision improves the quality and safety level established by these examinations.
- 5) The alternate examinations proposed below provide an acceptable level of quality and safety without causing undue hardship or difficulties.

PROPOSED ALTERNATE EXAMINATION

The following rules will be used to select and examine integrally welded attachments:

- This Relief Request is limited to Examination Categories B-H, B-K-1, C-C, D-A, D-B and D-C.
- Class 1, 2 and 3 component supports shall be selected for examination in accordance with Code Case N-491.
- Except for the selection of component supports for examination, all references to Section XI
 within this Relief Request are from the 1989 Edition.

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Scope

These requirements apply to examination and sample selection of Class 1, 2, and 3 integrally welded attachments of vessels, piping, pumps, and valves listed in Table 2500-1 as follows:

- a) Table 2500-1, Examination Category B-K shall be used for Class 1 integrally welded attachments in Examination Categories B-H and B-K-1 of IWB.
- b) Table 2500-1, Examination Category C-C shall be used for Class 2 integrally welded attachments in Examination Category C-C of IWC.
- c) Table 2500-1, Examination Category D-A shall be used for Class 3 integrally welded attachments in Examination Categories D-A, D-B, and D-C of IWD.

Exemption Criteria

- a) The exemption criteria provided in IWB-1220, IWC-1220, and IWD-1220 may be applied to Class 1, 2, and 3 components respectively, with integrally welded attachments, required to be examined in accordance with Table 2500-1.
- b) Class 1, 2, and 3 integrally welded attachment examinations performed as a result of component support deformation cannot be credited under the requirements of IWB-2411 or IWB-2412, IWC-2411 or IWC-2412, and IWD-2411 or IWD-2412, respectively.

Inspection Schedule

Class 1, 2, or 3 integrally welded attachments selected for examination by sample selection criteria in accordance with Table 2500-1, Examination Categories B-K, C-C and D-A, shall meet the requirements of IWB-2411 or IWB-2412, IWC-2411 or IWC-2412, or IWD-2411 or IWD-2412, respectively.

Additional and Successive Examinations

a) Class 1, 2, and 3 additional and successive examination requirements of IWB-2430 and IWB-2420 for Class 1, IWC-2430 and IWC-2420 for Class 2 and 3 as applicable, shall be applied to integrally welded attachments whose examinations reveal flaws or relevant conditions that exceed the acceptance standards of IWB-3000, IWC-3000, and IWD-3000, respectively.

When integrally welded attachments are examined as a result of identified component support deformation and the results of these examinations exceed the applicable acceptance standards listed above, additional or successive examinations shall be performed when determined necessary based on an evaluation.

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TABLE 2500-1 EXAMINATION CATEGORIES

Item No.	Parts Examined ¹	Examination Requirements/Fig. No.	Examination Method	Acceptance Standard	Extent of Examination ^{2,3}	Frequency of Examination ⁶
B10.10	Pressure Vessels Integrally Welded Attachments	IWB-2500-13, -14, and -	Surface ⁷	IWB-3516	100% of required areas of each welded attachment	Each identified occurrence and each inspection interval ⁴
B10.20	Piping Integrally Welded Attachments	IWB-2500-13, -14, and -	Surface	IWB-3516	100% of required areas of each welded attachment	Each identified occurrence and each inspection interval ⁵
B10.30	Pumps Integrally Welded Attachments	IWB-2500-13, -14, and -	Surface	IWB-3516	100% of required areas of each welded attachment	Each identified occurrence and each inspection interval ⁵
B10.40	Valves Integrally Welded Attachments	IWB-2500-13, -14, and -	Surface	IWB-3516	100% of required areas of each welded attachment	Each identified occurrence and each inspection interval ⁵

NOTES:

- (1) Examination is limited to those integrally welded attachments that meet the following conditions:
 - (a) the attachment is on the outside surface of the pressure retaining component;
 - (b) the attachment provides component support as defined in NF-1110; and
 - (c) the attachment weld joins the attachment either directly to the surface of the component or to an integrally cast or forged attachment to the component.
- (2) The extent of the examination includes essentially 100% of the length of the attachment weld at each attachment subject to examination.
- (3) Selected samples of integrally welded attachments shall be examined each inspection interval.
- (4) In the case of multiple vessels of similar design, function and service, only one integrally welded attachment of only one of the multiple vessels shall be selected for examination.
- (5) For piping, pumps, and valves, a sample of 10% of the welded attachments associated with the component supports selected for examination under Code Case N-491 shall be examined.
- (6) Examination is required whenever component support member deformation (e.g., broken, bent, or pulled out parts) is identified during operation, refueling, maintenance, examination, inservice inspection, or testing.
- (7) For the configuration shown in Fig. IWB-2500-14, a volumetric examination of volume A-B-C-D from side (B-C) of the circumferential welds may be performed in lieu of the surface examination of surfaces A-D and B-C.

RELIEF REQUEST NUMBER: ISI-06 (Page 5 of 6)

TABLE 2500-1 (cont'd) EXAMINATION CATEGORIES

Item No.	Parts Examined ¹	Examination Requirements/Fig. No.	Examination Method	Acceptance Standard	Extent of Examination ^{2,3}	Frequency of Examination ⁶
C3.10	Pressure Vessels Integrally Welded Attachments	IWC-2500-5	Surface	IWB-3512	100% of required areas of each welded affachment	Each identified occurrence and each inspection interval ⁴
C3.20	Piping Integrally Welded Attachments	IWC-2500-5	Surface	IWB-3512	100% of required areas of each welded attachment	Each identified occurrence and each inspection interval ⁵
C3.30	Pumps Integrally Welded Attachments	IWC-2500-5	Surface	IWB-3512	100% of required areas of each welded attachment	Each ide-tified occurrence and each inspection interval ^s
C3.40	Valves Integrally Welded Attachments	TG-C-2500-5	Surface	IWB-3512	100% of required areas of each welded attachment	Each identified occurrence and each inspection interval ⁵

NOTES

- (1) Examination is limited to those integrally welded attachments that meet the following conditions:
 - the attachment is on the outside surface of the pressure retaining component;
 - (b) the attachment provides component support as defined in NF-1110; and
 - (c) the attachment weld joins the attachment either directly to the surface of the component or to an integrally cast or forged attachment to the component.
- (2) The extent of the examination includes essentially 100% of the length of the attachment weld at each attachment subject to examination.
- (3) Selected samples of integrally welded attachments shall be examined each inspection interval.
- (4) In the case of multiple vessels of similar design, function and service, only one integrally welded attachment of only one of the multiple vessels shall be selected for examination.
- (5) For piping, pumps, and valves, a sample of 10% of the welded attachments associated with the component supports selected for examination under Code Case N-491 shall be examined.
- (6) Examination is required whenever component support member deformation (e.g., broken, bent, or pulled out parts) is identified during operation, refueling, maintenance, examination, inservice inspection, or testing.

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TABLE 2500-1 (cont'd) EXAMINATION CATEGORIES

Item No.	Parts Examined ¹	Examination Requirements/Fig. No.	Examination Method	Acceptance Standard	Extent of Examination ^{2,3}	Frequency of Examination ^{3,4}
D1.10	Pressure Vessels Integrally Welded Attachments	IWD-2500-1	Visual, VT-1	IWD-3000	100% of required areas of each welded attachment	Each identified occurrence and each inspection interval
D1.20	Piping Integrally Welded Attachments	IWD-2500-1	Visual, VT-1	IWD-3000	100% of required areas of each welded attachment	Each identified occurrence and each inspection interval
D1.30	Pumps Integrally Welded Attachments	IWD-2500-1	Visual, VT-1	IWD-3000	100% of required areas of each welded attachment	Each identified occurrence and each inspection interval
D1.40	Valves Integrally Welded Attachments	IWD-2500-1	Visual, VT-1	IWD-3000	100% of required areas of each welded attachment	Each identified occurrence and each inspection interval

NOTES:

- (1) Examination is limited to those integrally welded attachments that meet the following conditions:
 - (a) the attachment is on the outside surface of the pressure retaining component;
 - (b) the attachment provides component support as defined in NF-1110; and
 - (c) the attachment weld joins the attachment either directly to the surface of the component or to an integrally cast or forged attachment to the component.
- (2) The extent of the examination includes essentially 100% of the length of the attachment weld at each attachment subject to examination.
- (3) Selected samples of integrally welded attachments shall be examined each inspection interval. All integrally welded attachments selected for examination shall be subject to corrosion, such as the integrally welded attachments of the Service Water or Emergency Service Water systems. In the case of multiple vessels of similar design, function and service, the integrally welded attachments of only one of the multiple vessels shall be selected for examination. For integrally welded attachments of piping, pumps, and valves, a 10% sample shall be selected for examination. This percentage sample shall be proportional to the total number of nonexempt integrally welded attachments connected to the piping, pumps, and valves, located within each system subject to these examinations.
- (4) Examination is required whenever component support member deformation (e.g., broken, bent, or pulled out parts) is identified during operation, refueling, maintenance, examination, inservice inspection, or testing.

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COMPONENT IDENTIFICATION

Code Classes:

References: IWA-5242(a) Examination Categories: B-P and C-H

Item Numbers: All Item Numbers Listed Under Examination Categories B-P

and C-H

1 and 2

Description: Alternate Rules for Insulation Removal During IWB-5000

and IWC-5000 Pressure Tests at Bolted Connections in

Systems Borated for the Purpose of Controlling Reactivity

Component Numbers: Bolted Connections in Systems Borated for the Purpose of

Controlling Reactivity

CODE REQUIREMENT

ASME Section XI, 1989 Edition, Paragraph IWA-5242(a) states, "For systems borated for the purpose of controlling reactivity, insulation shall be removed from pressure retaining bolted connections for visual examination VT-2."

BASIS FOR RELIEF

Relief is requested from the requirement to remove insulation for visual VT-2 examination of bolted connections during a system pressure test on systems borated for the purpose of controlling reactivity. Union Electric believes that removal of insulation at bolted connections in borated systems solely for a visual VT-2 examination is impractical for the reasons listed below:

- Code Class 1 and 2 systems borated for the purpose of controlling reactivity are extensive and large systems covering many areas and elevations. Scaffolding will be required to access many of the bolted connections. In addition, many of the bolted connections are located in difficult to access areas and in medium to high radiation areas. Insulation removal combined with scaffolding requirements will increase the financial cost, personnel exposure, and generation of radwaste associated with performance of visual VT-2 examinations.
- 2) The visual VT-2 examination of Class 1 systems, primarily the Reactor Coolant System (RCS) piping and components, is performed between plant mode 3 and 2 ascending. As required by IWB-5221, the RCS is at a normal operating pressure of 2235 psig. Between modes 3 and 2 ascending, the temperature is approximately 557 °F. Performance of a visual VT-2 examination, installation of insulation, and disassembly of scaffolding at bolted connections under these operating conditions is a personnel safety hazard. The visual VT-2 examination is a critical path activity and normally has a duration of six to eight hours. Since the majority of Class 1 piping is inside the containment building bio-shield wall, insulation installation and disassembly of scaffolding will add to the outage duration. Critical path cost is currently estimated at \$207,000 per day.

(Page 2 of 3)

Union Electric believes that the established Callaway programs described below in addition to the alternative examination proposed below, provide an acceptable level of safety and quality for bolted connections in systems borated for the purpose of controlling reactivity.

- 1) In response to NRC Generic Letter 88-05, Union Electric has established a program for Engineering to inspect all boric acid leaks discovered in the containment building and to evaluate the impact of those leaks on carbon steel or low alloy steel components. All evidence of leaks, including boric acid crystals or residue, is inspected and evaluated regardless of whether the leak was discovered at power or during an outage. Issues such as the following are considered in the inspection and evaluation: 1) evidence of corrosion or metal degradation, 2) effect the leak may have on the pressure boundary, 3) possibility of boric acid traveling along the inside of insulation on piping, and 4) possibility of dripping or spraying on other components. Based on this evaluation, Engineering initiates appropriate corrective actions to prevent reoccurrence of the leak and to repair, if necessary, any degraded materials or components.
- 2) In addition to the nondestructive examinations required by ASME Section XI, Union Electric has committed to the bolting examination requirements of NRC Bulletin 82-02. In accordance with this Bulletin, at least two nondestructive examination techniques (e.g., ultrasonic, liquid penetrant, magnetic particle, or visual VT-1) are performed on bolted connections of the following components: Steam Generator primary manways, Pressurizer primary manway, Pressurizer safety valves, and a total of 22 Reactor Coolant System isolation valves that are greater than 6" NPS. As a minimum, two nondestructive examination techniques are used whenever the bolted connection of one of the subject components is disassembled for maintenance or other inspection. These additional examinations ensure that degradation mechanisms such as Stress Corrosion Cracking or corrosion do not go undetected in bolted connections critical to reactor safety.
- 3) The only carbon steel components at the Callaway Plant that are in systems borated for the purpose of controlling reactivity are clad with stainless steel. Specifically, these clad components are the Reactor Vessel, Steam Generators (primary side), and Pressurizer. All other piping and components in borated systems that are within inservice inspection boundaries are fabricated of stainless steel. There is substantial information, such as EPRI NP-5679, attesting to the resistance of stainless steels to boric acid corrosion. To ensure that degradation mechanisms in stainless steels are mitigated, Union Electric maintains a program at the Callaway Plant that controls materials (insulation, thread lubricant, boron, etc.) that may come in contact with safety related components, including bolting. This program ensures that impurities are not present in concentrations that would promote development of Stress Corrosion Cracking in stainless steel bolted connections.

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PROPOSED ALTERNATE EXAMINATION

Bolted connections in systems borated for the purpose of controlling reactivity shall receive a visual VT-2 examination during the system pressure tests of IWB-5000 and IWC-5000 with the insulation installed. If evidence of leakage is detected, either by discovery of active leakage or evidence of boric acid crystals, the insulation shall be removed and the bolted connection shall be re-examined and, if necessary, evaluated in accordance with the corrective measures of Subarticle IWA-5250.

If insulation is removed for planned maintenance, repair, or other inspection at a bolted connection in a system borated for the purpose of controlling reactivity, a visual VT-2 examination shall be performed on the bolted connection prior to disassembly and, if evidence of leakage is discovered, evaluated in accordance with the corrective measures of Subarticle IWA-5250.

(Page 1 of 2)

COMPONENT IDENTIFICATION

Code Classes:

1. 2. and 3

Reference:

IWA-5250(a)(2)

Examination Category:

N/A N/A

Item Number: Description:

Alternate Rules for Corrective Measures if Leakage Occurs

at a Bolted Connection

Component Numbers:

Class 1, 2, and 3 Pressure-Retaining Bolted Connections

CODE REQUIREMENT

ASME Section XI, 1989 Edition, Subparagraph IWA-5250(a)(2) states, "if leakage occurs at a bolted connection, the bolting shall be removed, VT-3 visually examined for corrosion, and evaluated in accordance with IWA-3100."

BASIS FOR RELIEF

A leaking environment at a bolted connection may be a significant variable in the degradation mechanism of bolted connections. However, leakage is not the only variable, and in some cases may not be the degradation mechanism. Other variables to be considered are: bolting materials, leaking medium, duration of the leak, and orientation of the leak (not all the bolts may be wetted). These variables are important to consider before disassembling a bolted connection for a visual VT-3 examination. Removal of bolting at a mechanical connection may not be the most prudent decision and may cause undue hardship without a compensating increase in the level of quality or safety. Union Electric proposes an alternative to the requirements of IWA-5250(a)(2) that will provide an equivalent level of quality and safety at Class 1, 2, and 3 bolted connections.

PROPOSED ALTERNATE EXAMINATION

Leakage discovered at a bolted connection by visual VT-2 examination during system pressure test will be evaluated to determine the susceptibility of the bolting to corrosion and potential future failure. The evaluation will, as a minimum, consider the following variables:

- 1) Location of leakage
- 2) History of leakage
- 3) Bolted connection materials
- 4) Visual evidence of corrosion with the connection assembled
- 5) Corrosiveness of the process fluid
- 6) History and studies of similar bolted material in a similar environment
- 7) Other components in the vicinity that may be degraded due to the leakage

(Page 2 of 2)

When evaluation of the variables above indicates the need for further evaluation, the bolt closest to the source of leakage will be removed, receive a visual VT-3 examination, and be evaluated in accordance with IWA-3100(a). If the leakage was identified with the bolted connection in service and evaluation supports continued service, this VT-3 examination may be deferred to the next outage of sufficient duration. When the removed bolt has evidence of rejectable degradation, all remaining bolts shall be removed and subsequently receive a visual VT-3 examination and evaluated in accordance with IWA-3100(a).

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COMPONENT IDENTIFICATION

Code Class:

2

Reference:

Table IWC-2500-1

Examination Category:

C-H

Item Numbers:

C7.30, C7.40, C7.70, and C7.80

Description:

Alternate Provisions for Pressure Testing Code Class 2 Piping and

Valves at Containment Penetrations Where the Balance of the System is Outside the Scope of Section XI.

Component Numbers:

Line Number	Penetration	Description		
BB-103-HCB-1"	P-62	Pressurizer relief tank gas line		
BL-028-HCB-3"	P-25	Reactor water storage tank to RCP standpipes		
BM-053-HBB-3"	P-78	Steam Generator drain		
EC-067-HCB-6"	P-53	Fuel pool cooling return		
EC-072-HCB-6"	P-54	Refueling pool to fuel pool cooling pump suction		
EC-081-HCB-3"	P-55	Refueling pool to fuel pool skimmer pump		
EM-071-BCB-3/4"	P-92	SIS pump test line return to Reactor water storage tank		
GP-003-HBB-1"	P-51	ILRT test connection lines		
GP-005-HBB-1"	P-51	ILRT test connection lines		
GS-025-HBB-6"	P-65	Hydrogen purge subsystem to ESF filters		
GT-007-HBB-36"	V-160	Containment shutdown purge		
GT-004-HBB-36"	V-161	Containment shutdown purge		
GT-029-HBB-18"	V-161	Containment shutdown purge		
GT-034-HBB-18"	V-160	Containment shutdown purge		
GT-033-HBB-18"	V-160	Containment shutdown purge		
GT-030-HBB-18"	V-161	Containment shutdown purge		
HB-015-HCB-3"	P-26	From reactor coolant drain tank heat exchanger		
HB-025-HBB-3/4"	P-44	Reactor coolant drain tank to waste gas compressor		
:ID-015-HBB-2"	P-43	Auxiliary steam for reactor vesse head decontamination		

(Page 2 of 3)

Component Numbers (Cont.):	Line Number P	enetratio	n Description
	KA-244-HCB-1 1/2"	P-30	Compressed air
	KA-259-HCB-1 1/2"	P-30	Compressed air
	KA-051-HBB-5"	P-63	Service air
	KA-261-HBB-1"	P-63	Reactor building service air
	KA-732-HBB-1"	N/A	Personnel hatch penetration test lines
	KA-733-HBB-1"	N/A	Personnel hatch penetration test lines
	KB-001-HCB-2"	P-98	Breathing air
	KC-560-HBB-4"	P-67	Fire protection
	LF-842-HCB-6"	P-32	Containment building floor drain header
	SJ-002-BCB-1"	P-69	Nuclear sampling from pressurizer vapor space
	SJ-003-ECB-1"	P-95	Nuclear sampling from accumulator tanks
	SJ-001-BCB-1"	P-93	Loop 1 hot leg liquid sample to PASS
	SJ-029-BCB-1"	P-93	Loop 1 hot leg liquid sample to PASS
	SJ-021-BCB-1"	P-64	Loop 3 hot leg & pressurizer liquid sample to PASS
	SJ-024-BCB-1"	P-57	PASS to reactor coolant drain tank
	SJ-025-BCB-1"	P-58	PASS to reactor coolant drain tank

CODE REQUIREMENT

ASME Section XI, Table IWC-2500-1, Examination Category C-H, requires the performance of a visual VT-2 examination during a system pressure test on Code Class 2 pressure retaining components. Note 7 of this table states, "The pressure boundary includes only those portions of the system required to operate or support the safety system function up to and including the first normally closed valve (including a safety or relief valve) or valve capable of automatic closure when the safety function is required."

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BASIS FOR RELIEF

Union Electric Company requests relief from the requirement to perform a pressure test in accordance with ASME Section XI, Table IWC-2500-1, Examination Category C-H on the Code Class 2 lines listed above. Based on the discussion below, these pressure tests are considered redundant and without a compensating increase in the level of quality or safety.

The lines listed above are portions of non-safety related piping systems that penetrate the primary reactor containment. At each containment penetration, the process pipe is classified Code Class 2 and provided with isolation valves that are either locked shut during normal operation, capable of automatic closure, or capable of remote closure to support the containment safety function. The piping and valves are considered part of the primary reactor containment and upgraded to Code Class 2 at the penetration only to support the primary reactor containment safety function. Except for this, the lines listed above provide no safety function.

The primary reactor containment integrity, including all containment penetrations, is periodically verified by performing leakage tests in accordance with a 10 CFR 50, Appendix J. Each of the Code Class 2 lines listed above and their associated isolation valves are tested during an Appendix J, Type A, B or C leakage test at a pressure not less than 48.1 psig. The Type A leakage test is performed three times in a ten year interval and the Type B and C leakage tests are performed at intervals not greater than 24 months. Performance of these Appendix J leak tests will verify the integrity of the subject Code Class 2 lines at each respective penetration. The performance of ASME Section XI, Examination Category C-H pressure tests on these same lines will provide little, if any, additional verification of primary reactor containment integrity. Based on this, the performance of Examination Category C-H pressure tests on these lines is considered by Union Electric to be unnecessary and provides a negligible increase in the level of quality or safety.

PROPOSED ALTERNATE PROVISIONS

Union Electric shall perform 10 CFR 50, Appendix J leakage tests on the primary reactor containment penetration lines listed above, and on their associated valves, in accordance with Callaway Technical Specification 3/4.6.

Callaway Plant 2nd Interval Inservice Inspection Plan

SECTION 5.0 NRC CORRESPONDENCE

This Section contains all NRC correspondence related to the Second Interval Inservice Inspection Plan at the Callaway Plant. The purpose of this Section is to incorporate NRC correspondence directly into the Inspection Plan so that related requests for information, submittals, decisions and approvals are permanently documented.



UNITED STATES NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

May 23, 1995

Mr. Donald F. Schnell Senior Vice President - Nuclear Union Electric Company Post Office Box 149 St. Louis, Missouri 63166

SUBJECT: CALLAWAY - REQUEST FOR ADDITIONAL INFORMATION ON SECOND 10-YEAR

INTERVAL INSERVICE INSPECTION PROGRAM PLAN AND ASSOCIATED REQUEST

FOR RELIEF (TAC NO. M90859)

Dear Mr. Schnell:

By letter dated October 10, 1994, you submitted for NRC review the Callaway Second 10-Year Interval Inservice Inspection Program Plan. The staff, with assistance from its contractor, Idaho National Engineering Laboratory (INEL) has completed its initial review of your submittal. Additional information is required from you in order for the staff to complete its review.

We request you provide your response within 90-days to meet the staff's inservice inspection program plan review schedule. The enclosure summarizes the specific areas for which additional information is needed. In addition, to expedite the review process, please send a copy of your response to the NRC's contractor, INEL, at the following address:

Boyd W. Brown
INEL Research Center
2151 North Boulevard
P.O. Box 1625
Idaho Falls, Idaho 83415-2209

This request for information affects nine or fewer respondents and, therefore, is not subject to the Office of Management and Budget review under P.L. 96-511.

If you have any questions, please feel free to contact me on (301) 415-1396.

Sincerely,

L. Raynard Wharton, Project Manager Project Directorate III-3

Division of Reactor Projects - III/IV Office of Nuclear Reactor Regulation

Docket No. 50-483

Enclosure: Request for Additional Information

cc w/encl: See next page

DEGENVEN MAY 3 1 1995

A. C. PASSWATER

Mr. D. F. Schnell Union Electric Company

cc:

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REQUEST FOR ADDITIONAL INFORMATION

RELATED TO THE

SECOND 10-YEAR INTERVAL SERVICE INSPECTION PROGRAM PLAN

CALLAWAY NUCLEAR POWER PLANT, UNIT 1

DOCKET NO. 50-483

1. Scope/Status of Review

Throughout the service life of a water-cooled nuclear power facility, 10 CFR 50.55a(g)(4) requires that components (including supports) that are classified as American Society of Mechanical Engineers (ASME) Boiler and Pressure Vessel Code Class 1, Class 2, and Class 3 meet the requirements, except design and access provisions and preservice examination requirements, set forth in the ASME Code Section XI. "Rules for Inservice Inspection of Nuclear Power Plant Components," to the extent practical within the limitations of design, geometry, and materials of construction of the components. This section of the regulations also requires that inservice examinations of components and system pressure tests conducted during the successive 120-month inspection interval comply with the requirements in the latest edition and addenda of the Code incorporated by reference in 10 CFR 50.55a(b) 12 months prior to the start of a successive 120-month interval, subject to the limitations and modifications listed therein. The components (including supports) may meet requirements set forth in subsequent editions and addenda of the Code that are incorporated by reference in 10 CFR 50.55a(b) subject to the limitations and modifications listed therein and subject to Nuclear Regulatory Commission (NRC) approval. The licensee, Union Electric Company, has prepared the Callaway Nuclear Power Plant, Unit 1, Second 10-Year Interval Inservice Inspection Program Plan to meet the requirements of the 1989 Edition of Section XI of the ASME Code. As required by 10 CFR 50.55(g)(5), if the licensee determines that certain Code examination requirements are impractical and requests relief, the licensee shall submit information to the NRC to support that determination. The staff has reviewed the available information in the Callaway Nuclear Power Plant, Unit 1, Second 10-Year Interval Inservice Inspection Program Plan, submitted October 10, 1994.

2. Additional Information Required

Based on the above review, the staff has concluded that the following information and/or clarification is required to complete the review of the Inservice Inspection (ISI) Program.

A. Provide isometric and/or component drawings showing the Code Class 1 and 2 piping welds, components, and supports that Section XI of the ASME Code requires to be examined during the second 10-year inspection interval. Also provide a listing and schedule of examinations to be performed during the second 10-year inspection interval. The requested items will permit the staff to determine if

the extent of ISI examinations meet the Code requirements of Tables IMB-2500-1, IWC-2500-1, and IWD-2500-1.

- B. Table 3.1, ASME Section XI Inservice Inspection Summary Table, of the Callaway Nuclear Power Plant, Unit 1, Second 10-Year Interval Inservice Inspection Program Plan, contains a listing of all items subject to inservice inspection. However, it doesn't appear that any heat exchanger inspections are scheduled in Examination Categories B-B, B-D or any other examination category. Provide technical discussion as to why these examinations are not required.
- C. Request for Relief ISI-02 pertains to the volumetric examination of 16 high energy break locations in the reactor coolant pump seal water lines. It is stated that volumetric examination of these 1-1/2 inch NPS Schedule 160 piping welds yield meaningless results. Provide technical discussion describing why an ultrasonic examination of the above mentioned piping would yield meaningless results. In addition, provide technical discussion describing why other types of volumetric examination (i.e. radiography) may not be applicable. Also, explain how a surface examination would provide adequate assurance of piping integrity as inservice degradation normally occurs from the pipe ID.
- D. In Request for Relief ISI-03, ASME Code, Section XI, Table IWF-2500-1, Examination Category F-A, Item No. F1.40 requires a visual examination of 100% of the four reactor vessel support assemblies. Each of these support assemblies consist of the nozzle weld build up, shoe plate, air-cooled box, and steel support structure embedded in the primary shield wall. The majority of the air cooled box and the entire steel support structure are located beneath a steel walk plate and only the top of the air-cooled box is directly accessible. Provide the percentage of examination that is possible without removing the steel walk plate. It is stated that "an additional 20 to 30 percent (examination coverage) of the aircooled box and a very small percentage of the steel support structure would be made accessible if the steel walk plate and insulation were removed." Provide the specific percentage that can be examined with the walk plate and insulation removed. Has Union Electric considered a remote VT-3 visual examination? What percentage of examination would be possible with a remote examination?
- E. Request for Relief ISI-05 concerns the scheduling requirements of IWB-2420(a) for the Reactor Pressure Vessel nozzle-to-vessel welds, inside radius sections, and nozzle-to-safe end welds. Was 100% of these welds examined during the third period of the first 10-year inspection interval?
- F. As part of the augmented inspection program, vessel inspections are performed per Section 1.4.7, which requires examination per "Code of Federal Regulations, Part 10, [subparagraph] 50.55a(g)(6)(ii)(A), December 31, 1992, 'Augmented Examination of Reactor Vessel'."

 Provide the staff with the projected schedule and a technical discussion describing how the regulation will be implemented for

these welds at Callaway during the second interval. Include in the discussion a description of the intended approach and any specialized techniques or equipment that will be used to complete the required augmented examinations.

G. Please verify that there are no additional relief requests, other than those submitted on October 12, 1994. If additional relief requests are required, the licensee should submit them for staff review.

The schedule for timely completion of this review requires that the licensee provide, by the requested date, the above requested information and/or clarifications regarding the Union Electric Company, Callaway Nuclear Power Plant, Unit 1, Second 10-Year Interval Inservice Inspection Program Plan, Revision 0.

ADDITIONAL INFORMATION SECOND TEN-YEAR INTERVAL INSERVICE INSPECTION PLAN CALLAWAY NUCLEAR POWER PLANT **DOCKET NO. 50-483**

- References: 1) Second Ten-Year Interval Inservice Inspection Plan for the Callaway Nuclear Power Plant, Revision 1, Dated August 11, 1995.
 - 2) NRC Letter dated May 23, 1995, Callaway Request for Additional Information on Second 10-Year Interval Inservice Inspection Program Plan and Associated Request for Relief (TAC No. M90859).

The following paragraphs provide additional information for the Second Ten-Year Interval Inservice Inspection Plan (2nd Interval Inservice Inspection Plan) for the Callaway Nuclear Plant (Reference 1). The NRC staff formally requested this information from Union Electric on May 23, 1995 (Reference 2).

Each paragraph below corresponds directly to the additional information requested in the Section 2 subparagraphs of Reference 2.

A. The additional information necessary to facilitate review of the 2nd Interval Inservice Inspection Plan is provided in Attachments A, B-1, B-2, B-3, and B-4. Attachment A provides a schedule of inservice inspections planned for the second ten-year interval. This attachment lists the ASME Section XI Examination Category, Item Number, drawing reference, component identification number, schedule (i.e., Period 1, 2, or 3), and examination method for each weld, component, or support selected for inservice inspection at the Callaway Plant. Attachment A also includes a section for inservice inspection of welds at high energy break locations (no break zone welds) and Reactor Coolant Pump flywheels. Attachments B-1, B-2, B-3, and B-4 contain the Inservice Inspection Boundary Drawings, Piping Isometrics, Equipment Detail Drawings, and Component/Equipment Support Drawings, respectively. A listing and description of the attached drawings are contained in the 2nd Interval Inservice Inspection Plan, Sections 2.2, 2.3, 2.4, and 2.5.

The attachments described above provide the information necessary to verify that the extent of inservice inspections scheduled at the Callaway Plant meet the requirements of ASME Section XI, Tables IWB-2500-1, IWC-2500-1, and IWD-2500-1.

The 2nd Interval Inservice Inspection Plan, Table 3.1, correctly omits Examination Category B-B and B-D Code Class 1 heat exchangers. There are no Code Class 1 heat exchangers at the Callaway Plant.

C. Relief Request ISI-02 pertains to 16 Reactor Coolant Pump (RCP) Seal Water Injection System pipe welds at high energy break locations. These are stainless steel welds joining 1 1/2 NPS schedule 160 pipe. The wall thickness of the pipe is 0.281 inch and the outside diameter is 1.9 inches. The combination of thin wall thickness and small outside diameter make the results obtained with volumetric nondestructive examination methods (i.e., ultrasonic and radiographic) unreliable, and at best, questionable.

A surface examination technique, such as a liquid penetrant, is considered by Union Electric to be more adequate to ensure the structural integrity of the RCP seal injection welds than either a radiographic or ultrasonic examination. This position is based on the following:

- 1. A volumetric examination on the 1 1/2 NPS seal injection welds goes beyond the current requirements of ASME Section XI Examination Categories B-J, C-F-1, or C-F-2 for Code Class 1 and 2 pipe welds. Code Class 2 pipe welds less than 2 NPS are exempt entirely from nondestructive examination. Only a surface (i.e., liquid penetrant or magnetic particle) examination is required on Code Class 1 pipe welds between 4 NPS and 1 NPS. Code Class 1 Reactor Coolant System (RCS) components operate at nearly the same pressure and at higher temperatures than the RCP Seal Water Injection System at the high energy break locations. It is reasonable to assume that the liquid penetrant examination required to ensure the integrity of small bore RCS pipe welds is equally adequate to ensure the structural integrity of the high energy RCP seal injection welds.
- 2. Ultrasonic examination of the RCP seal injection welds is questionable due to transducer near field effects, beam redirection, and excessive sound attenuation in stainless steel welds. The 1/2 vee ultrasonic examination technique is questionable due to transducer near field effects. When performing calibrations on 1 1/2 NPS schedule 160 pipe the 1/2 t calibration hole cannot be resolved. Extended beam path techniques are also questionable due to beam redirection and excessive sound attenuation in the weld volume. Recent Appendix VIII performance demonstrations at the EPRI NDE Center reveal that extended beam path techniques performed from one side of the weld are unreliable for detection of the Appendix VIII implanted mechanical and thermal fatigue cracks in stainless steel welds.
- 3. Radiography is ideal for detection of construction type weld flaws, however, tight service induced (i.e., crack like) flaws are not typically detected. Radiographic examination for crack like indications is almost totally ineffective unless the indication is aligned exactly perpendicular to the radiation source. In addition, with small diameter pipe, a double wall shot technique is required, thus further reducing the probability of detecting a crack like indication.
- There is no history of inside diameter service induced degradation mechanisms,

such as Intergranular Stress Corrosion Cracking (IGSCC), on small bore RCP seal injection lines in Pressurized Water Reactor plants. A search on the Nuclear Plant Reliability Data System (NPRDS) revealed only one failure of a pressure boundary in an RCP seal injection line. In this instance the crack initiated from a construction flaw at the weld root due to mechanical vibration. A similar failure is unlikely at the Callaway Plant since the subject RCP seal injection welds are high quality full penetration butt welds installed and tested in accordance with ASME Section III.

- 5. The primary flaw initiator for small bore pipe is typically vibration induced fatigue. In the absence of an internal flaw where a crack may propagate, fatigue cracks typically initiate from the pipe outside diameter since the maximum stress due to cyclic bending is located on the outside surface membrane. Furthermore, fatigue failures normally occur at gross structural discontinuities such as socket welded joints. The subject RCP seal injection welds are all full penetration single vee butt joints.
- D. Code Case N-491, Table -2500-1 requires a 100% visual examination of the Reactor Vessel supports. As stated in Callaway Relief Request ISI-03, only the nozzle weld buildup and shoe plate are completely accessible for visual inspection. ISI-03 requests relief from visual inspection of the support components beneath the walk plate and insulation. The information below is provided to supplement ISI-03. Detail drawings of the Reactor Vessel supports are provided in Attachment C.

With the walk plate and insulation installed only the topmost portion, or approximately 30 percent, of the air cooled box is accessible for visual inspection. As stated in ISI-03, removal of the insulation and walk plate would increase the inspection coverage by 20 to 30 percent or result in visual inspection of approximately 36 to 39 percent of the air cooled box.

Union Electric has neither performed nor attempted a remote visual inspection of those parts of the Reactor Vessel supports below the walk plate. A remote inspection has not been performed or attempted for the following reasons:

- Callaway Relief Request Q granted relief from the inaccessible portions of the Reactor Vessel supports beneath the walk plate. Relief Request Q was approved by the staff for the first ten-year interval.
- 2. The only additional parts of the Reactor Vessel support accessible to remote visual inspection are the end sections of two I-beams, the corresponding I-beam end stiffener plates, and the vessel side of the air cooled box. These accessible parts are loaded in compression with no bending or tension loads. Visual inspection of these parts would provide little, if any, indication of support structural integrity.
- It is Union Electric's position that the additional man-rem exposure and cost expended to remove and re-install the insulation and walk plate to increase direct

visual inspection coverage or to perform a limited remote visual inspection of the Reactor Vessel supports with the insulation and walk plates installed provides no additional assurance of structural integrity or material condition.

Based on the above and on information provided in ISI-03, Union Electric considers the slight increase in visual inspection coverage to be impractical.

- E. Callaway Relief Request ISI-05 requests deferral to the end of the second ten-year interval examination of the Reactor Vessel nozzle-to-vessel welds, nozzle inside radius sections, and nozzle-to-safe end welds. These examinations will not be deferred over a time period exceeding the ten years, plus one year, allowed by ASME Section XI. One hundred percent of these nozzle examinations were performed during the third period of the first ten-year interval and are scheduled again for the third period of the second ten-year interval.
- F. As noted in Reference 2, the Code of Federal Regulations, Part 10, Subparagraph 50.55a(g)(6)(ii)(A) invokes new augmented Reactor Vessel shell weld examinations. These new rules are incorporated in the 2nd Interval Inservice Inspection Plan as documented in Subsection 1.4. Specifically, 100% of the Reactor Vessel sl ell welds are scheduled for volumetric examination in the third period of the second ten-year interval.
 - The Reactor Vessel shell weld volumetric examinations will be performed in accordance with ASME Section XI, Article IWB and Regulatory Guide 1.150. These examinations are specified in the 2nd Interval Inservice Inspection Plan, Table 3.1 (Examination Category B-A) and Table 3.2.
- G. There are no additional relief requests, other than those submitted with the 2nd Interval Inservice Inspection Plan on October 12, 1994. As noted in Paragraph 4.4.3 of the 2nd Interval Inservice Inspection Plan, relief requests for incomplete examinations will be submitted as examinations are performed and conditions identified which prevent less than 90% completion.

Additional Information Second Ten-Year Interval Inservice Inspection Plan Callaway Nuclear Power Plant, Unit 1 Docket No. 50-483

ATTACHMENT A

CALLAWAY 2ND INTERVAL INSERVICE INSPECTION SCHEDULE

Callaway 2nd Interval Inservice Inspection Schedule

11-Aug-95

Exam Category	ltem Number	Drawing	Component	Period	Exam Method
B-A	PRESSURE	RETAINING WE	LDS IN REACTOR VESSEL		
	B1.11	CIRCUMFERE	NTIAL SHELL WELDS		VOLU METRIC
		ISI-RBB01 Sh	t. 1		
			2-RV-101-171	3	
			2-RV-103-121	3	
	B1.12	LONGITUDINA	L SHELL WELDS		VOLUMETRIC
		ISI-RBB01 Sh	t. 1		
			2-RV-101-122A	3	
			2-RV-101-122B	3	
			2-RV-101-122C	3	
			2-RV-101-124A	3	
			2-RV-101-124B	3	
			2-RV-101-124C	3	
			2-RV-101-142A	3	
			2-RV-101-142B	3	
			2-RV-101-142C	3	
	B1.21	CIRCUMFERE	NTIAL HEAD WELDS		VOLUMETRIC
		ISI-RBB01 Sh	. 1		
			2-CH-103-101	1	
			2-RV-101-141	3	
			2-RV-102-151	3	

Exam Category	Number Number	Drawing	Component	Period	Exam Method
	B1.22	MERIDIONAL H	HEAD WELDS		VOLUMETRIC
		ISI-RBB01 Sh	t. 1		
			2-CH-101-104A	1	
			2-CH-101-104B	1	
			2-CH-101-104C	1	
			2-CH-101-104D	1	
			2-RV-101-154A	3	
			2-RV-101-154B	3	
			2-RV-101-154C	3	
			2-RV-101-154D	3	
	B1.30	SHELL TO FLA	NGE WELD		VOLUMETRIC
		ISI-RBB01 Sh	t. 1		
			2-RV-101-121	13	
	B1.40	HEAD TO FLAM	NGE WELD		SURF. & VOL.
		ISI-RBB01 Sh	. 1		
			2-CH-101-101	1	
B-B	PRESSURE VESSEL	RETAINING WE	LDS IN VESSELS OTHER THAN REACTOR		
	B2.11	PRESSURIZER	R CIRCUMFERENTIAL SHELL TO HEAD WELDS		VOLUMETRIC
		ISI-TBB03 Sht	1182		
			2-TBB03-CIRCUM-1-W	2	
				2	

Exam Category	<u>Item</u> Number	Drawing	Component	Period	Exam Method
	B2.12	PRESSURIZER	R LONGITUDINAL SHELL TO HEAD WELDS		VOLUMETRIC
		ISI-TBB03 Sh	t.1 & 2		
			2-TBB03-SEAM-1-W	2	
			2-TBB03-SEAM-4-W	2	
	B2.40	STEAM GENER	RATOR TUBE SHEET TO HEAD WELD		VOLUMETRIC
		ISI-EBB01A S	ht. 1		
			2-EBB01A-SEAM-1-W	1	
		ISI-EBB01B S	ht. 1		
			2-EBB01B-SEAM-1-W	2	
		ISI-EBB01C S	ht. 1		
			2-EBB01C-SEAM-1-W	2	
		ISI-EBB01D S	ht. 1		
			2-EBB01D-SEAM-1-W	3	
B-D	FULL PEN	ETRATION OF NO	DZZLES IN VESSELS		
	B3.100	REACTOR VES	SSEL NOZZLE INSIDE RADIUS SECTION		VOLUMETRI
		ISI-RBB01 Sh	t.1		
			2-RV-105-121-A-IR	3	
			2-RV-105-121-B-IR	3	
			2-RV-105-121-C-IR	3	
			2-RV-105-121-D-IR	3	
			2-RV-107-121-A-IR	3	
			2-RV-107-121-B-IR	3	
			2-RV-107-121-C-IR	3	
			2-RV-107-121-D-IR	3	

Exam Category	Item Number	Drawing	Component	Period	Exam Method
	B3.110	PRESSURIZER	R NOZZLE TO VESSEL WELDS		VOLUMETRIC
		ISI-TBB03 Sh			
			2-TBB03-10A-W	3	
			2-TBB03-10B-A-W	1	
			2-TBB03-10B-B-W	1	
			2-TBB03-10B-C-W	2	
			2-TBB03-10B-D-W	2	
			2-TBB03-10C-W	3	
	B3.120	PRESSURIZER	R NOZZLE INSIDE RADIUS SECTION		VOLUMETRIC
		ISI-TBB03 Sh	11		
		101 10000 011	2-TBB03-10A-IR	3	
			2-TBB03-10B-A-IR	1	
			2-TBB03-10B-B-IR	1	
			2-TBB03-10B-C-IR	2	
			2-TBB03-10B-D-IR	2	
			2-TBB03-10C-IR	3	
	B3.140	STEAM GENER SECTION	RATOR (PRIMARY SIDE) INSIDE RADIUS	3	VOLUMETRIC
		ISI-EBB01A S	ht. 1		
			2-EBB01A-1-A-IR	1	
			2-EBB01A-1-B-IR	1	
		ISI-EBB01B S	he 4		
		101-200010 3	2-EBB01B-1-A-IR		
				2	
			2-EBB01B-1-B-IR	2	
		ISI-EBB01C S			
			2-EBB01C-1-A-IR	2	

Exam Category	Item Number	Drawing	Component	Period	Exam Method
		NACES AND PROPERTY OF THE PROP	2-EBB01C-1-B-IR	2	
		ISI-EBB01D S	ht. 1		
			2-EBB01D-1-A-IR	3	
			2-EBB01D-1-B-IR	3	
	B3.90	REACTOR VES	SSEL NOZZLE TO VESSEL WELDS		VOLUMETRIC
		ISI-RBB01 Sh	t. 1		
			2-RV-105-121-A	3	
			2-RV-105-121-B	3	
			2-RV-105-121-C	3	
			2-RV-105-121-D	3	
			2-RV-107-121-A	3	
			2-RV-107-121-B	3	
			2-RV-107-121-C	3	
			2-RV-107-121-D	3	
3-E	PRESSURE	RETAINING PAR	RTIAL PENETRATION WELDS IN VESSELS		
	B4.11	PARTIAL PENE 3)	TRATION VESSEL NOZZLE WELDS (See note		VISUAL, VT-2
		ISI-RBB01 Shi	1.1		
			2-CH-107-101	1	
	B4.12	PARTIAL PENE WELDS (See no	ETRATION CONTROL ROD DRIVE NOZZLE ote 3)		VISUAL, VT-2
		ISI-RBB01 Shi	t. 1		
			2-CH-112-201	1	
			2-CH-112-202-(2-5)	1	
			2-CH-112-203-(6-9)	1	
			2-CH-112-204-(10-13)	1	
			2-CH-112-205-(14-17)	1	

Exam Category	<u>Item</u> Number	Drawing	Component	Period	Exam Method
PERSONAL PROPERTY OF THE PERSON	PS CONTROL OF THE PROPERTY OF THE CO		2-CH-112-206-(18-21)	1	
			2-CH-112-207-(22-29)	1	
			2-CH-112-208-(30-37)	1	
			2-CH-112-209-(38-41)	1	
			2-CH-112-210-(42-49)	1	
			2-CH-112-211-(50-53)	1	
			2-CH-112-212-(54-61)	1	
			2-CH-112-213-(62-65)	1	
			2-CH-112-214-(66-73)	1	
			2-CH-112-215-(74-78)	1	
	B4.13	PARTIAL PENE WELDS (See n	ETRATION INSTRUMENTATION NOZZLE ote 3)		VISUAL, VT-2
		ISI-RBB01 Sh	t.1		
			2-RV-201-151-(1-58)W	1	
	B4.20	PARTIAL PENE PENETRATION	ETRATION PRESSURIZER HEATER WELDS		VISUAL, VT-2
		ISI-TBB03 Shi	1.1		
			2-TBB03-HTRS-(1-78)W	1	
B-F	PRESSURI	E RETAINING DIS	SIMILAR METAL WELDS		
	B5.10		SSEL DISSIMILAR METAL NOZZLE TO SAFE LDS 4 NPS OR LARGER		SURF. & VOL.
		ISI-RBB01 Sh	t.1		
			2-RV-301-121-A	3	
			2-RV-301-121-B	3	
			2-RV-301-121-C	3	
			2-RV-301-121-D	3	
			2-RV-302-121-A	3	
			2-RV-302-121-B	3	
Chiaman Company and	A STATE OF THE STA			William States And A Day About States	NAME AND ADDRESS OF THE OWNER, A PROPERTY OF

Exam Category	<u>Number</u>	Drawing	Component	Period	Exam Method
O MISS PER SECURATION OF THE SECURIS		In the State of the Control of the C	2-RV-302-121-C	3	CANAMORISMA PROPERTY OF THE STATE OF THE STA
			2-RV-302-121-D	3	
	B5.40		DISSIMILAR METAL NOZZLE TO SAFE END IPS 4 OR LARGER		SURF. & VOI
		ISI-TBB03 Sht.	1		
			2-TBB03-1-W	3	
			2-TBB03-2-W	3	
			2-TBB03-3-A-W	1	
			2-TBB03-3-B-W	1	
			2-TBB03-3-C-W	2	
			2-TBB03-4-W	2	
	B5.70		ATOR DISSIMILAR METAL NOZZLE TO VELDS NPS 4 OR LARGER		SURF. & VO
		BB-01-01 Sht.			
			2-BB-01-F104	3	
			2-BB-01-F105	3	
		BB-02-01 Sht.	1		
			2-BB-01-F204	2	
			2-BB-01-F205	2	
		BB-03-01 Sht.	1		
			2-BB-01-F304	1	
			2-BB-91-F305	1	
		BB-04-01 Sht.	1		
			2-BB-01-F404	2	

Exam Category	item Number	Drawing	Component	Period	Exam Method
B-G-1	PRESSURE DIAMETER		LTING GREATER THAN 2 INCHES IN		
	B6.10	REACTOR VE	SSEL CLOSURE HEAD NUTS		VISUAL, VT-
		ISI-RBB01 Sh	ıt. 1		
			2-CH-NUT-01	1	
			2-CH-NUT-02-R1	3	
			2-CH-NUT-03	1	
			2-CH-NUT-04-R1	3	
			2-CH-NUT-05-R1	3	
			2-CH-NUT-06	1	
			2-CH-NUT-07-R1	3	
			2-CH-NUT-08	3	
			2-CH-NUT-09-R1	3	
			2-CH-NUT-10	1	
			2-CH-NUT-11	3	
			2-CH-NUT-12	3	
			2-CH-NUT-13	1	
			2-CH-NUT-14	3	
			2-CH-NUT-15	3	
			2-CH-NUT-16	1	
			2-CH-NUT-17	3	
			2-CH-NUT-18	3	
			2-CH-NUT-19	1	
			2-CH-NUT-20	3	
			2-CH-NUT-21	3	
			2-CH-NUT-22	1	
			2-CH-NUT-23	3	
			2-CH-NUT-24-R1	1	
			2-CH-NUT-25	1	
			2-CH-NUT-26	3	

Exam Category	Item Number	Drawing	Component	Period	Exam Method
PERSONAL PROPERTY OF THE PERSON OF THE PERSO	and the emission of the purpose of the second of the secon	ALCOHOLOGIC PROPERTY.	2-CH-NUT-27	3	
			2-CH-NUT-28	1	
			2-CH-NUT-29	3	
			2-CH-NUT-30	3	
			2-CH-NUT-31	1	
			2-CH-NUT-32	3	
			2-CH-NUT-33	3	
			2-CH-NUT-34	1	
			2-CH-NUT-35-R1	3	
			2-CH-NUT-36	3	
			2-CH-NUT-37	1	
			2-CH-NUT-38	3	
			2-CH-NUT-39	3	
			2-CH-NUT-40	1	
			2-CH-NUT-41	3	
			2-CH-NUT-42-R1	1	
			2-CH-NUT-43	1	
			2-CH-NUT-44	3	
			2-CH-NUT-45	3	
			2-CH-NUT-46	1	
			2-CH-NUT-47	3	
			2-CH-NUT-48	3	
			2-CH-NUT-49	1	
			2-CH-NUT-50	3	
			2-CH-NUT-51	3	
			2-CH-NUT-52	1	
			2-CH-NUT-53-R1	3	
			2-CH-NUT-54	3	

Exam Category	<u>Item</u> Number	Drawing	Component	Period	Exam Method
	B6.180	BOLTS & STU	OS IN PUMPS		VOLUMETRI
		ISI-PUMPS SI	nt. 1		
			2-PBB01A-BOLT (1-24)	3	
	B6.190	FLANGE SURF PUMPS	ACE WHEN CONNECTION DISASSEMBLED IN		VISUAL VT-
		ICI DUMDE CI			
		ISI-PUMPS SI	2-PBB01A-FLANGE	1	
	B6.200	NUTS, BUSHIN	IGS & WASHERS IN PUMPS		VISUAL VT-
		ISI-PUMPS SI	nt. 1		
			2-PBB01A-WASHER (1-24)	3	
	B6.30	REACTOR VES	SSEL CLOSURE STUDS, WHEN REMOVED		SURF. & VOL
		ISI-RBB01 Sh	t. 1		
			2-CH-STUD-01	1	
			2-CH-STUD-02-R1	1	
			2-CH-STUD-03	1	
			2-CH-STUD-04-R1	1	
			2-CH-STUD-05-R1	1	
			2-CH-STUD-06	1	
			2-CH-STUD-07-R1	1	
			2-CH-STUD-08	3	
			2-CH-STUD-09-R1	1	
			2-CH-STUD-10	1	
			2-CH-STUD-11	3	
			: CH-STUD-12	3	
			2-CH-STUD-13	1	

Exam Category	Item Number	Drawing	Component	Period	Exam Method
			2-CH-STUD-14	3	
			2-CH-STUD-15-R1	1	
			2-CH-STUD-16	1	
			2-CH-STUD-17	3	
			2-CH-STUD-18	3	
			2-CH-STUD-19	1	
			2-CH-STUD-20	3	
			2-CH-STUD-21	3	
			2-CH-STUD-22	-1	
			2-CH-STUD-23	3	
			2-CH-STUD-24-R1	1	
			2-CH-STUD-25	1	
			2-CH-STUD-26	3	
			2-CH-STUD-27	3	
			2-CH-STUD-28	1	
			2-CH-STUD-29	3	
			2-CH-STUD-30	3	
			2-CH-STUD-31	1	
			2-CH-STUD-32	3	
			2-CH-STUD-33	3	
			2-CH-STUD-34	1	
			2-CH-STUD-35-R1	1	
			2-CH-STUD-36	3	
			2-CH-STUD-37	1	
			2-CH-STUD-38	3	
			2-CH-STUD-39	3	
			2-CH-STUD-40	1	
			2-CH-STUD-41	3	
			2-CH-STUD-42-R1	3	
			2-CH-STUD-43	1	
			2-CH-STUD-44	3	
			2-CH-STUD-45	3	

Exam Category	Item Number	Drawing	Component	Period	Exam Method
		MANAGEMENTE MENTE PER PER MENTE AN	2-CH-3TUD-46	1	
			2-CH-STUD-47	3	
			2-CH-STUD-48	3	
			2-CH-STUD-49	1	
			2-CH-STUD-50	3	
			2-CH-STUD-51	3	
			2-CH-STUD-52	1	
			2-CH-STUD-63-R1	3	
			2-CH-STUD-54	3	
	B6.40	THREADS IN V	ESSEL FLANGE		VOLUMETRI
		ISI-RBB01 Sh	t. 1		
			2-RV-LIG-01	3	
			2-RV-LIG-02	3	
			2-RV-LIG-03	3	
			2-RV-LIG-04	3	
			2-RV-LIG-05	3	
			2-RV-LIG-06	3	
			2-RV-LIG-07	3	
			2-RV-LIG-08	3	
			2-RV-LIG-09	3	
			2-RV-LIG-10	3	
			2-RV-LIG-11	3	
			2-RV-LIG-12	3	
			2-RV-LIG-13	3	
			2-RV-LIG-14	3	
			2-RV-LIG-15	3	
			2-RV-LIG-16	3	
			2-RV-LIG-17	3	
			2-RV-LIG-18	3	
			2-RV-LIG-19	3	

Exam Category	Item Number	Drawing	Component	Period	Exam Method
	An area maken and a second of the		2-RV-LIG-20	3	
			2-RV-LIG-21	3	
			2-RV-LIG-22	3	
			2-RV-LIG-23	3	
			2-RV-LIG-24	3	
			2-RV-LIG-25	3	
			2-RV-LIG-26	3	
			2-RV-LIG-27	3	
			2-RV-LIG-28	3	
			2-RV-LIG-29	3	
			2-RV-LIG-30	3	
			2-RV-LIG-31	3	
			2-RV-LIG-32	3	
			2-RV-LIG-33	3	
			2-RV-LIG-34	3	
			2-RV-LIG-35	3	
			2-RV-LIG-36	3	
			2-RV-LIG-37	3	
			2-RV-LIG-38	3	
			2-RV-LIG-39	3	
			2-RV-LIG-40	3	
			2-RV-LIG-41	3	
			2-RV-LIG-42	3	
			2-RV-LIG-43	3	
			2-RV-LIG-44	3	
			2-RV-LIG-45	3	
			2-RV-LIG-46	3	
			2-RV-LIG-47	3	
			2-RV-LIG-48	3	
			2-RV-LIG-49	3	
			2-RV-LIG-50	3	
			2-RV-LIG-51	3	

Exam Category	Item Number	Drawing	Component	Period	Exam Method
ANN AND AND AND AND AND AND AND AND AND	MATERIA DE DOMINA A INCLÚSICA CALCA		2-RV-LIG-52	3	AND THE PERSON NAMED IN COLUMN TO PERSON.
			2-RV-LIG-53	3	
			2-RV-LIG-54	3	
	B6.50	REACTOR VE	SSEL CLOSURE WASHERS, BUSHINGS		VISUAL, VT-
		ISI-RBB01 Sh	nt. 1		
			2-CH-WASH-01	1	
			2-CH-WASH-02	1	
			2-CH-WASH-03	1	
			2-CH-WASH-04	1	
			2-CH-WASH-05	1	
			2-CH-WASH-06	1	
			2-CH-WASH-07	1	
			2-CH-WASH-08	1	
			2-CH-WASH-09	1	
			2-CH-WASH-10	1	
			2-CH-WASH-11	1	
			2-CH-WASH-12	1	
			2-CH-WASH-13	1	
			2-CH-WASH-14	1	
			2-CH-WASH-15	1	
			2-CH-WASH-16	1	
			2-CH-WASH-17	1	
			2-CH-WASH-18	1	
			2-CH-WASH-19	1	
			2-CH-WASH-20	1	
			2-CH-WASH-21	1	
			2-CH-WASH-22	1	
			2-CH-WASH-23	1	
			2-CH-WASH-24	1	
			2-CH-WASH-25	1	

Exam Category	Item Number	Drawing	Component	Period Ex Met	hod
DENTIL SERVICE PER CONTROL PER	our recommendation of the state	1965-1760-1868-1868-1874-1875-1875-1875-1875-1875-1875-1875-1875	2-CH-WASH-26	1	RANGE STORY
			2-CH-WASH-27	1	
			2-CH-WASH-28	1	
			2-CH-WASH-29	1	
			2-CH-WASH-30	1	
			2-CH-WASH-31	1	
			2-CH-WASH-32	1	
			2-CH-WASH-33	1	
			2-CH-WASH-34	1	
			2-CH-WASH-35	1	
			2-CH-WASH-36	1	
			2-CH-WASH-37	1	
			2-CH-WASH-38	1	
			2-CH-WASH-39	1	
			2-CH-WASH-40	1	
			2-CH-WASH-41	1	
			2-CH-WASH-42	1	
			2-CH-WASH-43	1	
			2-CH-WASH-44	1	
			2-CH-WASH-45	1	
			2-CH-WASH-46	1	
			2-CH-WASH-47	1	
			2-CH-WASH-48	. 1	
			2-CH-WASH-49	1	
			2-CH-WASH-50	1	
			2-CH-WASH-51	1	
			2-CH-WASH-52	1	
			2-CH-WASH-53	1	
			2-CH-WASH-54	1	

Exam Category	Item Number	Drawing	Component	Period	Exam Method
B-G-2	PRESSURE	RETAINING BO	LTING, 2 INCHES AND LESS IN DIAMETER		
	B7.20	BOLTS, STUD	S, & NUTS IN PRESSURIZER		VISUAL, VT-1
		ISI-TBB03 Sh	1.1		
			2-TBB03-10D-(1-16)-b	1	
	B7.30	BOLTS, STUD	S, & NUTS IN STEAM GENERATOR		VISUAL, VT-1
		ISI-EBB01A S	ht. 1		
			2-EBB01A-2-A-(1-16)-b	1	
			2-EBB01A-2-B-(1-16)-b	1	
		ISI-EBB01B S	ht. 1		
			2-EBB01B-2-A-(1-16)-b	2	
			2-EBB01B-2-B-(1-16)-b	2	
		ISI-EBB01C S	ht. 1		
			2-EBB01C-2-A-(1-16)-b	2	
			2-EBB01C-2-B-(1-16)-b	2	
		ISI-EBB01D S	int. 1		
			2-EBB01D-2-A-(1-16)-b	3	
			2-EBB01D-2-B-(1-16)-b	3	
	B7.50	BOLTS, STUDS	S, & NUTS IN PIPING		VISUAL, VT-1
		BB-01-08 Sht.	.1		
			2-BB-08-FL1-b	1	
		BB-02-02 Sht.	1		
			2-BB-02-8010A-FL-b	2	
			2-BB-02-8010B-FL-b	3	
			2-BB-02-8010C-FL-b	1	

Exam Category	Item Number	Drawing	Component	Period	Exam Method
		BB-02-11 Sht. 1			
			2-BB-11-FL1-b	3	
		BB-03-09 Sht.			
			2-BB-09-FL1-b	3	
		BB-04-07 Sht.	1		
			2-BB-07-FL1-b	2	
		EM-06-03 Sht.	1		
			2-EM-03-FE-924-FL-b	3	
			2-EM-03-FE-925-FL-b	2	
			2-EM-03-FE-926-FL-b	2	
			2-EM-03-FE-927-FL-b	1	
	B7.70	BOLTS, STUDS	& NUTS IN VALVES		VISUAL, VT-
			2-BB-02-8010A-b 2-BB-02-8010B-b 2-BB-02-8010C-b	2 3 1	
		EJ-01-04 Sht. 1			
			2-EJ-04-BB-PV8702A-b	3	
			2-EJ-04-HV-8701A-b	3	
		EJ-02-04 Sht. 1			
			2-EJ-04-BB-PV8702B-b	1	
			2-EJ-04-HV-8701B-b	1	
		EJ-02-04 Sht. 3			
			2-EM-03-EJ-8841A-b	3	
			2-EM-03-EJ-8841B-b	1	
		EM-03-05 Sht.	1		
			2-EM-05-BB-8949A-b	1	
			2-EM-05-BB-8949D-b	3	
			2-EM-05-BB-8949A-b		

Exam Category	item Number	Drawing	Component	Period	Exam Method
		EM-04-03 Sht.	1		
			2-EM-03-BB-8949B-b	2	
			2-EM-03-BB-8949C-b	2	
		EP-01-01 Sht.			
			2-EP-01-8818A-b	1	
			2-EP-01-8956A-b	1	
			2-EP-01-BB-8948A-b	1	
		EP-02-02 Sht.			
			2-EP-02-8818B-b	2	
			2-EP-02-8956B-b	2	
			2-EP-02-BB-8948B-b	2	
		EP-03-02 Sht. 1			
			2-EP-02-8818C-b	2	
			2-EP-02-8956C-b	2	
			2-EP-02-BB-8948C-b	2	
		EP-04-01 Sht. 1			
			2-EP-01-8818D-b	3	
			2-EP-01-8956D-b	3	
			2-EP-01-BB-8948D-b	3	
1-1	PRESSURE	RETAINING WEL	DS IN PIPING		
	B9.11	CIRCUMFEREN	TIAL WELDS IN PIPING NPS 4 OR LARGER		SURF. & VOI
		BB-01-01 Sht. 1			
			2-BB-01-F101	3	
			2-BB-01-F107	3	
			2-BB-01-S102-2	3	
		BB-01-02 Sht. 1			
			2-BB-02-F008	1	
			2-BB-02-F009	1	

Exam Category	Item Number	Drawing	Component	Period	<u>Method</u>
			2-BB-02-FW2	1	PARTIE COME ESSA SANCIA DA PARTIE DA PAR
			2-BB-02-S010-B	1	
		BB-01-04 Sht.			
			2-BB-04-F001	3	
			2-BB-04-F015	3	
		BB-01-04 Sht.	2		
			2-BB-04-F008	3	
			2-BB-04-F012	3	
			2-BB-04-F022	3	
		BB-02-01 Sht.	1		
			2-BB-01-F201	2	
			2-BB-01-F207	2	
		BB-02-02 Sht.	1		
			2-BB-02-F001	2	
			2-BB-02-F005	2	
			2-BB-02-F006	2	
			2-BB-02-S001-J	2	
			2-BB-02-S006-E	2	
			2-BB-02-S007-D	2	
			2-BB-02-S007-E	2	
			2-BB-02-S007-J	2	
		BB-03-01 Sht.			
			2-BB-01-F301	1	
			2-BB-01-F307	1	
			2-BB-01-S305-4	1	
		BB-04-01 Sht.			
			2-BB-01-F401	2	
			2-BB-01-F407	2	
			2-BB-01-S402-2	2	
		BB-05-01 Sht.		-	
			2-BB-01-F001	1	

Exam Category	<u>Number</u>	Drawing	Component	Period	Method
	Maria da Cara de Assas de Cara		2-BB-01-F002	1	AS WHEN THE SAME THE
			2-BB-01-F004	1	
		EJ-01-04 Sht.			
			2-EJ-04-F030	3	
			2-EJ-04-F031	3	
			2-EJ-04-F034	2	
		EJ-02-04 Sht.	1		
			2-EJ-04-F023	1	
			2-EJ-04-F024	3	
			2-EJ-04-F026	2	
			2-EJ-04-S014-D	1	
			2-EJ-04-S015-C	1	
		EM-03-05 Sht.	1		
			2-EM-05-F007	1	
			2-EM-05-S005-B	1	
		EM-04-03 Sht.	1		
			2-EJ-04-F017	2	
			2-EM-03-F009	3	
			2-EM-03-F010	2	
			2-EM-03-F016	1	
			2-EM-03-S011-F	2	
			2-EM-03-S015-B	2	
		EP-01-01 Sht.	1		
			2-EP-01-F007	1	
			2-EP-01-F008	3	
			2-EP-01-F010	1	
			2-EP-01-F012	1	
			2-EP-01-S003-E	1	
			2-EP-01-S003-F	1	
		EP-02-02 Sht.	1		
			2-EP-02-F007	1	

Exam Category	<u>Item</u> Number	Drawing	Component	Period	Exam Method
CALCULAR SOFTER HUMP FOR HEAVY CO.	STOCKET STOCKE BY TO BE ARREST MINE OF		2-EP-02-F008	2	
			2-EP-02-F009	2	
			2-EP-02-F010	2	
			2-EP-02-S003-F	2	
			2-EP-02-S003-G	2	
		EP-03-02 Sht.	1		
			2-EP-02-F018	2	
			2-EP-02-F019	1	
			2-EP-02-F020	3	
			2-EP-02-F021	3	
			2-EP-02-S008-D	3	
			2-EP-02-S008-F	3	
		EP-04-01 Sht.	1		
			2-EP-01-F019	1	
			2-EP-01-F020	2	
			2-EP-01-F022	2	
			2-EP-01-S013-B	3	
			2-EP-01-S013-C	3	
			2-EP-01-S013-K	2	
		ISI-RBB01 Sht	. 4		
			2-BB-01-F102	3	
			2-BB-01-F103	3	
			2-BB-01-F202	3	
			2-BB-01-F203	3	
			2-BB-01-F302	3	
			2-BB-01-F303	3	
			2-BB-01-F402	3	
			2-BB-01-F403	3	

Exam Category	Item Number	Drawing	Component	Period	Exam Method
	B9.21	CIRCUMFERENT	TIAL WELDS IN PIPING LESS THAN NPS 4		SURFACE
		BB-01-01 Sht. 1			
			2-BB-01-S101-13	3	
		BB-01-02 Sht. 1			
			2-BB-02-F012	1	
			2-BB-02-F019	1	
			2-BB-02-FW25	1	
			2-BB-02-FW3	1	
			2-BB-02-S010-F	1	
			2-BB-02-S010-G	1	
			2-BB-02-S010-H	1	
			2-BB-02-S014-B	1	
			2-BB-02-S014-C	1	
		BB-01-08 Sht. 1			
			2-BB-08-FW040	1	
			2-BB-08-FW041-A-R-1	1	
			2-BB-08-FW053	1	
			2-BB-08-V121-2	1	
		BB-02-01 Sht. 1			
			2-BB-01-S201-15	2	
			2-BB-06-F001	2	
			2-BB-06-F006	2	
		BB-02-11 Sht. 1			
			2-BB-11-FW017	3	
			2-BB-11-V151-2	3	
		BB-03-01 Sht. 1			
			2-BB-01-S301-9	1	
		BB-03-09 Sht. 1			
			2-BB-09-V181-1	3	

Exam Category	<u>Item</u> Number	Drawing	Component	Period	Exam Method
	and the state of t		2-BB-09-V181-2	3	AND MESON AND DESCRIPTION
		BB-04-01 Sht. 1			
			2-BB-01-S401-10	2	
		BB-04-07 Sht. 1			
			2-BB-07-FW001	2	
			2-BB-07-FW030	2	
		BB-06-01 Sht. 1			
			2-HB-24-FW079-A	3	
			2-HB-24-FW080	3	
		BG-01-21 Sht. 1			
			2-BG-21-F011	2	
			2-BG-21-F012	2	
			2-BG-21-F013	2	
			2-BG-21-F014	3	
			2-BG-21-F024	2	
			2-BG-21-F025	2	
			2-BG-21-F026	2	
			2-BG-21-F027	2	
		BG-02-22 Sht. 1			
			2-HB-24-FW051	3	
			2-HB-24-FW052	3	
		BG-03-23 Sht. 1			
			2-BG-23-FW145	1	
			2-BG-23-FW149	1	
			2-BG-23-FW150	1	
		BG-05-24 Sht. 1			
			2-BG-24-FW060	3	
			2-BG-24-FW067	3	
			2-BG-24-FW068	3	
		EM-03-05 Sht. 1			
			2-EM-05-FW005	1	

Exam Category	<u>Item</u> Number	Drawing	Component	Period	Exam Method
			2-EM-05-FW016	1	
			2-EM-05-FW025	1	
			2-EM-05-FW076	1	
			2-EM-05-FW082	1	
			2-EM-05-FW083	1	
			2-EM-05-FW084	1	
			2-EM-05-FW087	2	
			2-EM-05-FW098	2	
			2-EM-05-FW108	2	
			2-EM-05-FW109	2	
		EM-06-03 Sht.	1		
			2-EM-03-BBV001-1	3	
			2-EM-03-BBV022-1	2	
			2-EM-03-BBV040-1	1	
			2-EM-03-BBV059-1	2	
			2-EM-03-F002	3	
			2-EM-03-FW113	2	
			2-EM-03-FW2	3	
			2-EM-03-FW227	2	
			2-EM-03-FW235	3	
			2-EM-03-FW242	3	
			2-EM-03-FW259	1	
			2-EM-03-FW269	2	
			2-EM-03-FW270	3	
			2-EM-03-FW272	3	
			2-EM-03-FW274	3	
		EP-01-01 Sht.			
			2-EP-01-FW049	3	
			2-EP-01-FW060	3	
		EP-02-02 Sht.	1		
			2-EP-02-FW008	2	
			2-EP-02-FW009	2	

Exam Category	<u>Item</u> Number	Drawing	Component	Period	Exam Method
A APPARENT AND PARENT OF THE P	CLASSICAL STATE OF STREET	A STATE OF THE STA	2-EP-02-V020-2	2	ministra erabilitaria titalvi konsultaria erabilita
		EP-03-02 Sht.	1		
			2-EP-02-V030-2	3	
		EP-04-01 Sht.	1		
			2-EP-01-FW046	3	
			2-EP-01-FW047	3	
			2-EP-01-V040-2	3	
	B9.31	BRANCH PIPE	CONNECTION WELDS NPS 4 OR LARGER		SURF. & VOL
		BB-01-01 Sht.	1		
			2-BB-01-S101-7	3	
		BB-03-01 Sht.	1		
			2-BB-01-S302-3	1	
		BB-04-01 Sht.	1		
			2-BB-01-S402-3	2	
		EJ-01-04 Sht.	1		
			2-FJ-04-S018-F	3	
	B9.32	BRANCH PIPE	CONNECTION WELDS LESS THAN NPS 4		SURFACE
		BB-01-04 Sht.	1		
			2-BB-04-S002-L	3	
		BG-02-22 Sht.	1		
		00 02 12 011.	2-BG-22-S001-F	3	
		EM-04-03 Sht.			
		EM-04-03 SIIC.	2-EM-03-S013-B	2	
				-	
		EM-06-03 Sht.			
			2-EM-03-S003-F	2	
			2-EM-03-S003-G	3	
			2-EM-03-S003-N	3	

Exam Category	item Number	Drawing	Component	Period	Exam Method
		EP-01-01 Sht. 1			
			2-EP-01-S005-F	1	
		EP-02-02 Sht. 1			
			2-EP-02-S005-B	2	
		EP-03-02 Sht. 1			
			2-EP-02-S010-B	3	
		EP-04-01 Sht. 1			
			2-EP-01-S011-J	1	
	B9.40	SOCKET WELD	S		SURFACE
		BB-02-01 Sht.			
			2-BB-06-FW120	2	
		EM-03-05 Sht.			
			2-EM-05-FW10	1	
		EP-04-01 Sht. 1			
			2-EP-01-FW045	3	
B-K	VALVES	ATTACHMENTS F	FOR CLASS 1 VESSELS, PIPING, PUMPS &		
	B10.10	INTEGRALLY W	ELDED ATTACHMENTS TO VESSELS		SURFACE
		ISI-TBB03 Sht.	,		
			2-TBB03-LUG-C-W	2	
B-L-2	PUMP CASINGS				
	B12.20	PUMP CASING	INTERNAL SURFACF (See note 1)		VISUAL, VT-
		ISI-PUMPS Sht	.1		
			2-PBB01A-SURF	1	

Exam Category	Item Number	Drawing	Component	Period	Exam Method
			2-PBB01B-SURF	1	and the second s
			2-PBB01C-SURF	1	
			2-PBB01D-SURF	1	
B-M-2	VALVE BO	DIES			
	B12.50-1	VALVE BODIES	S. EXCEEDING NPS 4 - GROUP 1 (See note 2)		VISUAL, VT-
		BB-02-02 Sht.			
			2-BB-02-8010A-SURF	1	
			2-BB-02-8010B-SURF	1	
			2-BB-02-8010C-SURF	1	
	B12.50-2	VALVE BODIES	S, EXCEEDING NPS 4 - GROUP 2 (See note 2)		VISUAL, VT-
		EJ-01-04 Sht.	1		
			2-EJ-04-BB-PV-8702A-SURF	1	
		EJ-02-04 Sht.	1		
			2-EJ-04-BB-PV-8702B-SURF	1	
	B12.50-3	VALVE BODIES	S, EXCEEDING NPS 4 - GROUP 3 (See note 2)		VISUAL, VT-3
		EJ-01-04 Sht.	1		
			2-EJ-04-HV-8701A-SURF	1	
		EJ-02-04 Sht.	1		
			2-EJ-04-HV-8701B-SURF	1	
	B12.50-4	VALVE BODIES	S. EXCEEDING NPS 4 - GROUP 4 (See note 2)		VISUAL, VT-
		EM-03-05 Sht.	1		
			2-EM-05-BB-8949A-SURF	1	

Exam Category	Item Number	Drawing	Component	Period	Exam Method
			2-EM-05-BB-8949D-SURF	1	
		EM-04-03 Sht.	1		
			2-EM-03-BB-8949B-SURF	1	
			2-EM-03-BB-8949C-SURF	1	
	B12.50-5	VALVE BODIES	, EXCEEDING NPS 4 - GROUP 5 (See note 2)		VISUAL, VT-3
		EP-01-01 Sht.	1		
			2-EP-01-8956A-SURF	1	
		EP-02-02 Sht.	1		
			2-EP-02-8956B-SURF	1	
		EP-03-02 Sht.	1		
			2-EP-02-8956C-SURF	1	
		EP-04-01 Sht.	1		
			2-EP-01-8956D-SURF	1	
	B12.50-6	VALVE BODIES	EXCEEDING NPS 4 - GROUP 6 (See note 2)		VISUAL, VT-3
		EJ-02-04 Sht. :	3		
			2-EM-03-EJ-8841A-SURF	1	
			2-EM-03-EJ-8841B-SURF	1	
		EP-01-01 Sht.			
			2-EP-01-8818A-SURF	1	
		EP-02-02 Sht.			
			2-EP-02-8818B-SURF	1	
		EP-03-02 Sht.			
			2-EP-02-8818C-SURF	1	
		EP-04-01 Sht.			
			2-EP-01-8818D-SURF	1	

Exam Category	Item Number	Drawing	Component	Period	Exam Method
	B12.50-7	VALVE BODIES	S, EXCEEDING NPS 4 - GROUP 7 (See note 2)		VISUAL, VT-3
		EP-01-01 Sht.	1		
			2-EP-01-BB8948A-SURF	1	
		EP-02-02 Sht.	1		
			2-EP-02-BB8948B-SURF	1	
		EP-03-02 Sht.	1		
			2-EP-02-BB8948C-SURF	1	
		EP-04-01 Sht.	1		
			2-EP-01-BB8948D-SURF	1	
B-N-1	INTERIOR OF REACTOR VESSEL				
	B13.10	VESSEL INTER	NOR		VISUAL, VT-3
		1447E54			
			2-RV-INTERIOR	123	
B-N-2		LY WELDED COR	E SUPPORT STRUCTURES & INTERIOR DR VESSEL		
	813.60	INTERIOR ATTA	ACHMENTS BEYOND BELTLINE REGION IN SEL		VISUAL, VT-3
		ISI-RBB01 Sht	4		
			2-RV-CSL-104-142-A	3	
			2-RV-CSL-104-142-B	3	
			2-RV-CSL-104-142-C	3	
			2-RV-CSL-104-142-D	3	
			2-RV-CSL-104-142-E	3	
			2-RV-CSL-104-142-F	3	

Exam Category	Item Number	Drawing	Component	Period	Exam Method
B-N-3	REMOVAB	LE CORE SUPPO	ORT STRUCTURES		Section Expense Community is may recent
	B13.70	CORE SUPPOR	RT STRUCTURE IN REACTOR VESSEL		VISUAL, VT-3
		1447E54			
			2-RV-CORE STRUCT	3	
B-0	PRESSURE	RETAINING WE	LDS IN CONTROL ROD HOUSINGS		
	B14.10	WELDS IN CRE	HOUSING		SURF. OR VOL
		ISI-RBB01 Sh			
		101-110001 011	2-CH-214-112-B	3	
			2-CH-214-112-F	3	
			2-CH-215-112-A	3	
			2-CH-215-112-D	3	
C-A	PRESSURE	RETAINING WE	LDS IN PRESSURE VESSELS		
	C1.10	SHELL CIRCUM	MFERENTIAL WELDS		VOLUMETRIC
		ISI-EBB01B S	ht. 1		
			2-EBB01B-SEAM-3-W	1	
			2-EBB01B-SEAM-5-W	2	
			2-EBB01B-SEAM-6-W	1	
		ISI-EEJ01A SH	nt. 1		
			2-EEJ01A-SEAM-1-W	3	
			2-EEJ01A-SEAM-2-W	3	

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Exam Category	Item Number	Drawing	Component	Period	Exam Method
	C1.20	HEAD CIRCUN	FERENT AL WELDS		VOLUMETRIC
		ISI-EBB01B S	ht. 1		
			2-EBB01B-SEAM-8-W	3	
	C1.30	TUBESHEET T	O SHELL WELDS		VOLUMETRIC
		ISI-EBB01B S	ht. 1		
			2-EBB01B-SEAM-2-W	1	
С-В	PRESSURE	RETAINING NO	ZZLE WELDS IN VESSELS		
	C2.21		HELL (OR HEAD) WELD WITHOUT PLATE IN VESSEL > 1/2 INCH NOMINAL		SURF. & VOL.
		ISI-EBB01B S	ht. 1		
			2-EBB01B-11-W	1	
			2-EBB01B-16-W	3	
	C2.22	NOZZLE INSID	E RADIUS SECTION		VOLUMETRIC
		ISI-EBB01B S	ht. 1		
			2-EBB01B-11-IR	1	
			2-EBB01B-16-IR	3	
	C2.33	VESSEL IS INA	HELL (OR HEAD) WELDS WHEN INSIDE OF CCESSIBLE, FOR VESSELS > 1/2 INCH CKNESS WITH REINFORCING PLATES		SURFACE
		ISI-EEJ01A SI	nt. 1		
			2-EEJ01A-NA-1	3	
			2-EEJ01A-NB-1	3	

Exam Category	Number Number	Drawing	Component	Period	Exam Method
c-c	INTEGRAL AND VALV	ATTACHMENTS ES			
	C3.10	INTEGRALLY VESSELS	WELDED ATTACHMENTS TO PRESSURE		SURFACE
		ISI-EEJ01A S	ht. 1		
			2-EEJ01A-SKIRT-W	3	
	C3.20	INTEGRALLY	WELDED ATTACHMENTS TO PIPING		SURFACE
		AE-04-05 Sht.			
			2-AE-05-R028	3	
		EJ-02-02 Sht.	2		
			2-EJ-02-C022	2	
		EJ-02-02 Sht.	3		
			2-EJ-02-C018	2	
			2-EJ-03-A003	1	
		EJ-02-02 Sht.	4		
			2-EJ-02-C020	2	
			2-EJ-02-R032	1	
		EJ-02-04 Sht.	•		
			2-EJ-02-R016	2	
		EM-05-01 Sht.	.2		
			2-EM-01-A003	3	
			2-EM-01-A004	3	
			2-EM-01-H010	3	
	C3.30	INTEGRALLY	NELDED ATTACHMENTS TO PUMPS		SURFACE
		ISI-PUMPS SH	nt. 1		
			2-PBG05A-SUP-4	2	

Exam Category	Item Number	Drawing	Component	Period	Exam Method
			2-PEJ01A-SUP-1	1	
			2-PEN01A-SUP-1	3	
C-D	PRESSURE	and the second of the second o	LTING GREATER THAN 2 INCHES IN		
	C4.40	BOLTS AND ST	TUDS IN VALVES		VOLUMETRIC
		AB-01-01 Sht.	•		
			2-AB-01-HV-14-b	3	
C-F-1	PRESSURE HIGH ALLO		LDS IN AUSTENITIC STAINLESS STEEL OR		
	C5.11	STEEL OR HIG	NTIAL WELDS IN AUSTENITIC STAINLESS TH ALLOY PIPING GREATER THAN OR EQUAL OMINAL WALL THICKNESS FOR PIPING IN NPS 4		SURF. & VOL
		AE-02-04 Sht.	1		
			2-AE-04-C525735-FW02	3	
		BG-06-02 Sht	1		
			2-BG-02-S011-A	3	
			2-BG-02-S015-B	3	
			2-BG-02-S036-A	3	
			2-BG-02-S037-A	3	
		EJ-01-01 Sht.	1		
			2-EJ-01-FW75	3	
			2-EJ-01-S004-F	3	
		EJ-01-01 Sht.	2		
			2-E.I-01-S010-A	1	
			2-EJ-01-S014-A	1	
			2-EJ-01-S021-J	1	
		EJ-01-01 Sht.	3		
			2-EJ-01-F027	1	
			2-EJ-01-S015-H	1	
			2-EJ-01-S017-B	1	

	Number Number	Drawing	Component	Period	Exam Method
Maria Cara Cara Cara Cara Cara Cara Cara		MANAGEMENT DE ANGLE ET AUTORIT DE SELECTION DE	2-EJ-01-S018-C	1	
			2-EJ-01-S023-E	1	
		EJ-01-01 Sht.	4		
			2-EJ-01-F045	1	
			2-EJ-01-S035-B	1	
			2-EJ-04-S026-C	1	
			2-EJ-04-S026-F	1	
		EJ-01-01 Sht.	5		
			2-EJ-01-F034	3	
			2-EJ-01-S025-B	3	
		EJ-02-02 Sht.			
			2-EJ-02-F064	2	
			2-EJ-02-S008-B	2	
		EJ-02-02 Sht. 2			
			2-EJ-02-F014	2	
			2-EJ-02-F021	2	
		EJ-02-02 Sht. :			
			2-EJ-02-S016-M	2	
			2-EJ-02-S017-C	2	
			2-EJ-02-S020-K	2	
			2-EJ-02-S022-B	2	
			2-EJ-03-S003-F	2	
		EJ-02-02 Sht. 4			
			2-EJ-02-F027	2	
			2-EJ-02-F029	2	
			2-EJ-02-S027-C	2	
		EJ-02-02 Sht. 8			
			2-EJ-02-F050	3	
		EJ-02-04 Sht. 1			
		EU UL UN OIL	2-EJ-02-FW96	2	

Exam Category	<u>Number</u>	Drawing	Component	Period	Exam Method
Any Constitution of the Co		EJ-02-04 Sht. 2			Allenda Septimental Paris
			2-EJ-02-F041	3	
		EJ-02-04 Sht. 3			
			2-EJ-04-F008	1	
			2-EJ-04-F016	1	
			2-EJ-04-S010-C	1	
		EM-01-01 Sht. 1			
			2-EM-01-S027-D	3	
			2-EM-01-S027-G	3	
			2-EM-01-S040-D	3	
			2-EM-01-S041-E	3	
		EM-05-01 Sht. 1			
			2-EM-01-S035-C	1	
			2-EM-01-S039-C	1	
		EM-05-01 Sht. 2			
			2-EM-01-S022-K	2	
			2-EM-01-S029-E	2	
		EN-01-01 Sht. 1			
			2-BN-01-F044	2	
			2-EN-01-F007	2	
		EN-01-01 Sht. 2			
			2-EN-01-S013-B	1	
		EN-02-02 Sht. 1			
		T. C. T.	2-EN-02-S003-A	2	
			2-EN-02-S005-B	2	
		EN-02-02 Sht. 2			
		211 12 12 1111 2	2-EN-02-F012	3	
			2-EN-02-S008-A	3	
			2-EN-02-S009-C	3	

Exam Category	Item Number	Drawing	Component	Period	Exam Method
	C5.21	STEEL OR HIGH TO 1/5 INCH NO	TIAL WELDS IN AUSTENITIC STAINLESS ALLOY PIPING GREATER THAN OR EQUAL MINAL WALL THICKNESS FOR PIPING I OR EQUAL TO NPS 2 AND LESS THAN OR 4		SURF. & VOL
		BG-06-02 Sht. 2			
			2-BG-02-F063	1	
			2-BG-02-F076	1	
			2-BG-02-S040-A	1	
			2-BG-02-S040-E	1	
		BG-06-10 Sht. 1			
			2-BG-10-FW212	3	
		EM-06-02 Sht. 1			
			2-EM-02-A556572A-FW02	3	
			2-EM-02-S008-D	3	
			2-EM-02-S014-C	3	
			2-EM-02-S015-B	3	
		EM-06-02 Sht. 2			
			2-EM-02-S012-G	2	
			2-EM-02-S012-L	2	
			2-EM-02-S013-D	2	
			2-EM-02-S013-H	2	
			2-EM-02-S013-M	2	
			2-EM-02-S013-N	2	
	C5.30	SOCKET WELDS			SURFACE
		BG-06-02 Sht. 2			
			2-BG-09-FW464	1	
		BG-06-10 Sht. 1		. 1	
		DG-00-10 SML 1	2 DC 40 FM452		
			2-BG-10-FW152	2	

Exam Category	ltem Number	Drawing	Component	Period	Exam Method
	C5.41		NTIAL WELDS IN PIPE BRANCH S OF BRANCH PIPING GREATER THAN OR S 2		SURFACE
		BG-06-02 Sht	. 2		
			2-BG-02-A-34-112-SW14	1	
C-F-2	PRESSURE PIPING	RETAINING WE	LDS IN CARBON OR LOW ALLOY STEEL		
	C5.51	STEEL PIPING	NTIAL WELDS IN CARBON OR LOW ALLOY GREATER THAN OR EQUAL TO 3/8 INCH L THICKNESS FOR PIPING GREATER THAN		SURF. & VOL
		AB-01-01 Sht.			
			2-AB-01-F006	2	
			2-AB-01-F011	1	
			2-AB-01-S006-G	1	
		AB-02-01 Sht.	4		
			2-AB-01-F020	2	
			2-AB-01-F026	2	
			2-AB-01-F031	2	
			2-AB-01-F099	2	
			2-AB-01-S018-S	2	
		AB-03-01 Sht.	1		
			2-AB-01-F050	3	
			2-AB-01-F055	3	
			2-AB-01-F101	3	
			2-AB-01-S032-L	3	
		AB-04-01 Sht.	4		
			2-AB-01-F074	1	
			2-AB-01-F095	1	
		AE-01-04 Sht.	1		
			2-AE-04-F005	1	
			2-AE-04-F015	1	

Exam Category	<u>Number</u>	Drawing	Component	Period	Exam Method
ALL DO MANIE SEAT MANIE DE M	S WEEK CERNOPEN WOLLERAN, A WOLD	AE-02-04 Sht.	1		INCOMES OF PROPERTY OF PARTY.
			2-AE-04-F019	2	
			2-AE-04-F024	2	
			2-AE-04-S016-B	3	
		AE-03-05 Sht.	1		
			2-AE-05-F024	3	
		AE-04-05 Sht.	1		
			2-AE-05-F008	3	
			2-AE-05-F009	3	
			2-AE-05-F011	3	
		GN-01-03 Sht.	,		
			2-EF-03-C528892-FW01	1	
		GN-02-04 Sht.			
		517 02 07 011.	2-EF-04-C528895-FW01	1	
			2-GN-02-C528899-FW01	3	
		GN-02-05 Sht.			
		GN-02-03 SIII.	2-EF-05-C528898-FW01	1	
			2-GN-02-C528900-FW01	3	
	C5.81	STEEL PIPE BE	NTIAL WELDS IN CARBON OR LOW ALLOY RANCH CONNECTIONS OF BRANCH PIPING IN OR EQUAL TO NPS 2		SURFACE
		GN-01-03 Sht.	1		
			2-EF-03-S018-D	1	
-G	PRESSURE	E RETAINING WE	LDS IN PUMPS AND VALVES		
	C6.10	PUMP CASING	WELDS		SURFACE
		ISI-PUMPS Sh	nt. 1		
			2-PBG05A-F-1	2	
			2-PBG05A-F-2	2	

Exam Category	Item Number	Drawing	Component	Period	Exam Method
DAVED STATE OF STATE			2-PEJ01A-F-2	1	
			2-PEM01A-F-1	3	
D-A	INTEGRAL VALVES	ATTACHMENTS	FOR CLASS 3 VESSELS, PIPING, PUMPS &		
	D1.10	INTEGRALLY V	NELDED ATTACHMENTS TO PRESSURE		VISUAL, VT-1
		EF-01-01 Sht.	1		
			2-FEF02A-SUP-1	2	
		EG-01-01 Sht.	1		
			2-EEG01A-SUP-1	1	
		EG-01-02 Sht.	1		
			2-EEC01A-SUP-1	3	
		EG-05-06 Sht.	1		
			2-EBG01-SUP-1	1	
		ISI-EEJ01A SI	ht. 1		
			2-EEJ01A-SUP-1	3	
		KJ-01-01 Sht.	1		
			2-EKJ03A-SUP-1	2	
			2-EKJ04A-SUP-1	3	
			2-EKJ06A-SUP-1	1	
	D1.20	INTEGRALLY	VELDED ATTACHMENTS TO PIPING		VISUAL, VT-1
		AL-02-03 Sht.			
			2-AL-03-C005	2	
		EF-01-01 Sht.	2		
			2-EF-01-C002	1	
			2-EF-02-A010	3	
		EF-01-02 Sht.	1		
			2-EF-02-C007	1	

Exam Category	<u>Number</u>	Drawing	Component	Period	Exam Method
SACCOMINA PROPERSONAL ANNIES CALIFORNIA	ANALYSIS OF THE STATE OF THE ST	EF-01-02 Sht. 2			
			2-EF-06-A015	3	
		EF-01-03 Sht. 2			
			2-EF-01-C004	1	
		EG-01-02 Sht. 1			
			2-EG-02-A005	1	
		EG-03-07 Sht. 1			
			2-EG-07-C002	3	
			2-EG-07-C021	3	
			2-EG-07-R002	2	
		EG-04-01 Sht. 1			
			2-EG-01-R030	1	
		EG-04-01 Sht. 2			
			2-EG-04-A001	2	
		EG-06-09 Sht. 2			
			2-EG-15-A002	1	
		GN-01-01 Sht. 2			
			2-GN-01-C017	2	
		GN-01-01 Sht. 3			
			2-M-620-003-10	3	
			2-M-620-003-14	3	
		GN-02-02 Sht. 1			
			2-GN-02-C010	3	
		GN-02-02 Sht. 2			
			2-GN-02-C009	2	
	D1.30	INTEGRALLY WE	LDED ATTACHMENTS TO PUMPS		VISUAL, VT
		EF-01-01 Sht. 1			
			2-PEF01A-SUP-3	1	

Exam Category	ltem Number	Drawing	Component	Period	Exam Method
F-A	SUPPORTS				
	F1.10	CLASS 1 PIPIN	G SUPPORTS		VISUAL, VT-
		BB-01-01 Sht.	1		
			2-BB-01-SUP-1C	3	
		BB-01-02 Sht.	1		
			2-BB-02-H007	1	
			2-BB-02-R011	1	
			2-BB-02-R015	1	
			2-BB-02-R022	1	
			2-BB-02-R032	1	
		BB-01-04 Sht.	1		
			2-BB-04-H007	3	
			2-BB-04-R015	3	
			2-BB-04-R022	3	
		BB-01-04 Sht.	9		
		DD-01-04 SIII.	2-BB-04-C011	,	
			2-BB-04-H009	2 3	
			2-BB-04-H011	2	
			2-BB-04-R024	3	
			2-BB-04-R032	2	
		PP 04 00 PL		*	
		BB-01-08 Sht.			
			2-BB-08-H001	1	
		BB-02-01 Sht.			
			2-BB-01-SUP-2D	2	
		BB-02-02 Sht.	1		
			2-BB-02-H001	3	
			2-BB-02-R027	2	
			2-BB-02-R028	3	

Exam Category	item Number	Drawing	Component	Period	Exam Method
EDMONTHUM PART PROMISO			2-BB-02-R029	1	MARIN SPECIAL PROPERTY OF THE SPECIAL
		BB-02-11 Sht. 1			
			2-BB-11-H001	3	
		BB-03-01 Sht. 1			
			2-BB-01-SUP-3C	. 1	
		BB-03-09 Sht. 1			
			2-BB-09-R001	3	
		BB-04-01 Sht. 1			
			2-BB-01-SUP-4D	2	
		BB-04-07 Sht. 1			
			2-BB-07-H001	2	
		BB-05-01 Sht. 1			
			2-BB-01-H001	1	
			2-BB-01-H002	1	
			2-BB-01-R001	1	
		BB-06-01 Sht. 1			
			2-HB-24-R009	3	
			2-HB-24-R015	3	
		BG-01-21 Sht. 1			
			2-BG-21-H008	2	
		BG-02-22 Sht. 1			
			2-BG-22-H007	3	
			2-BG-22-H010	3	
			2-BG-22-R012	3	
			2-BG-22-R013	3	
			2-BG-22-R017	3	
		BG-03-23 Sht. 1			
			2-BG-23-H004	1	
			2-BG-23-R004	1	

Exam Category	Item Number	Drawing	Component	Period	Exam Method
		BG-05-24 Sht.	1		end at charactery, any man ext
			2-BG-24-H001	3	
			2-BG-24-R003	3	
		EJ-01-04 Sht.	1		
			2-EJ-04-H008	3	
			2-EJ-04-R018	3	
		EJ-02-04 Sht.	1		
			2-EJ-04-C020	1	
		EM-03-05 Sht.	1		
			2-EM-05-C008	1	
			2-EM-05-C010	1	
			2-EM-05-R011	1	
		EM-04-03 Sht.	1		
			2-EM-03-C014	2	
			2-EM-03-H003	2	
			2-EM-03-R008	2	
		EM-06-03 Sht.	1		
			2-EM-03-A005	2	
			2-EM-03-C007	3	
			2-EM-03-C009	3	
			2-EM-03-C026	2	
			2-EM-03-C033	2	
			2-EM-03-C037	1	
			2-EM-03-H006	3	
			2-EM-03-R003	2	
			2-EM-03-R011	3	
			2-EM-03-R014	3	
			2-EM-03-R015	2	
			2-EM-03-R017	2	
			2-EM-03-R024	2	

Exam Category	Number	Drawing	Component	Period	Exam Method
		EP-01-01 Sht.	1		
			2-EP-01-H009	1	
			2-EP-01-H011	2	
			2-EP-01-R011	2	
			2-EP-01-R016	2	
		EP-02-02 Sht.	1		
			2-EP-02-H006	2	
			2-EP-02-R012	2	
		EP-03-02 Sht.	1		
			2-EP-02-C004	3	
			2-EP-02-H011	3	
		EP-04-01 Sht.	1		
			2-EP-01-C004	3	
			2-EP-01-H004	2	
			2-EP-01-H006	3	
			2-EP-01-R008	2	
			2-EP-01-R019	2	
	F1.20	CLASS 2 PIPIN	IG SUPPORTS		VISUAL, VT-
		AB-01-01 Sht.	1		
			2-AB-01-R009	1	
		AB-02-01 Sht.	1		
			2-AB-01-H010	2	
			2-AB-01-R032	2	
		AB-03-01 Sht.			
			2-AB-01-C011	3	
			2-AB-01-R031	3	
		AB-04-01 Sht.	1		
			2-AB-01-H012		

Exam Category	<u>Item</u> Number	Drawing	Component	Period	Exam Method
		AE-01-04 Sht. 1			
			2-AE-04-C002	1	
		AE-02-04 Sht. 1			
			2-AE-04-C001	3	
			2-AE-04-R011	2	
		AE-03-05 Sht. 1			
			2-AE-05-C001	1	
			2-AE-05-R017	1	
		AE-04-05 Sht. 1			
			2-AE-05-R028	3	
		BG-96-02 Sht. 1			
			2-BG-02-A002	1	
			2-BG-02-R012	1	
			2-BG-02-R015	1	
		BG-06-02 Sht. 2			
			2-BG-02-H003	3	
			2-BG-02-H011	3	
		BG-06-10 Sht. 1			
			2-BG-10-C528	1	
		EJ-01-01 Sht. 1			
			2-EJ-01-C002	3	
			2-EJ-01-H018	3	
		EJ-01-01 Sht. 2			
			2-EJ-01-C014	1	
			2-EJ-01-R030	1	
		EJ-01-01 Sht. 3			
			2-EJ-01-R023	1	
		EJ-01-01 Sht. 4			
			2-EJ-04-R004	2	

Exam Category	item Number	Drawing	Component	Period	Exam Method
SHI SHI KARIMAN SAME IN EMECAD	OR CONTRACTOR OF THE STREET	EJ-02-02 Sht. 1			de en altra securità di després presi del colo
			2-EJ-02-H019	2	
		EJ-02-02 Sht. 2			
			2-EJ-02-C022	2	
		EJ-02-02 Sht. 3			
			2-EJ-02-C018	2	
			2-EJ-03-A003	1	
		EJ-02-02 Sht. 4			
			2-EJ-02-C020	2	
			2-EJ-02-H021	3	
			2-EJ-02-R032	1	
		EJ-02-04 Sht. 1			
			2-EJ-02-R016	2	
		EJ-02-04 Sht. 2			
			2-EJ-04-R010	3	
		EJ-02-04 Sht. 3			
			2-EJ-04-C024	3	
		EM-01-01 Sht. 1			
			2-EM-01-C043	3	
			2-EM-01-R030	3	
		EM-02-01 Sht. 1			
			2-EM-01-R008	1	
		EM-05-01 Sht. 1			
			2-EM-01-A002	1	
			2-EM-01-H004	1	
		EM-05-01 Sht. 2			
			2-EM-01-A003	3	
			2-EM-01-A004	3	
			2-EM-01-C019	1	
			2-EM-01-H010	3	

Exam Category	Item Number	Drawing	Component	Period	Exam Method
		EM-06-02 Sht.	1		
			2-EM-02-C023	3	
			2-EM-02-C030	3	
			2-EM-02-H009	1	
			2-EM-02-R001	3	
		EM-06-02 Sht.	2		
			2-EM-02-C013	2	
			2-EM-02-R004	2	
		EN-01-01 Sht.	1		
			2-EN-01-H005	2	
			2-EN-01-R010	2	
		EN-01-01 Sht.	2		
			2-EN-01-H003	2	
		EN-02-02 Sht.	1		
			2-EN-02-R005	1	
		EN-02-02 Sht.	2		
			2-EN-02-C004	2	
			2-EN-02-R004	2	
		GN-01-03 Sht.	1		
		- 1000-100-100-100-100-100-100-100-100-1	2-EF-03-C034	1	
		GN-02-05 Sht.			
		511-02-00 511L	2-EF-05-H004	3	
	F1.30	CLASS 3 PIPING			VISUAL, VT-
		AL-01-02 Sht.	1		
			2-AL-02-C005	1	
		AL-02-03 Sht.			
			2-AL-03-C005	2	

Exam Category	Item Number	Drawing	Component	Period	Exam Method
EAS. BY COLUMN STATES	a-year and it is a second and a second	AL-03-04 Sht. 1			
			2-AL-04-C002	3	
			2-AL-04-C006	3	
			2-AL-04-H007	2	
		EF-01-01 Sht. 1			
			U-EF11-C001	1	
		EF-01-01 Sht. 2			
			2-EF-01-C002	1	
			2-EF-02-A010	3	
		EF-01-02 Sht. 1			
			2-EF-02-C003	1	
			2-EF-02-C007	1	
		EF-01-02 Sht. 2			
			2-EF-02-A007	3	
			2-EF-02-C013	2	
			2-EF-06-A015	3	
		EF-01-03 Sht. 2			
			2-EF-01-C004	1	
			2-EF-01-C012	1	
			2-EF-01-H003	1	
			2-EF-03-R020	3	
		EF-01-07 Sht. 1			
			2-EF-07-C007	1	
		EF-01-08 Sht. 1			
			2-EF-07-R001	2	
			2-EF-08-C002	2	
		EF-02-01 Sht. 1			
			U-EF11-H002	2	
		EF-02-01 Sht. 2			
			2-EF-01-C005	3	

xam ategory	Item Number	Drawing	Component	Period	Exam Method
		EF-02-01 Sht. 3			
			2-EF-01-R010	3	
			2-EF-06-C010	1	
		EF-02-04 Sht. 1			
			2-EF-04-C005	3	
			2-EF-04-R005	3	
		EF-02-05 Sht. 1			
			2-EF-05-H010	3	
			2-EF-05-R025	3	
		EF-02-05 Sht. 2			
			2-EF-05-C014	1	
		EF-02-06 Sht. 1			
			2-EF-04-C015	1	
			2-EF-06-A014	1	
			2-EF-06-C004	1	
		EF-02-08 Sht. 1			
			2-EF-08-C017	1	
		EF-02-08 Sht. 2			
		LI -02-00 Silc. 2	2-EF-08-C011	1	
		EC 04 02 Ph. 4			
		EG-01-02 Sht. 1	2-EG-02-A005		
				1	
		EG-02-03 Sht. 1			
			2-EG-03-H013	2	
			2-EG-03-R018	2	
		EG-02-03 Sht. 2			
			2-EG-03-H012	3	
			2-EG-03-R006	3	
		EG-02-05 Sht. 1			
			2-EG-18-R005	2	

Exam Category	Number Number	Drawing	Component	Period	Exam Method
		EG-03-07 Sht.	1		
			2-EG-07-C002	3	
			2-EG-07-C021	3	
			2-EG-07-R002	2	
		EG-04-01 Sht.	1		
			2-EG-01-R030	1	
			2-EG-04-C012	3	
			2-EG-04-R004	3	
		EG-04-01 Sht.	2		
			2-EG-04-A001	2	
			2-EG-04-R007	2	
			2-EG-04-R008	2	
			2-EG-04-R009	2	
		EG-05-06 Sht.	1		
			2-EG-06-C023	2	
			2-EG-06-C027	2	
			2-EG-06-R009	2	
		EG-05-06 Sht.	2		
			2-EG-06-C021	2	
			2-EG-06-H010	3	
			2-EG-06-R008	1	
		EG-06-09 Sht.			
			2-EG-10-A001	3	
			2-EG-10-H001	3	
		EG-06-09 Sht.	2		
			2-EG-09-C001	1	
			2-EG-15-A002	1	
		GN-01-01 Sht.			
			2-GN-01-C017	2	
		GN-01-01 Sht. :			
		Jir vi vi viit.	2-M-620-003-10	3	

MATERIAL PROPERTY AND ADDRESS OF THE PARTY AND	Number	Drawing	Component	Period	Exam Method
	Continues Incident actives and		2-M-620-003-14	3	
		GN-02-02 Sht.	1		
			2-GN-02-C010	3	
		GN-02-02 Sht.	2		
			2-GN-02-C009	2	
			2-GN-02-R003	2	
	F1.40	SUPPORTS OF	THER THAN PIPING SUPPORTS (CLASS 1, 2,		VISUAL, VT-
		11908662	2 EV IDAA CUD 4		
			2-EKJ04A-SUP-1 2-EKJ04A-SUP-2	3	
			2-2/3044-501-2	3	
		5736	2 55 1044 EVIDS W		
			2-EEJ01A-SKIRT-W	3	
			2-EEJ01A-SUP-1 2-EEJ01A-SUP-2	3	
			2-EEJ01A-SUP-3	3	
				3	
			2-EEJ01A-SUP-4	3	
			2-EEJ01A-SUP-5	3	
		BB-00-01 Sht.			
			2-BB-01-R005	3	
		C-2S1904			
			2-PEJ01A-SUP-1	2	
			2-PEJ01A-SUP-2	2	
			2-PEJ01A-SUP-3	2	
			2-PEJ01A-SUP-4	2	
			2-PEN01A-SUP-1	3	
			2-PEN01A-SUP-2	3	
			2-PEN01A-SUP-3	3	
		C-2S1908			
			2-PEJ01A-SUP-5	2	

Exam Category	<u>Number</u>	Drawing	Component	Period	Exam Method
			2-PE-J01A-SUP-6	2	
			2-PEJ01A-SUP-7	2	
			2-PEJ01A-SUP-8	2	
		M-018-00091			
			2-EKJ03A-SUP-1	2	
			2-EKJ03A-SUP-2	2	
		M-018-00724			
			2-EKJ06A-SUP-1	1	
			2-EKJ06A-SUP-2	1	
		M-021-00045			
			2-PAL01A-SUP-1	2	
		M-021-005		•	
		WI-021-005	2-PAL02-SUP-1		
			2-PAL02-50P-1	3	
		M-071-00001			
			2-EEC01A-SUP-1	3	
			2-EEC01A-SUP-2	2	
			2-EEC01A-SUP-3	2	
		M-072-0001			
			2-EEG01A-SUP-1	2	
			2-EEG01A-SUP-2	2	
			2-EEG01A-SUP-3	2	
			2-EEG01A-SUP-4	2	
		M-082-012			
			2-PEG01A-SUP-1	2	
		M-089-U0012			
			2-PEF01A-SUP-1	1	
			2-PEF01A-SUP-2	1	
			2-PEF01A-SUP-3	1	
			U-EF11-R008	1	
			U-EF11-R009	1	

Exam Category	Number	Drawing	Component	Period	Exam Method
		M-154-U0018			ACCURATION AND ACCURATIONS
			2-FEF02A-SUP-1	2	
			2-FEF02A-SUP-2	2	
		OP-1459F01			
			2-RBB01-SUP-1	1	
			2-RBB01-SUP-2	1	
			2-RBB01-SUP-3	1	
			2-RBB01-SUP-4	1	
		OP-1459F02			
			2-EBB01B-SUP-1	1	
			2-EBB01B-SUP-2	1	
			2-EBB01B-SUP-3	1	
			2-EBB01B-SUP-4	1	
			2-PBB01A-SUP-4	3	
			2-PBB01A-SUP-5	3	
			2-PBB01A-SUP-6	3	
		OP-1459F04			
			2-EBB01B-SUP-5	1	
			2-EBB01B-SUP-6	1	
			2-EBB01B-SUP-7	1	
		OP-1459F07			
			2-PBB01A-SUP-1	3	
			2-PBB01A-SUP-2	3	
			2-PBB01A-SUP-3	3	
		OP-1459F08			
			2-BB-00-R602-B	1	
			2-BB-00-R602-C	1	
			2-EBB01B-SUP-8	1	
		OP-1459F10			
			2-TBB03-LUG-A-W	3	
			2-TBB03-LUG-B-W	3	

Exam Category	Item Number	Drawing	Component	Period	Exam Method
			2-TBB03-LUG-C-W	3	ALAM PILLETT SPECIAL PROPERTY
			2-TBB03-LUG-D-W	3	
			2-TBB03-SKIRT-W	3	
		OP-300-B4973	39-S001		
			2-PBG05A-SUP-1	2	
			2-PBG05A-SUP-2	2	
			2-PBG05A-SUP-3	2	
			2-PBG05A-SUP-4	2	
			2-PBG05A-SUP-5	2	
		OP-300-J4972	8		
			2-PEM01A-SUP-1	3	
		OP-D-75-569			
			2-EBG01-SUP-1	1	
			2-EBG01-SUP-2	1	

Notes:

- Only the first pump disassembled for maintenance, repair, or other inspection requires Section XI internal surface examination.
- Only the first valve in each group disassembled for maintenance, repair, or other inspection requires Section XI internal surface examination.
- 3. Twenty-five percent of the partial penetration welds listed require Visual, VT-2 examination.

Callaway 2nd Interval High Energy Weld (NBZ) Inspections

11-Aug-95

Exam Category	Item Number	Drawing	Component	Period	Exam Method
C-F-1	PRESSURE HIGH ALLO	The second secon	LDS IN AUSTENITIC STAINLESS STEEL OR		
	NBZ EXAMS		FERENTIAL, LONGITUDINAL AND BRANCH N PIPING TWO INCHES NPS AND GREATER		VOLUMETRIC
		AE-01-04 Sht.			
			2-AE-04-C525734-FW01	1	
			2-AE-04-C525734-FW02	1	
			2-AE-04-S022-C	1	
			2-AE-04-S022-D	1	
		AE-02-04 Sht.	1		
			2-AE-04-C525735-FW01	3	
			2-AE-04-C525735-FW02	3	
			2-AE-04-S023-C	3	
			2-AE-04-S023-D	3	
		AE-03-05 Sht.	1		
			2-AE-05-C525736-FW01	3	
			2-AE-05-C525736-FW02	3	
			2-AE-05-S024-C	3	
			2-AE-05-S024-D	3	
		AE-04-05 Sht.	1		
			2-AE-05-C525737-FW01	3	
			2-AE-05-C525737-FW02	3	
			2-AE-05-S023-C	3	
			2-AE-05-S023-D	3	
		BB-01-08 Sht.	1		
			2-BB-08-F002	1	
			2-BB-08-FW080	1	
			2-BB-08-FW081	1	

Exam Category	Item Number	Drawing	Component	Period	Exam Method
e proposición es franças de			2-BB-08-FW082	1	
			2-BB-08-FW083	1	
			2-BB-08-FW084	1	
			2-BB-08-FW085	1	
			2-BB-08-FW086	1	
			2-BB-08-FW087	1	
			2-BB-08-FW088	1	
			2-BB-08-FW089	1	
			2-BB-08-V118-1	1	
			2-BB-08-V118-2	1	
		BB-02-11 Sht.	1		
			2-BB-11-F002	2	
			2-BB-11-FW10	2	
			2-BB-11-FW117	2	
			2-BB-11-FW118	2	
			2-BB-11-FW119	2	
			2-BB-11-FW120	2	
			2-BB-11-FW121	2	
			2-BB-11-V148-1	2	
			2-BB-11-V148-2	2	
		BB-03-09 Sht.	1		
			2-BB-09-F002	3	
			2-BB-09-FW098	3	
			2-BB-09-FW099	3	
			2-BB-09-FW100	3	
			2-BB-09-FW101	3	
			2-BB-09-FW102	3	
			2-BB-09-FW109	3	
			2-BB-09-FW110	3	
			2-BB-09-FW111	3	
			2-BB-09-FW190	3	
			2-BB-09-FW191	3	

Exam Category	Item Number	Drawing	Component	Period	Exam Method
			2-BB-09-V178-1	3	
			2-BB-09-V178-2	3	
		BB-04-07 Sht.	1		
			2-BB-07-F002-R3	2	
			2-BB-07-FW044	2	
			2-BB-07-FW045	2	
			2-BB-07-FW058	2	
			2-BB-07-FW059	2	
			2-BB-07-FW060	2	
			2-BB-07-FW061	2	
			2-BB-07-FW122	2	
			2-BB-07-FW130	2	
			2-BB-07-FW131	2	
			2-BB-07-V208-1	2	
			2-BB-07-V208-2	2	
		BG-01-01 Sht.	1		
			2-BG-01-F022	3	
			2-BG-01-F023	3	
			2-BG-01-F024	3	
			2-BG-01-F025	3	
			2-BG-01-F026	3	
			2-BG-01-FW175	3	
			2-BG-01-FW176	3	
			2-BG-01-FW178	3	
			2-BG-23-F002	3	
			2-BG-23-F003	3	
			2-BG-23-F004	3	
			2-BG-23-F005	3	
		BG-02-22 Sht.	2		
			2-BG-03-F001	1	
			2-BG-03-F002	1	
			2-BG-03-F003		

2.8G-03-FW5 1 2.8G-03-S001-A 1 2.8G-03-S001-B 1 2.8G-03-S001-C 1 2.8G-03-S002-A 1 2.8G-03-S002-A 1 2.8G-22-F019 2 2.8G-22-F020-A 2 2.8G-22-F021 2 2.8G-22-F022 2 2.8G-22-F023 2 2.8G-22-FW-2 2 2.8G-22-FW-2 2 2.8G-22-FW-5 2 2.8G-22-S018-A 2 2.8G-22-S018-A 2 2.8G-22-S018-B 2 2.8G-22-S018-B 2 2.8G-22-S019-D 2 2.8G-04-09 Sht. 1 2.8G-07-F001 2 2.8G-09-FW-5 1 2.8G-09-FW226 1 2.8G-09-FW226 1 2.8G-09-FW226 1 2.8G-09-FW226 1 2.8G-09-FW229 1 2.8G-09-FW226 1 2.8G-09-FW229 1 2.8G-09-FW229 1	Exam Category	Item Number	Drawing	Component	Period	Exam Method
2-BG-03-S001-B 2-BG-03-S001-C 1 2-BG-03-S002-A 1 2-BG-22-F019 2-BG-22-F020-A 2-BG-22-F021 2-BG-22-F022 2-BG-22-F023 2-BG-22-F023 2-BG-22-FW-2 2-BG-22-FW-2 2-BG-22-FW-5 2-BG-22-FW-6 2-BG-22-FW-6 2-BG-22-S018-A 2-BG-22-S018-B 2-BG-22-S018-B 2-BG-22-S019-A 2-BG-22-S019-B 2-BG-22-S019-B 2-BG-22-S019-C 2-BG-22-S019-D 2-BB-08-F001 1-2-BB-08-F001 2-BB-08-F001 1-2-BB-08-F001 1-2-BB-09-FW-6 1-2-BG-09-FW-26 1-2-BG-09-FW-29 1-2-BG-09-FW-29 1-2-BG-09-FW-29 1-2-BG-09-FW-267				2-BG-03-FW5	1	
2-BG-03-S001-C 2-BG-03-S002-A 1 2-BG-23-F019 2-BG-22-F020-A 2-BG-22-F021 2-BG-22-F022 2-BG-22-F023 2-BG-22-FW-2 2-BG-22-FW-2 2-BG-22-FW-5 2-BG-22-FW-6 2-BG-22-FW-6 2-BG-22-S018-A 2-BG-22-S018-B 2-BG-22-S018-B 2-BG-22-S019-A 2-BG-22-S019-B 2-BG-22-S019-C 2-BG-22-S019-D 2-BG-04-09 Sht. 1 2-BB-07-F001 2-BB-08-F001 3-2-BG-09-FW-6 1-2-BG-09-FW-26 1-2-BG-09-FW229 1-2-BG-09-FW226				2-BG-03-S001-A	1	
2-BG-03-S002-A 1 2-BG-22-F019 2 2-BG-22-F020-A 2 2-BG-22-F021 2 2-BG-22-F022 2 2-BG-22-F023 2 2-BG-22-FW-2 2 2-BG-22-FW-4 2 2-BG-22-FW-5 2 2-BG-22-S018-A 2 2-BG-22-S018-B 2 2-BG-22-S018-B 2 2-BG-22-S019-A 2 2-BG-22-S019-A 2 2-BG-22-S019-B 2 2-BG-22-S019-B 2 2-BG-22-S019-C 2 2-BG-22-S019-D 2 2-BG-04-09 Sht. 1 2-BB-08-F001 1 2-BB-09-F001 3 2-BB-11-F001 1 2-BB-09-FW-6 1 2-BG-09-FW-6 1 2-BG-09-FW-29 1 2-BG-09-FW229 1 2-BG-09-FW229 1 2-BG-09-FW229 1 2-BG-09-FW220 1				2-BG-03-S001-B	1	
2-BG-22-F019 2-BG-22-F020-A 2-BG-22-F021 2-BG-22-F022 2-BG-22-F023 2-BG-22-FW-2 2-BG-22-FW-2 2-BG-22-FW-4 2-BG-22-FW5 2-BG-22-S018-A 2-BG-22-S018-B 2-BG-22-S018-C 2-BG-22-S019-A 2-BG-22-S019-B 2-BG-22-S019-C 2-BG-22-S019-D 2-BB-08-F001 2-BB-08-F001 2-BB-09-FW26 2-BG-09-FW26 2-BG-09-FW226 2-BG-09-FW229 2-BG-09-FW226				2-BG-03-S001-C	1	
2-BG-22-F020-A 2-BG-22-F021 2-BG-22-F022 2-BG-22-F023 2-BG-22-FW-2 2-BG-22-FW-2 2-BG-22-FW-4 2-BG-22-FW5 2-BG-22-FW5 2-BG-22-S018-A 2-BG-22-S018-B 2-BG-22-S018-C 2-BG-22-S019-A 2-BG-22-S019-B 2-BG-22-S019-C 2-BG-22-S019-D 2-BG-22-S019-D 2-BG-02-FW1-D 3-2-BG-02-FW1-D 3-2-BG-03-FW1-D 3-2-BG-03-FW2-B 2-BG-03-FW2-B 2-BG-03-FW2				2-BG-03-S002-A	1	
2-BG-22-F021 2-BG-22-F022 2-BG-22-F023 2-BG-22-FW-2 2-BG-22-FW-2 2-BG-22-FW-4 2-BG-22-FW5 2-BG-22-S018-A 2-BG-22-S018-B 2-BG-22-S018-B 2-BG-22-S019-A 2-BG-22-S019-A 2-BG-22-S019-B 2-BG-22-S019-C 2-BG-22-S019-D 2-BG-22-S019-D 2-BG-04-09 Sht. 1 2-BB-07-F001 2-BB-08-F001 2-BB-09-FW1 2-BG-09-FW26 1-BG-09-FW29 1-BG-09-FW29 1-BG-09-FW29 1-BG-09-FW29 1-BG-09-FW267 2-BG-09-FW267				2-BG-22-F019	2	
2-BG-22-F022 2 2-BG-22-F023 2 2-BG-22-FW-2 2 2-BG-22-FW-4 2 2-BG-22-FW5 2 2-BG-22-S018-A 2 2-BG-22-S018-B 2 2-BG-22-S018-C 2 2-BG-22-S019-A 2 2-BG-22-S019-A 2 2-BG-22-S019-B 2 2-BG-22-S019-D 2 2-BG-22-S019-D 3 2-BG-04-09 Sht. 1 2-BB-08-F001 2 2-BB-09-F001 3 2-BB-01-F001 1 2-BB-09-FW26 1 2-BG-09-FW26 1 2-BG-09-FW26 1 2-BG-09-FW29 1 2-BG-09-FW29 1 2-BG-09-FW29 1				2-BG-22-F020-A	2	
2-BG-22-F023 2-BG-22-FW-2 2-BG-22-FW-4 2-BG-22-FW6 2-BG-22-S018-A 2-BG-22-S018-B 2-BG-22-S018-C 2-BG-22-S019-A 2-BG-22-S019-B 2-BG-22-S019-B 2-BG-22-S019-D 2-BG-04-09 Sht. 1 2-BB-07-F001 2-BB-08-F001 2-BB-09-F001 2-BG-09-FW-6 1-2-BG-09-FW-6 1-2-BG-09-FW26 2-BG-09-FW229 1-2-BG-09-FW229 1-2-BG-09-FW230 1-2-BG-09-FW267 2-BG-09-FW267				2-BG-22-F021	2	
2-BG-22-FW-2 2-BG-22-FW-4 2-BG-22-FW5 2-BG-22-S018-A 2-BG-22-S018-B 2-BG-22-S018-C 2-BG-22-S019-A 2-BG-22-S019-A 2-BG-22-S019-B 2-BG-22-S019-C 2-BG-22-S019-D 2-BG-04-09 Sht. 1 2-BB-07-F001 2-BB-08-F001 3-BB-08-F001 3-BB-11-F001 2-BG-09-FW-6 2-BG-09-FW-6 2-BG-09-FW29 1-2-BG-09-FW29 1-2-BG-09-FW267 2-BG-09-FW267 2-BG-09-FW267				2-BG-22-F022	2	
2-BG-22-FW-4 2-BG-22-FW5 2-BG-22-S018-A 2-BG-22-S018-B 2-BG-22-S018-C 2-BG-22-S019-A 2-BG-22-S019-A 2-BG-22-S019-B 2-BG-22-S019-C 2-BG-22-S019-D 2-BG-04-09 Sht. 1 2-BB-08-F001 2-BB-08-F001 3-2-BB-11-F001 2-BG-09-FW-6 1-2-BG-09-FW-6 1-2-BG-09-FW226 1-2-BG-09-FW229 1-2-BG-09-FW230 1-2-BG-09-FW230 1-2-BG-09-FW267 2-BG-09-FW267				2-BG-22-F023	2	
2-BG-22-FW5 2 2-BG-22-S018-A 2 2-BG-22-S018-B 2 2-BG-22-S018-C 2 2-BG-22-S019-A 2 2-BG-22-S019-A 2 2-BG-22-S019-B 2 2-BG-22-S019-D 2 2-BG-22-S019-D 2 BG-04-09 Sht. 1 2-BB-07-F001 2 2-BB-08-F001 1 2-BB-09-F001 3 2-BB-11-F001 1 2-BG-09-FW26 1 2-BG-09-FW226 1 2-BG-09-FW229 1 2-BG-09-FW229 1 2-BG-09-FW229 1 2-BG-09-FW230 1 2-BG-09-FW267 2				2-BG-22-FW-2	2	
2-BG-22-S018-A 2 2-BG-22-S018-B 2 2-BG-22-S018-C 2 2-BG-22-S019-A 2 2-BG-22-S019-B 2 2-BG-22-S019-D 2 2-BG-22-S019-D 2 BG-04-09 Sht. 1 2-BB-07-F001 2 2-BB-08-F001 1 2-BB-09-F001 3 2-BB-11-F001 1 2-BB-09-FW26 1 2-BG-09-FW26 1 2-BG-09-FW29 1 2-BG-09-FW29 1 2-BG-09-FW29 1 2-BG-09-FW29 1				2-BG-22-FW-4	2	
2-BG-22-S018-B 2 2-BG-22-S018-C 2 2-BG-22-S019-A 2 2-BG-22-S019-B 2 2-BG-22-S019-C 2 2-BG-22-S019-D 2 BG-04-09 Sht. 1 2-BB-07-F001 2 2-BB-08-F001 1 2-BB-09-F001 3 2-BB-11-F001 1 2-BG-09-FW-6 1 2-BG-09-FW-6 1 2-BG-09-FW229 1 2-BG-09-FW229 1 2-BG-09-FW230 1 2-BG-09-FW267 2				2-BG-22-FW5	2	
2-BG-22-S018-C 2 2-BG-22-S019-A 2 2-BG-22-S019-B 2 2-BG-22-S019-C 2 2-BG-22-S019-D 2 BG-04-09 Sht. 1 2-BB-07-F001 2 2-BB-08-F001 1 2-BB-09-F001 3 2-BB-11-F001 1 2-BG-09-FW-6 1 2-BG-09-FW-26 1 2-BG-09-FW229 1 2-BG-09-FW229 1 2-BG-09-FW230 1 2-BG-09-FW267 2				2-BG-22-S018-A	2	
2-BG-22-S019-A 2 2-BG-22-S019-B 2 2-BG-22-S019-C 2 2-BG-22-S019-D 2 BG-04-09 Sht. 1 2-BB-07-F001 2 2-BB-08-F001 1 2-BB-09-F001 3 2-BB-11-F001 1 2-BG-09-FW-6 1 2-BG-09-FW226 1 2-BG-09-FW229 1 2-BG-09-FW230 1 2-BG-09-FW230 1				2-BG-22-S018-B	2	
2-BG-22-S019-B 2 2-BG-22-S019-C 2 2-BG-22-S019-D 2 BG-04-09 Sht. 1 2-BB-07-F001 2 2-BB-08-F001 1 2-BB-09-F001 3 2-BB-11-F001 1 2-BG-09-FW-6 1 2-BG-09-FW26 1 2-BG-09-FW229 1 2-BG-09-FW229 1 2-BG-09-FW230 1 2-BG-09-FW267 2				2-BG-22-S018-C	2	
2-BG-22-S019-C 2 2-BG-22-S019-D 2 BG-04-09 Sht. 1 2-BB-07-F001 2 2-BB-08-F001 1 2-BB-09-F001 3 2-BB-11-F001 1 2-BG-09-FW-6 1 2-BG-09-FW226 1 2-BG-09-FW229 1 2-BG-09-FW230 1 2-BG-09-FW267 2				2-BG-22-S019-A	2	
2-BG-22-S019-D 2 BG-04-09 Sht. 1 2-BB-07-F001 2 2-BB-08-F001 1 2-BB-09-F001 3 2-BB-11-F001 1 2-BG-09-FW-6 1 2-BG-09-FW26 1 2-BG-09-FW229 1 2-BG-09-FW229 1 2-BG-09-FW230 1 2-BG-09-FW267 2				2-BG-22-S019-B	2	
BG-04-09 Sht. 1 2-BB-07-F001 2-BB-08-F001 1 2-BB-09-F001 3 2-BB-11-F001 1 2-BG-09-FW-6 1 2-BG-09-FW226 1 2-BG-09-FW229 1 2-BG-09-FW230 1 2-BG-09-FW267 2				2-BG-22-S019-C	2	
2-BB-07-F001 2 2-BB-08-F001 1 2-BB-09-F001 3 2-BB-11-F001 1 2-BG-09-FW-6 1 2-BG-09-FW226 1 2-BG-09-FW229 1 2-BG-09-FW230 1 2-BG-09-FW267 2				2-BG-22-S019-D	2	
2-BB-08-F001 1 2-BB-09-F001 3 2-BB-11-F001 1 2-BG-09-FW-6 1 2-BG-09-FW226 1 2-BG-09-FW229 1 2-BG-09-FW230 1 2-BG-09-FW267 2			BG-04-09 Sht	1		
2-BB-09-F001 3 2-BB-11-F001 1 2-BG-09-FW-6 1 2-BG-09-FW226 1 2-BG-09-FW229 1 2-BG-09-FW230 1 2-BG-09-FW267 2				2-BB-07-F001	2	
2-BB-11-F001 1 2-BG-09-FW-6 1 2-BG-09-FW226 1 2-BG-09-FW229 1 2-BG-09-FW230 1 2-BG-09-FW267 2				2-BB-08-F001	1	
2-BG-09-FW-6 1 2-BG-09-FW226 1 2-BG-09-FW229 1 2-BG-09-FW230 1 2-BG-09-FW267 2				2-BB-09-F001	3	
2-BG-09-FW226 1 2-BG-09-FW229 1 2-BG-09-FW230 1 2-BG-09-FW267 2				2-BB-11-F001	1	
2-BG-09-FW229 1 2-BG-09-FW230 1 2-BG-09-FW267 2				2-BG-09-FW-6	1	
2-BG-09-FW230 1 2-BG-09-FW267 2				2-BG-09-FW226	1	
2-BG-09-FW267 2				2-BG-09-FW229	1	
				2-BG-09-FW230	1	
2-BG-09-FW270 2				2-BG-09-FW267	2	
				2-BG-09-FW270	2	
2-BG-09-FW271 2				2-BG-09-FW271	2	

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Exam Category	Item_ Number	Drawing	Component	Period	Exam Method
			2-BG-09-FW308	3	COLUMN PROGRAMMENT AND ASSESSED.
			2-BG-09-FW311	3	
			2-BG-09-FW312	3	
			2-BG-09-FW383	1	
			2-BG-09-FW388	1	
			2-BG-09-FW389	1	
			2-BG-09-FW390	. 1	
			2-BG-09-FW391	1	
			2-BG-09-FW392	1	
			2-BG-09-FW393	1	
			2-BG-09-FW395	1	
			2-BG-09-FW396	1	
			2-BG-09-FW398	2	
			2-BG-09-FW403	2	
			2-BG-09-FW404	2	
			2-BG-09-FW405	2	
			2-BG-09-FW406	2	
			2-BG-09-FW407	2	
			2-BG-09-FW408	2	
			2-BG-09-FW410	2	
			2-BG-09-FW411	2	
			2-BG-09-FW413	3	
			2-BG-09-FW418	3	
			2-BG-09-FW419	3	
			2-BG-09-FW420	3	
			2-BG-09-FW421	3	
			2-BG-09-FW422	3	
			2-BG-09-FW423	3	
			2-BG-09-FW425	3	
			2-BG-09-FW426	3	
			2-BG-09-FW428	3	
			2-BG-09-FW433	3	

Exam Category	Item Number	Drawing	Component	Period	Exam Method
E EL POLITO DE SERVICIO DE LA CONTRE CIPITA EL CONTRE CIP	HAM BOOK STANISH TO HE WAS BEEN COMEDIA.		2-BG-09-FW434	3	
			2-BG-09-FW435	3	
			2-BG-09-FW436	1	
			2-BG-09-FW437	1	
			2-BG-09-FW438	1	
			2-BG-09-FW439	1	
			2-SG-09-FW440	1	
			2-BG-09-FW441	1	
			2-BG-09-FW442	1	
			2-BG-09-FW521	1	
	NBZ EXAMS		FERENTIAL WELDS IN PIPING ONE AND ONE- CHES NPS		SURFACE
		BG-04-09 Sht.	1		
			2-BG-09-FW384	1	
			2-BG-09-FW385	1	
			2-BG-09-FW386	1	
			2-03-09-FW387	1	
			2-BG-09-FW399	2	
			2-BG-09-FW400	2	
			2-BG-09-FW401	2	
			2-BG-09-FW402	2	
			2-BG-09-FW414	3	
			2-BG-09-FW415	3	
			2-BG-09-FW416	3	
			2-BG-09-FW417	3	
			2-BG-09-FW429	3	
			2-BG-09-FW430	3	
			2-BG-09-FW431	3	
			2-BG-09-FW432	3	

Exam Category	<u>Number</u>	Drawing	Component	Period	Exam Method
	NBZ EXAMS		WELDS IN PIPING ONE AND ONE-HALF NPS AND GREATER		SURFACE
		BB-01-08 Sht.	1		
			2-BB-08-FW076	1	
			2-BB-08-FW077	1	
			2-BB-08-FW078	1	
			2-BB-08-FW079	1	
		BB-02-11 Sht.	1		
			2-BB-11-FW115	2	
			2-BB-11-FW116	2	
		BB-03-09 Sht.	1		
			2-BB-09-FW112-A	3	
			2-BB-09-FW113	3	
			2-BB-09-FW33	3	
			2-BB-09-FW34	3	
			2-BB-09-FW37	3	
			2-BB-09-F W 38	3	
		BB-04-07 Sht.	1		
			2-BB-07-FW057	2	
			2-BB-07-FW121	2	
			2-BB-07-FW134	2	
C-F-2	PRESSURE F	RETAINING WE	LDS IN CARBON OR LOW ALLOY STEEL		
	NBZ EXAMS		ERENTIAL, LONGITUDINAL AND BRANCH N PIPING TWO INCHES NPS AND GREATER		VOLUMETRIC
		AB-01-01 Sht.	1		
			2-AB-01-F007	1	
			2-AB-01-F008	- 1	
			2-AB-01-F009	1	
			2-AB-01-F010	2	
			2-AB-01-F011	1	

Exam Category	Item Number	Drawing	Component	Period	Exam Method
			2-AB-01-F012	1	
			2-AB-01-F013	1	
			2-AB-01-F014	1	
			2-AB-01-F097	1	
			2-AB-01-S006-G	1	
			2-AB-01-S006-H	1	
			2-AB-01-S006-J	1	
			2-AB-01-S006-K	1	
			2-AB-01-S006-L	1	
			2-AB-01-S006-M	1	
			2-AB-01-S006-S	1	
			2-AB-01-S006-V	1	
			2-AB-01-S008-B	1	
			2-AB-01-S008-C	1	
			2-AB-01-S009-C	1	
			2-AB-01-S009-D	1	
			2-AB-01-TORSIONAL RESTRAINT-1	1	
		AB-02-01 Sht.	1		
			2-AB-01-F026	2	
			2-AB-01-F027	2	
			2-AB-01-F028	2	
			2-AB-01-F029	2	
			2-AB-01-F030	2	
			2-AB-01-F031	2	
			2-AB-01-F032	2	
			2-AB-01-F033	2	
			2-AB-01-F035	2	
			2-AB-01-F036	2	
			2-AB-01-F037	2	
			2-AB-01-F038	2	
			2-AB-01-F039	2	
			2-AB-01-F108	2	

Exam Category	<u>Number</u>	Drawing	Component	Period	Exam Method
			2-AB-01-FW1-B	2	
			2-AB-01-FW11	2	
			2-AB-01-S018-G	2	
			2-AB-01-S018-H	2	
			2-AB-01-S018-J	2	
			2-AB-01-S018-K	2	
			2-AB-01-S018-L	2	
			2-AB-01-S018-N	2	
			2-AB-01-S018-S	2	
			2-AB-01-S018-W	2	
			2-AB-01-S020-B	2	
			2-AB-01-S020-C	2	
			2-AB-01-S021-C	2	
			2-AB-01-S021-D	2	
			2-AB-01-S023-C	2	
			2-AB-01-S023-D	2	
			2-AB-01-TORSIONAL RESTRAINT-2	2	
		AB-03-01 Sht. 1			
			2-AB-01-F050	3	
			2-AB-01-F051	3	
			2-AB-01-F052	3	
			2-AB-01-F053	2	
			2-AB-01-F054	3	
			2-AB-01-F055	3	
			2-AB-01-F056	3	
			2-AB-01-F057	3	
			2-AB-01-F059	3	
			2-AB-01-F060	3	
			2-AB-01-F061	3	
			2-AB-01-F062	3	
			2-AB-01-F063	3	
			2-AB-01-F101	3	

Exam Category	<u>Number</u>	Drawing	Component	Period	Exam Method
			2-AB-01-FW12	3	
			2-AB-01-FW2-B	3	
			2-AB-01-S032-G	3	
			2-AB-01-S032-H	3	
			2-AB-01-S032-J	3	
			2-AB-01-S032-K	3	
			2-AB-01-S032-L	3	
			2-AB-01-S032-M	3	
			2-AB-01-S032-N	3	
			2-AB-01-S032-S	3	
			2-AB-01-S032-W	3	
			2-AB-01-S032-Z	3	
			2-AB-01-S034-C	3	
			2-AB-01-S034-D	3	
			2-AB-01-S035-B	3	
			2-AB-01-S035-C	3	
			2-# B-01-S037-C	3	
			2-AB-01-S037-E	3	
			2-AB-01-TORSIONAL RESTRAINT-3	3	
		AB-04-01 Sht. 1			
			2-AB-01-F074	1	
			2-AB-01-F075	2	
			2-AB-01-F076	2	
			2-AB-01-F077	2	
			2-AB-01-F078	1	
			2-AB-01-F079	1	
			2-AB-01-F080	1	
			2-AB-01-F081	1	
			2-AB-01-F095	1	
			2-AB-01-FW9	2	
			2-AB-01-S046-G	1	
			2-AB-01-S046-H	1	

Exam Category	Item Number	Drawing	Component	Period	Exam Method
	THE RESERVE OF THE PERSON OF T	200	2-AB-01-S046-J	1	
			2-AB-01-S046-K	1	
			2-AB-01-S046-L	1	
			2-AB-01-S046-M	2	
			2-AB-01-S046-S	2	
			2-AB-01-S046-V	2	
			2-AB-01-S048-C	1	
			2-AB-01-S048-D	1	
			2-AB-01-S049-C	1	
			2-AB-01-S049-D	1	
			2-AB-01-TORSIONAL RESTRAINT-4	2	
		AE-01-04 Sht. 1			
			2-AE-04-F001	2	
			2-AE-04-F004	1	
			2-AE-04-F005	1	
			2-AE-04-F006	1	
			2-AE-04-F007	1	
			2-AE-04-F008	1	
			2-AE-04-F009	1	
			2-AE-04-F031	1	
			2-AE-04-F039	1	
			2-AE-04-F042	1	
			2-AE-04-FW1	1	
			2-AE-04-FW5	1	
			2-AE-04-FW9	1	
			2-AE-04-S004-A	1	
			2-AE-04-S025-A	1	
			2-AE-04-S025-B	1	
			2-AE-04-S025-C	1	
		AE-02-04 Sht. 1			
			2-AE-04-F016	2	
			2-AE-04-F019	2	

Exam Category	Item Number	Drawing	Component	Period	Exam Method
			2-AE-04-F020	2	APPENDING TO THE SHEET HIS TON
			2-AE-04-F021	2	
			2-AE-04-F022	2	
			2-AE-04-F023	2	
			2-AE-04-F024	2	
			2-AE-04-F033	2	
			2-AE-04-F040	2	
			2-AE-04-F043	2	
			2-AE-04-FW2	2	
			2-AE-04-FW6	2	
			2-AE-04-S014-A	2	
			2-AE-04-S026-A	2	
			2-AE-04-S026-B	2	
			2-AE-04-S026-C	2	
		AE-03-05 Sht.	1		
			2-AE-05-F016	2	
			2-AE-05-F019	3	
			2-AE-05-F020	3	
			2-AE-05-F021	3	
			2-AE-05-F022	3	
			2-AE-05-F023	3	
			2-AE-05-F024	3	
			2-AE-05-F033	3	
			2-AE-05-F037	3	
			2-AE-05-F043	3	
			2-AE-05-FW1	3	
			2-AE-05-FW3	3	
			2-AE-05-FW4	3	
			2-AE-05-S014-A	3	
			2-AE-05-S025-B	3	
		AE-04-05 Sht.	1		
			2-AE-05-F001	2	

Exam Category	<u>Item</u> Number	Drawing	Component	Period	Exam Method
			2-AE-05-F004	3	ALAS MESIAS TERMINANTAS ARTISTA
			2-AE-05-F005	3	
			2-AE-05-F006	3	
			2-AE-05-F007	3	
			2-AE-05-F008	3	
			2-AE-05-F009	3	
			2-AE-05-F031	1	
			2-AE-05-F038	1	
			2-AE-05-F041	3	
			2-AE-05-FW2	3	
			2-AE-05-FW5	1	
			2-AE-05-S004-A	1	
			2-AE-05-S026-A	1	
		BM-01-01 Sht	1		
			2-BM-01-F001	1	
			2-BM-01-F003	2	
			2-BM-01-F004	3	
			2-BM-01-F006	2	
			2-BM-01-F007	1	
			2-BM-01-F008	1	
			2-BM-01-F009	1	
			2-BM-01-F010	1	
			2-BM-01-F011	1	
			2-BM-01-F017	1	
			2-BM-01-F018	1	
			2-BM-01-F019	3	
			2-BM-01-F020	3	
			2-BM-01-F021	3	
			2-BM-01-F027	1	
			2-BM-01-F028	1	
			2-BM-01-F029	1	
			2-BM-01-F030	1	

Exam Category	Item Number	Drawing	Component	Period	Exam Method
			2-BM-01-S003-D	1	BOST PARTITION OF THE
		BM-01-02 Sht.	1		
			2-BM-02-F001	3	
			2-BM-02-F003	2	
			2-BM-02-F004	2	
			2-BM-02-F006	2	
			2-BM-02-F007	3	
			2-BM-02-F008	3	
			2-BM-02-F009	3	
			2-BM-02-F010	3	
			2-BM-02-F011	3	
			2-BM-02-F018	2	
			2-BM-02-F019	2	
			2-BM-02-F020	2	
			2-BM-02-F021	2	
			2-BM-02-F022	2	
			2-BM-02-F030	3	
			2-BM-02-F031	2	
			2-BM-02-F032	2	
			2-BM-02-FW-2	3	
		FC-01-01 Sht.	1		
			2-FC-01-F020	2	
			2-FC-01-F021	1	
			2-FC-01-F022	1	
			2-FC-01-F023	1	
			2-FC-01-F024	1	
			2-FC-01-F025	3	
			2-FC-01-F026	3	
			2-FC-01-F027	3	
			2-FC-01-F028	3	
			2-FC-01-F029	2	
			2-FC-01-F030	2	

Exam Category		Drawing	Component	Period	Exam Method
AR AND DE TRANSPORTE DE LE SERVICIO			2-FC-01-S018-B	1	
			2-FC-01-S018-C	1	
			2-FC-01-S021-B	3	
			2-FC-01-S021-C	3	
			2-FC-01-S022-C	2	
			2-FC-01-S022-E	2	
	NBZ EXAMS -		WELDS IN PIPING ONE AND ONE-HALF NPS AND GREATER		SURFACE
		AB-01-01 Sht.	1		
			2-AB-01-FW022	1	
			2-AB-01-FW023	1	
			2-AB-01-FW024	1	
			2-AB-01-FW025	1	
			2-AB-01-FW026	1	
			2-AB-01-FW027	1	
			2-AB-01-FW265	1	
			2-AB-01-FW266	1	
			2-AB-01-FW267	1	
			2-AB-01-FW268	. 1	
			2-AB-01-FW269	1	
			2-AB-01-FW270	1	
			2-AB-01-FW271	1	
			2-AB-01-FW272	1	
			2-AB-01-FW273	1	
			2-AB-01-FW274	1	
			2-AB-01-FW275	1	
			2-AB-01-FW276	1	
			2-AB-01-FW277	1	
			2-AB-01-FW278	1	
			2-AB-01-FW279	1	
			2-AB-01-FW280	1	

Exam Category	<u>Number</u>	Drawing	Component	Period	Exam Method
OCCUPANTATION ASSUMED		AB-02-01 Sht.	1		ACTIVAÇÃOS EN ASTAPOS EN ASTA EM
			2-AB-01-FW038	2	
			2-AB-01-FW039	2	
			2-AB-01-FW040	2	
			2-AB-01-FW041	2	
			2-AB-01-FW042	2	
			2-AB-01-FW043	2	
			2-AB-01-FW281	2	
			2-AB-01-FW289	2	
			2-AB-01-FW290	2	
			2-AB-01-FW291	2	
			2-AB-01-FW292	2	
			2-AB-01-FW293	2	
			2-AB-01-FW294	2	
			2-AB-01-FW295	2	
			2-AB-01-FW296	2	
			2-AB-01-FW297	2	
			2-AB-01-FW298	2	
			2-AB-01-FW299	2	
			2-AB-01-FW300	2	
			2-AB-01-FW301	2	
			2-AB-01-FW302	2	
			2-AB-01-FW303	2	
		AB-03-01 Sht.	1		
			2-AB-01-FW054	3	
			2-AB-01-FW055	3	
			2-AB-01-FW056	3	
			2-AB-01-FW057	3	
			2-AB-01-FW058	3	
			2-AB-01-FW059	3	
			2-AB-01-FW304	3	
			2-AB-01-FW305	3	

Exam Category	Item Number	Drawing	Componert	Period	Exam Method
			2-AB-01-FW306	3	
			2-AB-01-FW307	3	
			2-AB-01-FW308	3	
			2-AB-01-FW309	3	
			2-AB-01-FW310	3	
			2-AB-01-FW311	3	
			2-AB-01-FW312	3	
			2-AB-01-FW313	3	
			2-AB-01-FW314	3	
			2-AB-01-FW315	3	
			2-AB-01-FW316	3	
			2-AB-01-FW317	3	
			2-AB-01-FW318	3	
			2-AB-01-FW319	3	
		AB-04-01 Sht.	1		
			2-AB-01-FW006	3	
			2-AB-01-FW007	3	
			2-AB-01-FW008	3	
			2-AB-01-FW009	2	
			2-AB-01-FW010	2	
			2-AB-01-FW011	2	
			2-AB-01-FW1	2	
			2-AB-01-FW2	2	
			2-AB-01-FW249	2	
			2-AB-01-FW250	3	
			2-AB-01-FW251	3	
			2-AB-01-FW252	3	
			2-AB-01-FW253	3	
			2-AB-01-FW254	3	
			2-AB-01-FW255	3	
			2-AB-01-FW256	3	
			2-AB-01-FW257	3	

Exam Category	Item Number	Drawing	Component	Period	Exam Method
Common trongs (Academic			2-AB-01-FW258	2	
			2-AB-01-FW259	2	
			2-AB-01-FW260	2	
			2-AB-01-FW261	2	
			2-AB-01-FW262	2	
			2-AB-01-FW263	2	
			2-AB-01-FW264	2	

Regulatory Guide 1.14 Flywheel Inspections

11-Aug-95

	Component	Period	Exam Method
REG GUIDE 1.14			
	RCP FLYWHEEL AT EXPOSED SURFACES		SURFACE
	2-PBB01A-FLYWHEEL-R1	3	
	2-PBB01B-FLYWHEEL-R1	3	
	2-PBB01C-FLYWHEEL-R1	3	
	2-PBB01D-FLYWHEEL	3	
	RCP FLYWHEEL IN AREAS OF HIGHER STRESS CONCENTRATION AT THE BORE AND KEYWAY (See Note 1)		VOLUMETRIC
	2-PBB01A-FLYWHEEL-R1	123	
	2-PBB01B-FLYWHEEL-R1	123	
	2-PBB01C-FLYWHEEL-R1	123	
	2-PBB01D-FLYWHEEL	123	

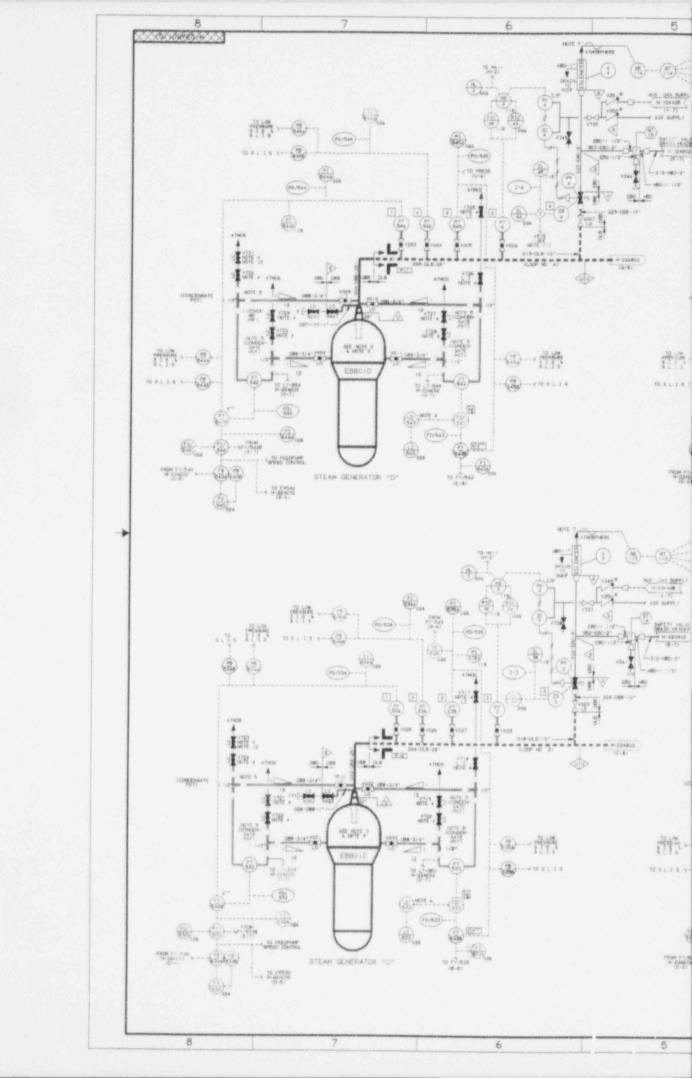
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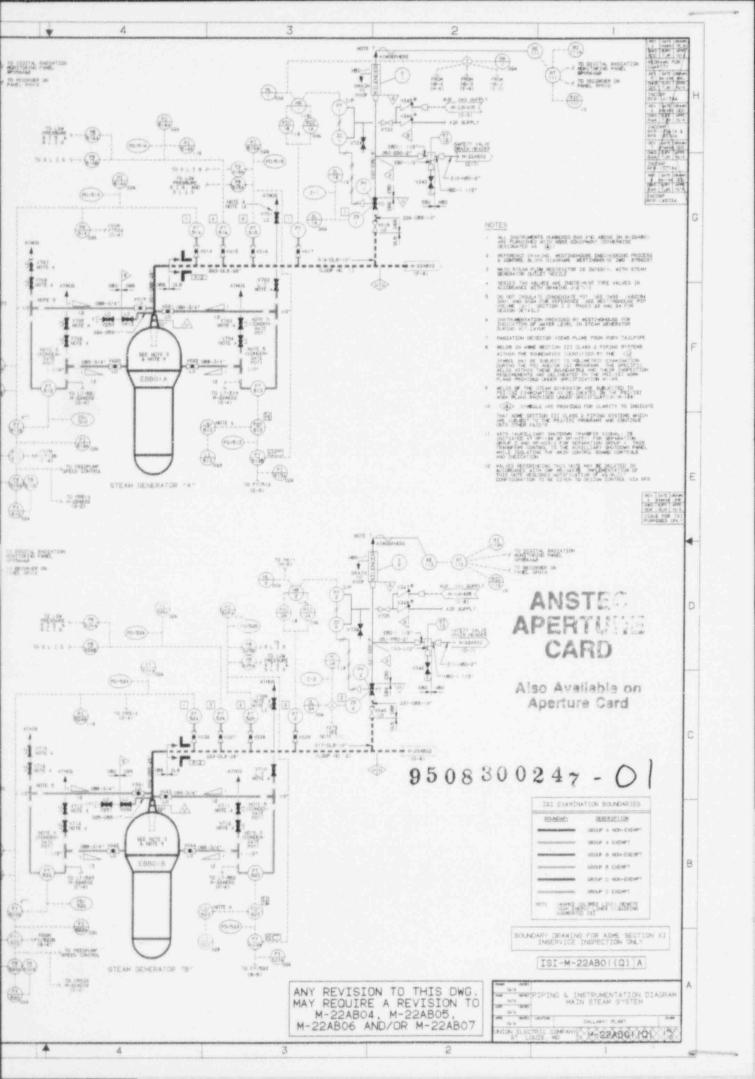
 A volumetric examination of the entire volume of the flywheel will be performed in the third period.

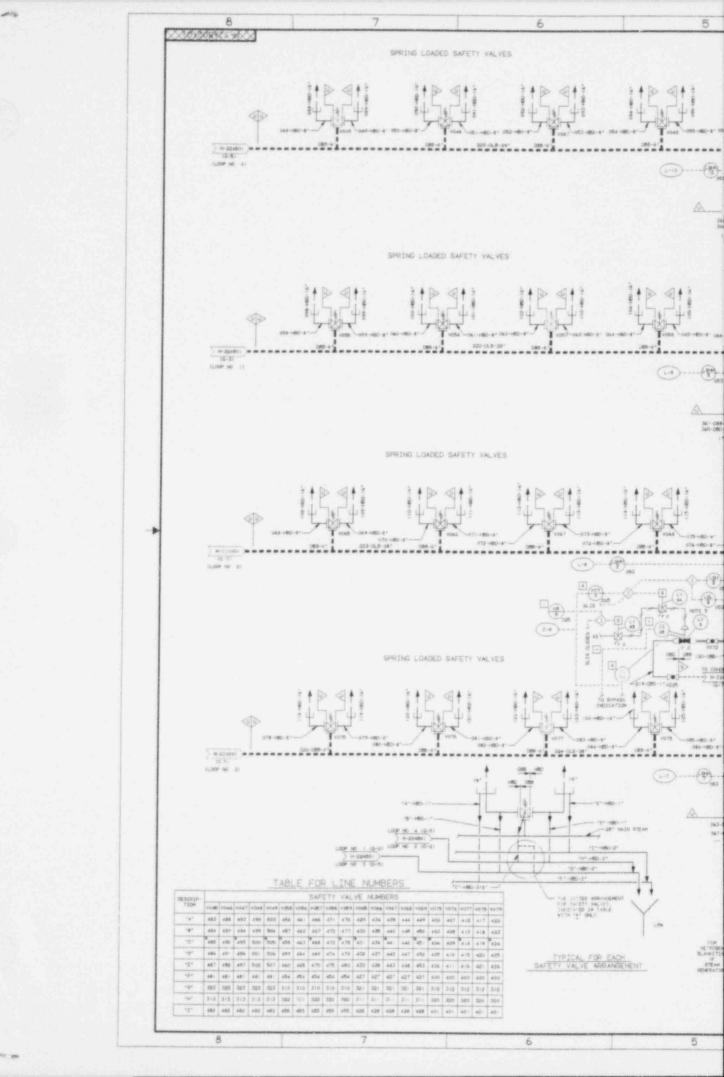
Additional Information Second Ten-Year Interval Inservice Inspection Plan Callaway Nuclear Power Plant, Unit 1 Docket No. 50-483

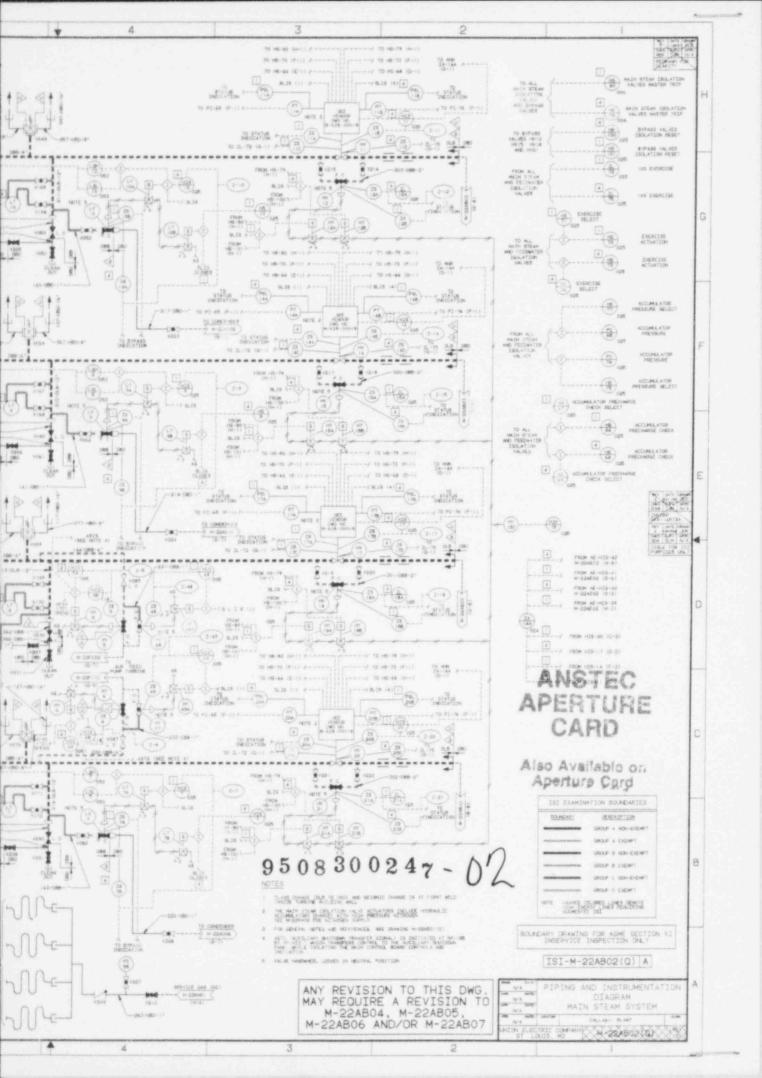
ATTACHMENT B-1

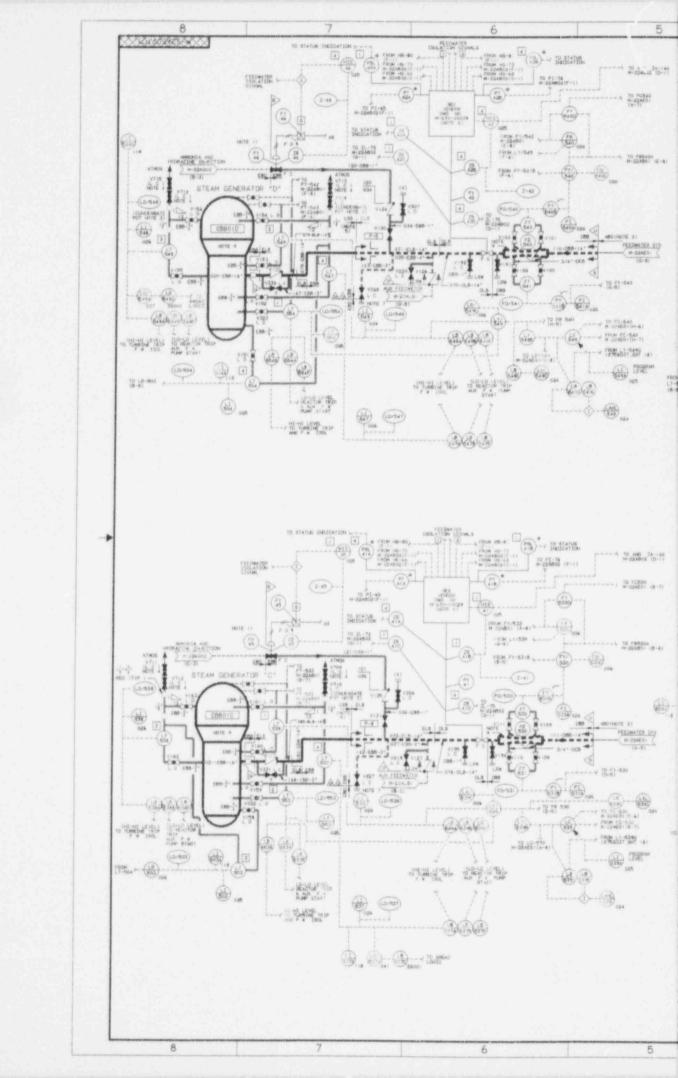
INSERVICE INSPECTION BOUNDARY DRAWINGS

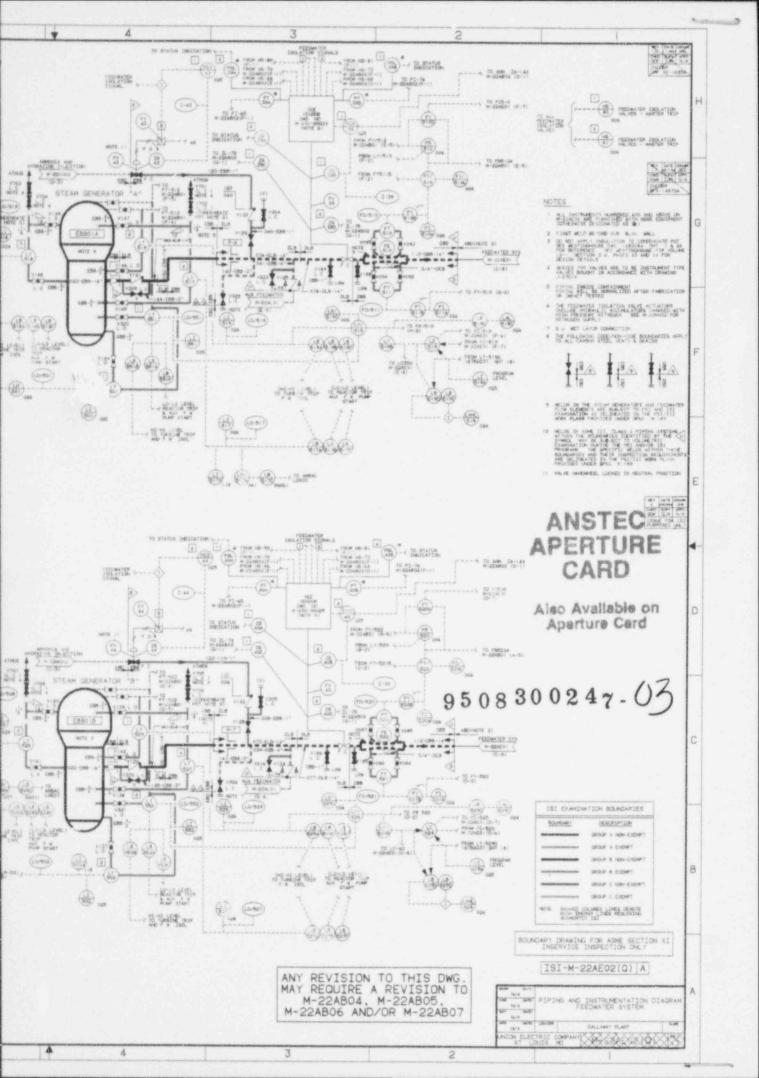


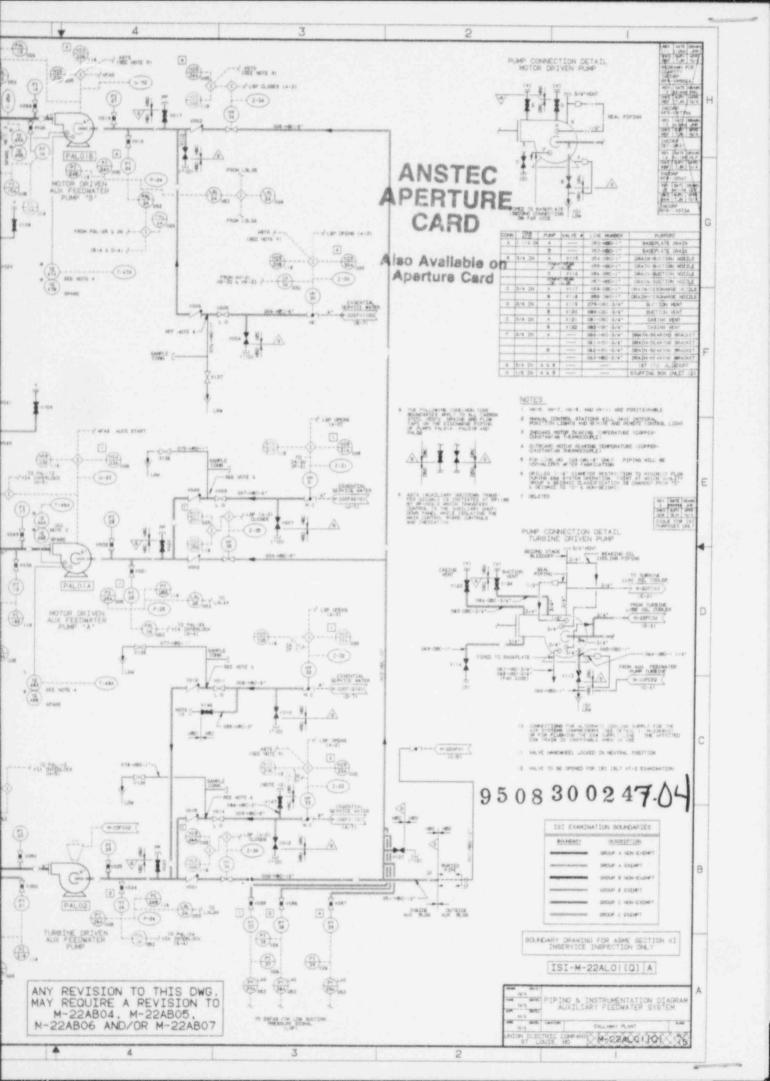


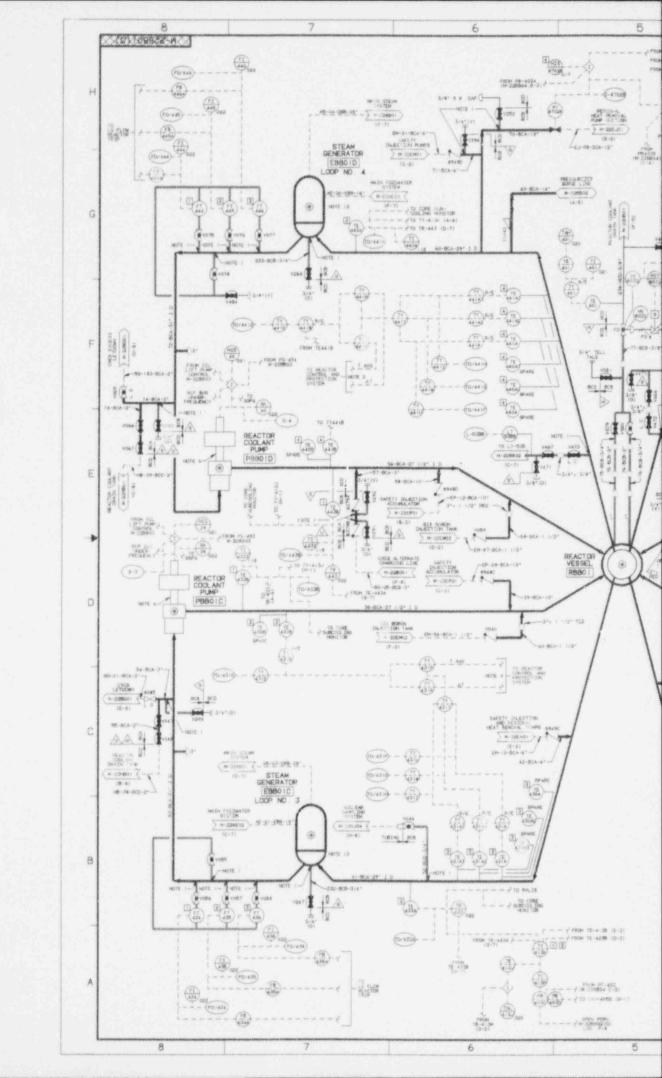


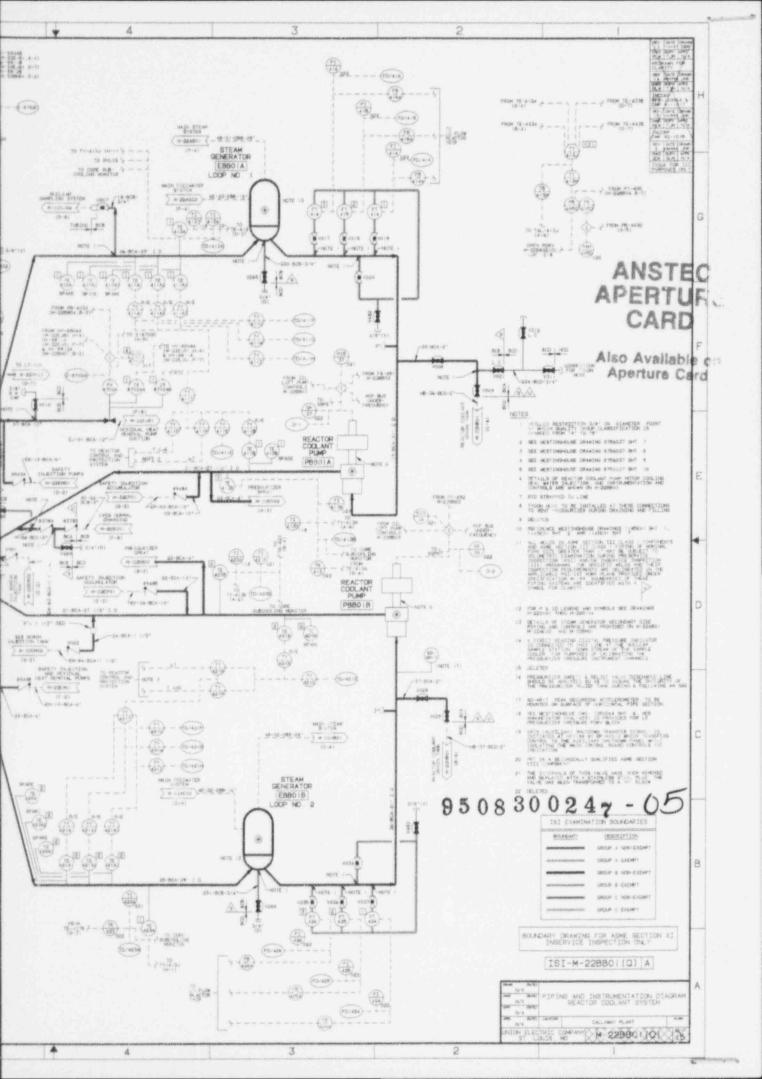


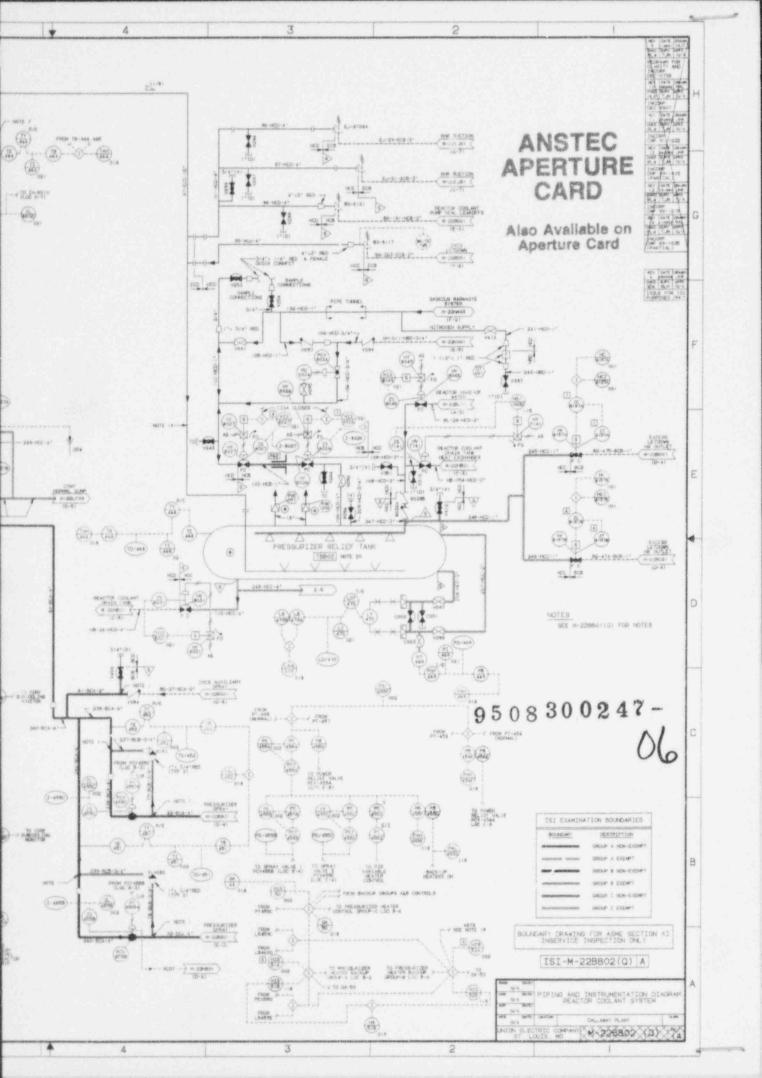


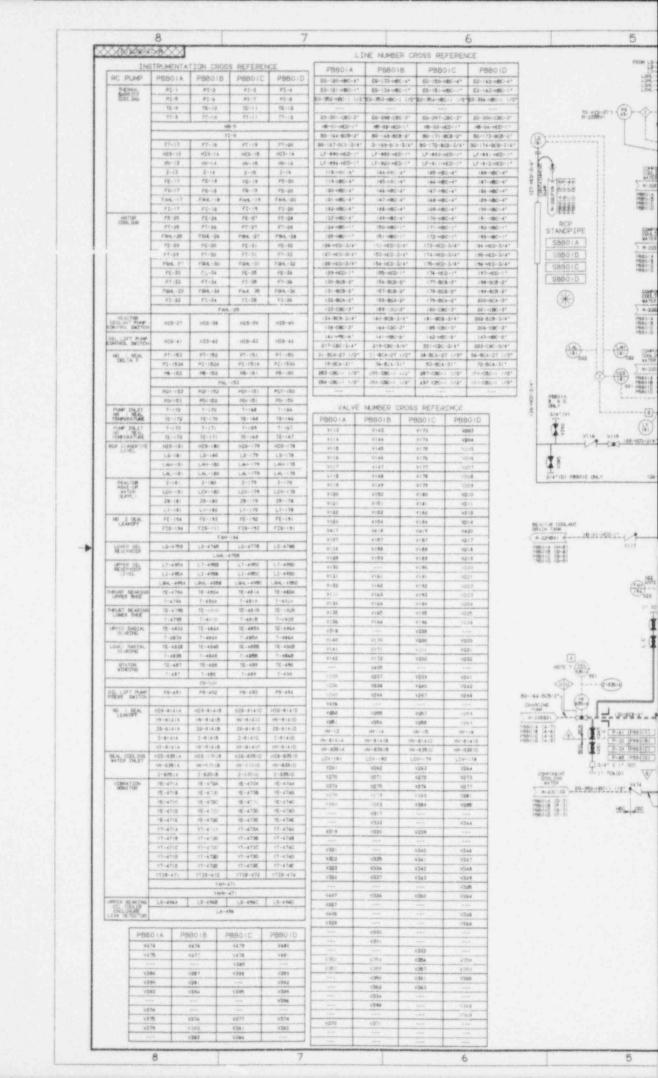


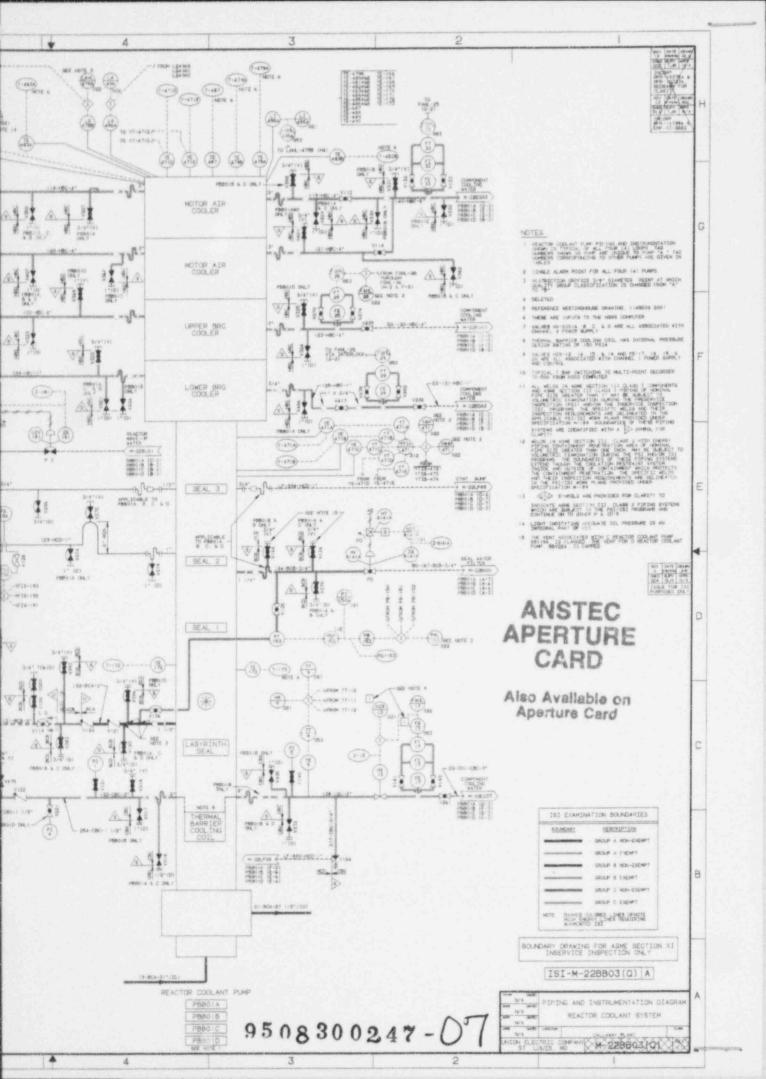


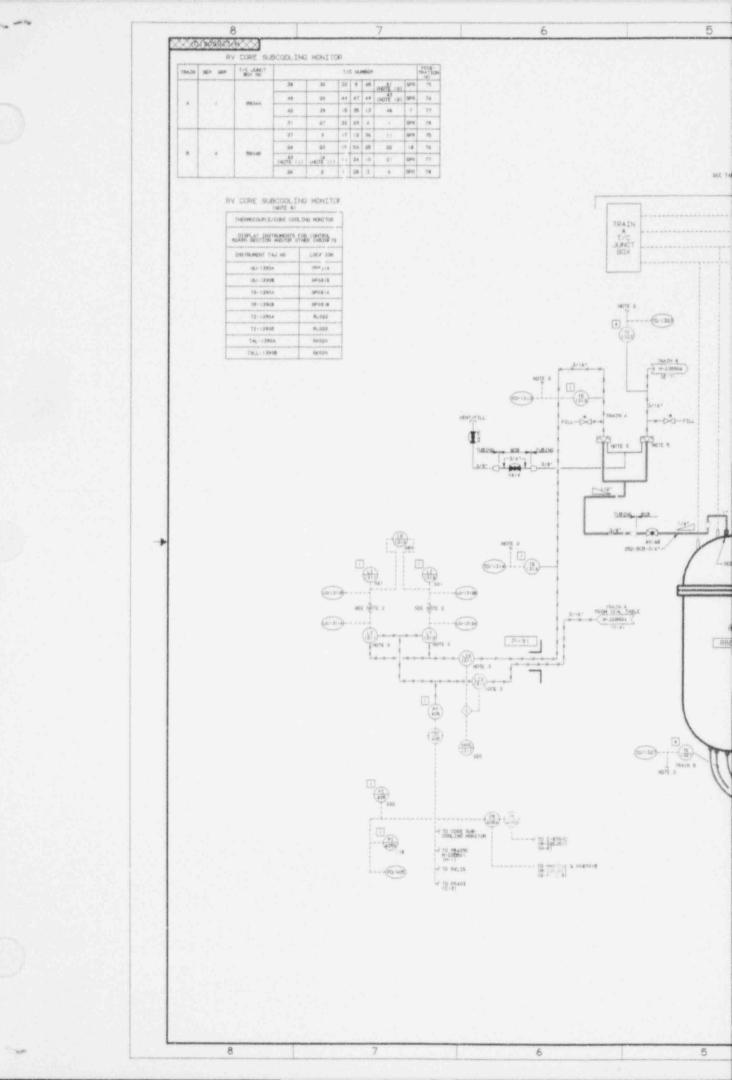


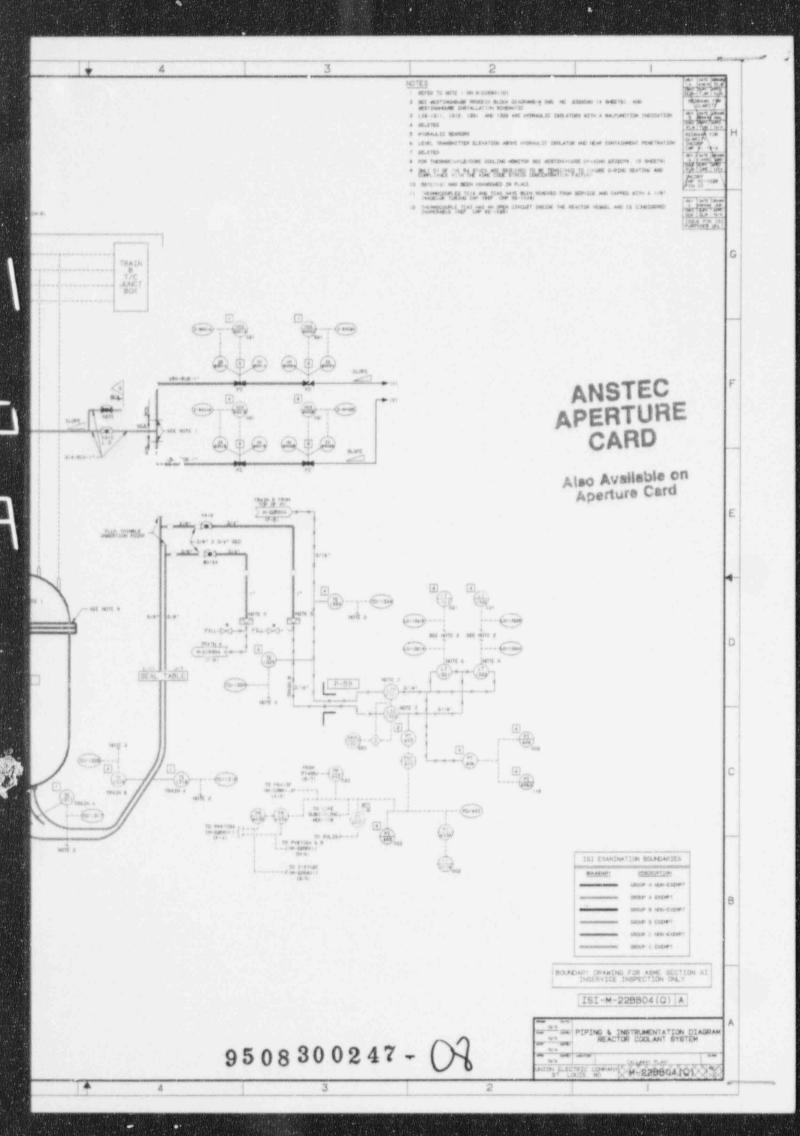




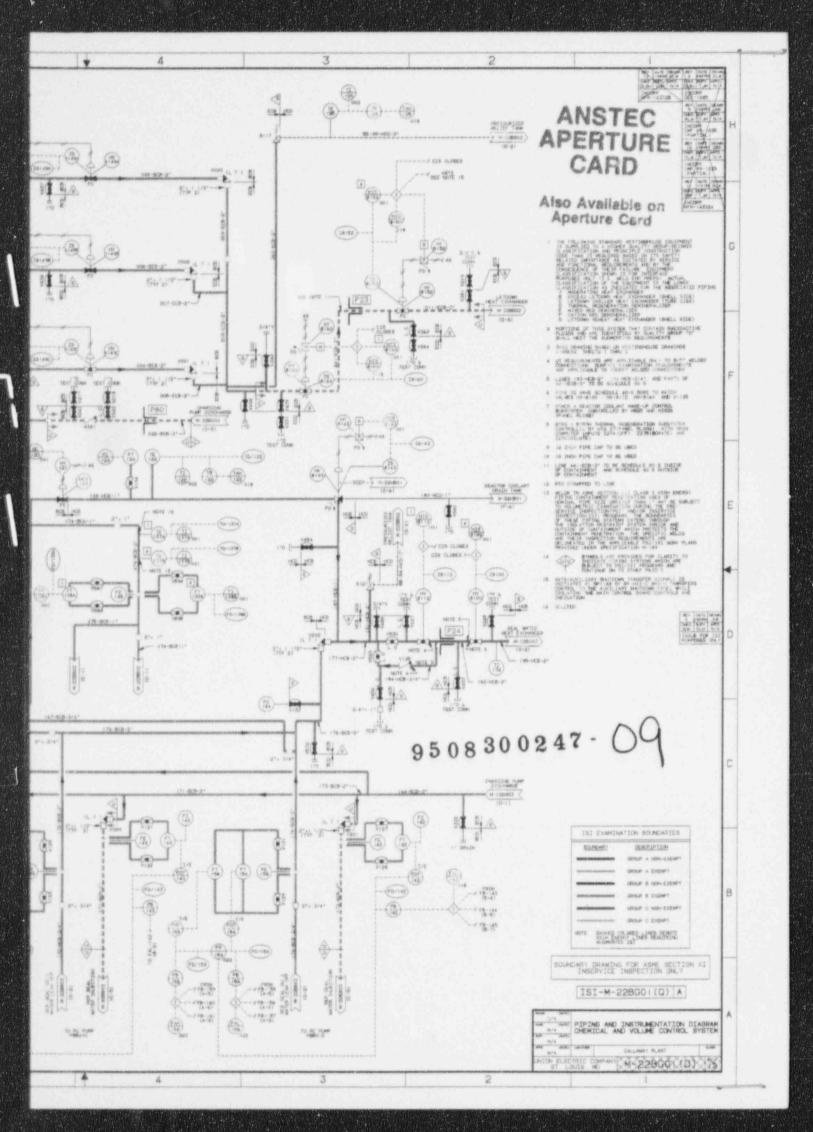




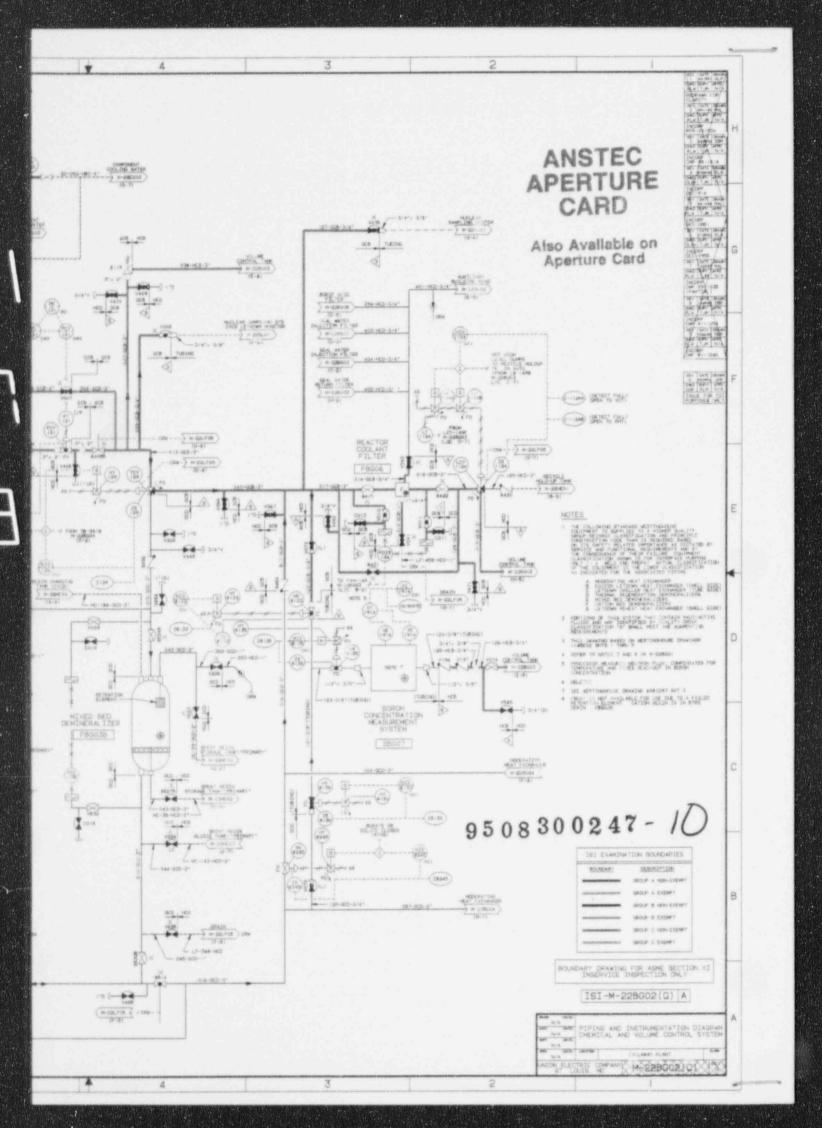


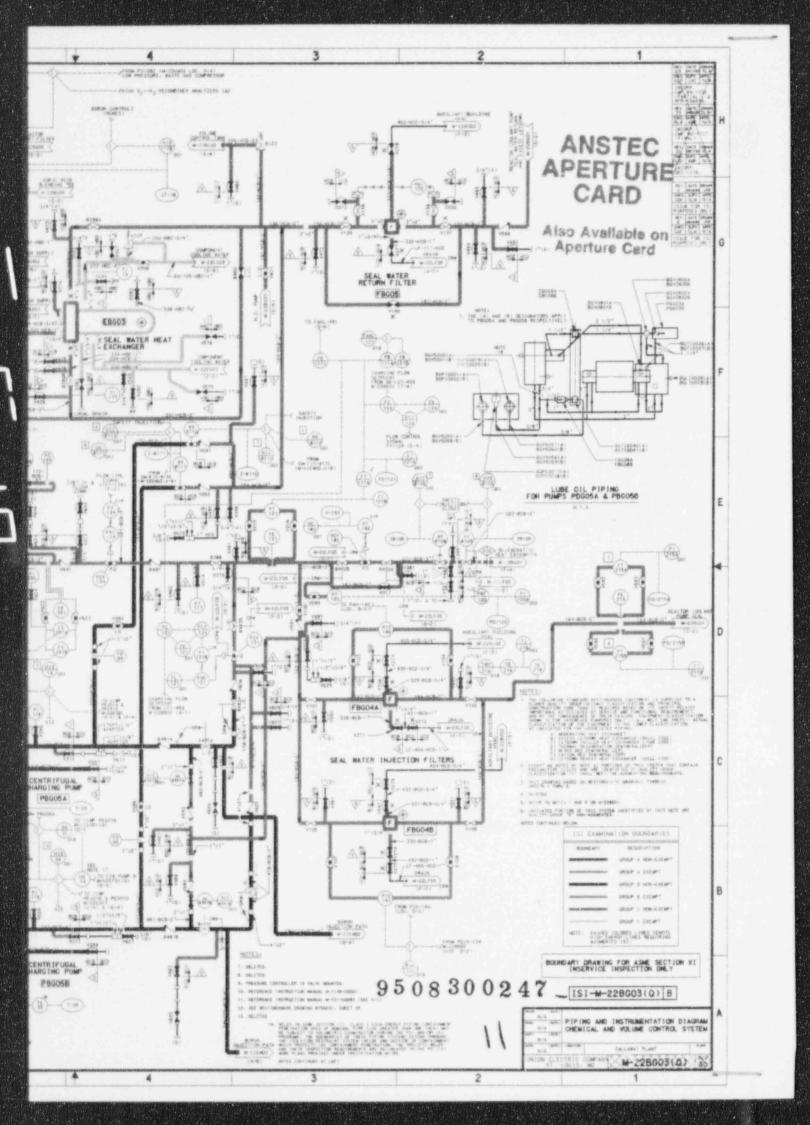


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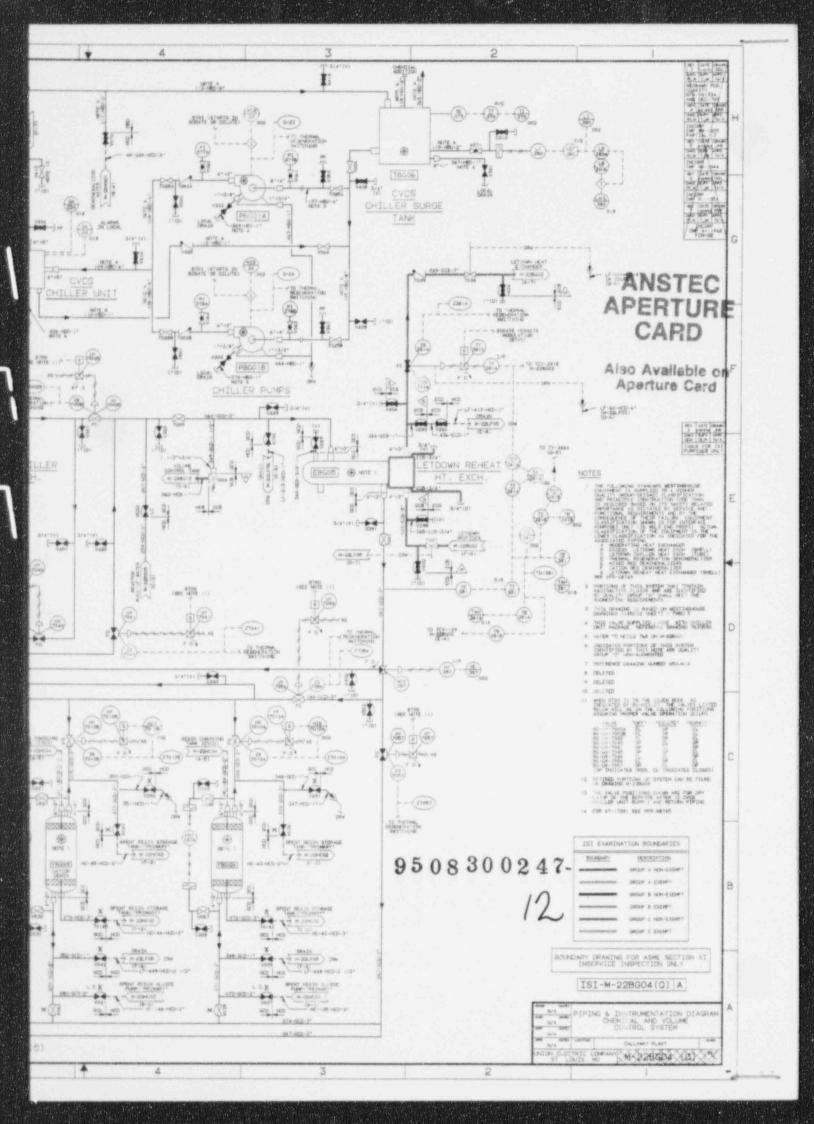


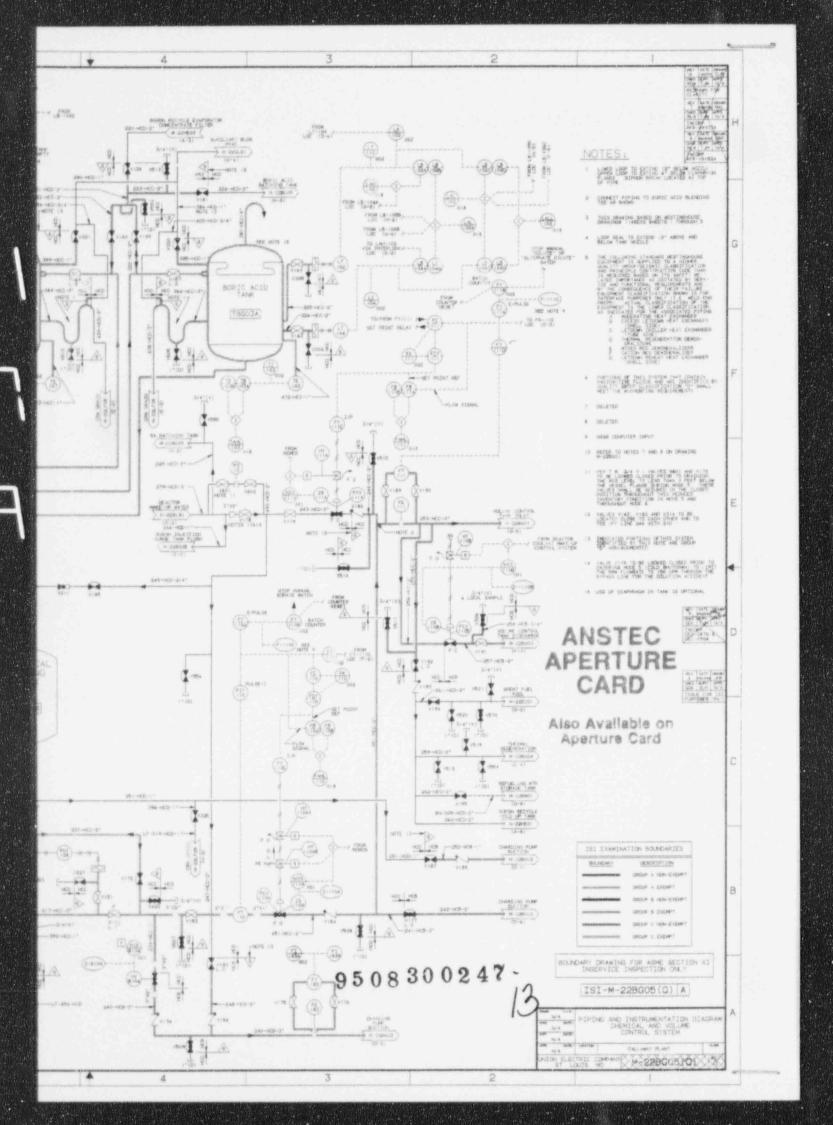
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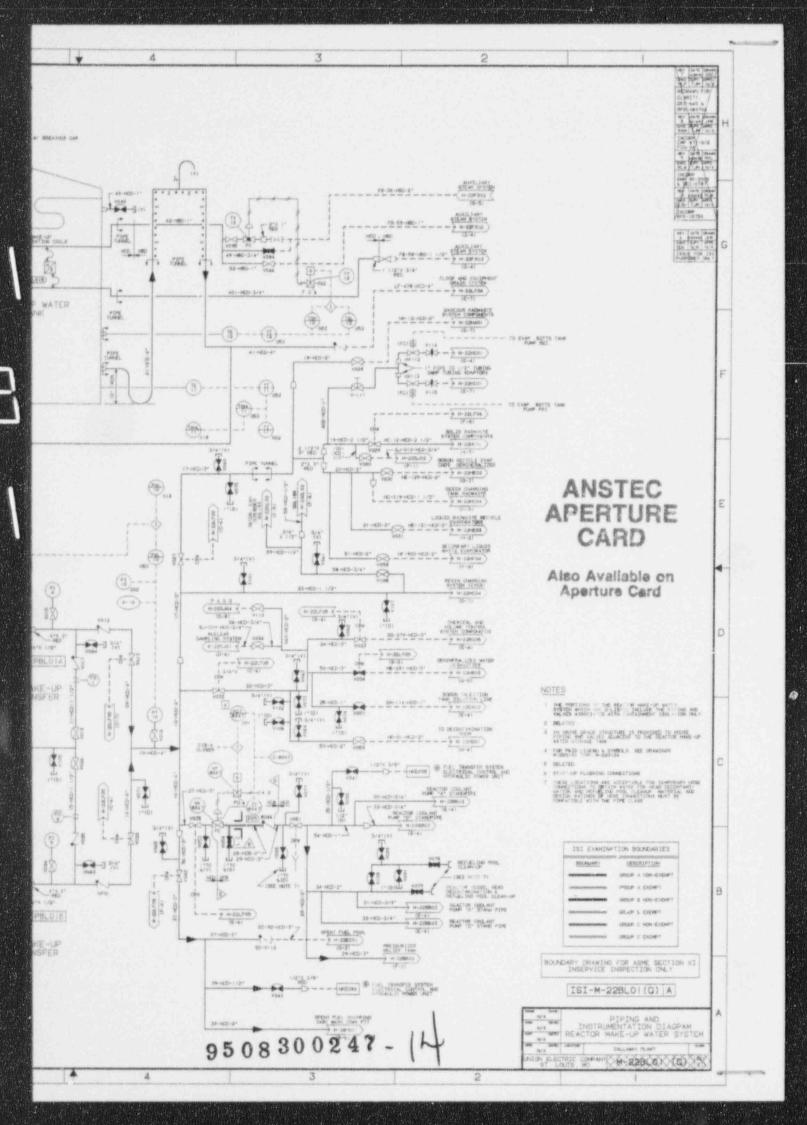


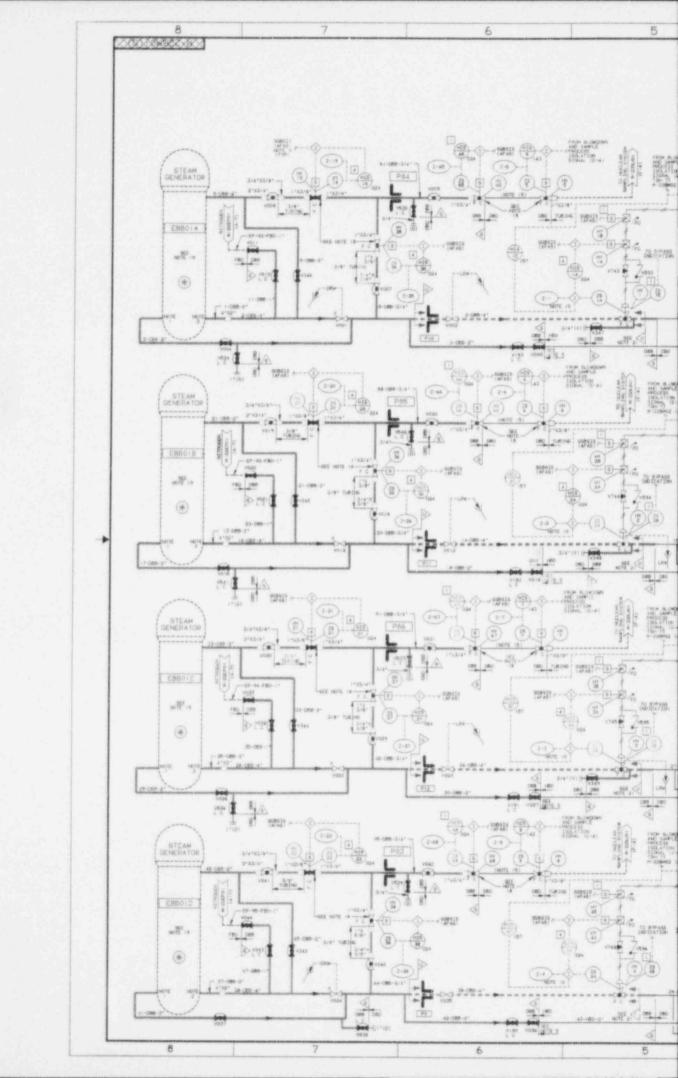


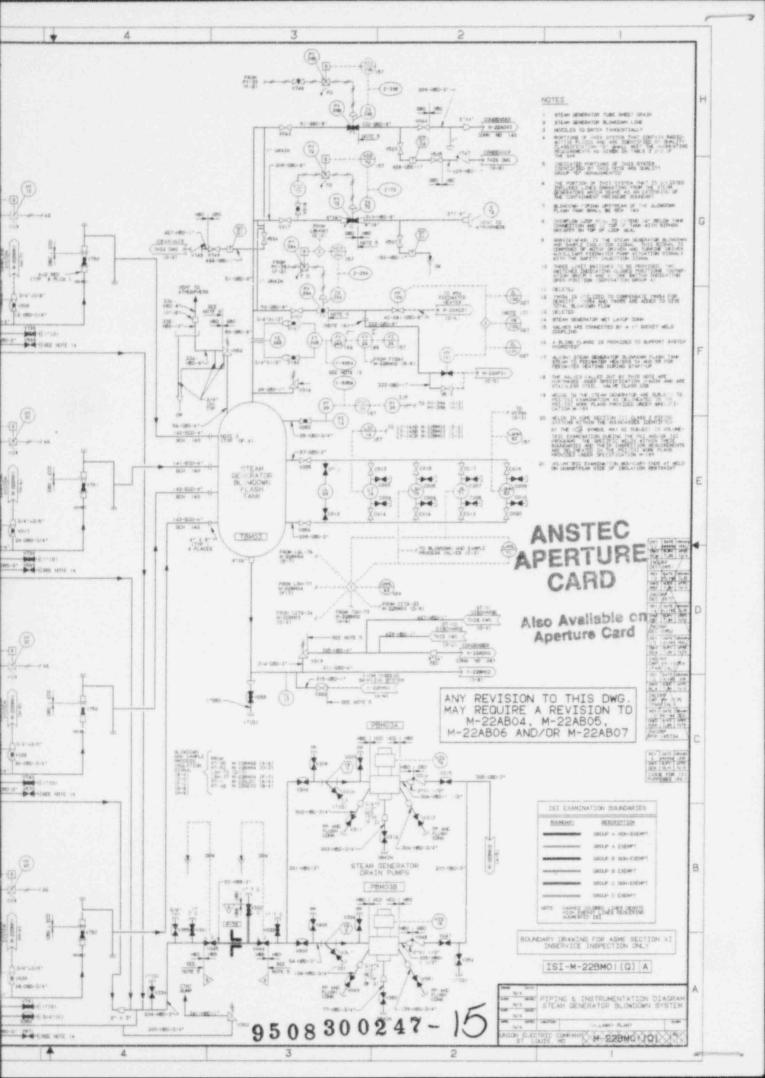
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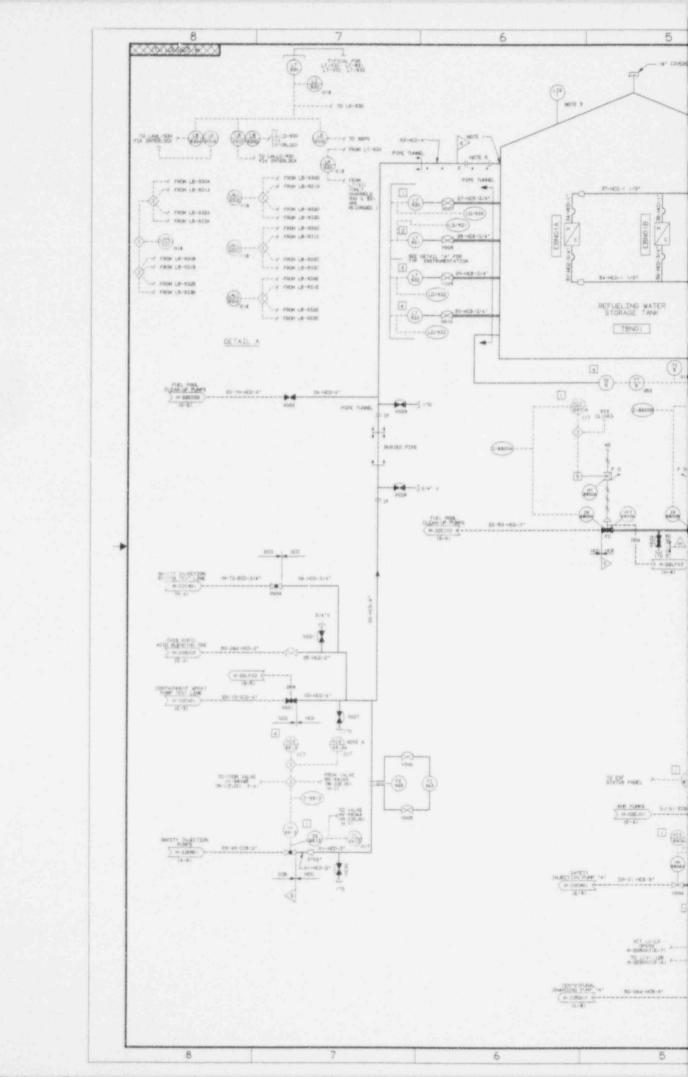


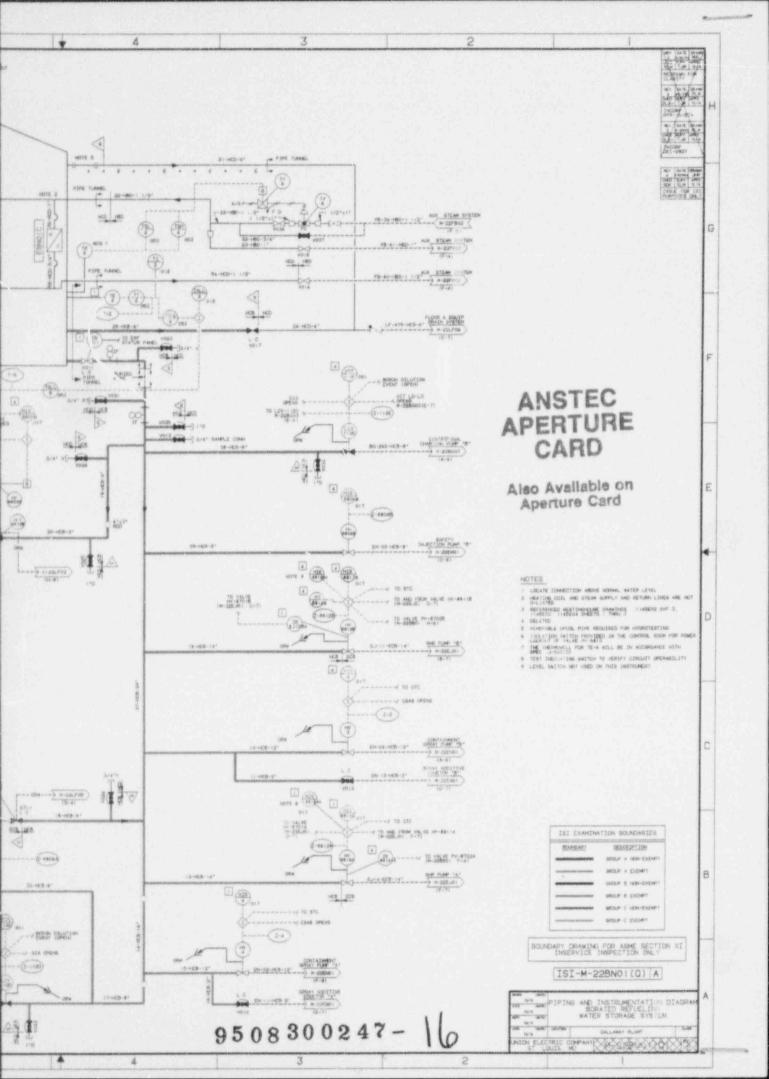


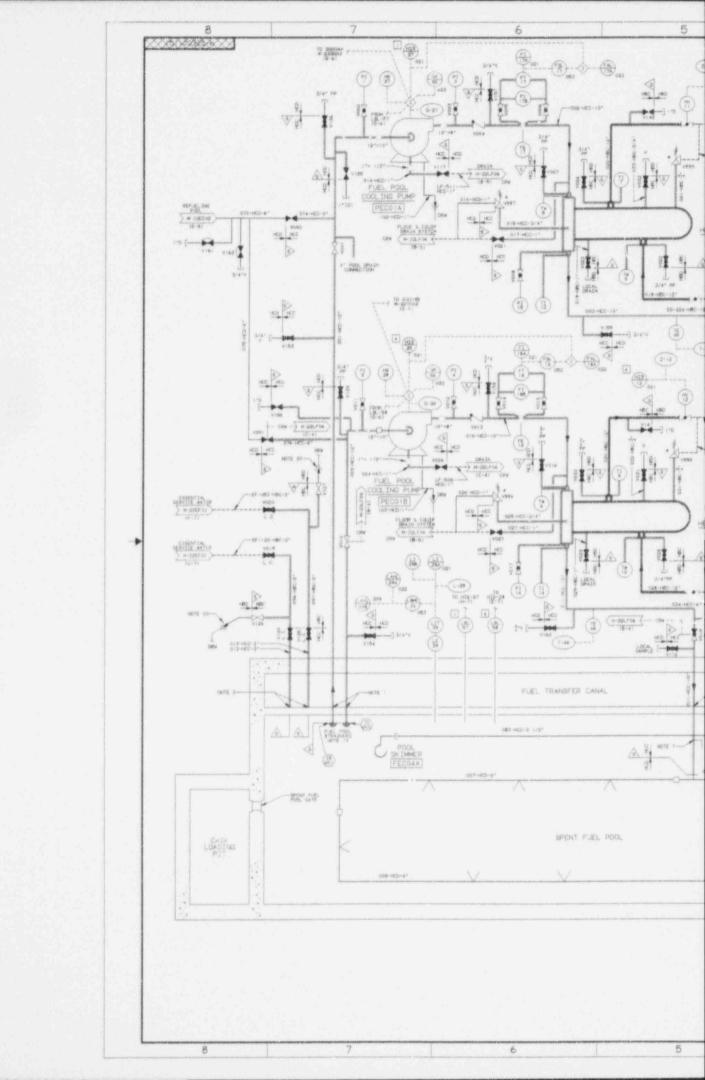


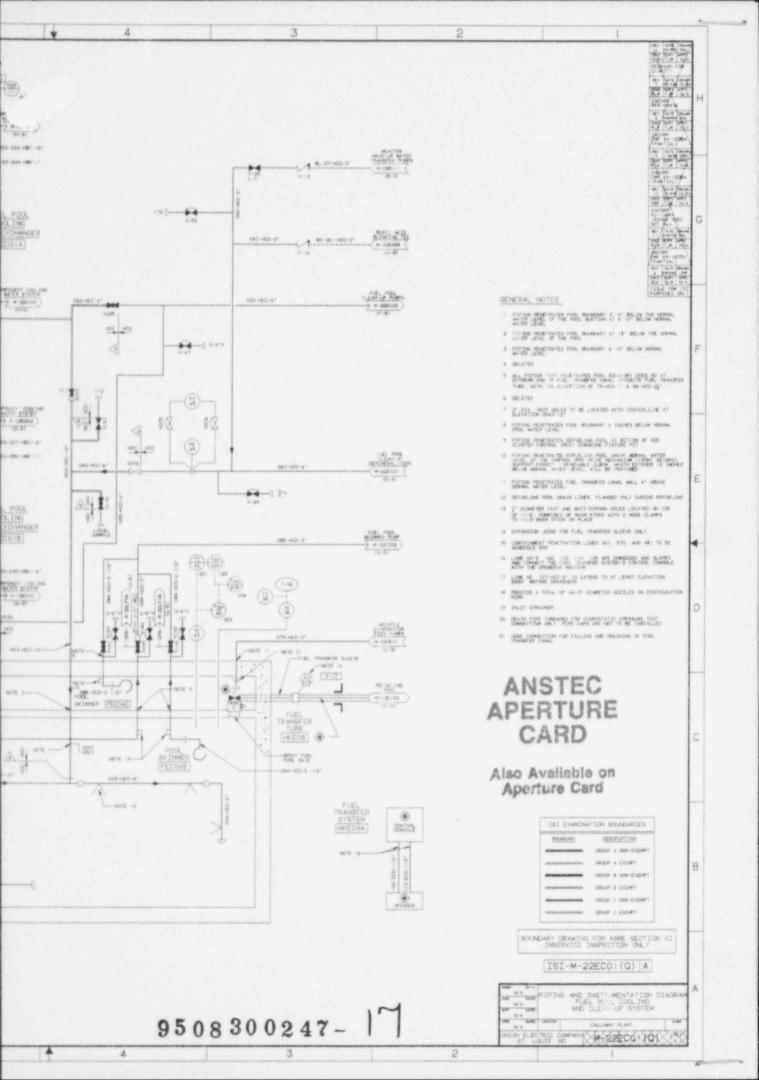


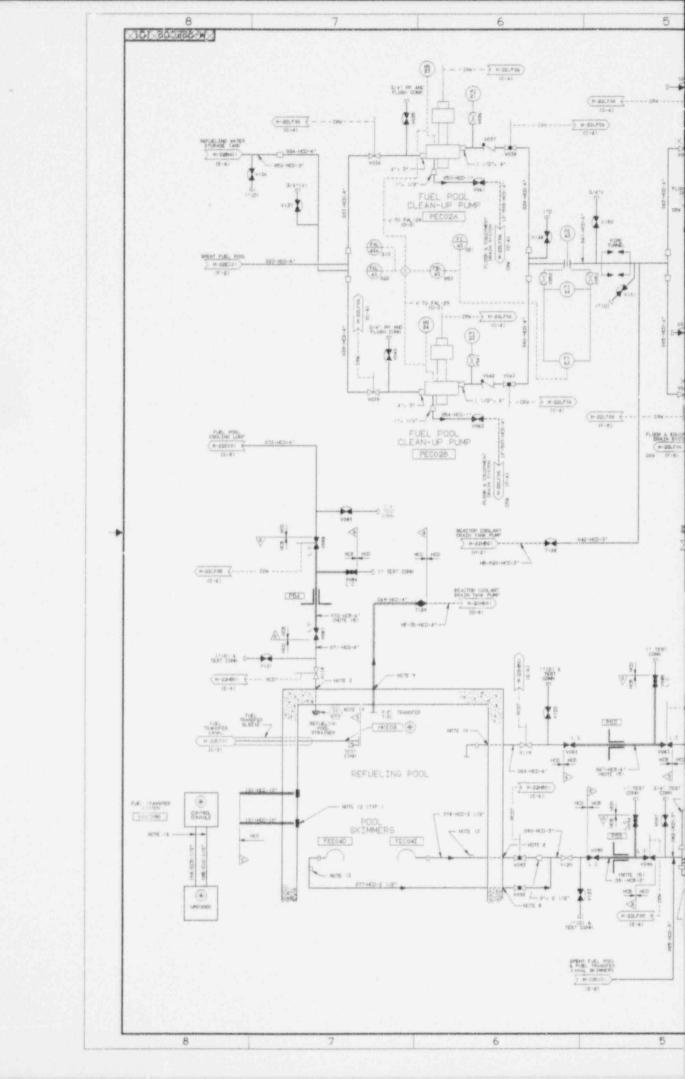


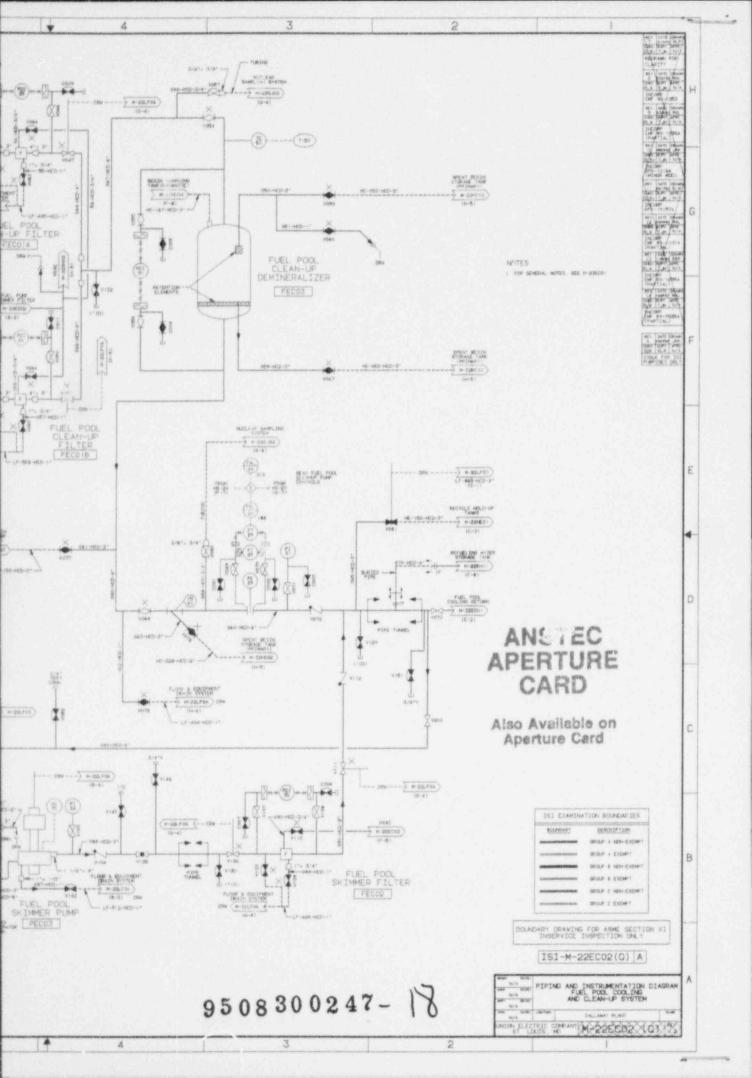


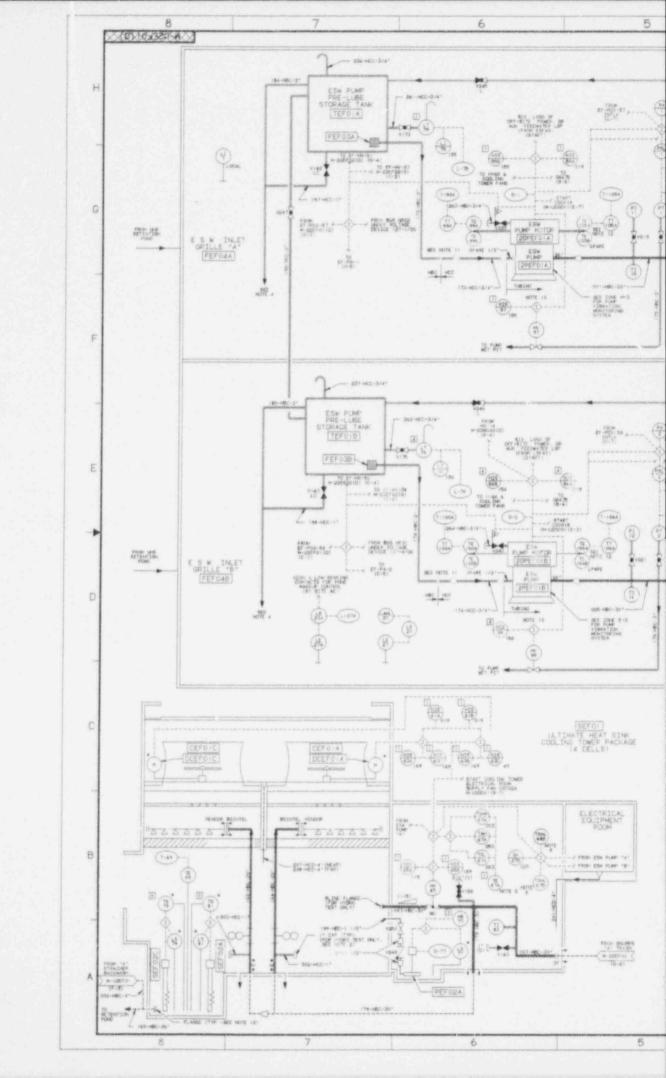


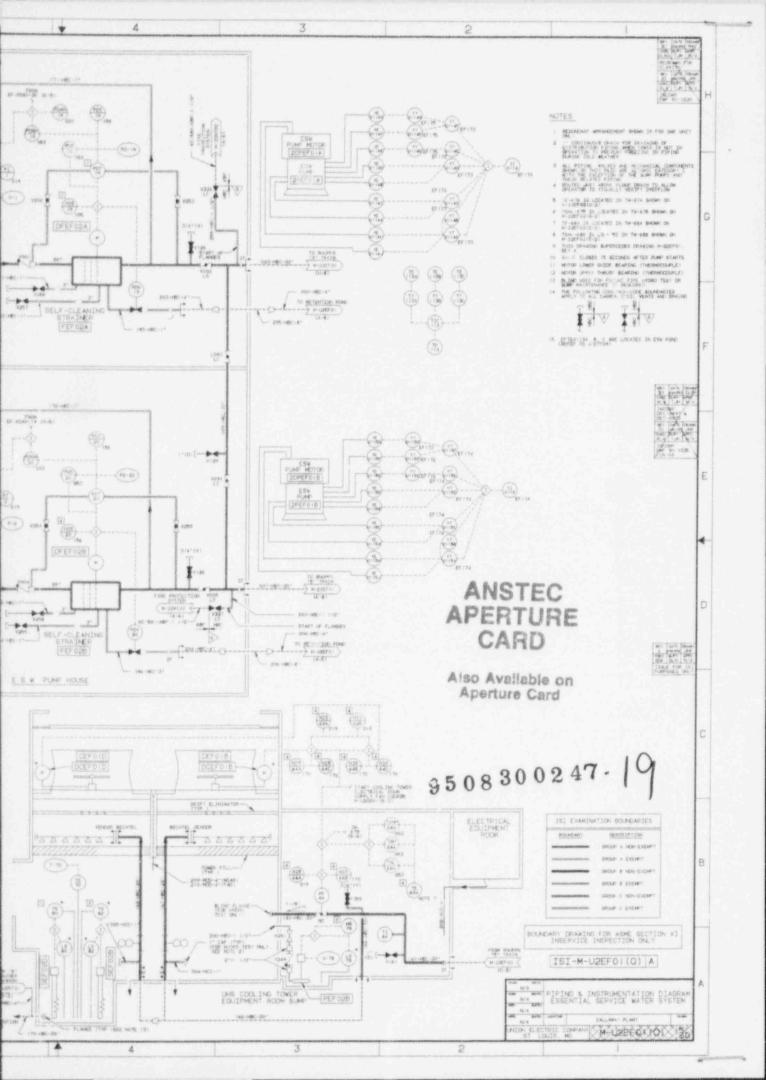


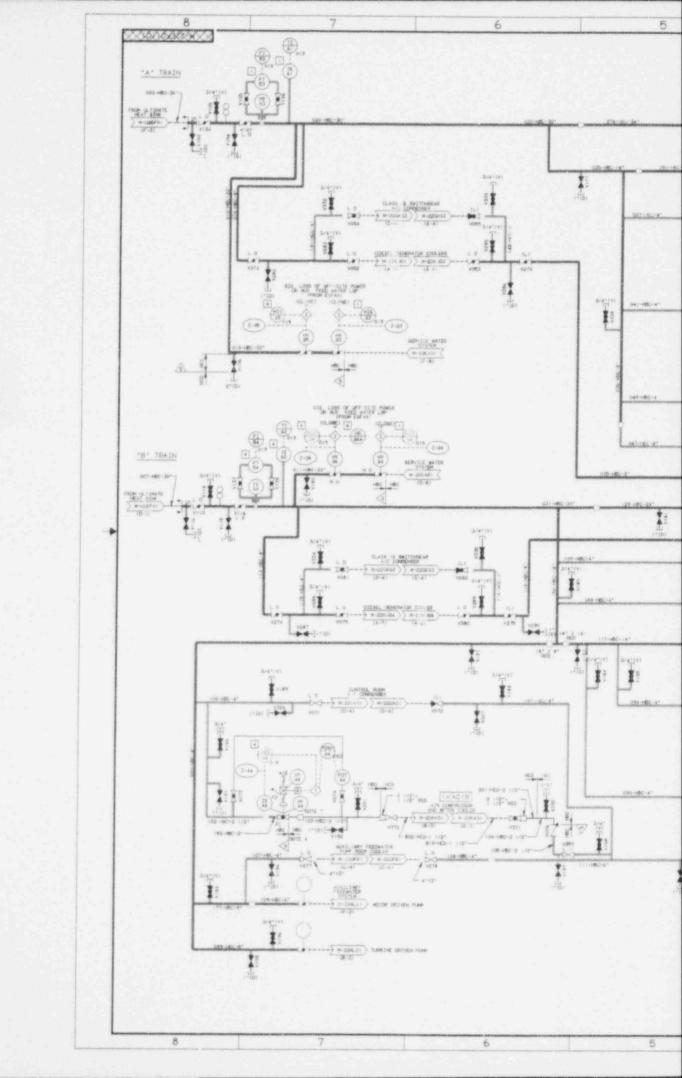


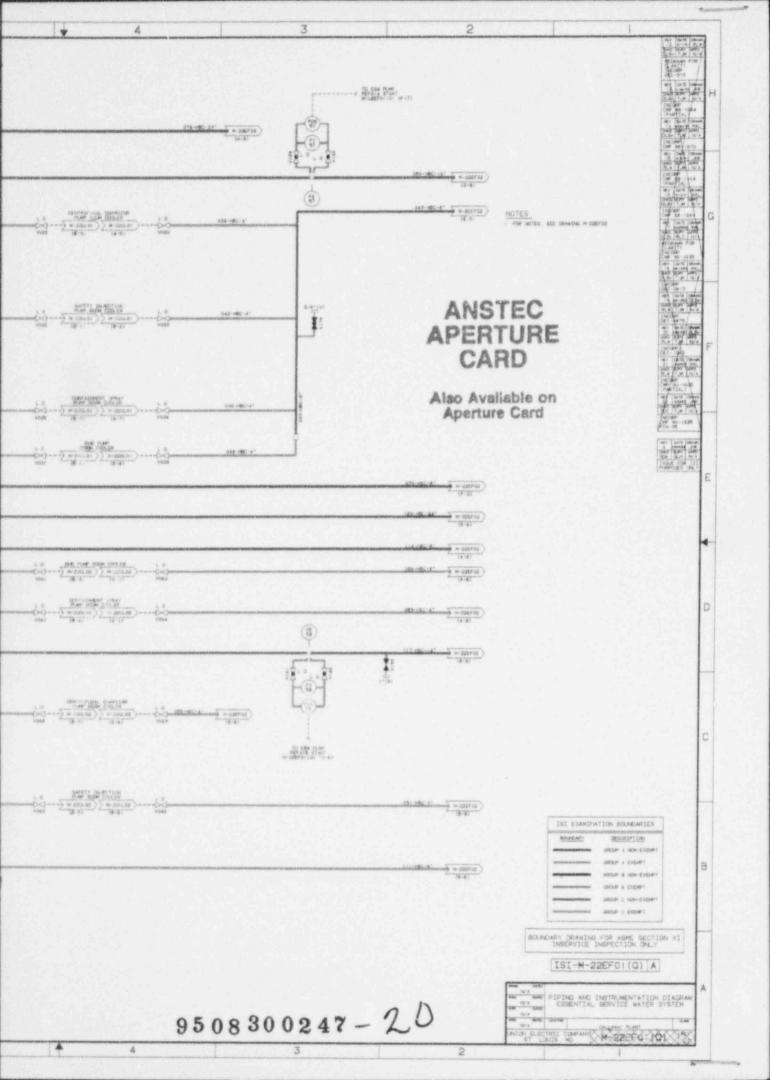


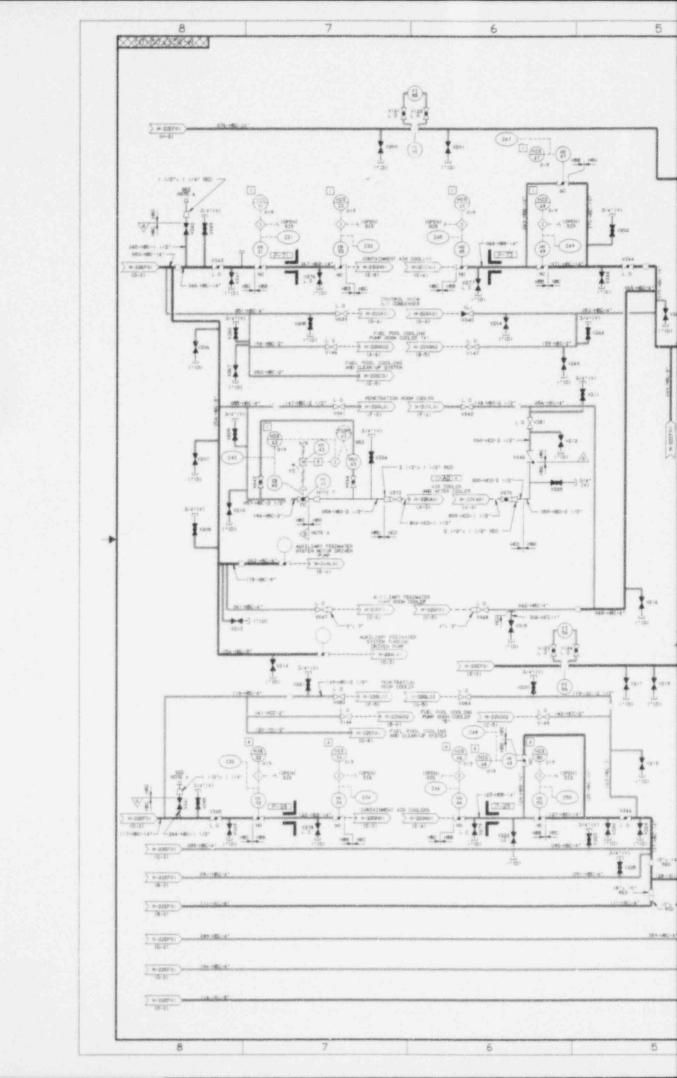


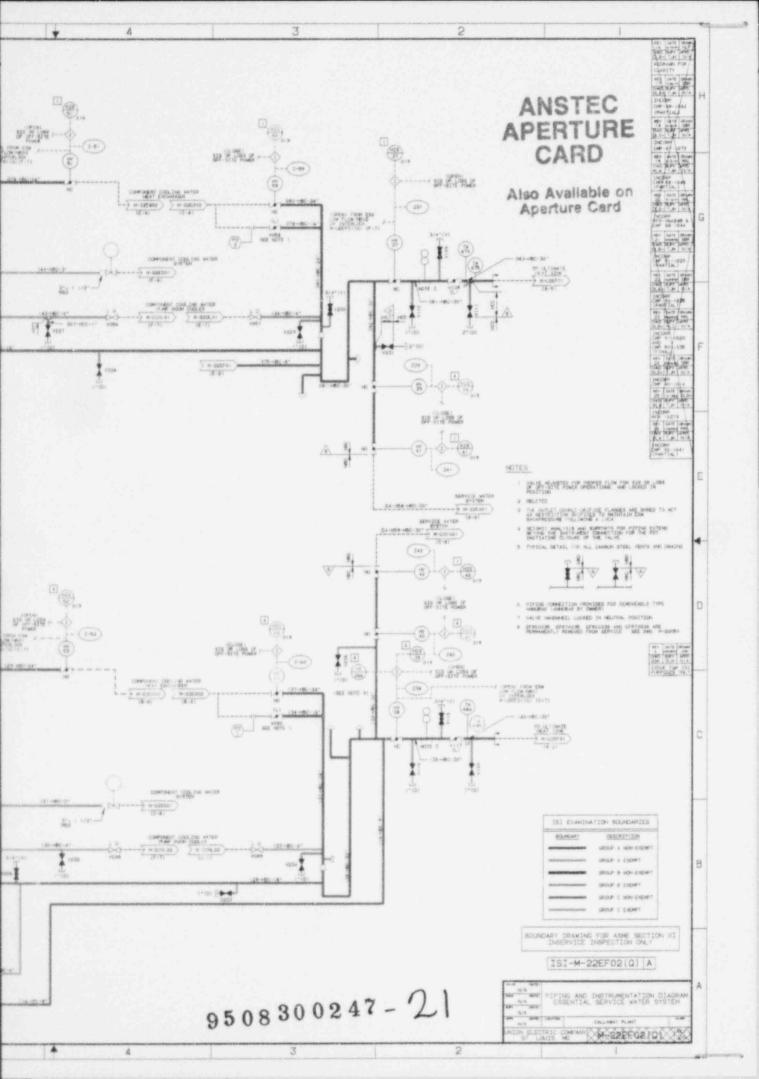


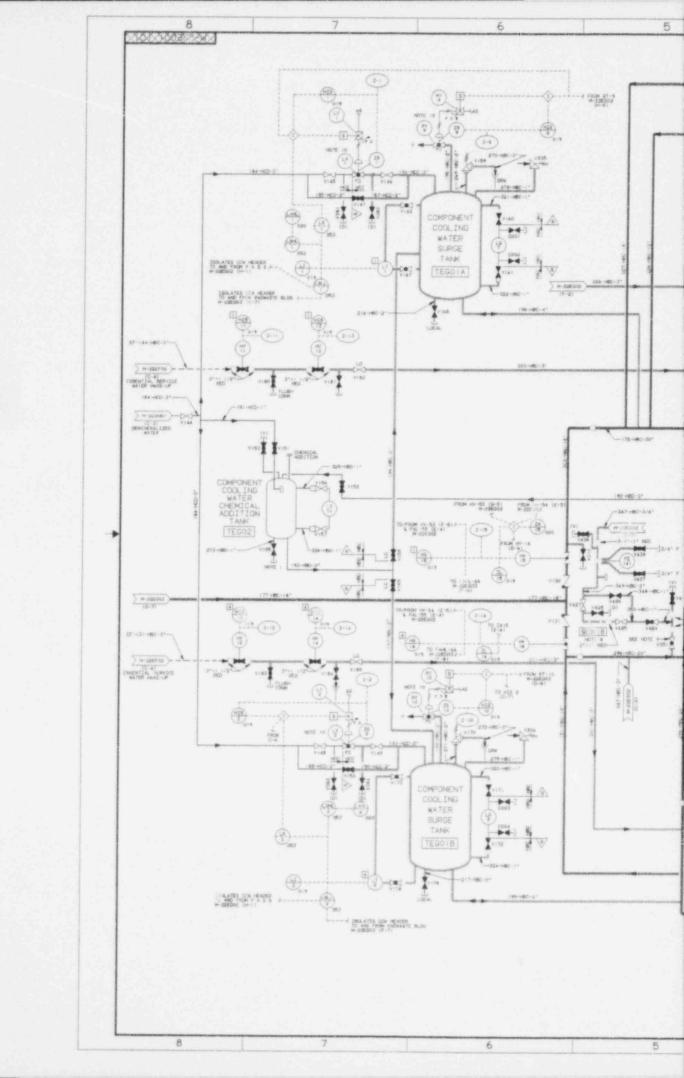


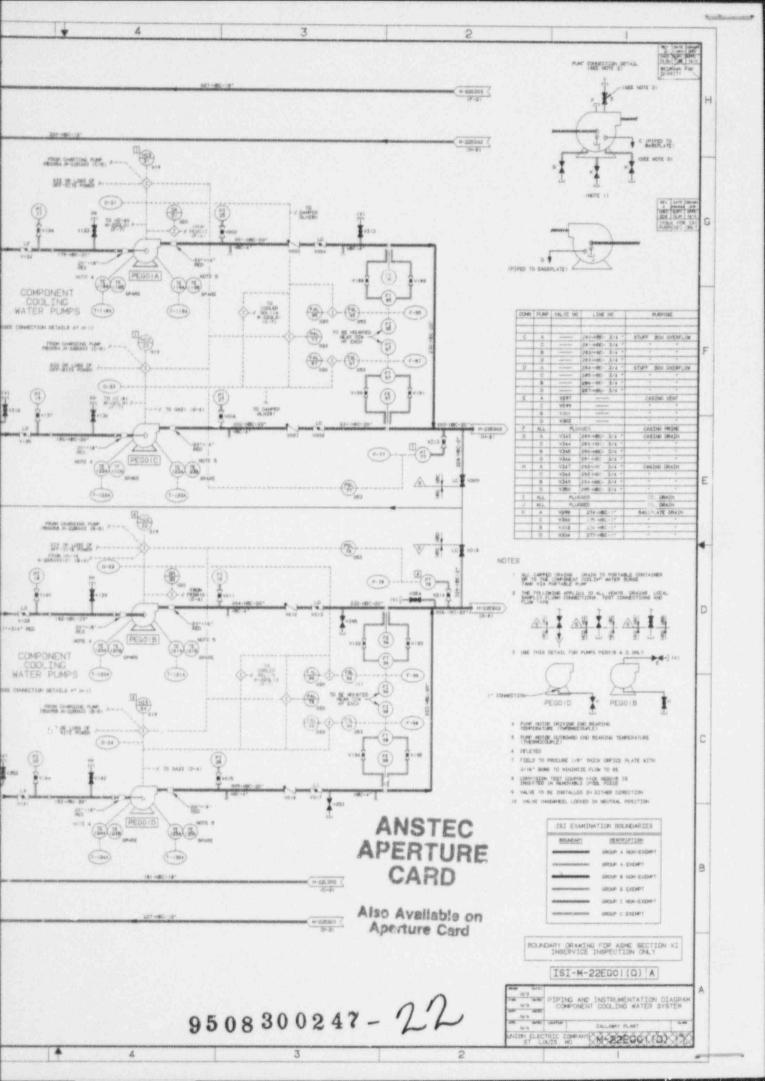


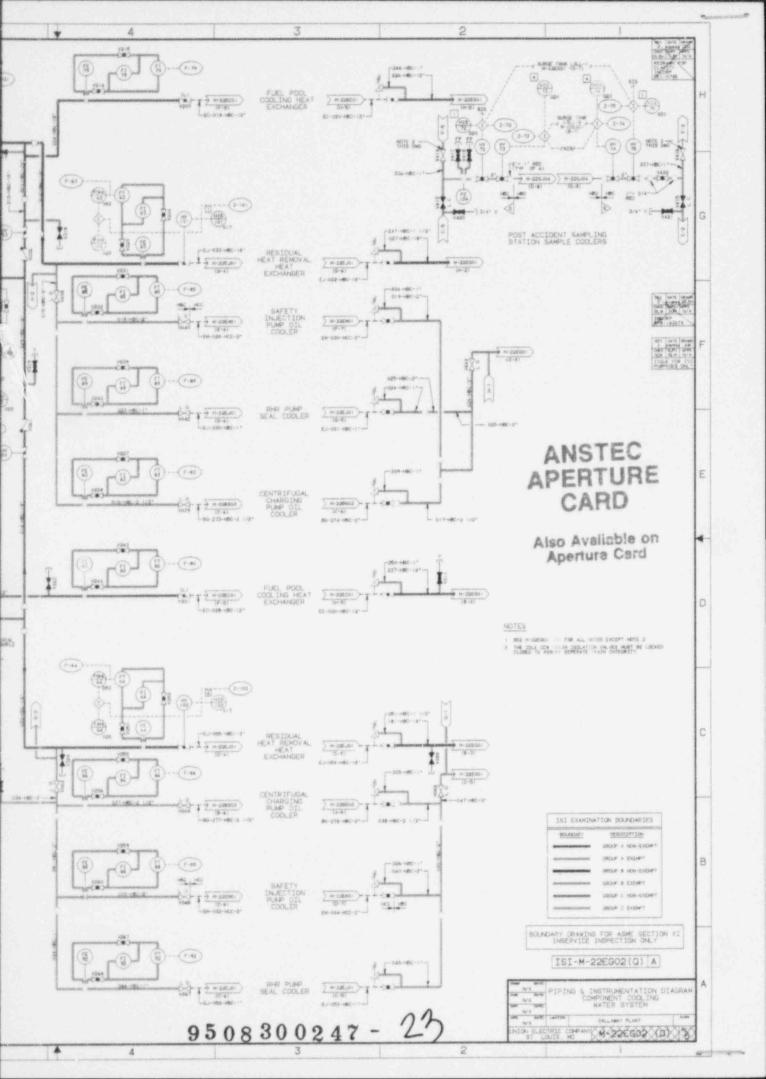


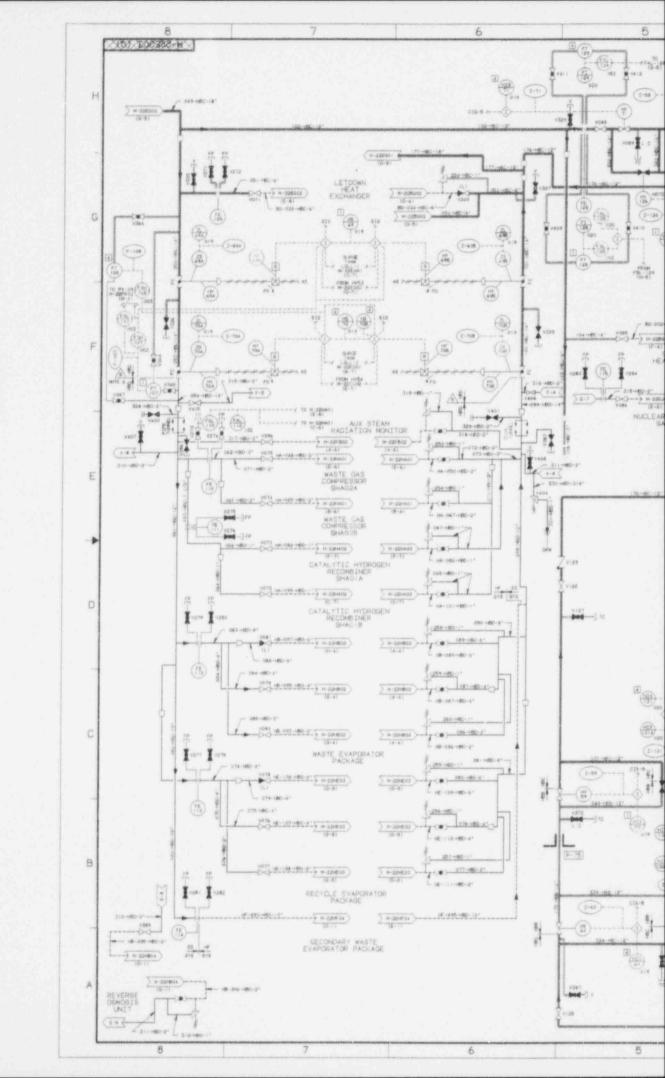


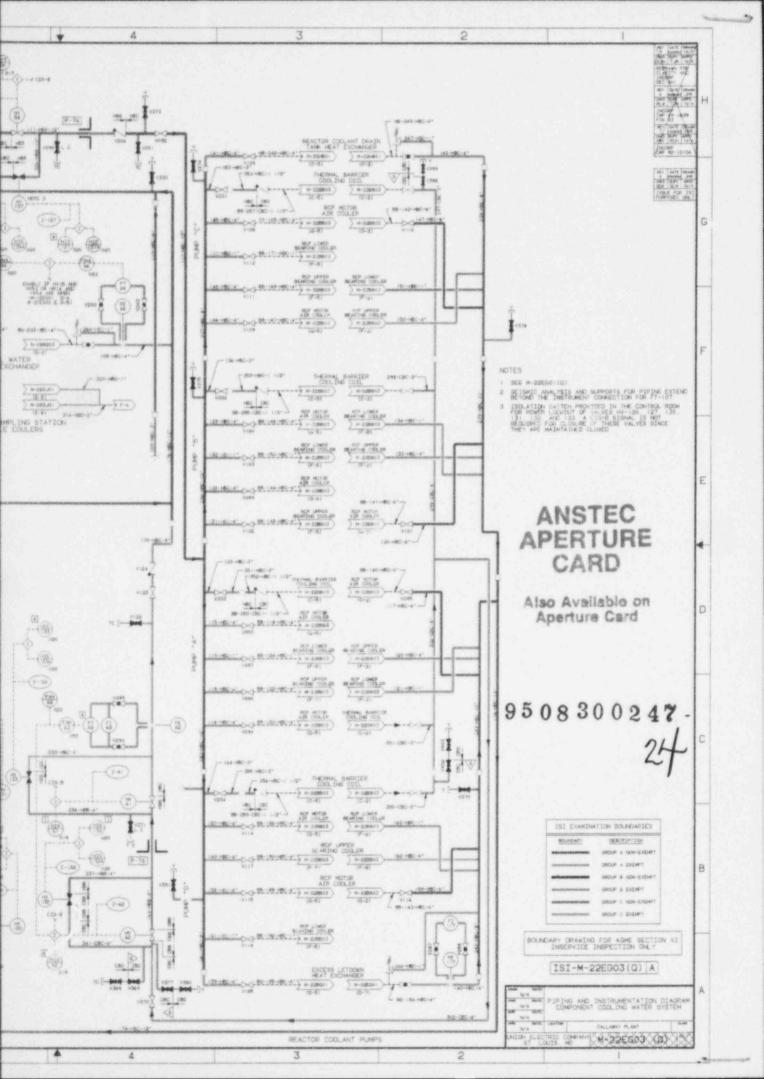


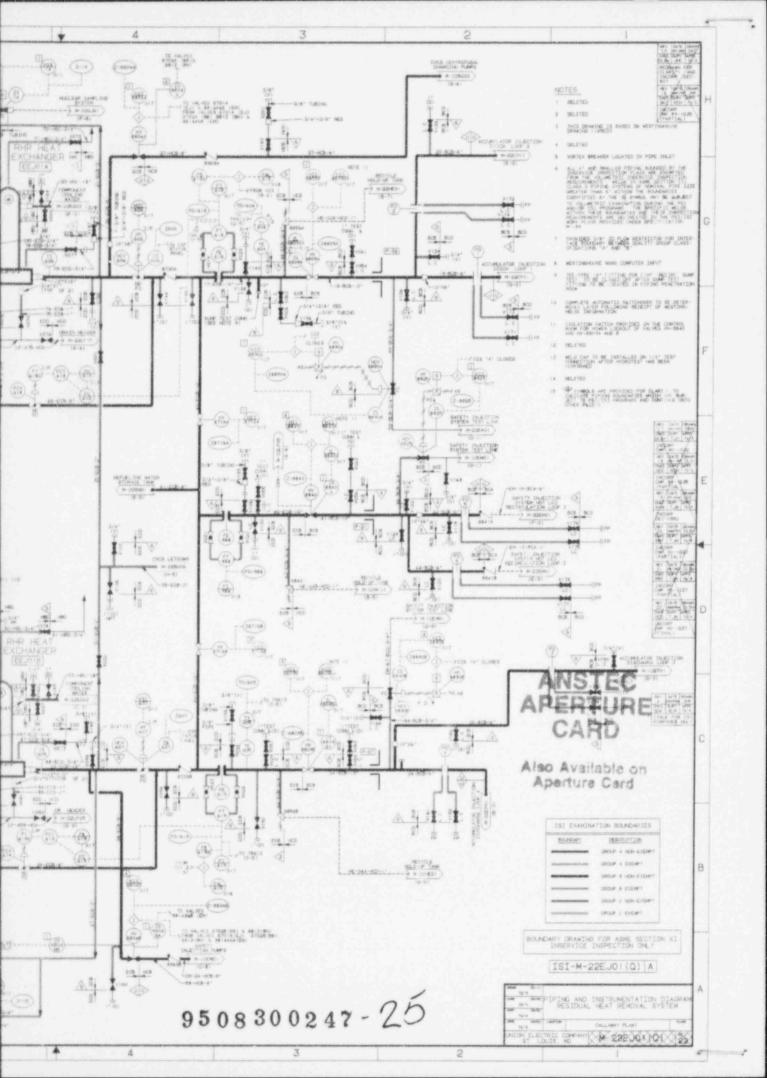


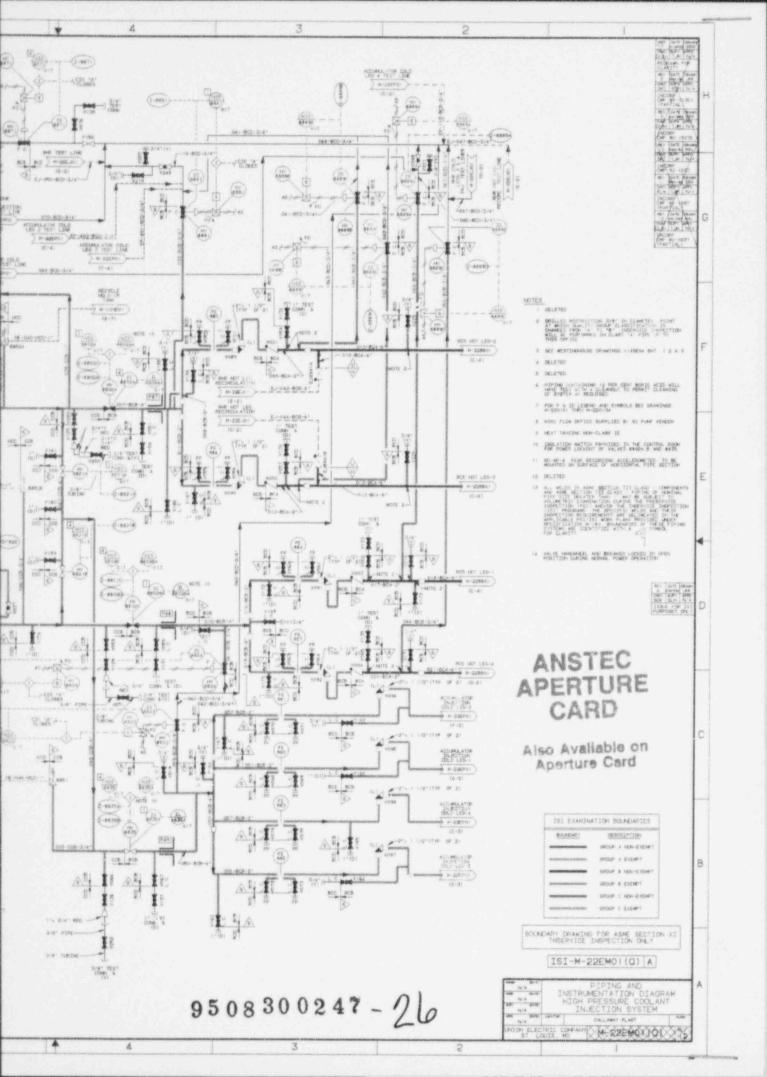


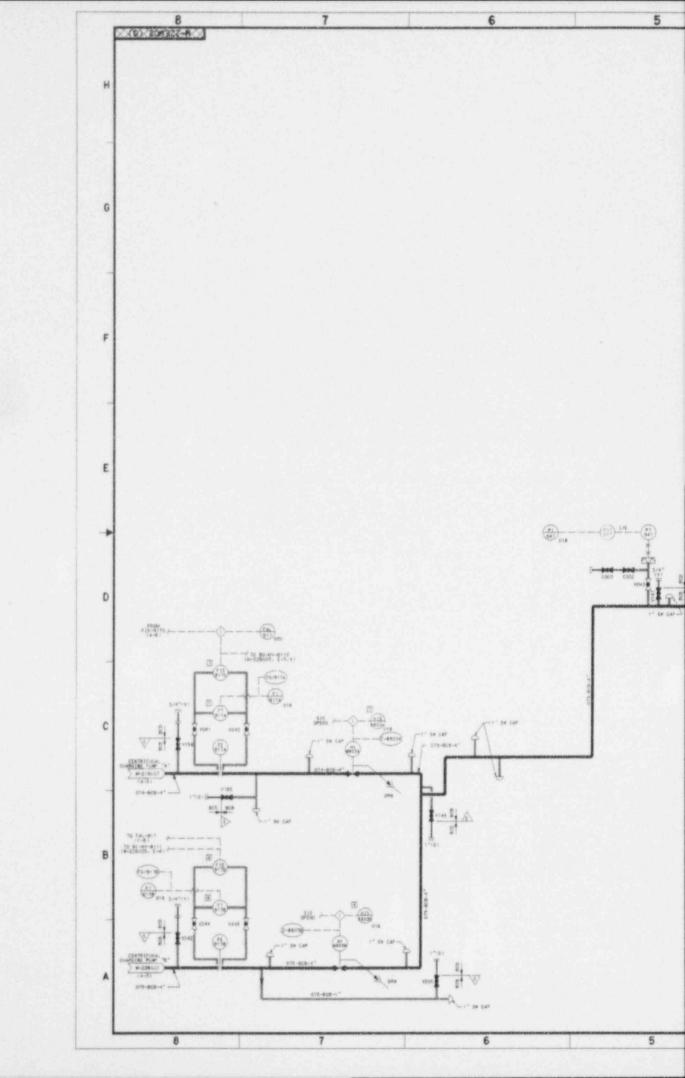


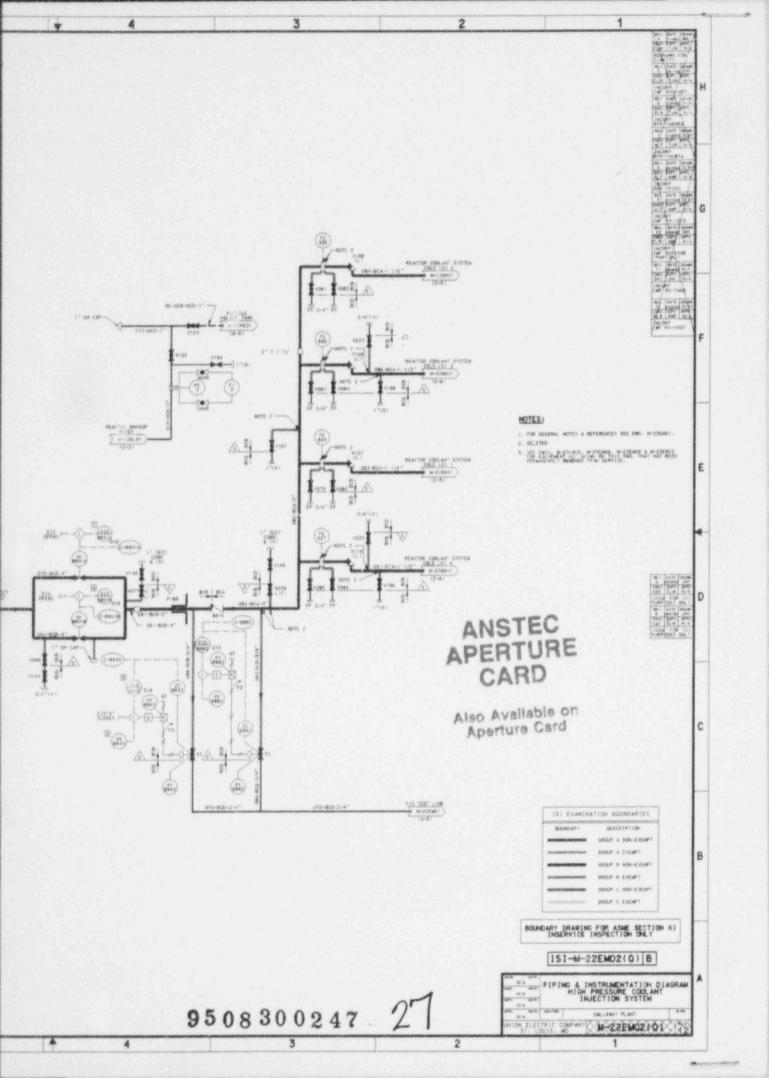


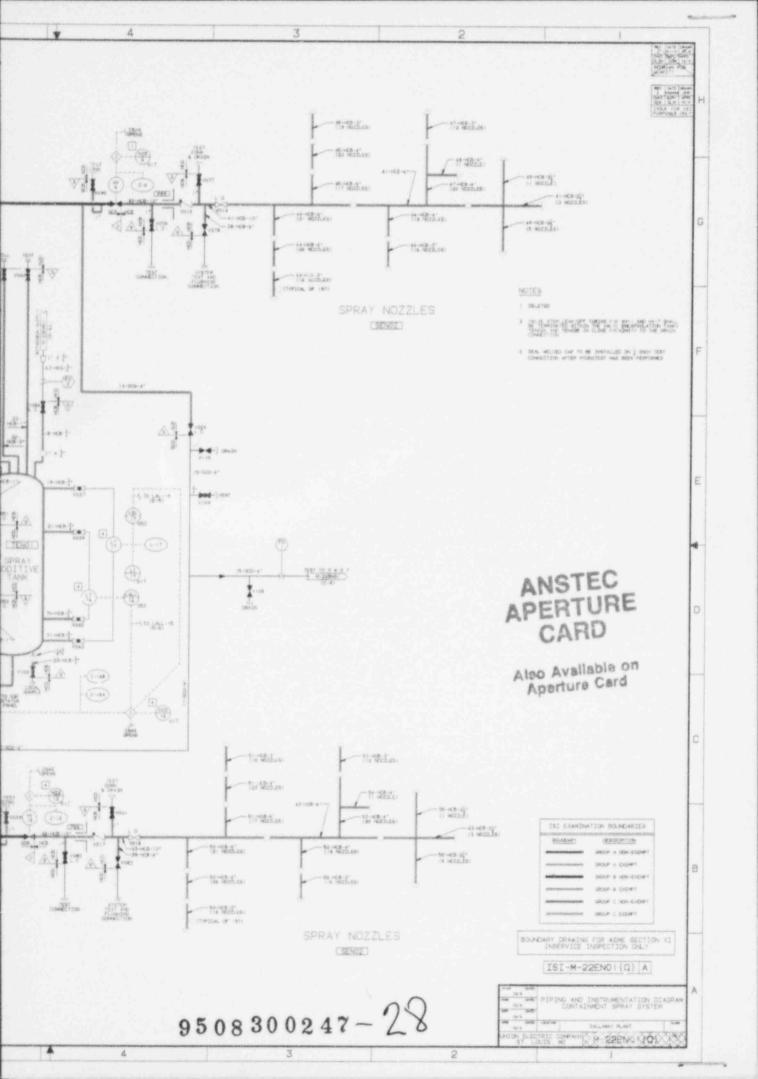


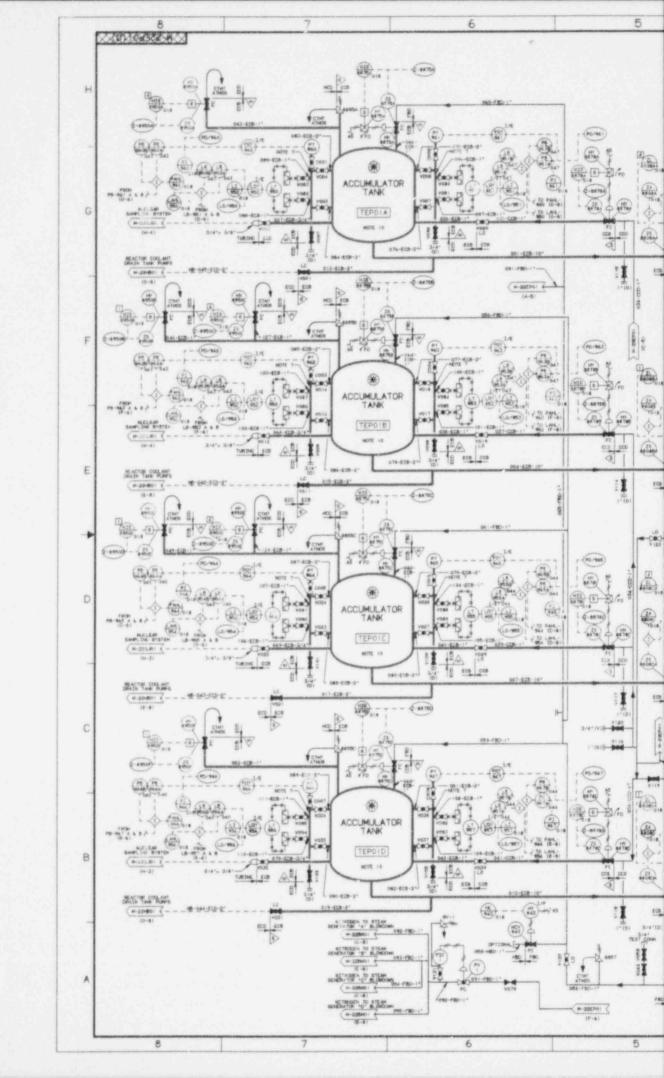


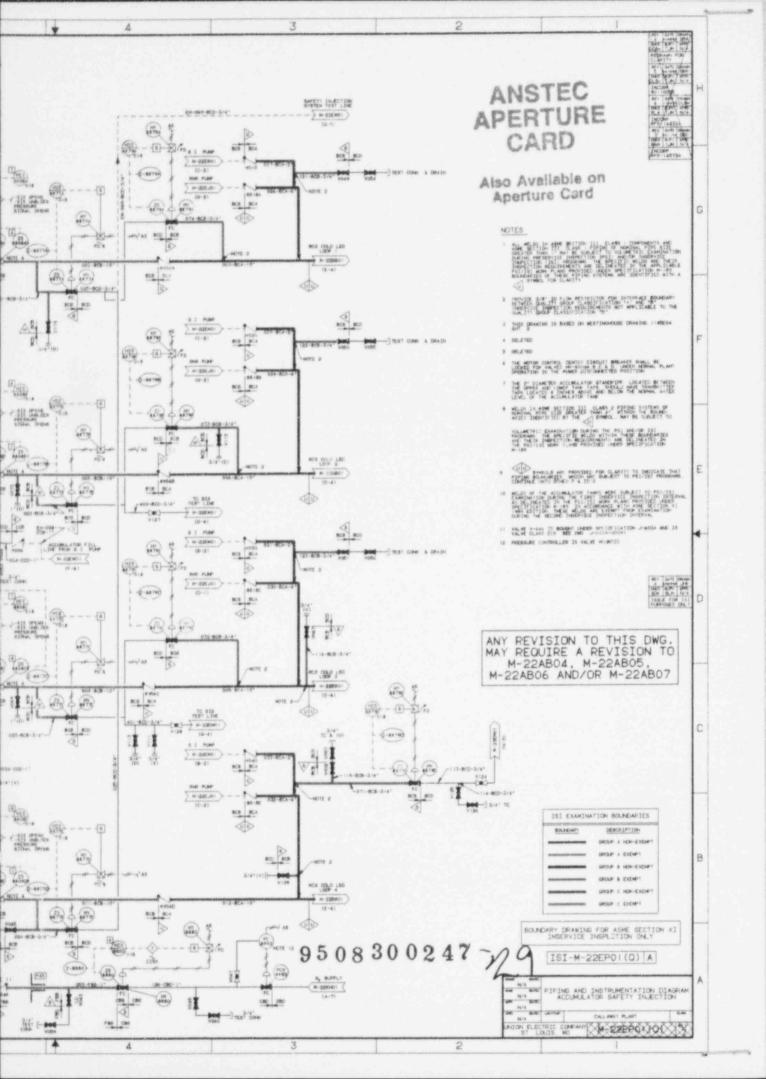


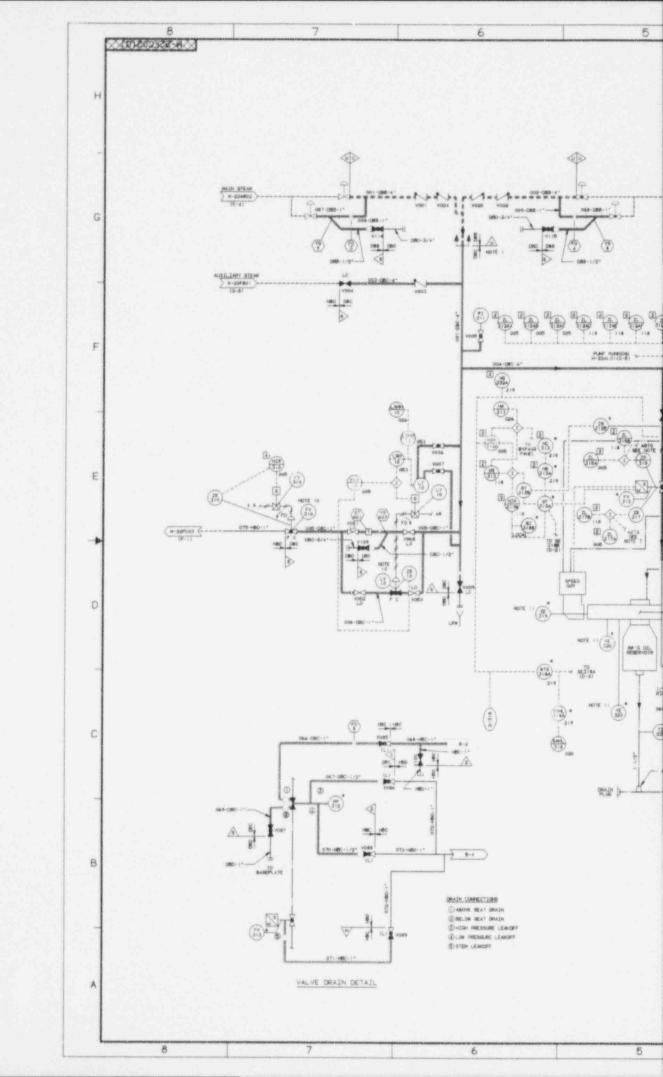


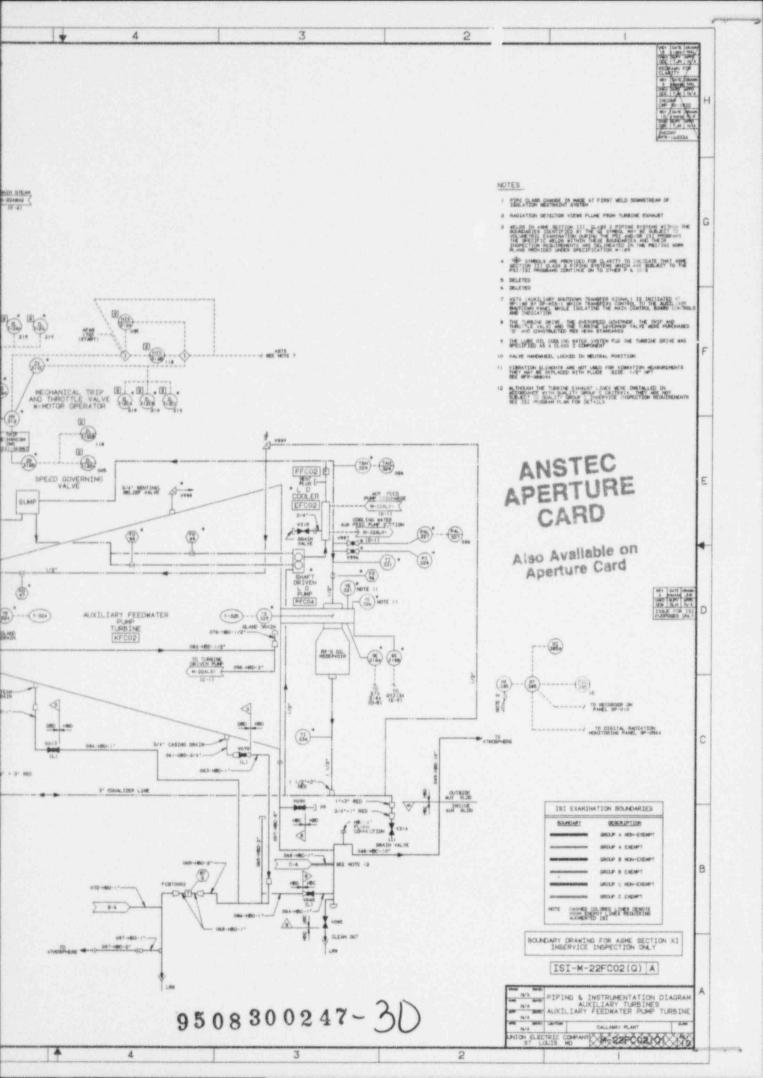


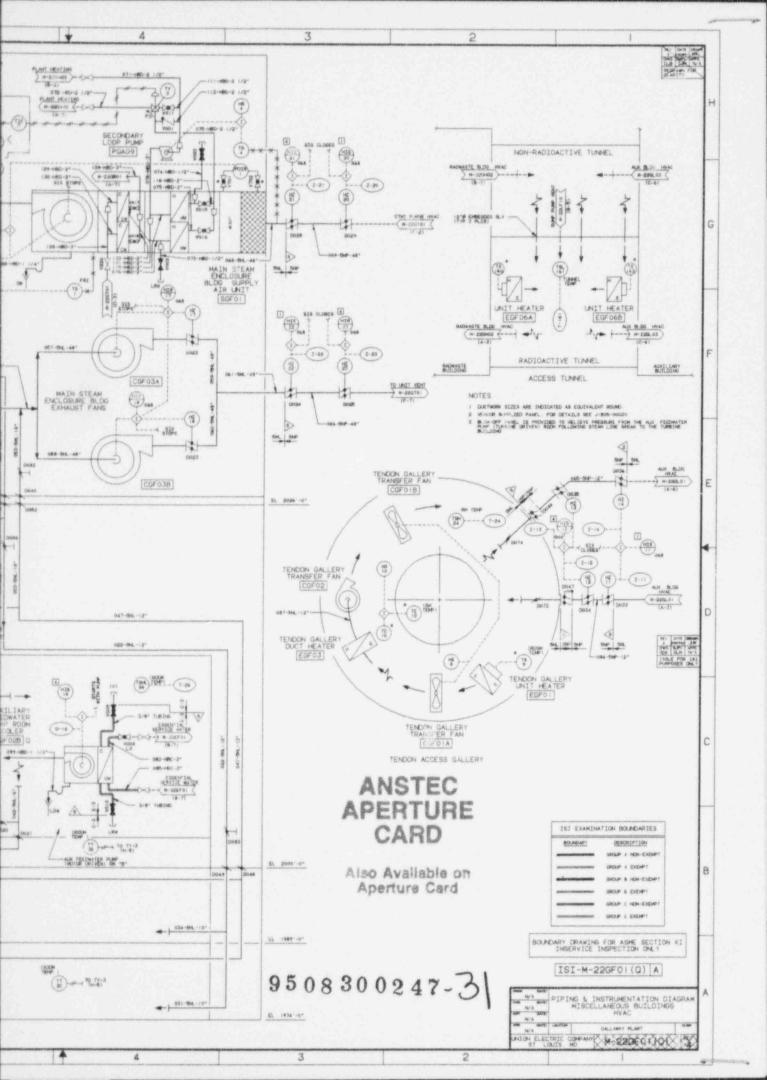


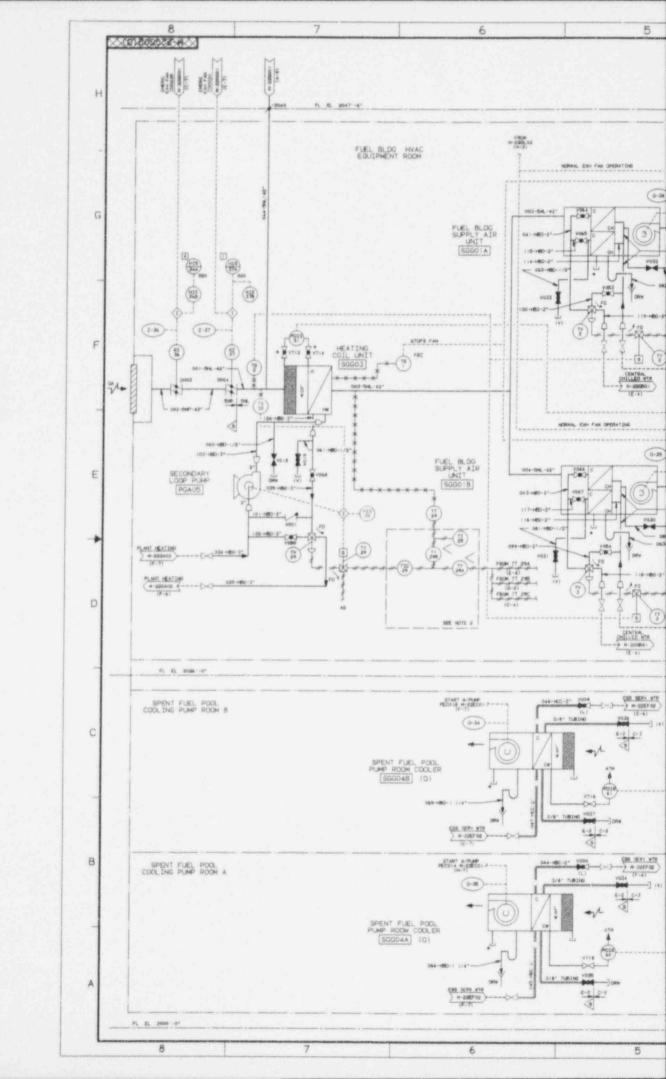


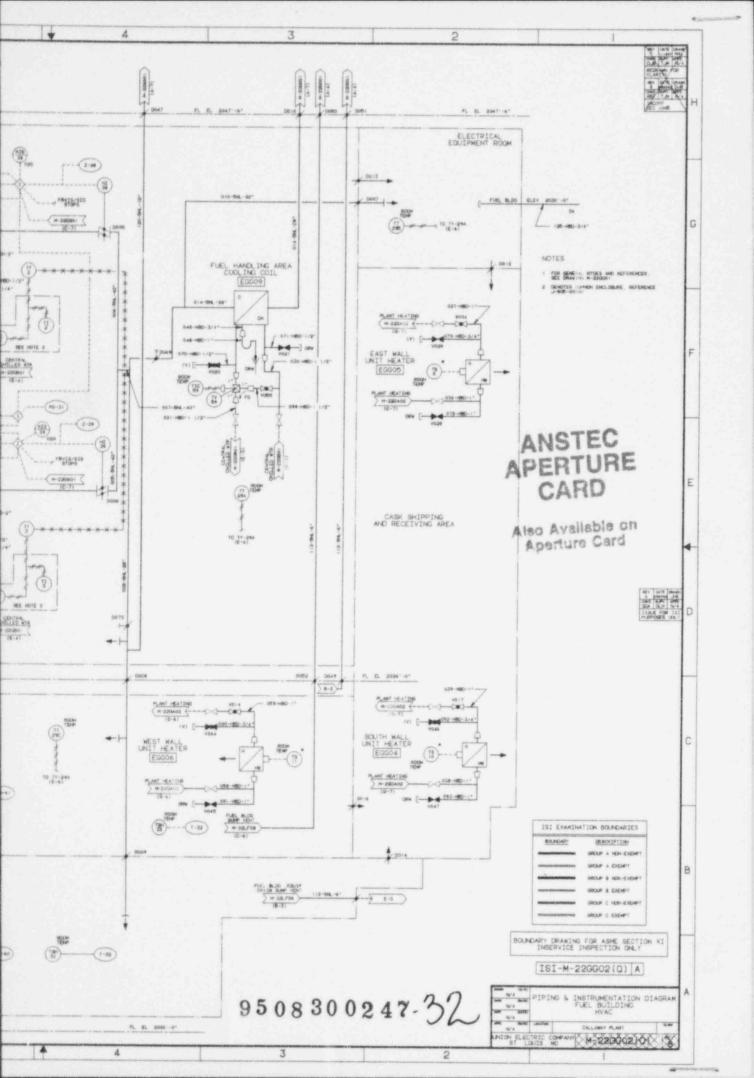


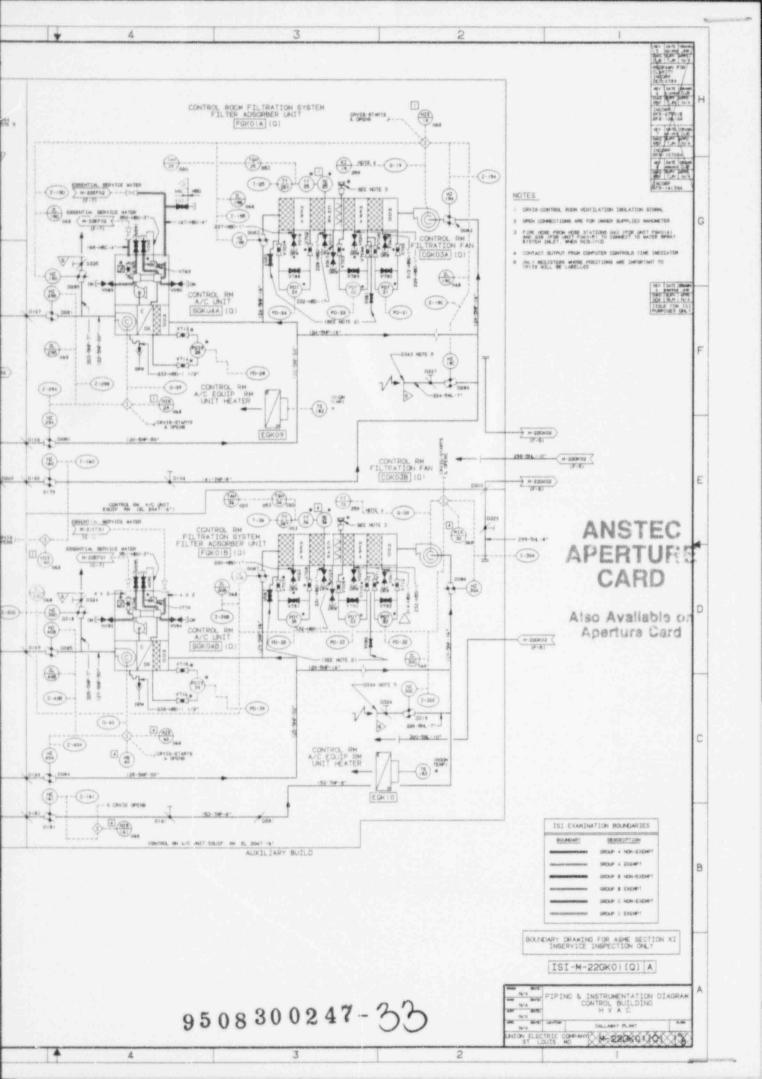


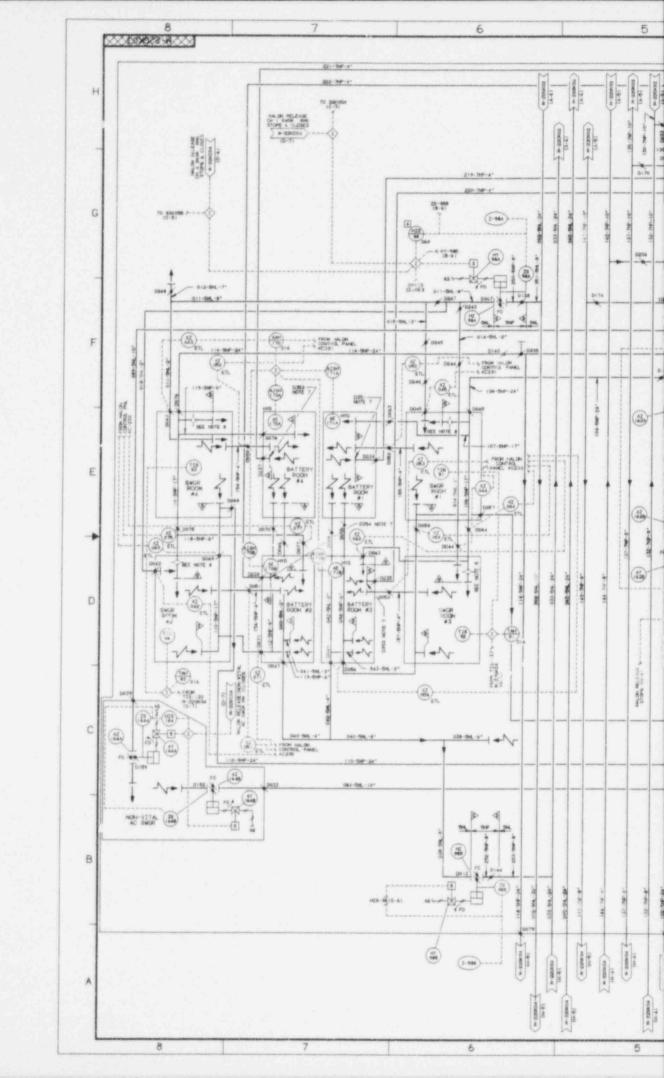


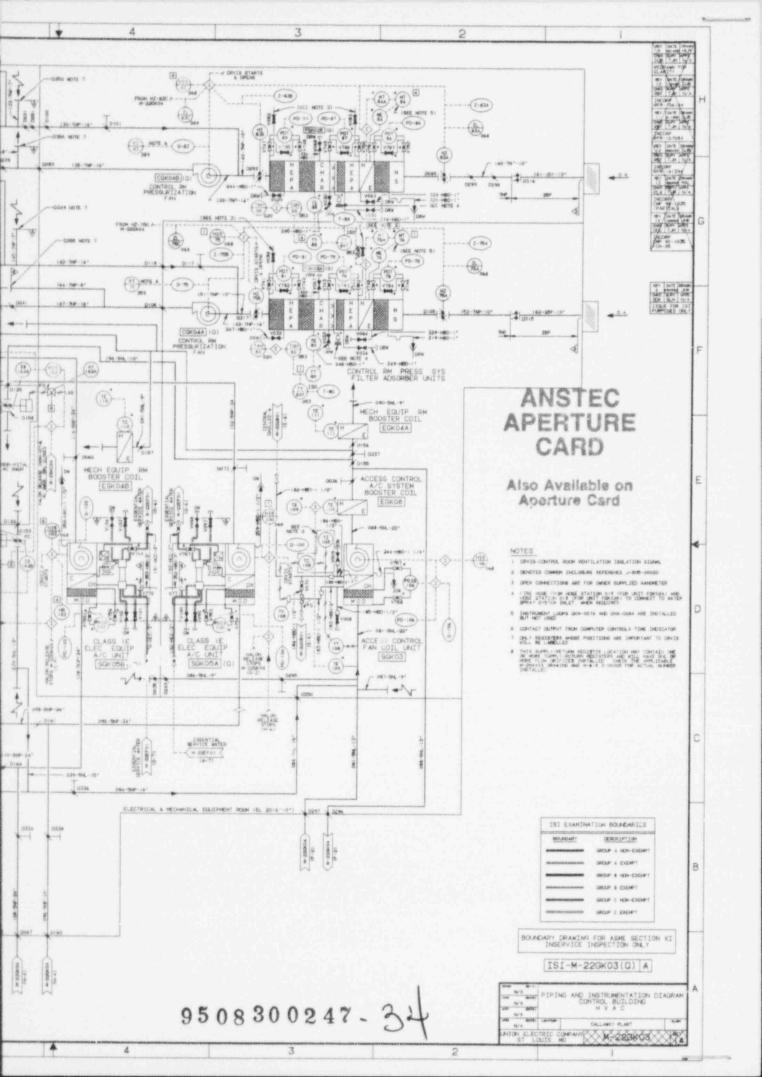




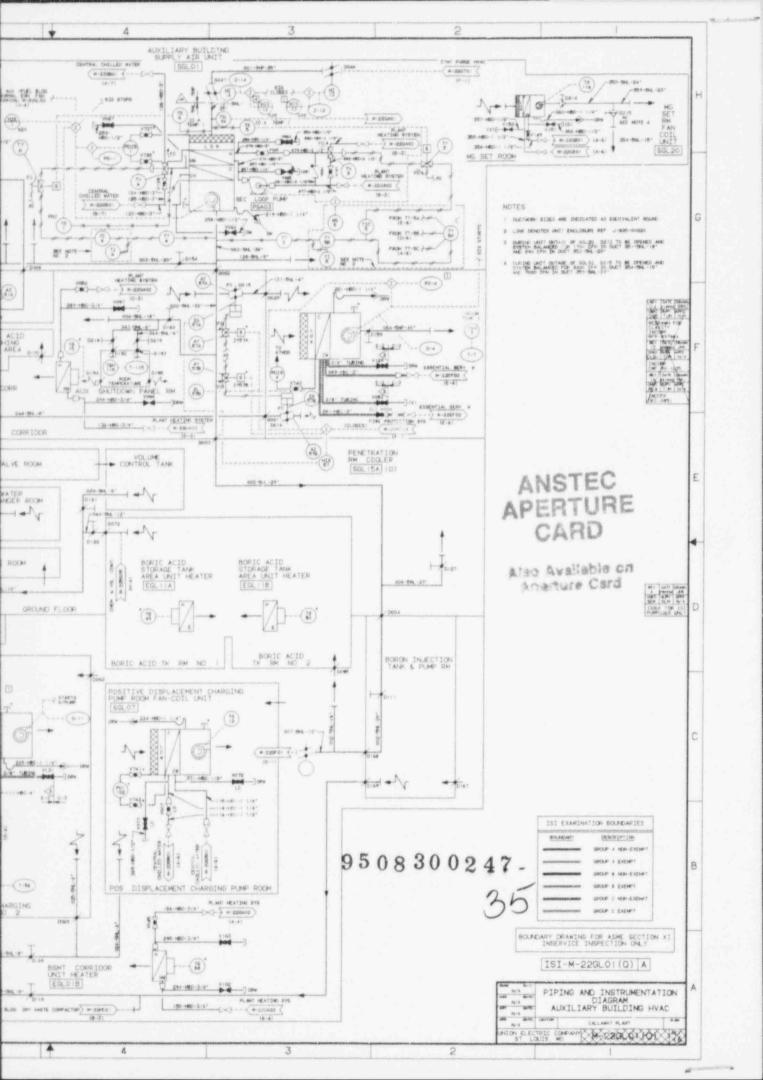


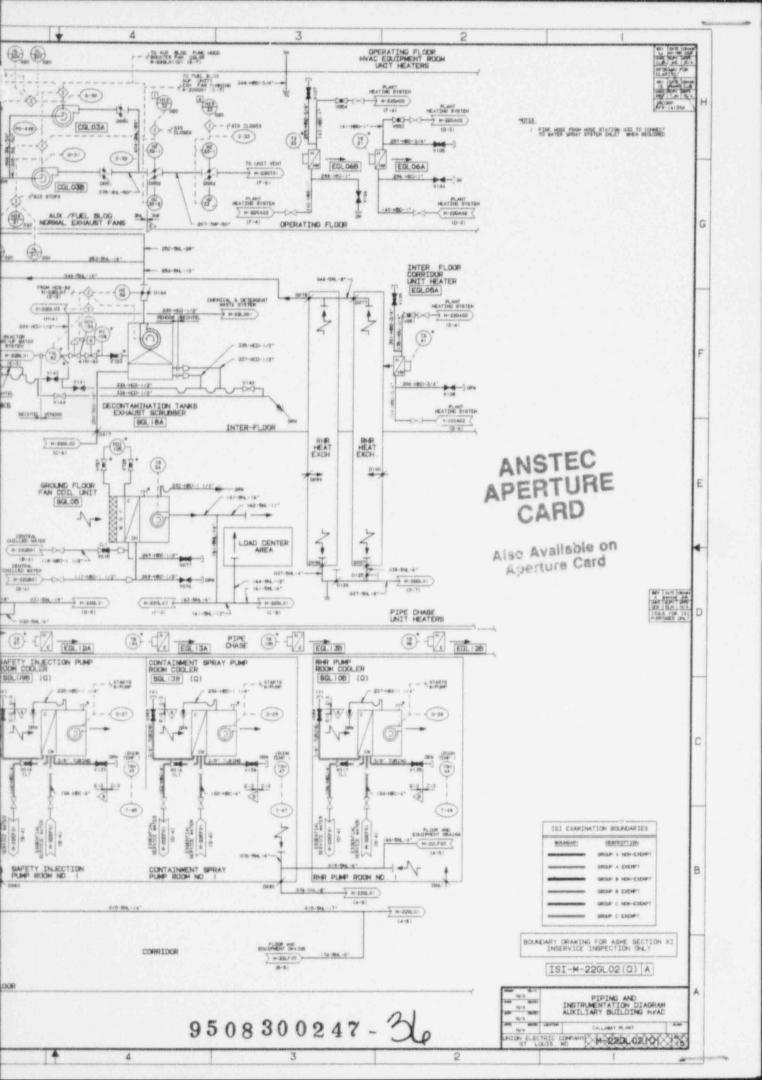


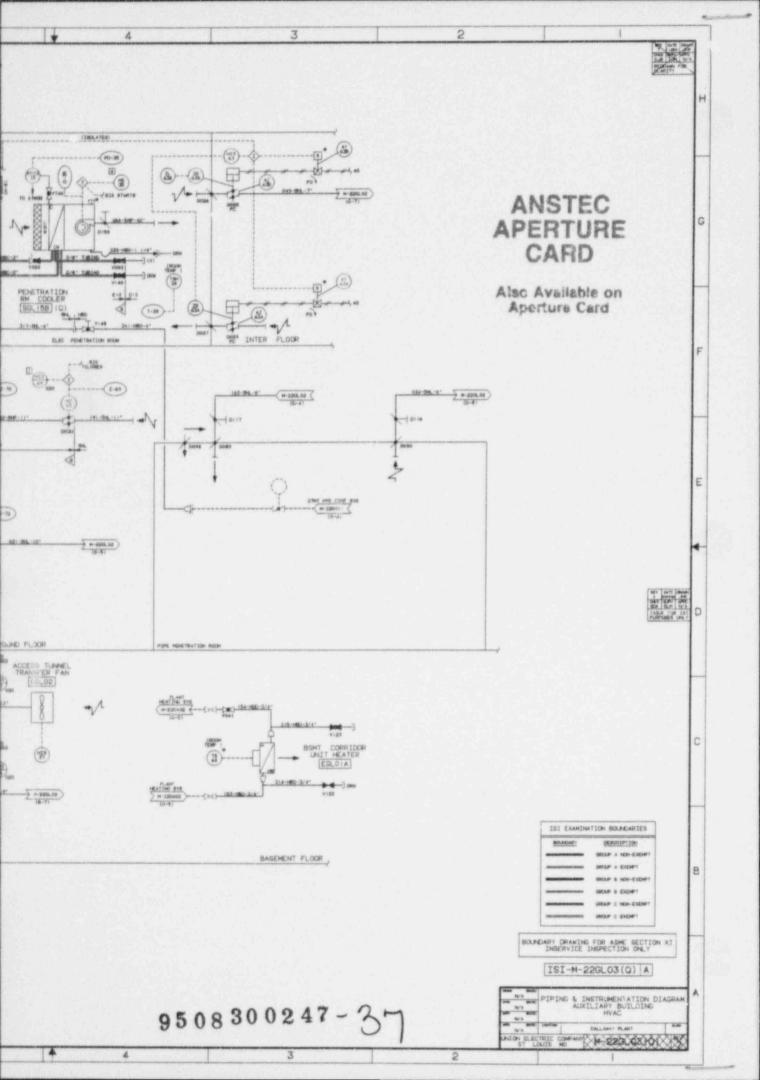


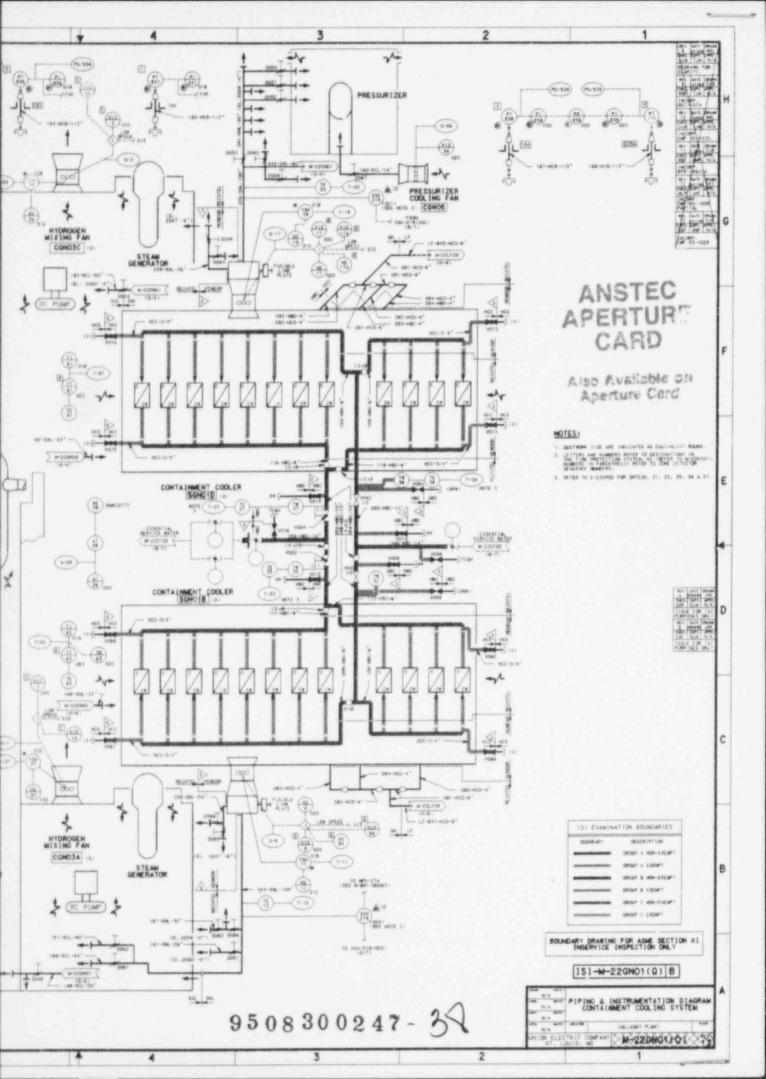


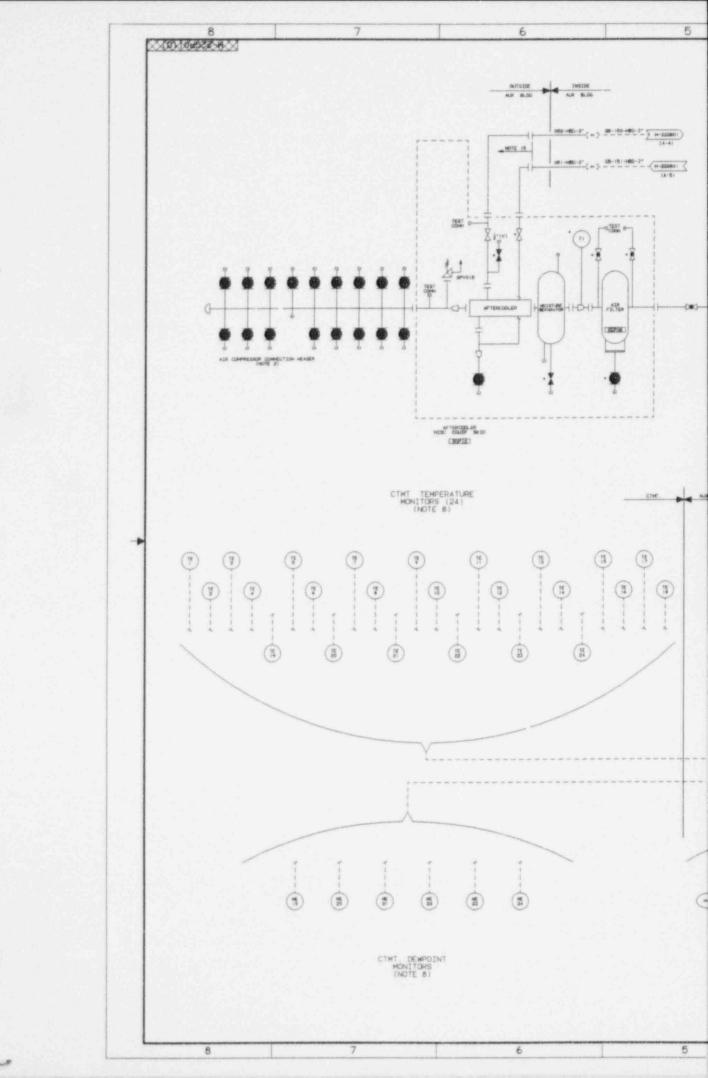
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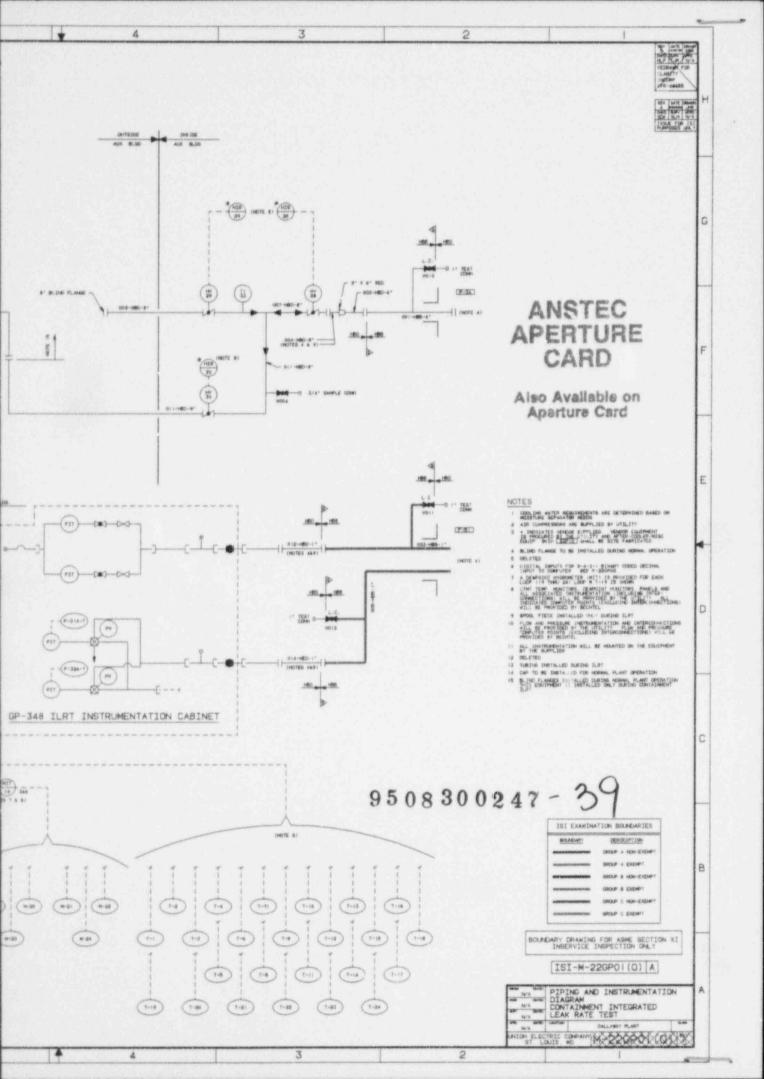


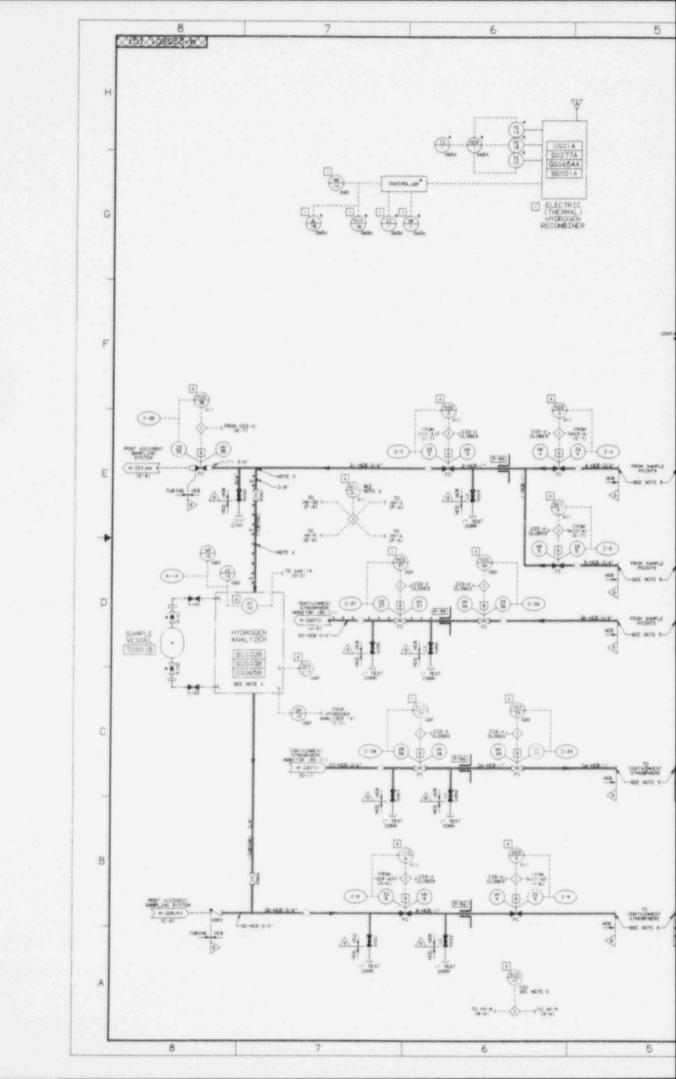


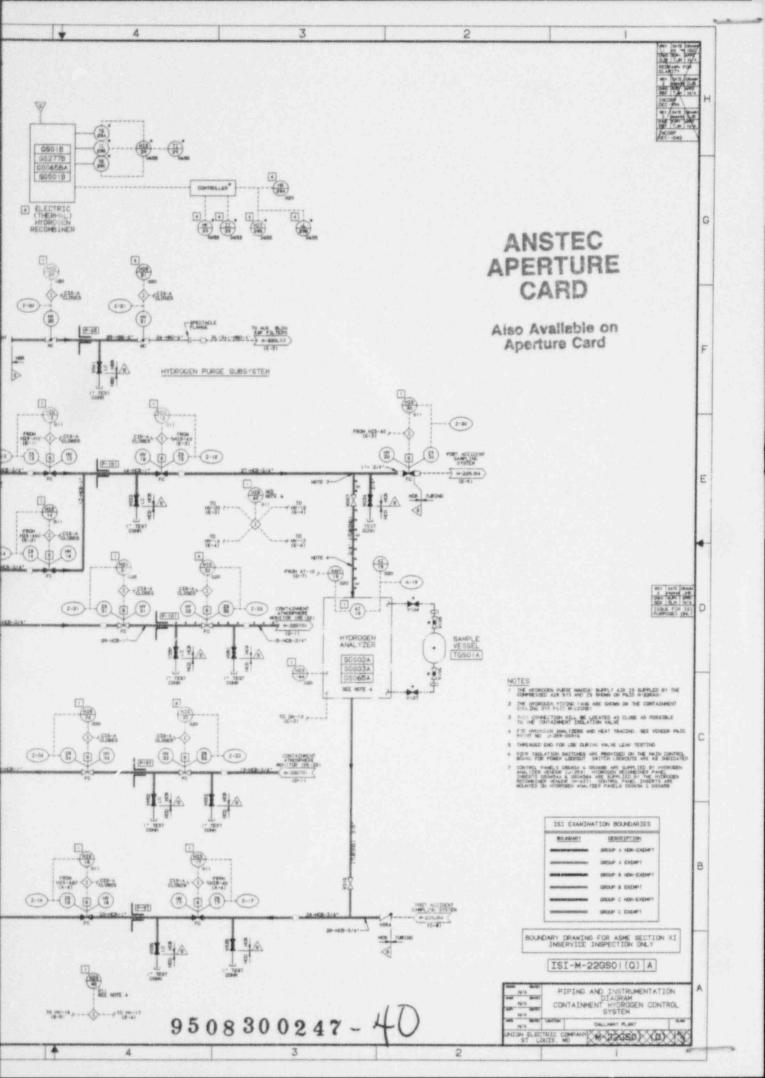


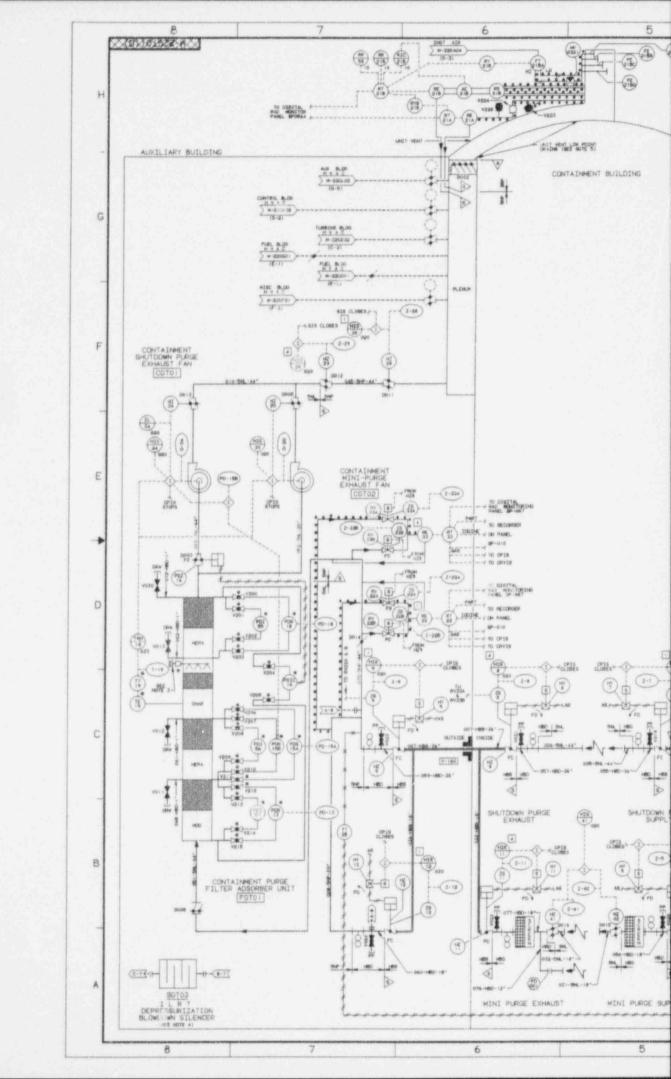


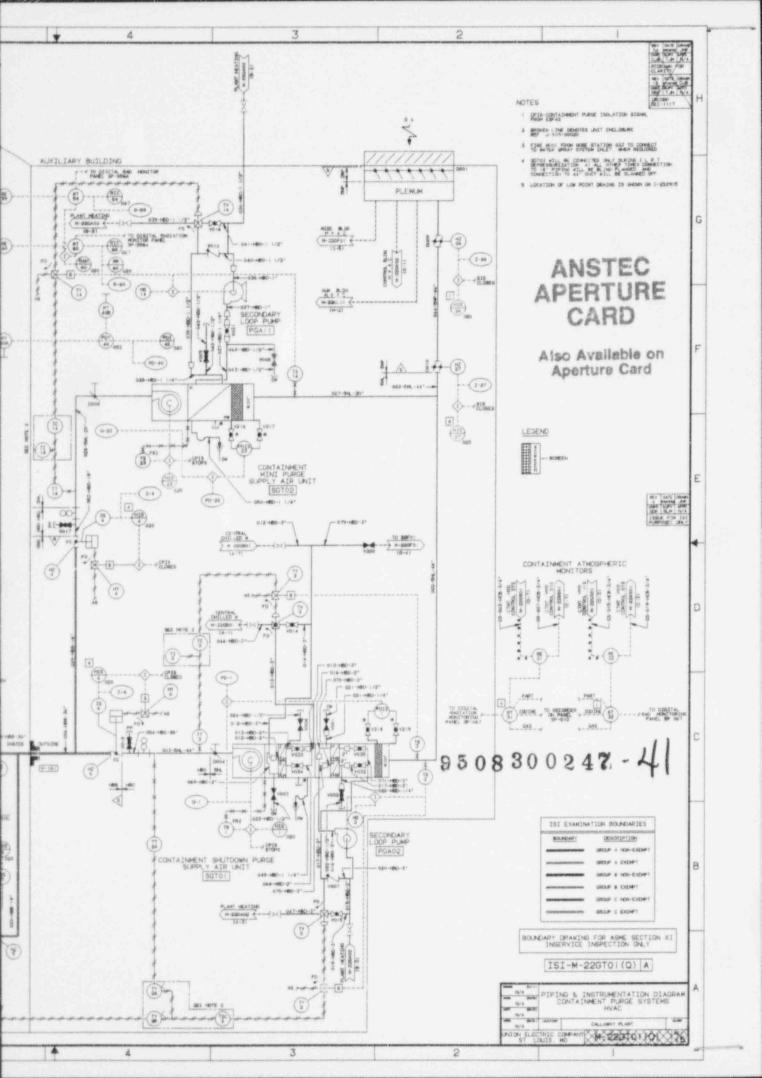


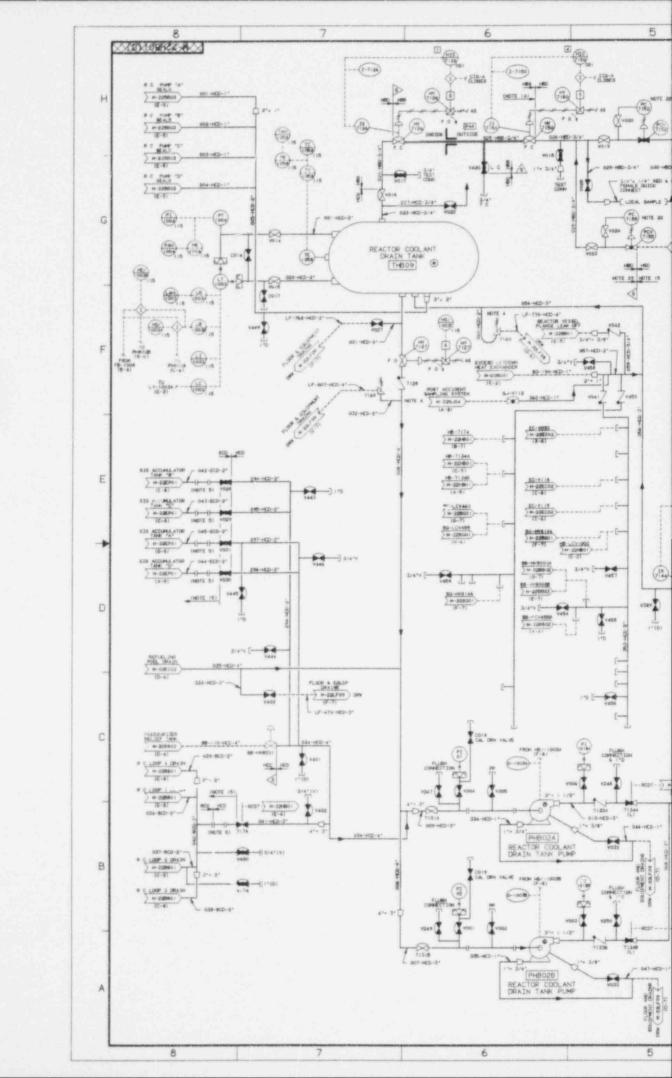


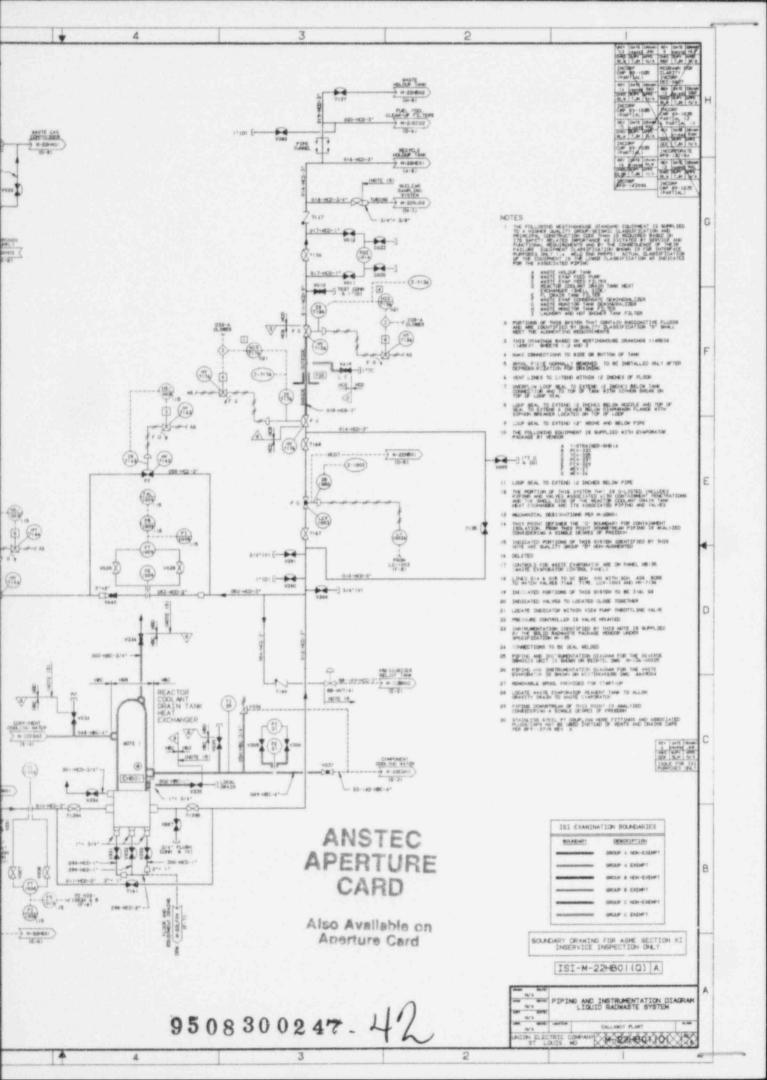


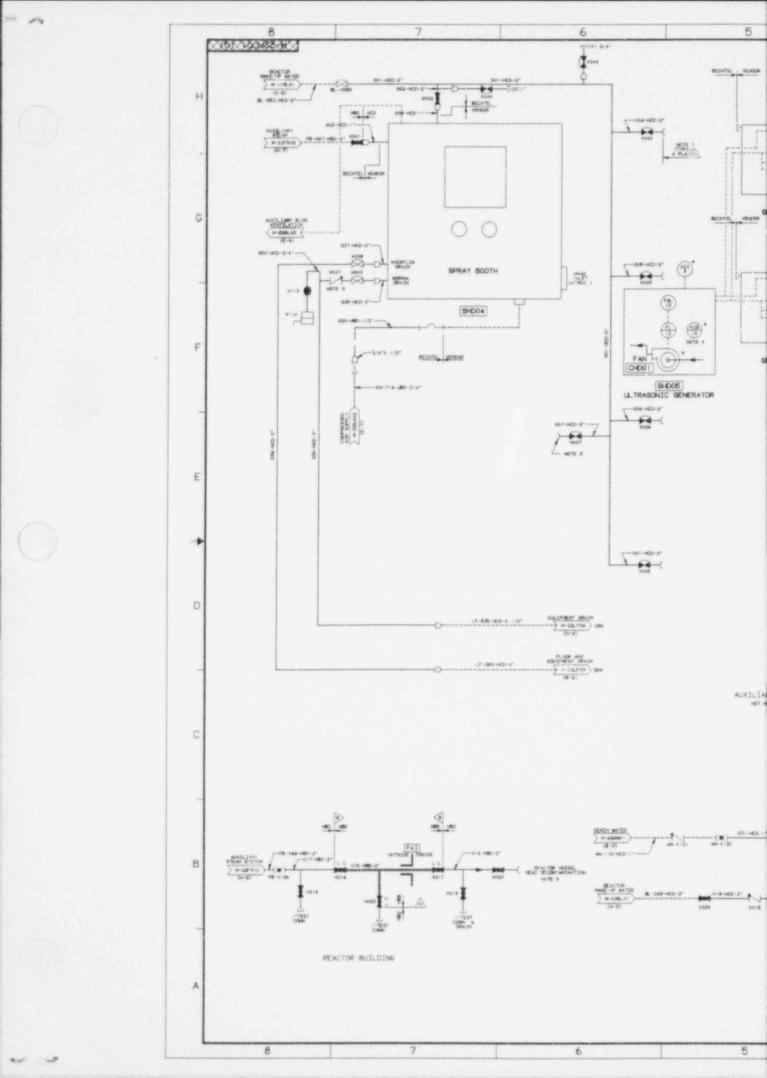


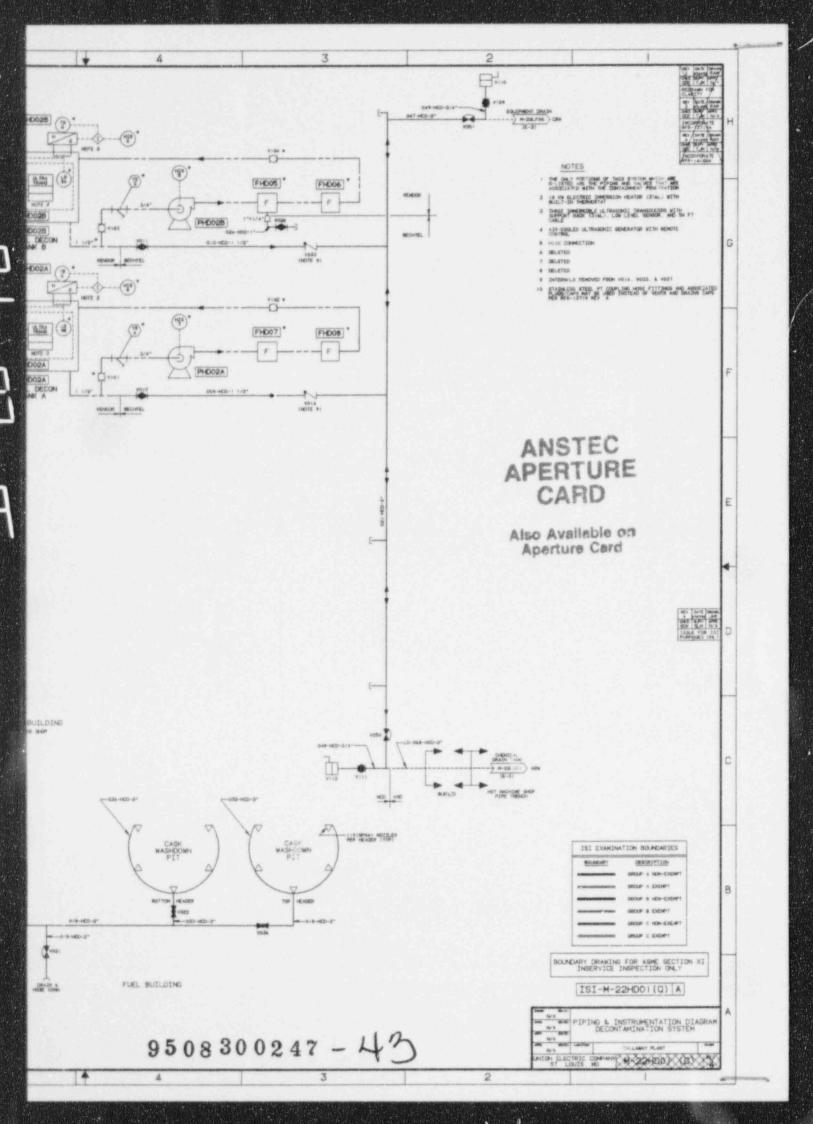


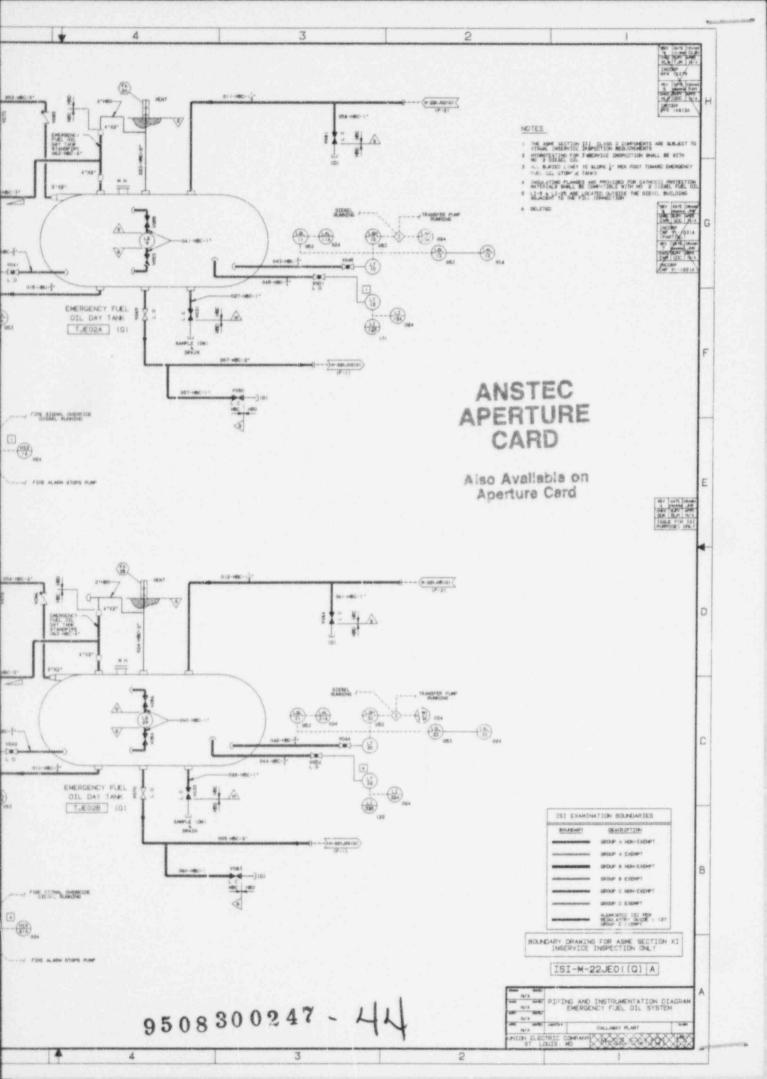


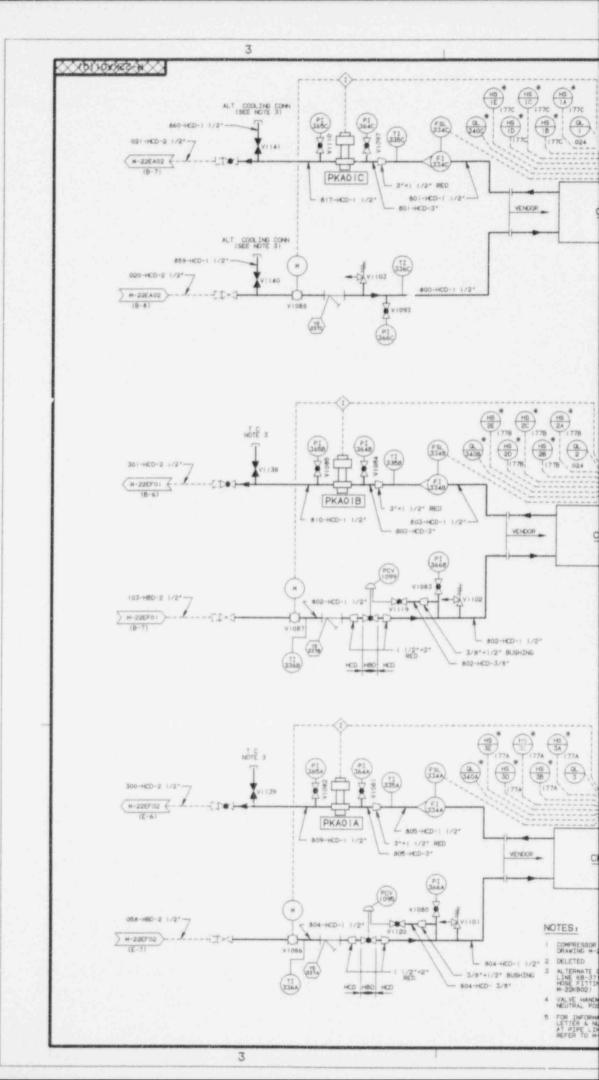


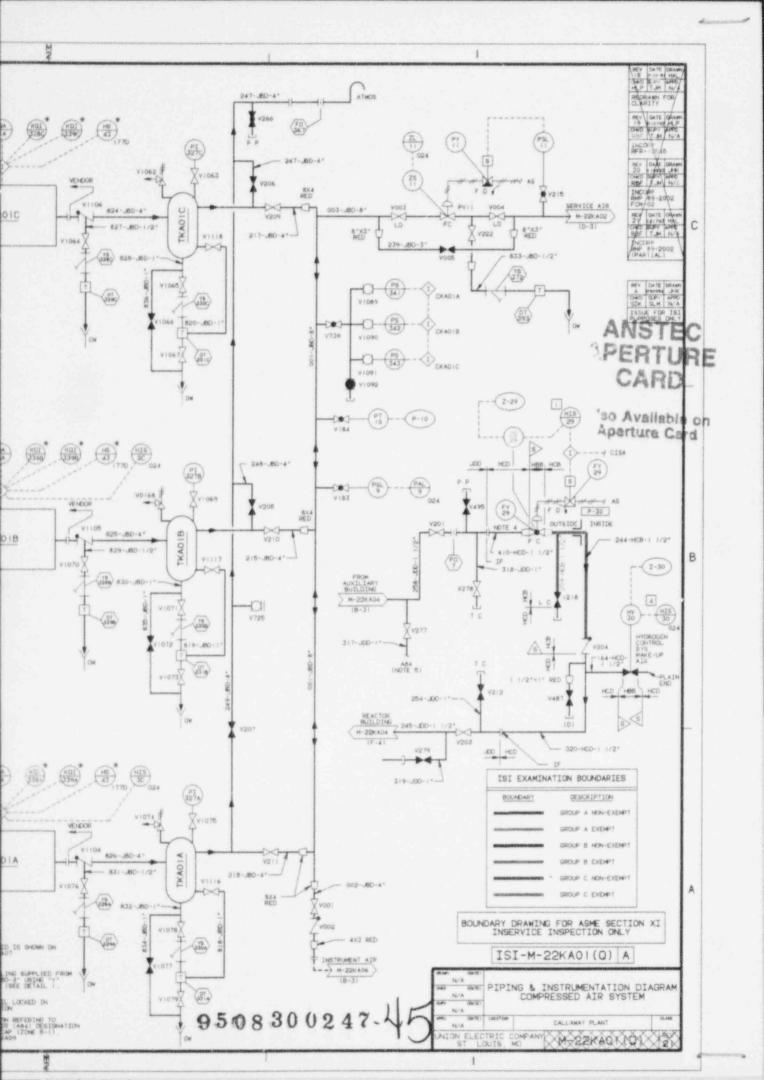


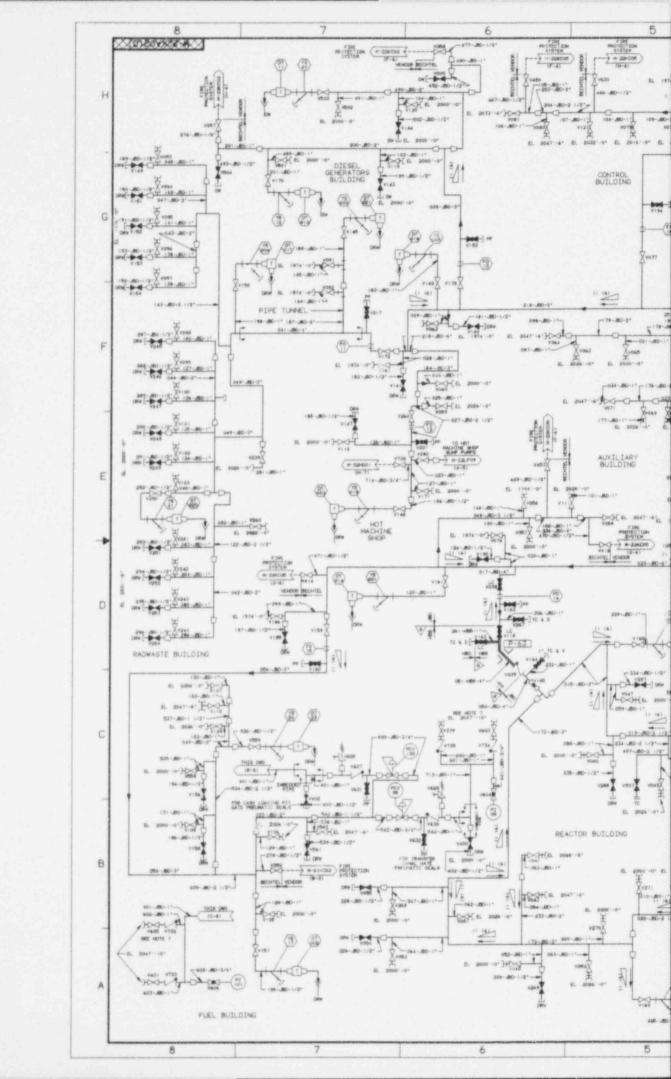


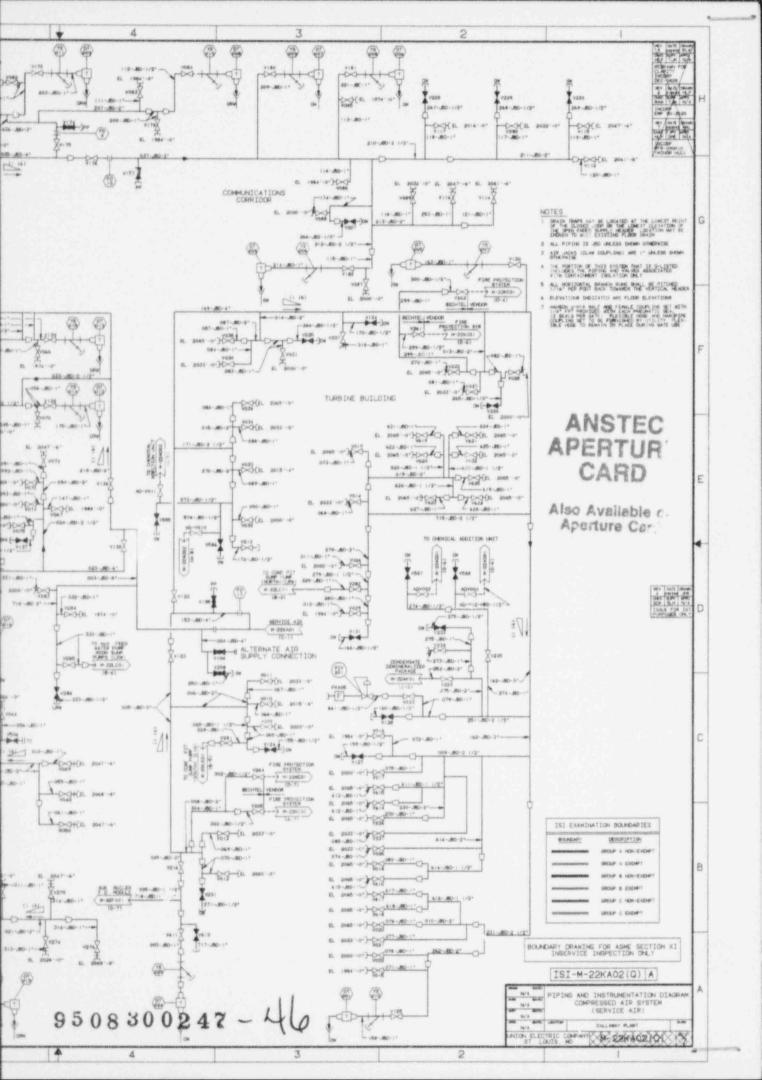


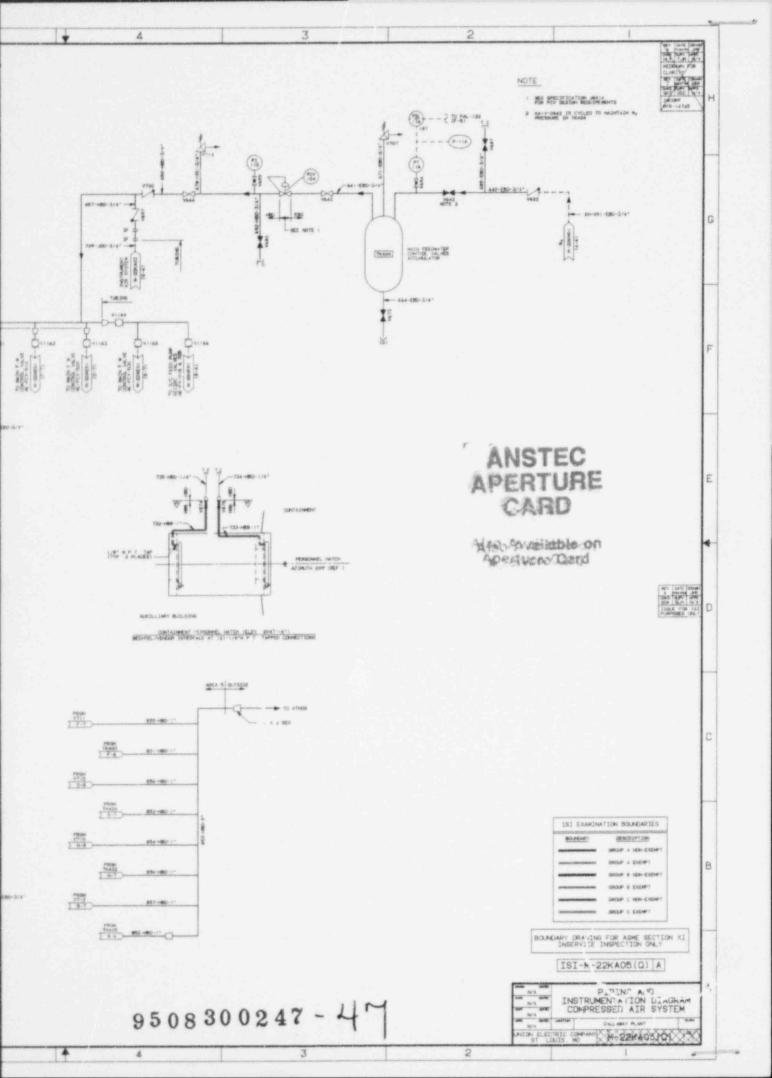


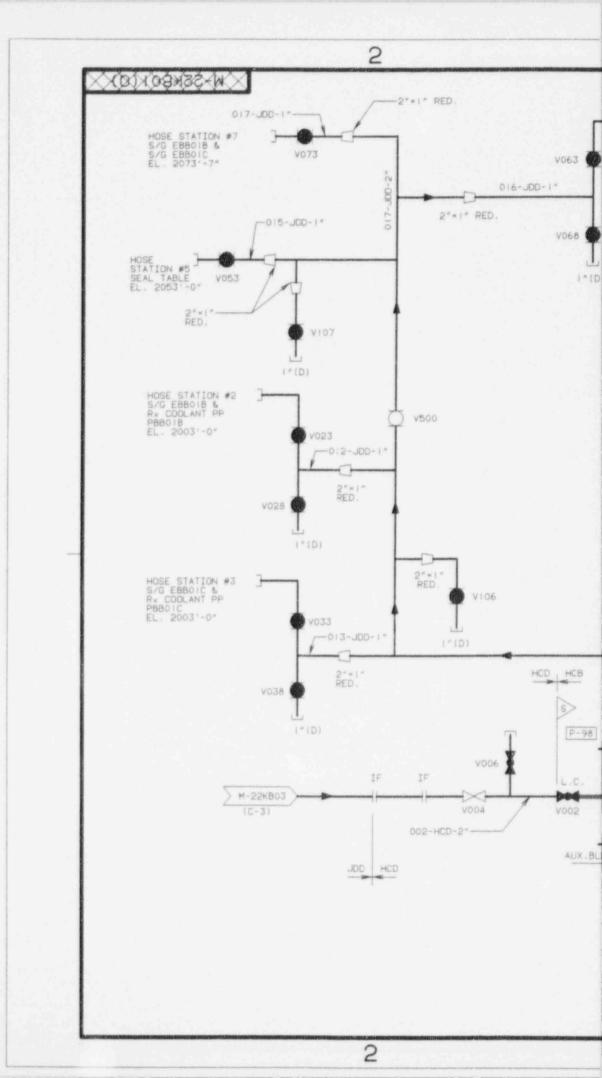


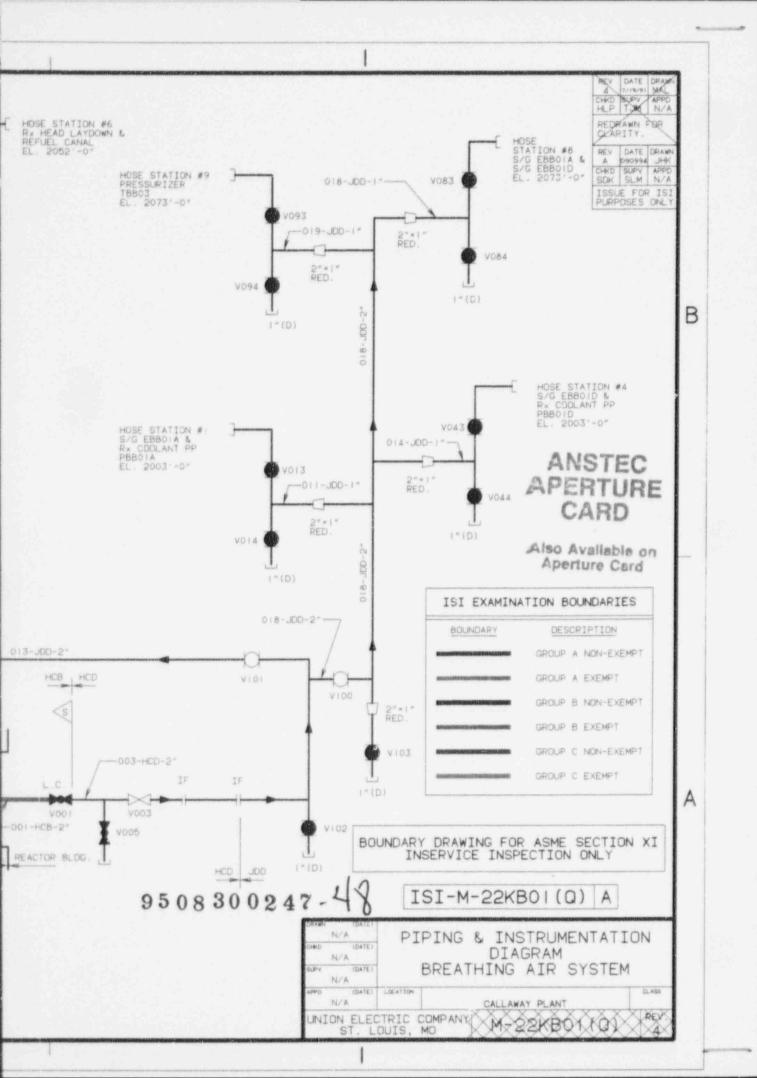


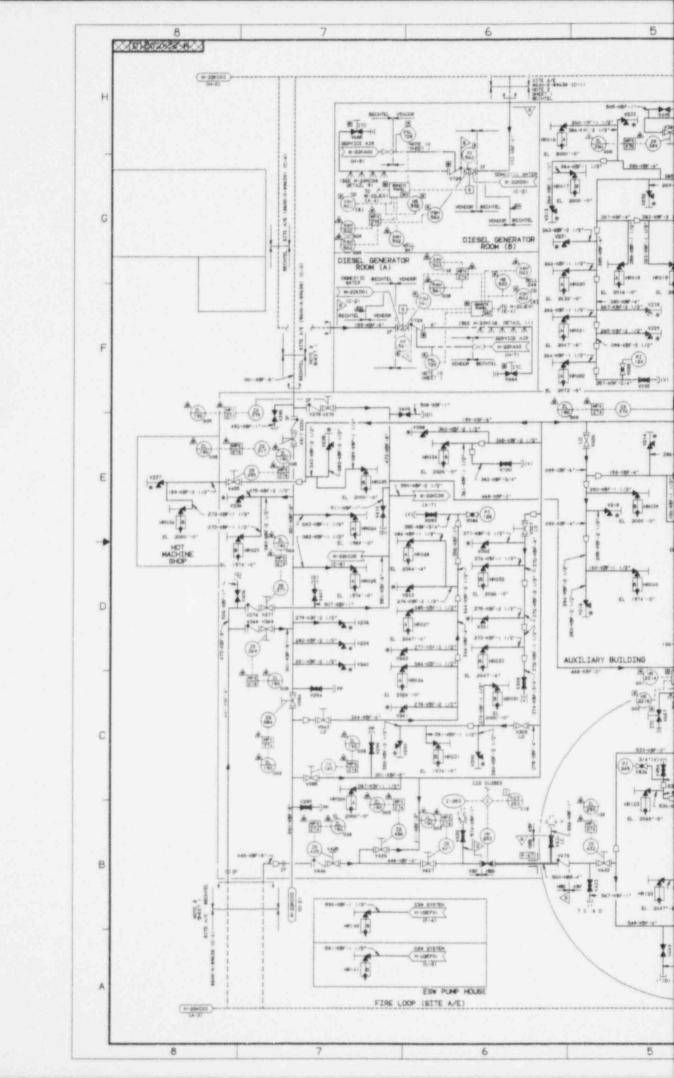


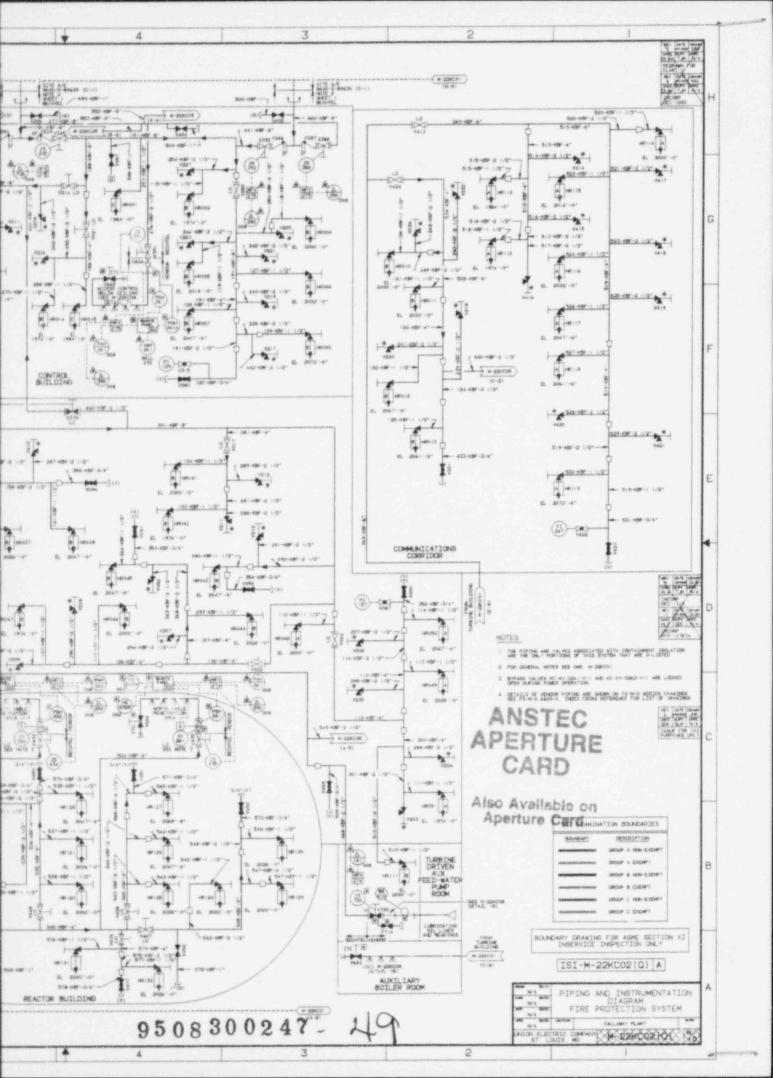


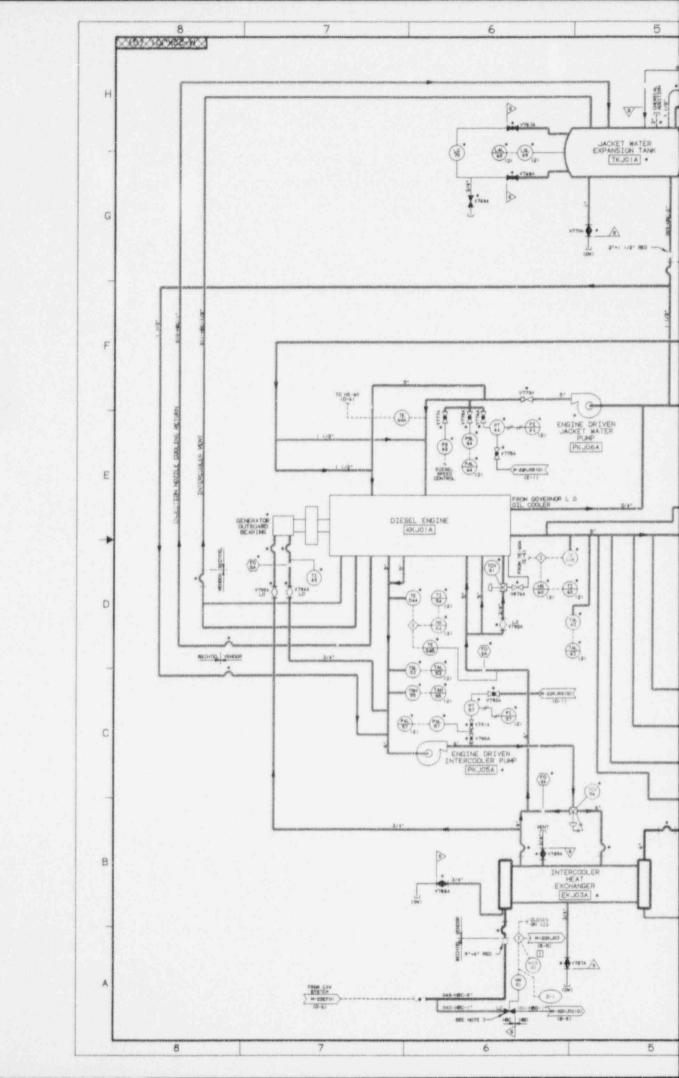


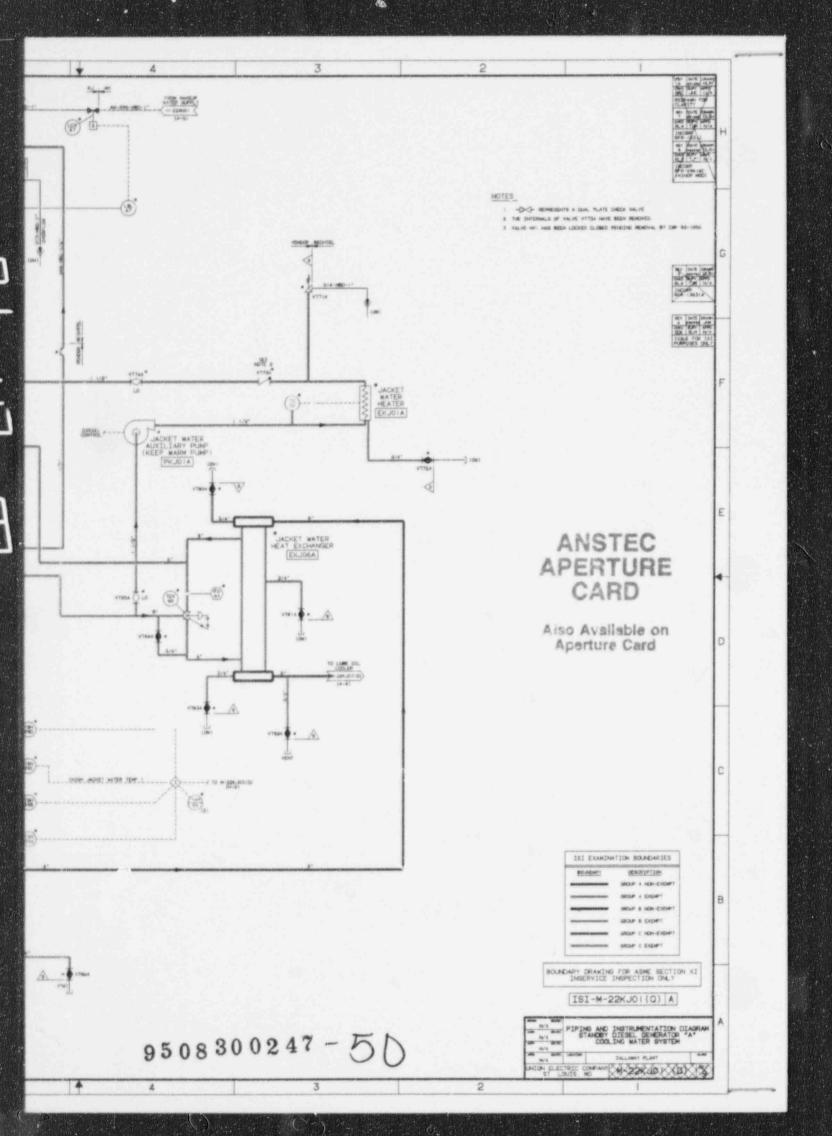




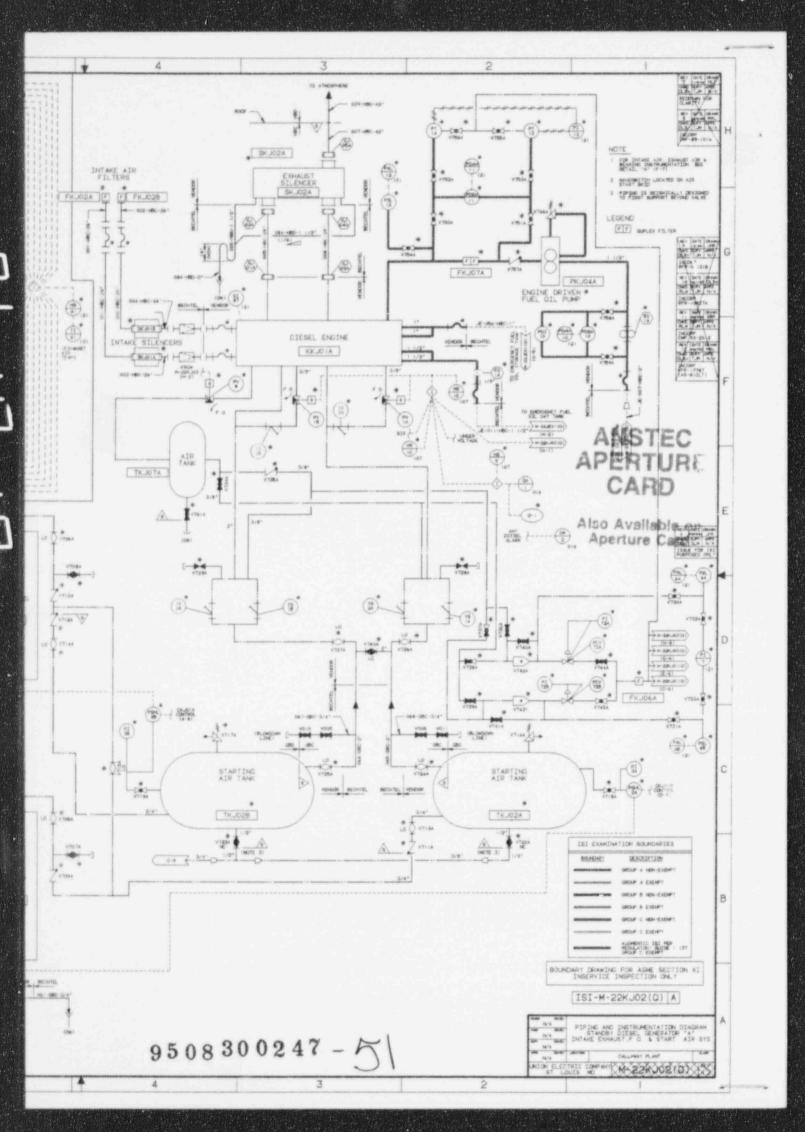


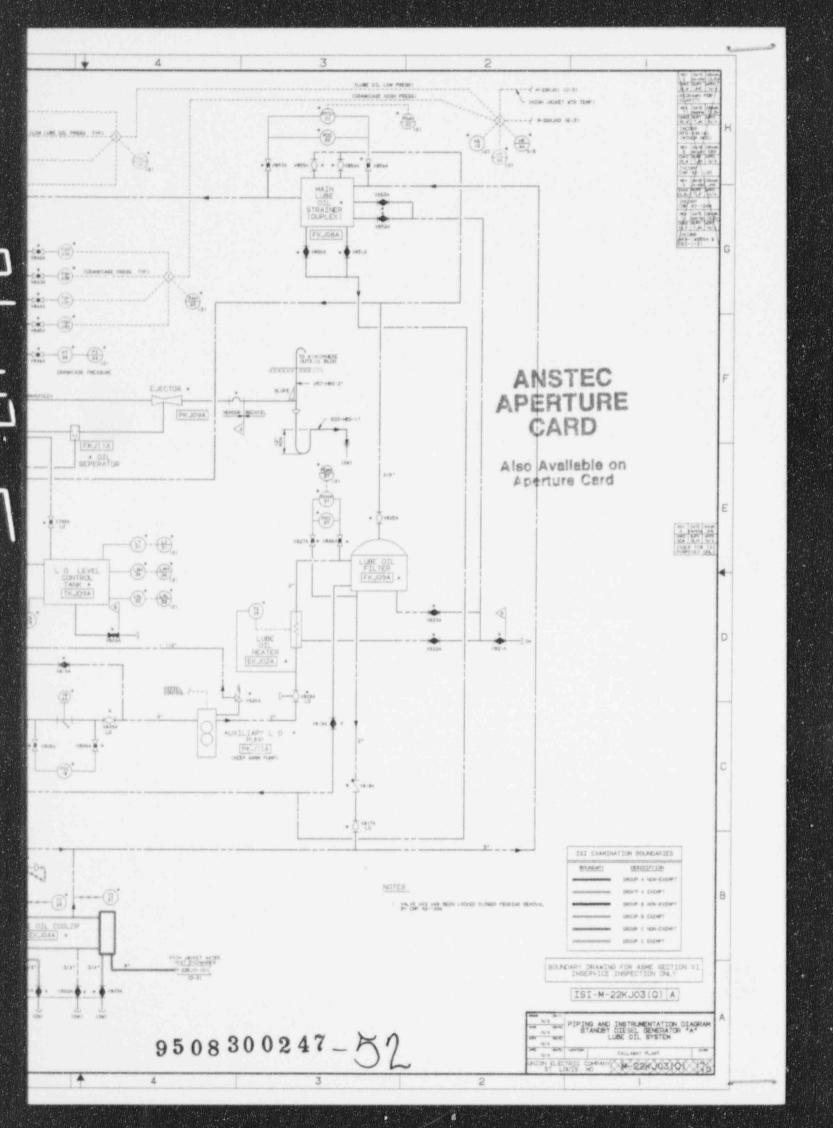




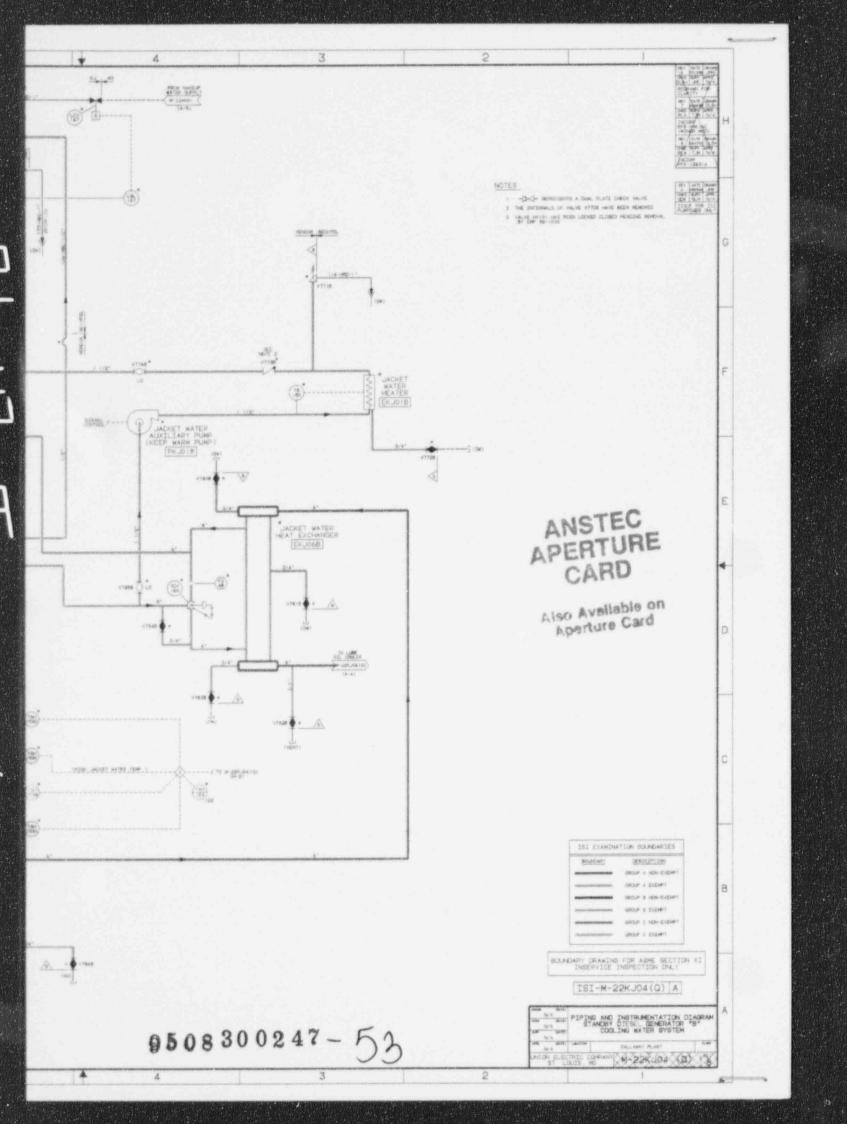


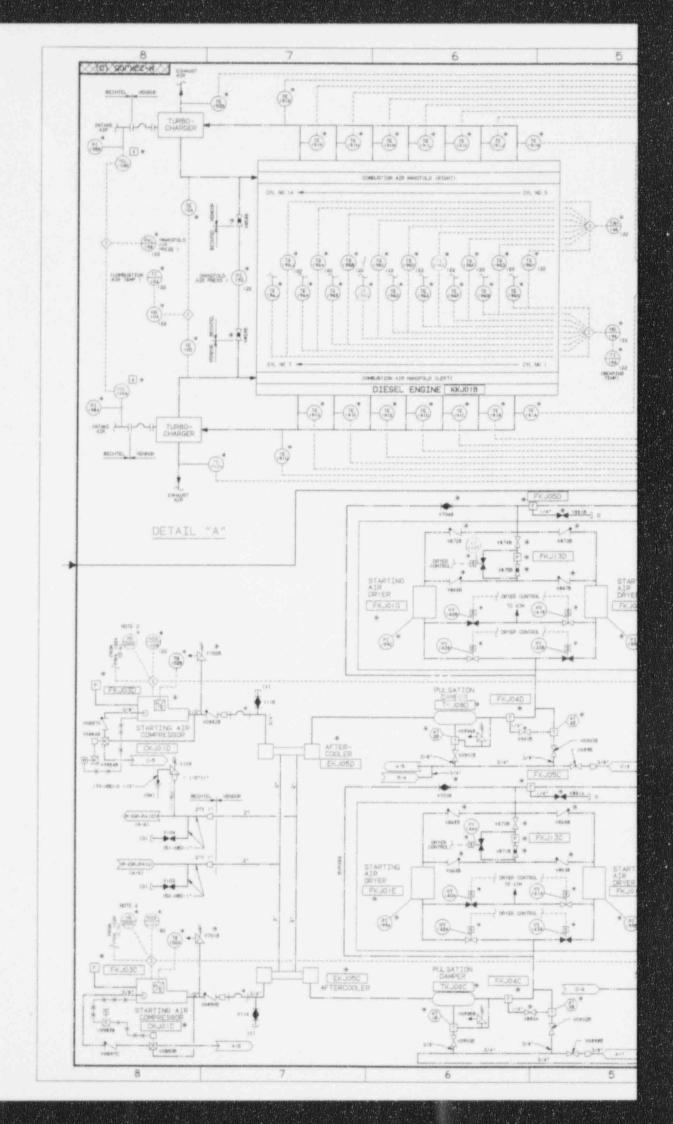
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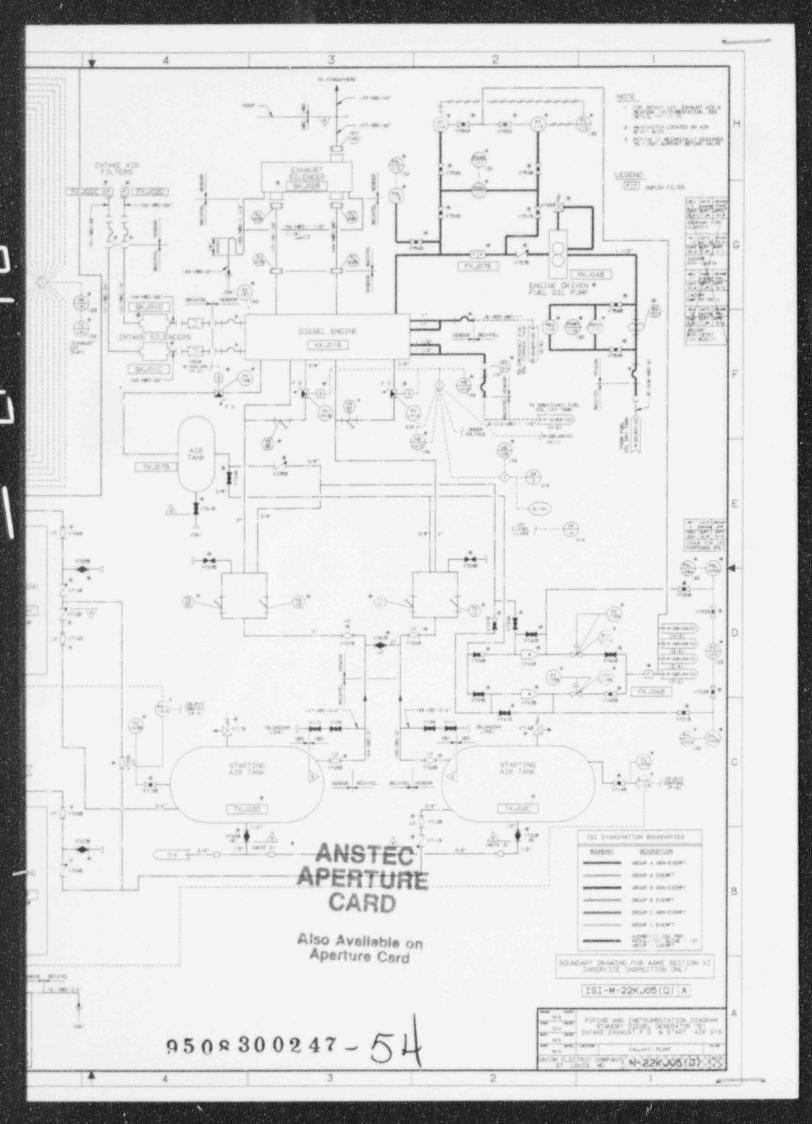




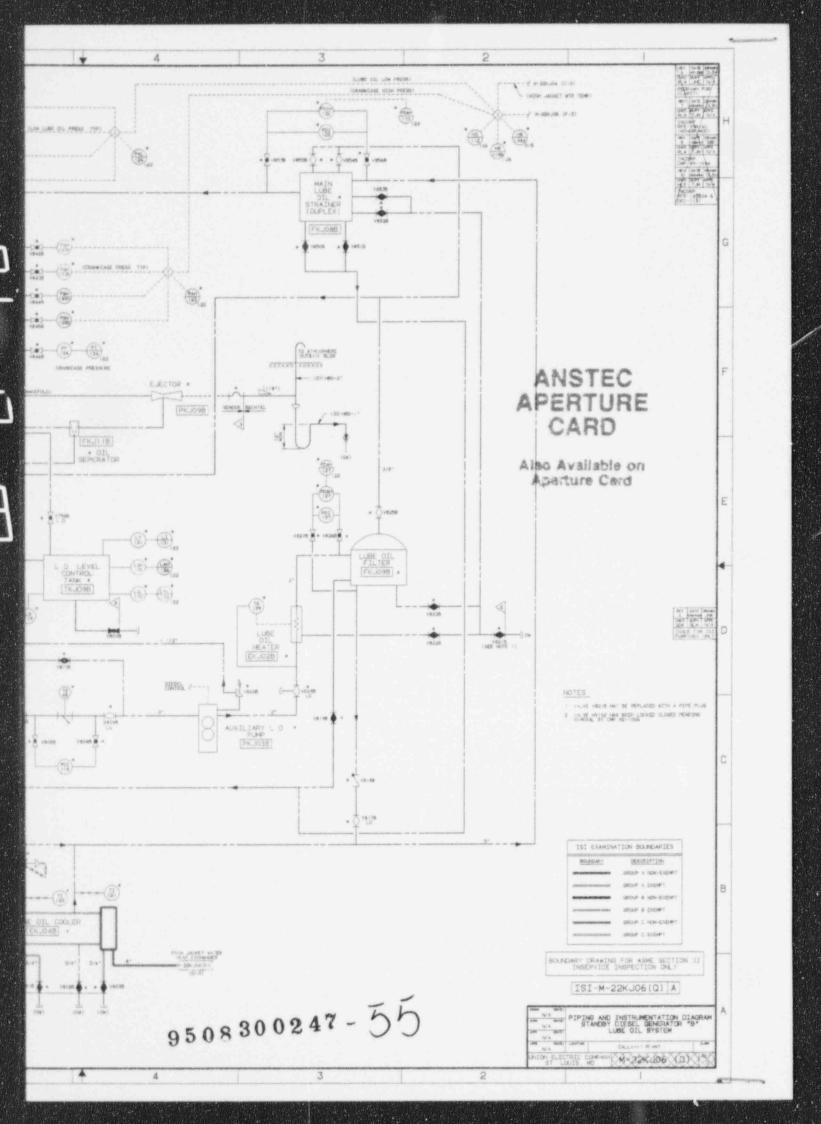
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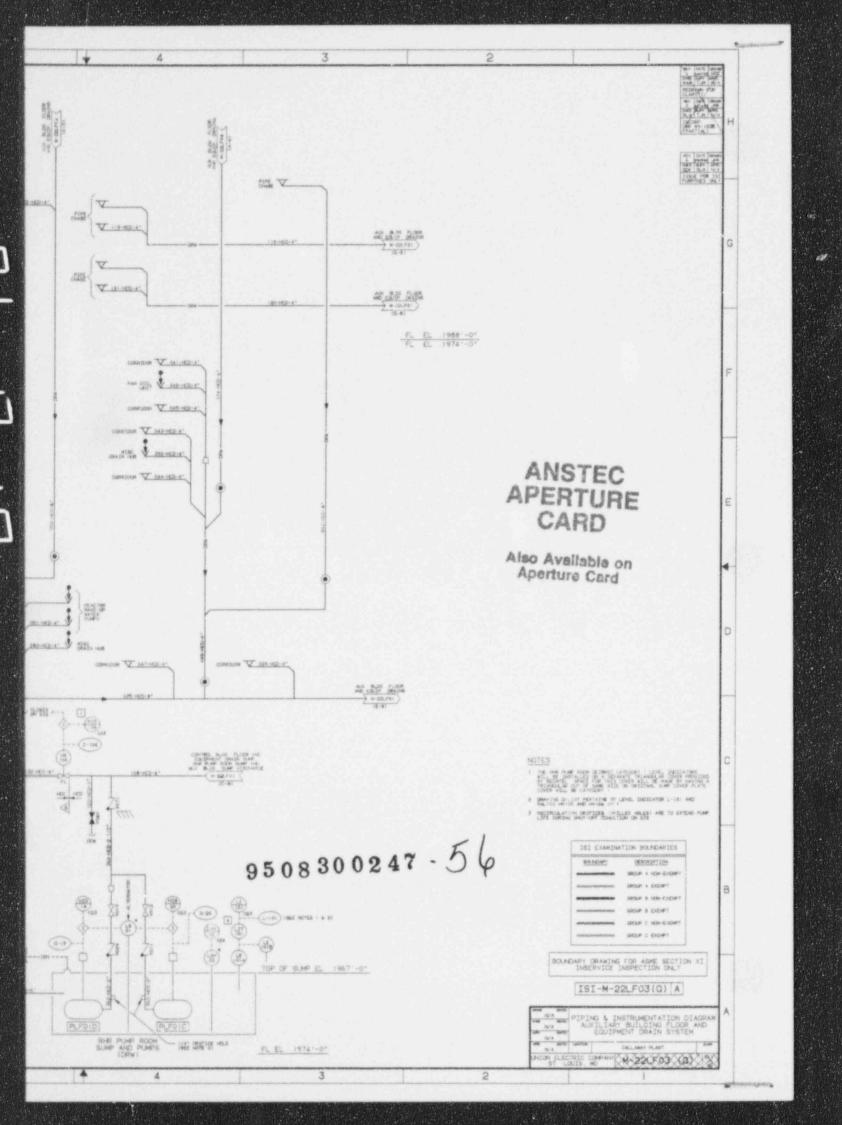


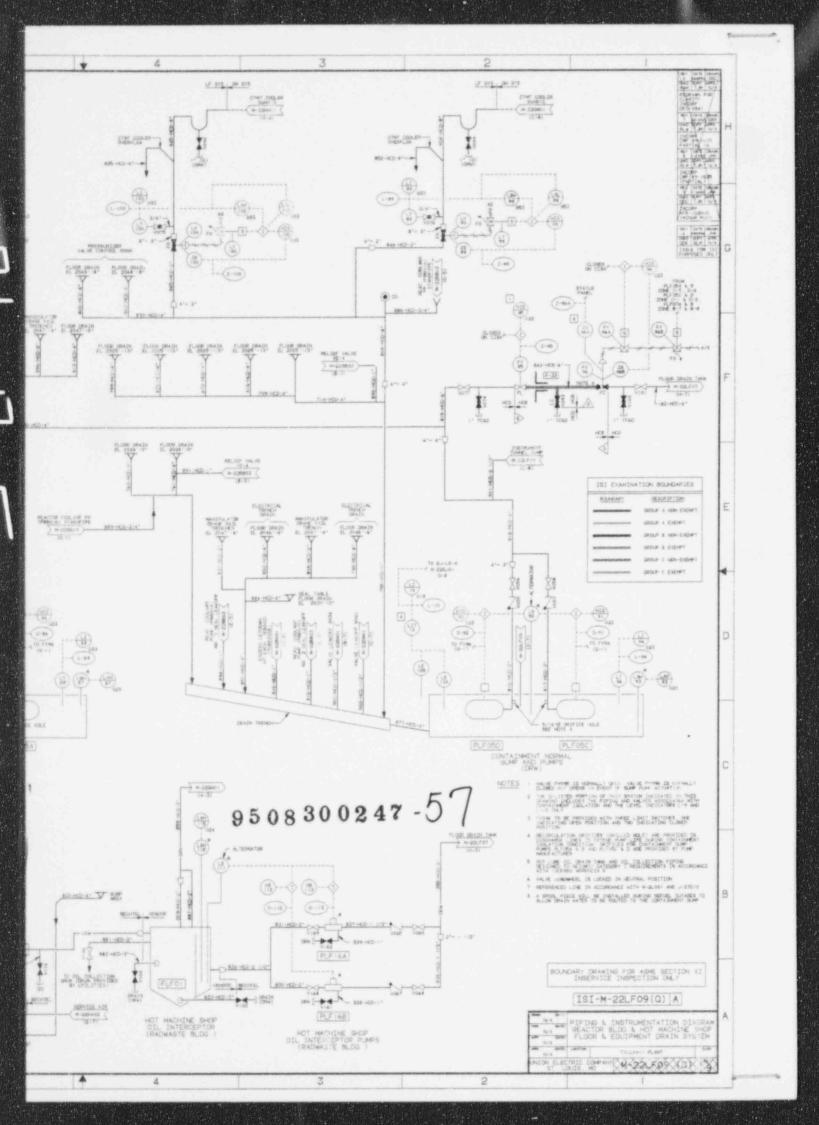


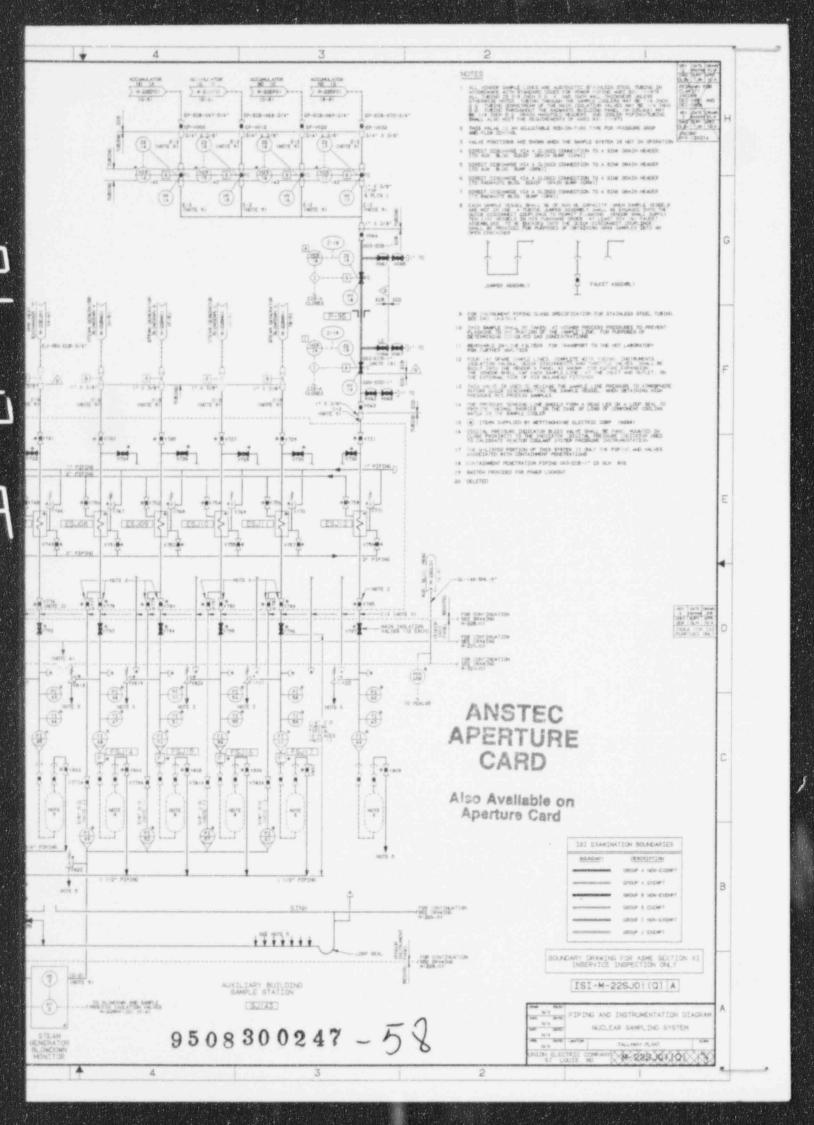


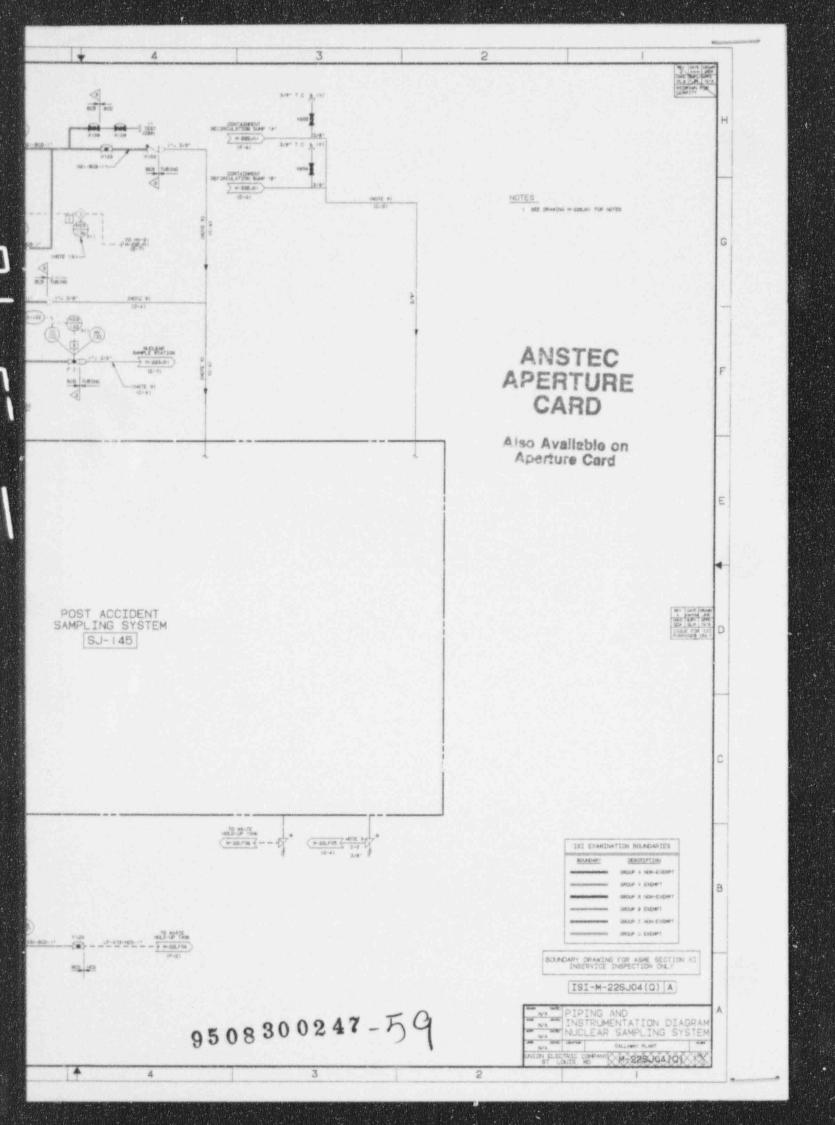
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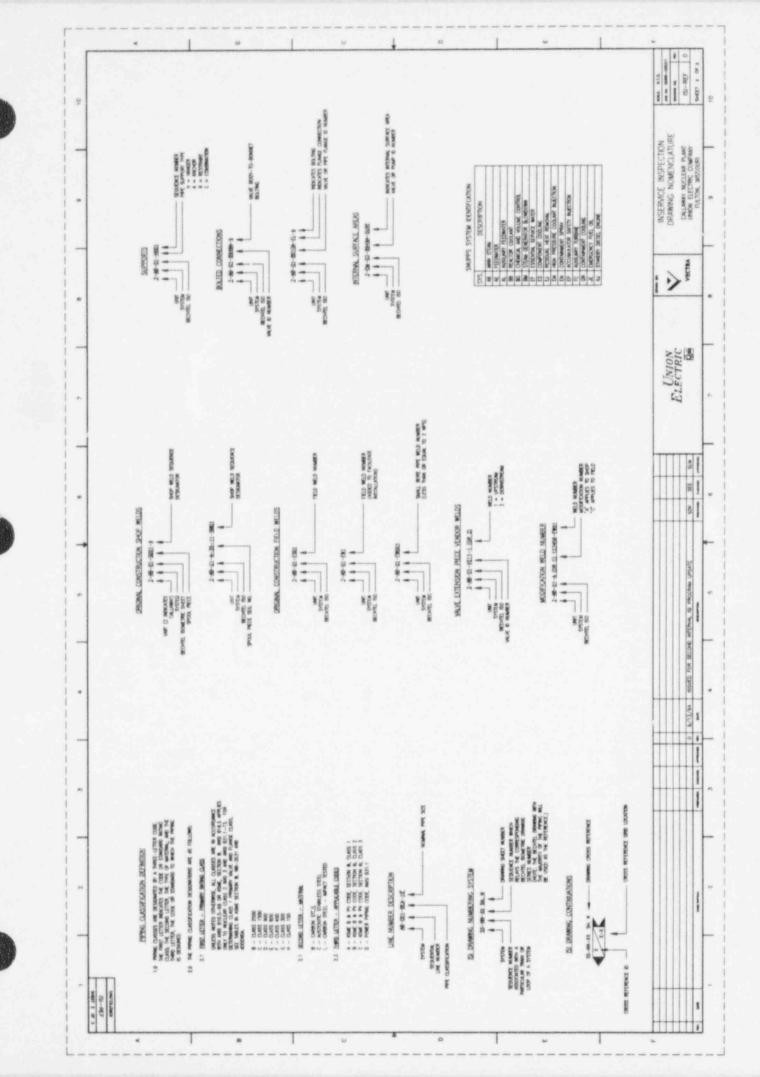
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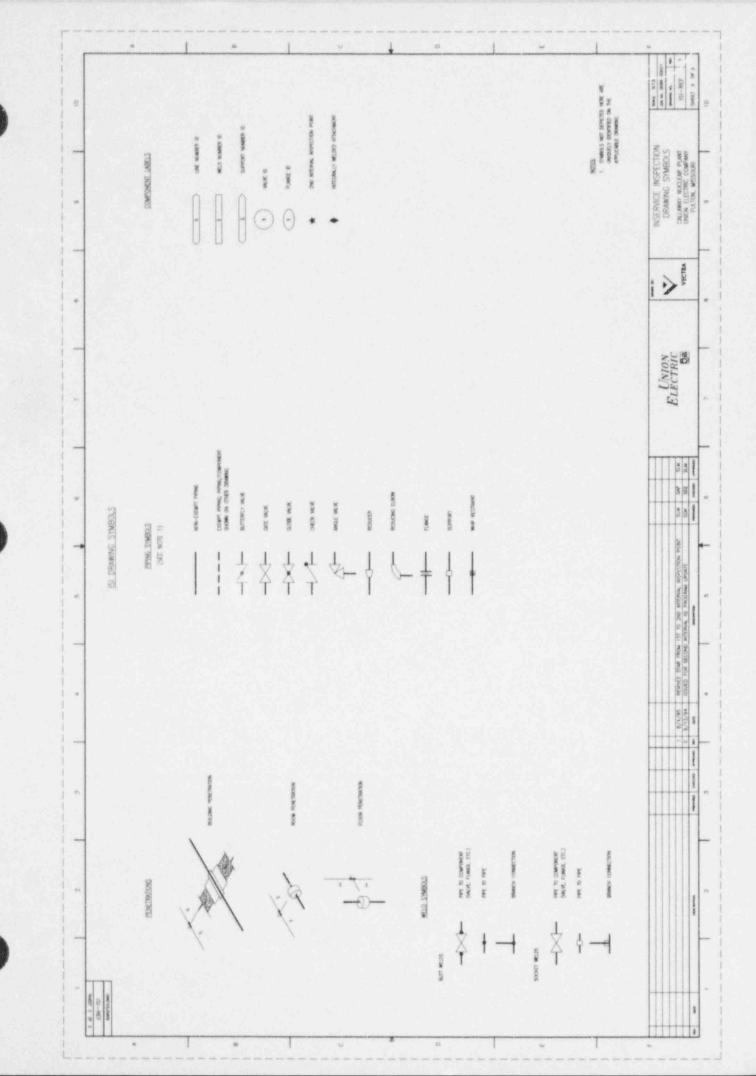
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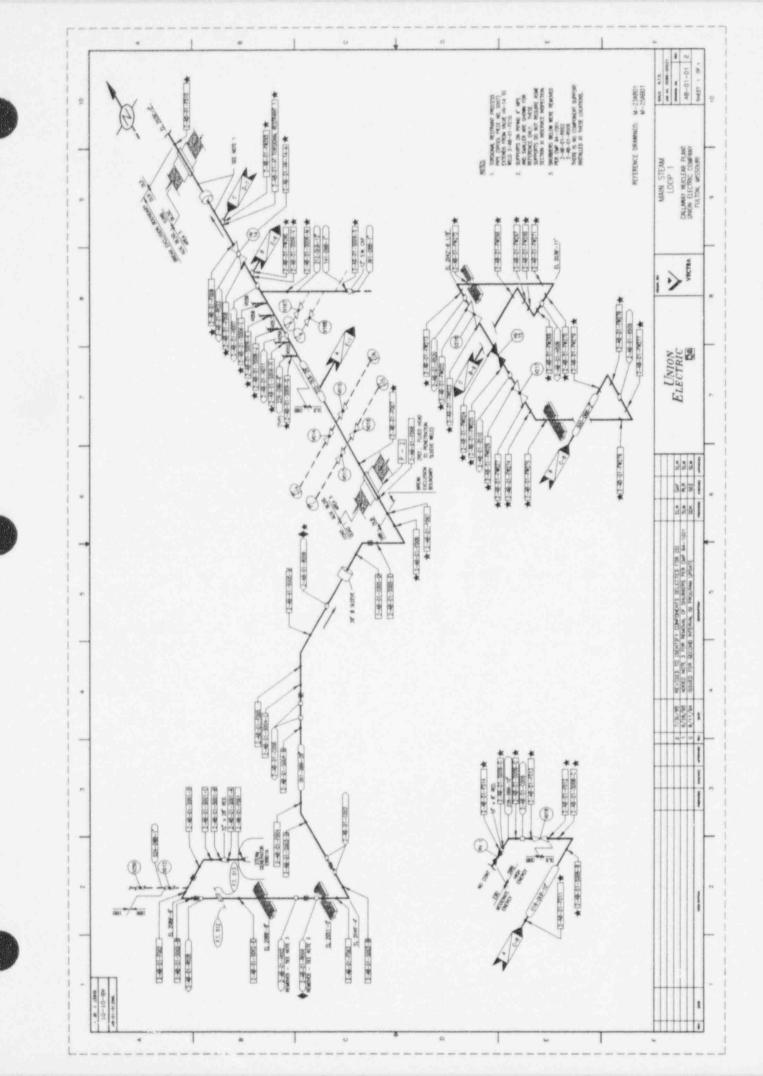
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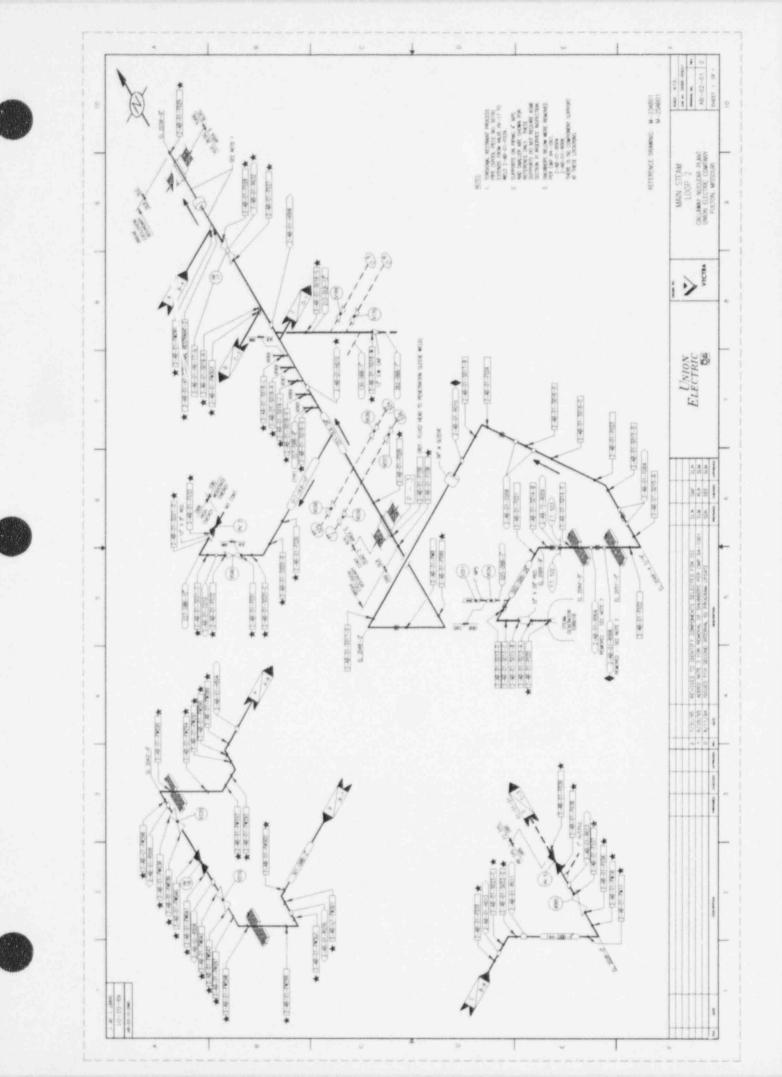
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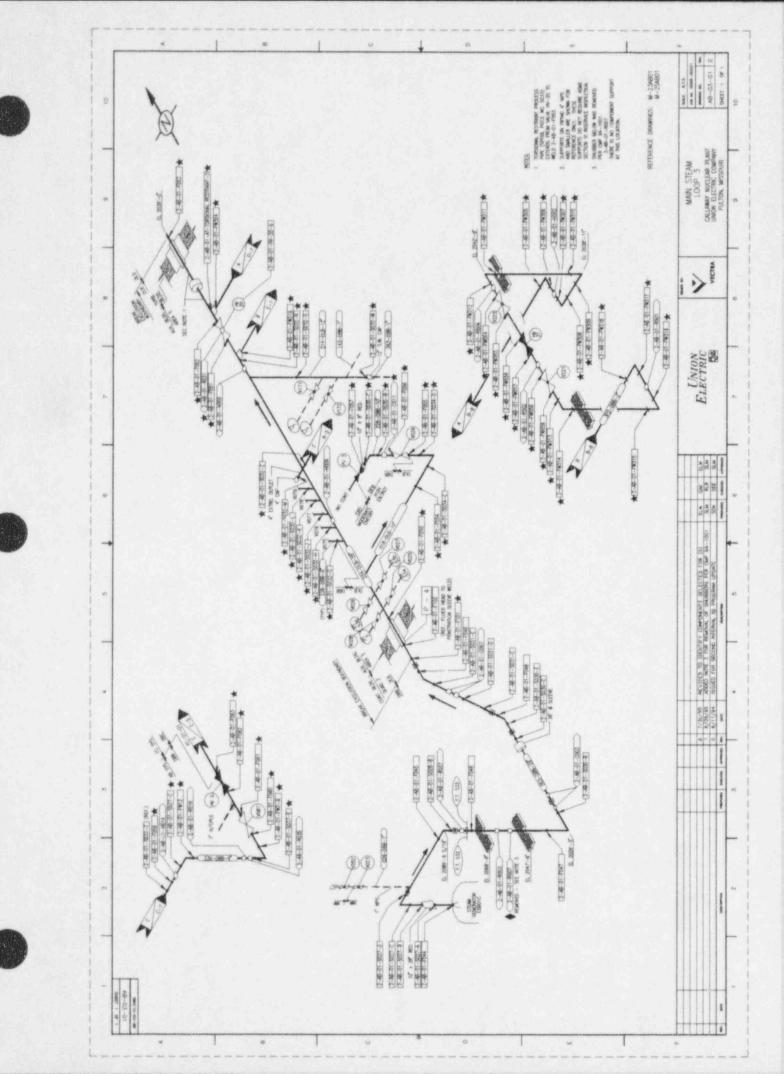
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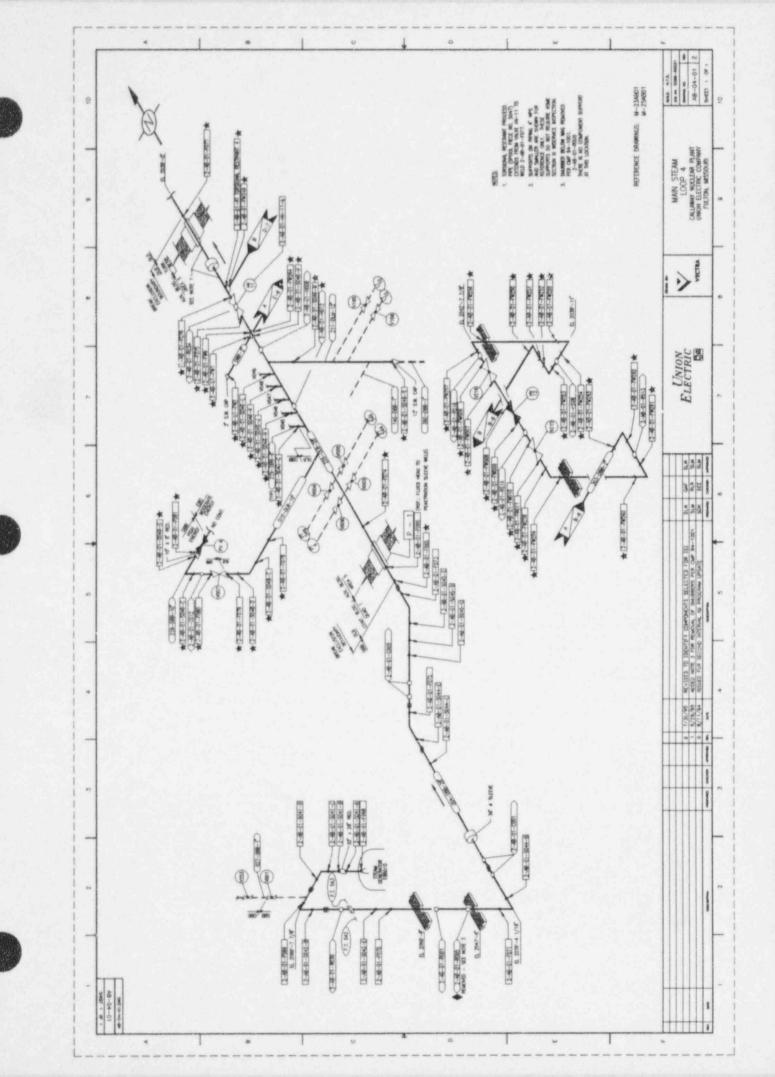


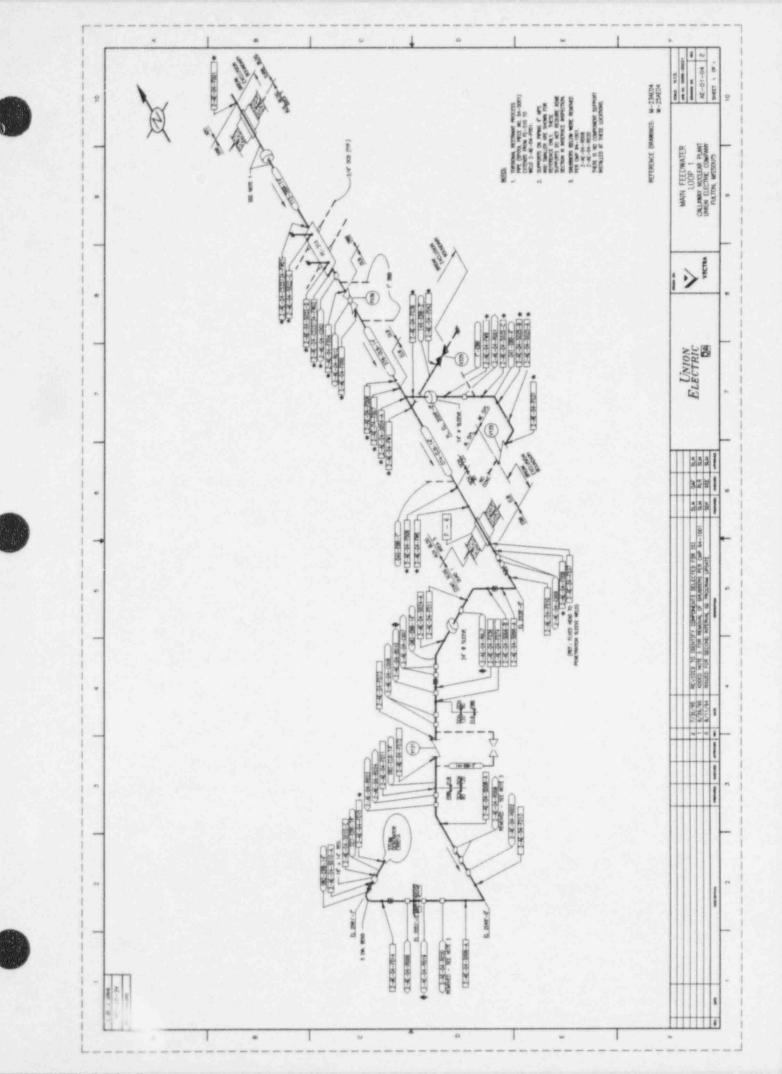


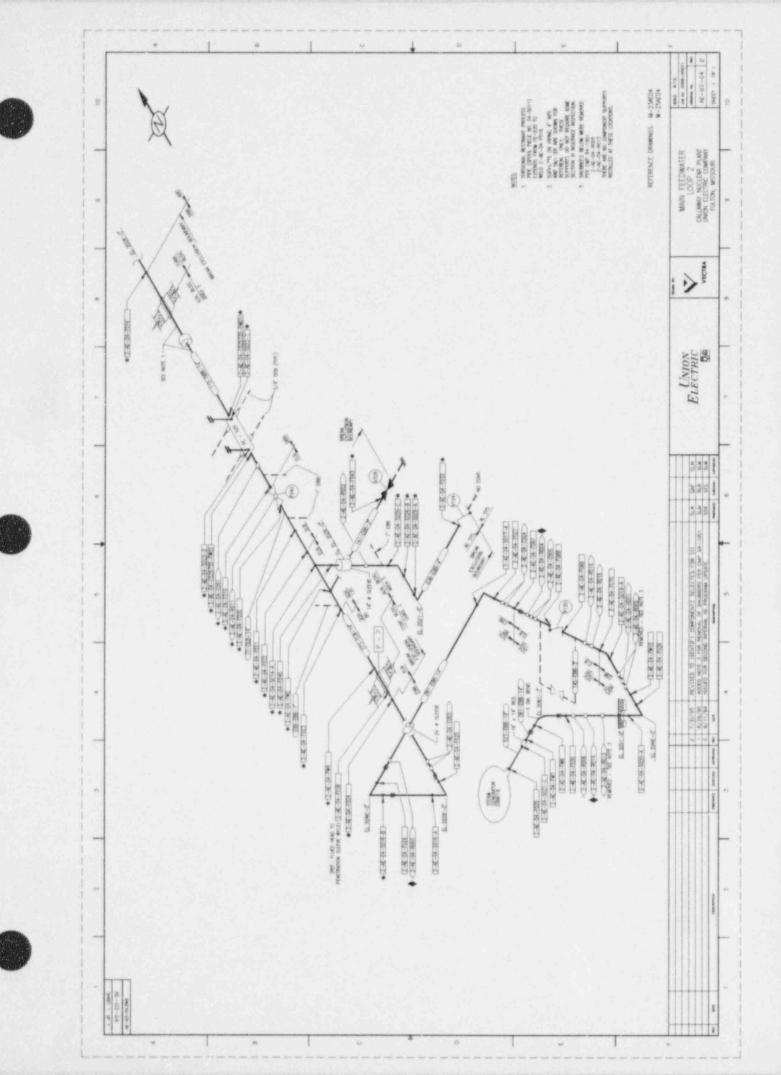


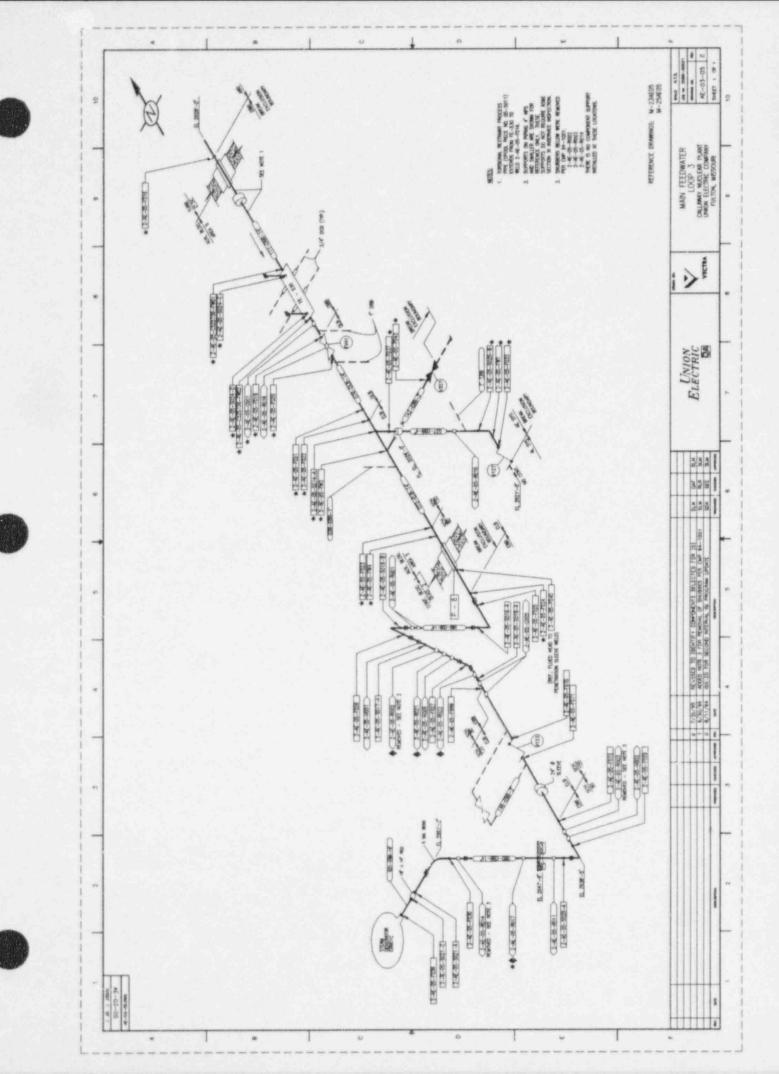


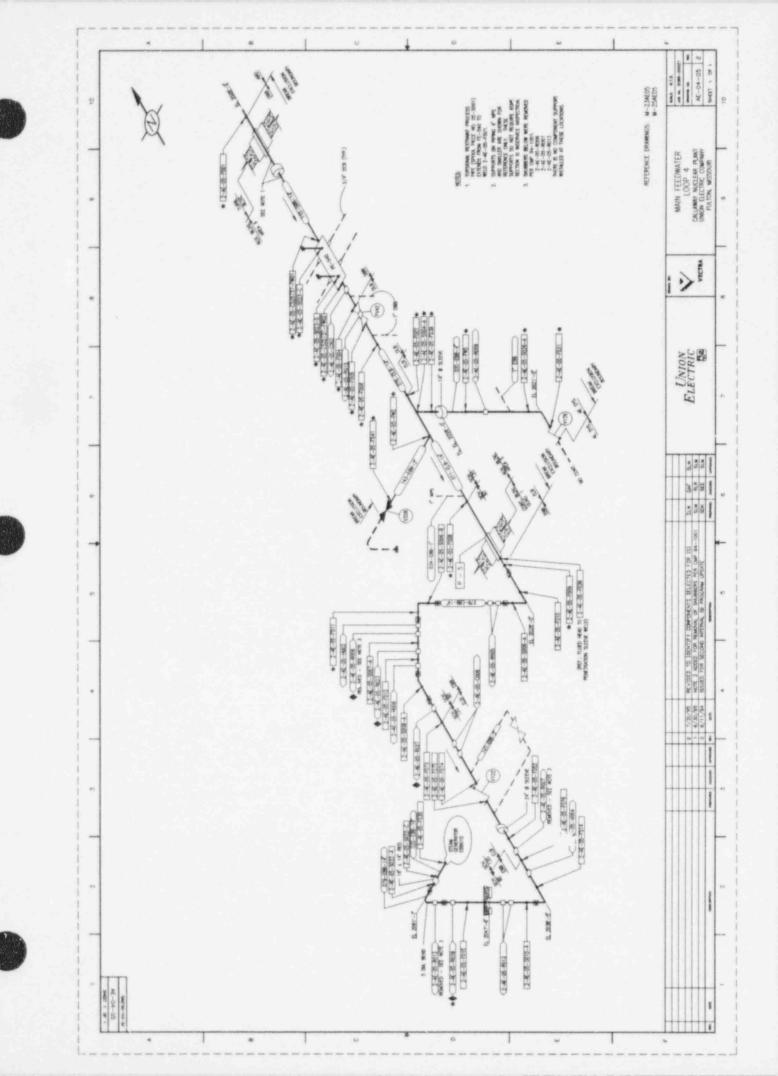


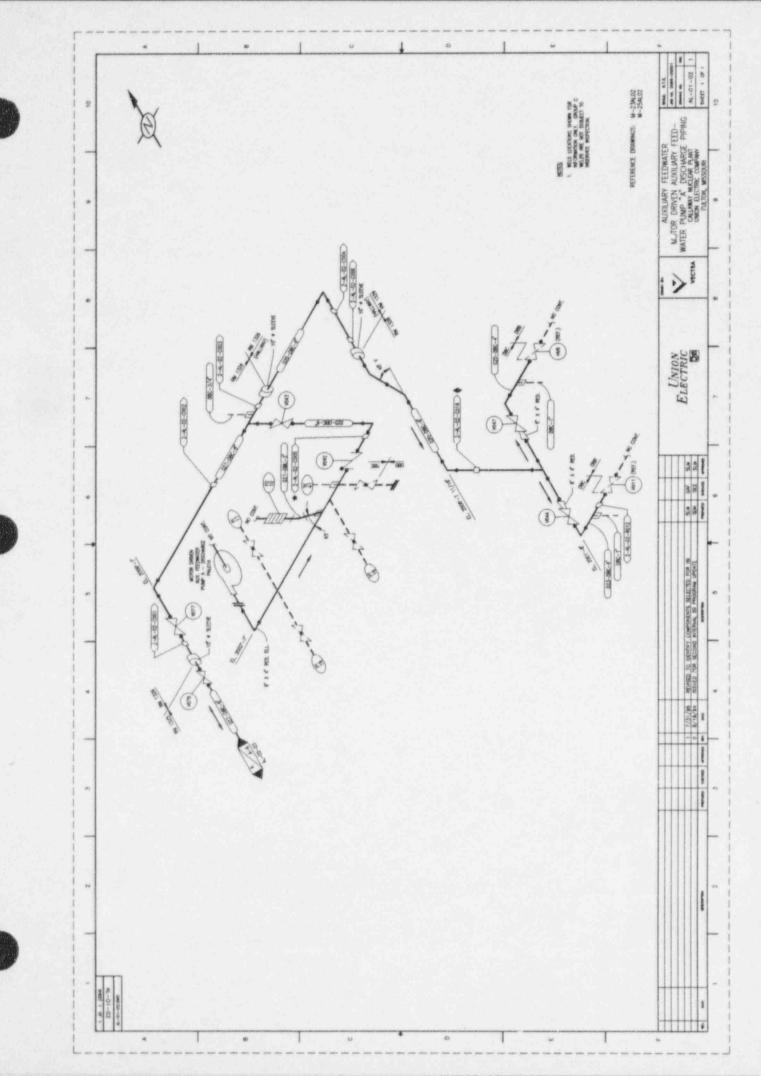


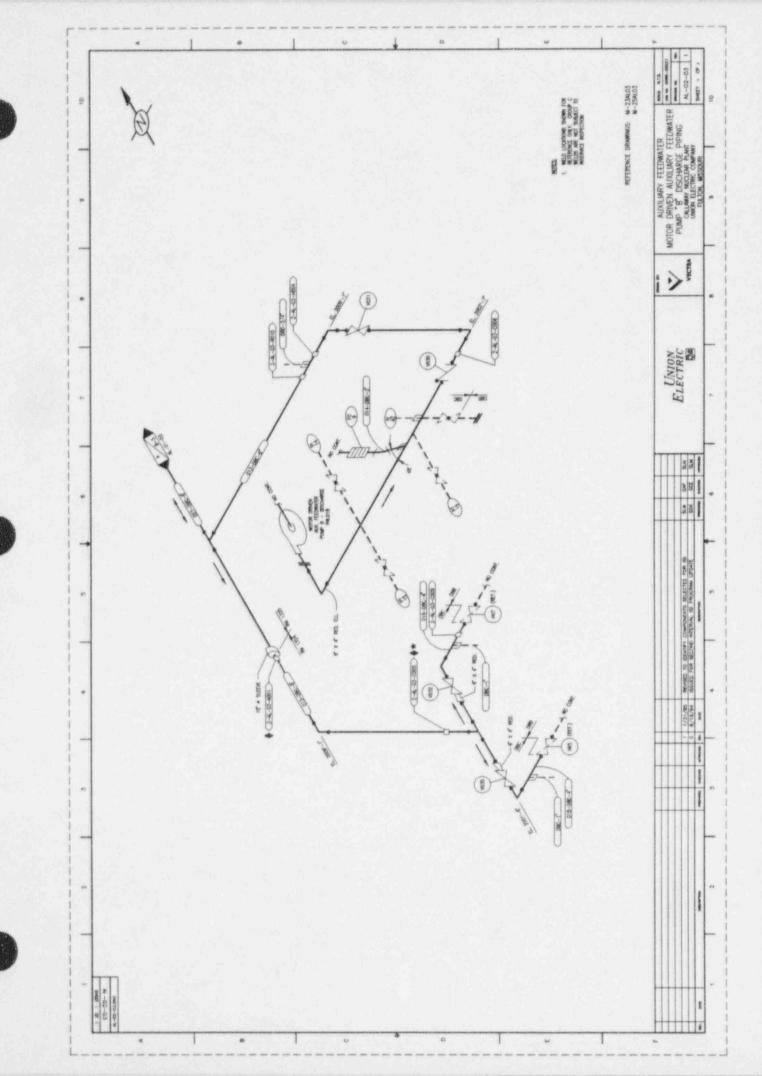


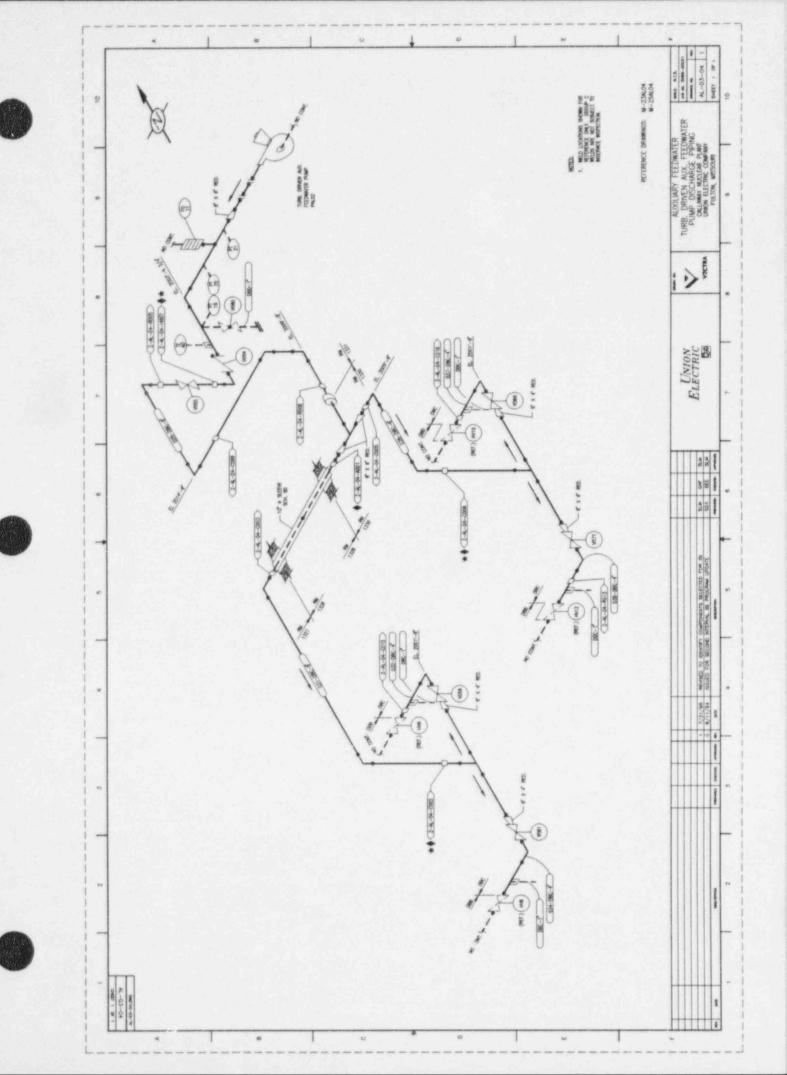


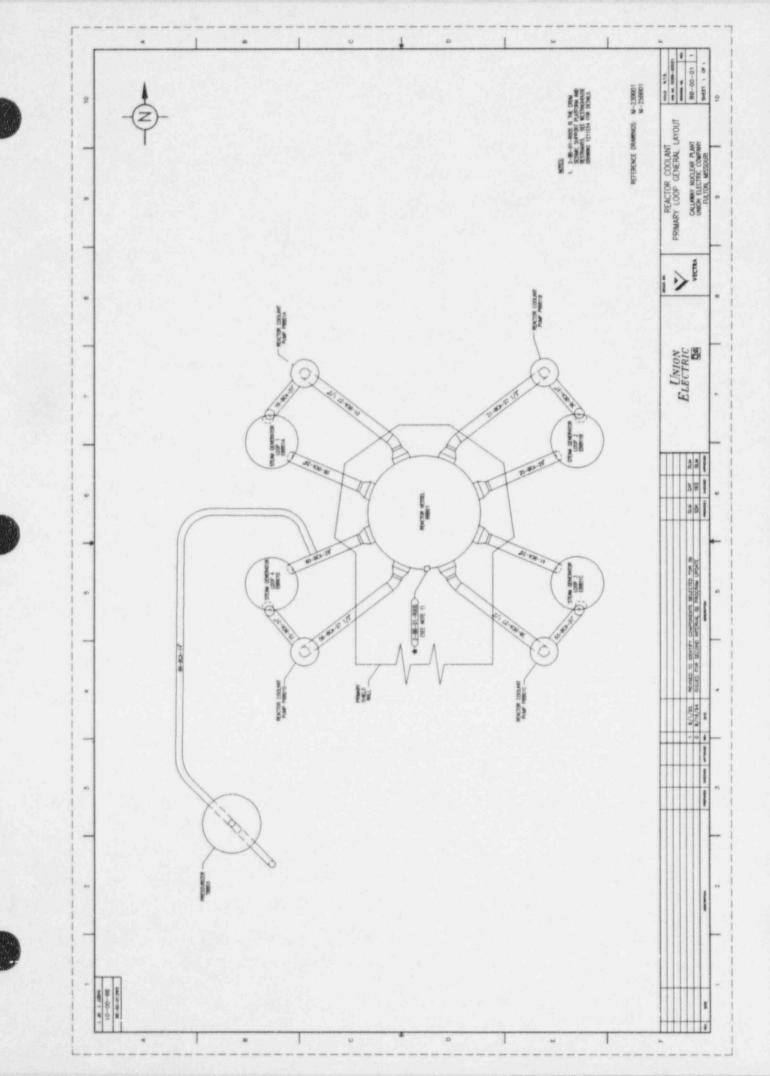


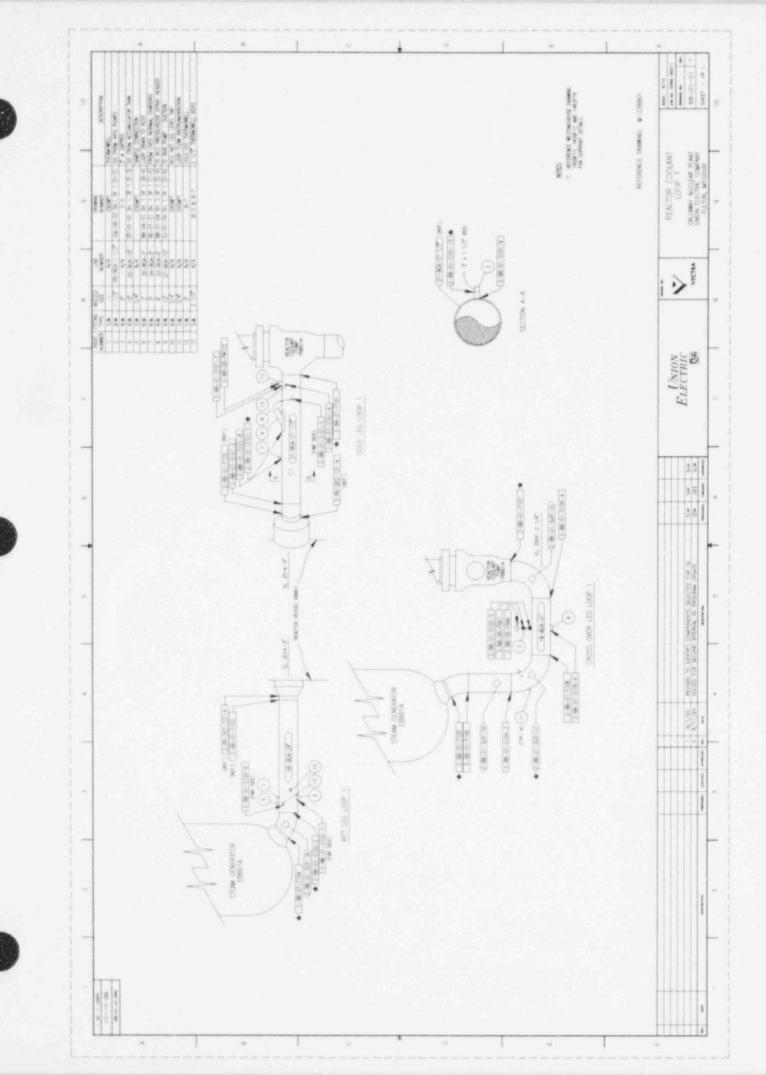


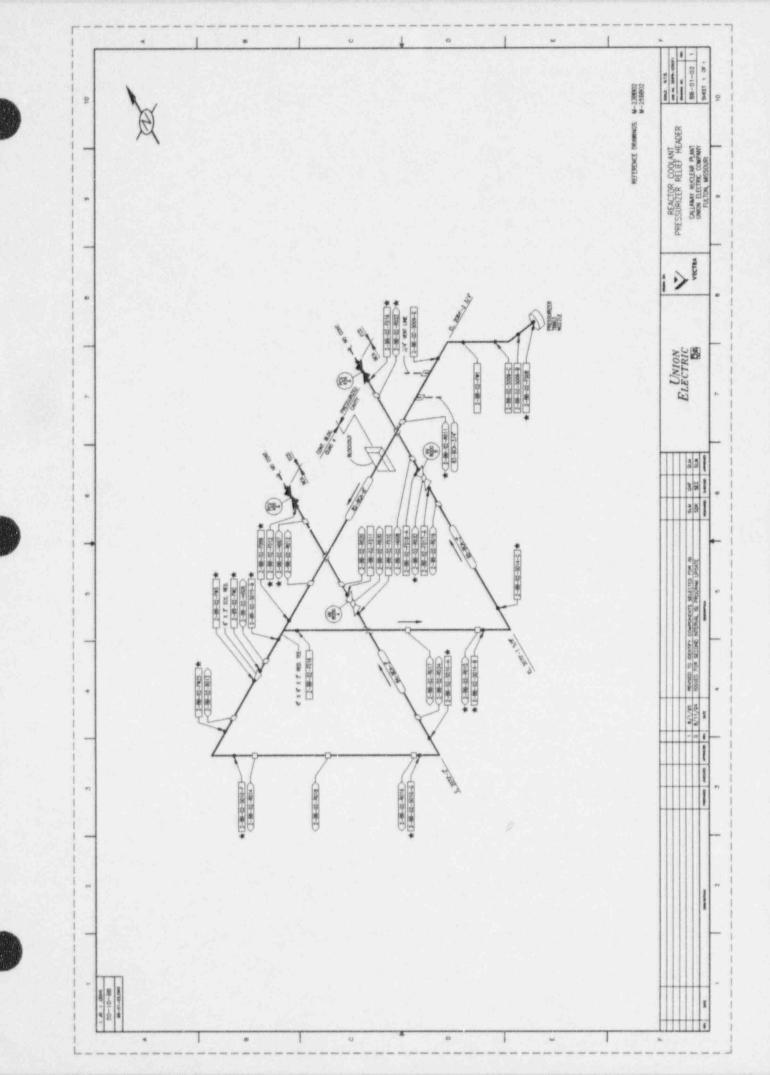


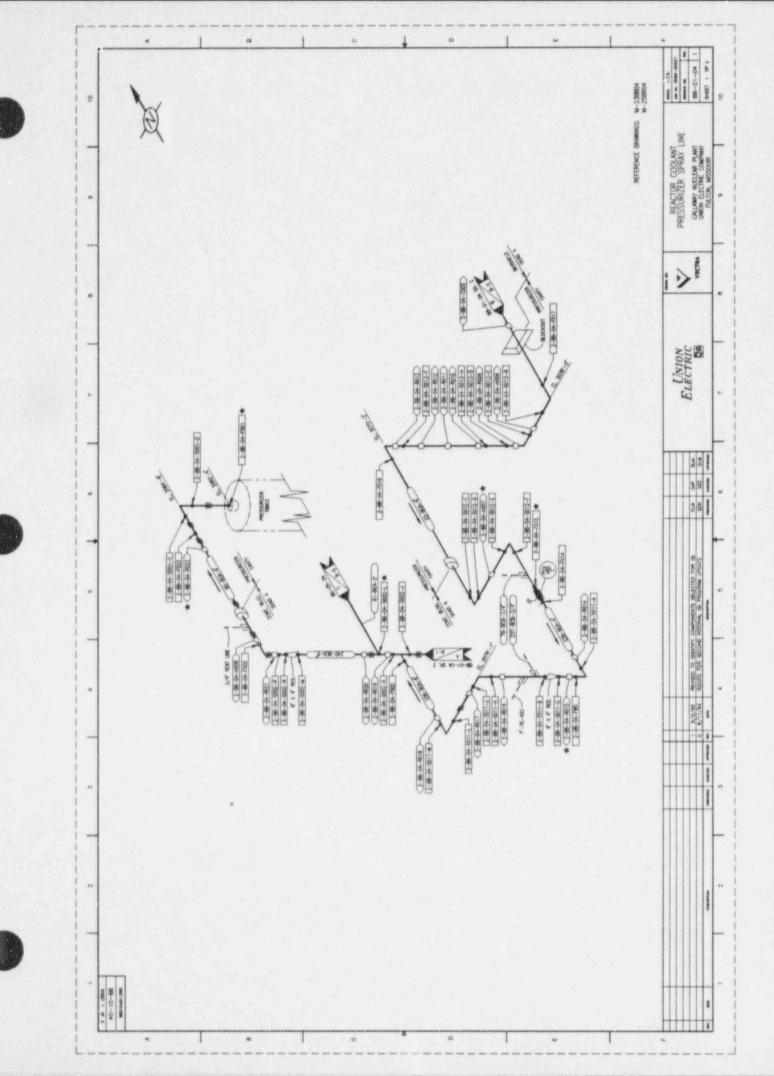


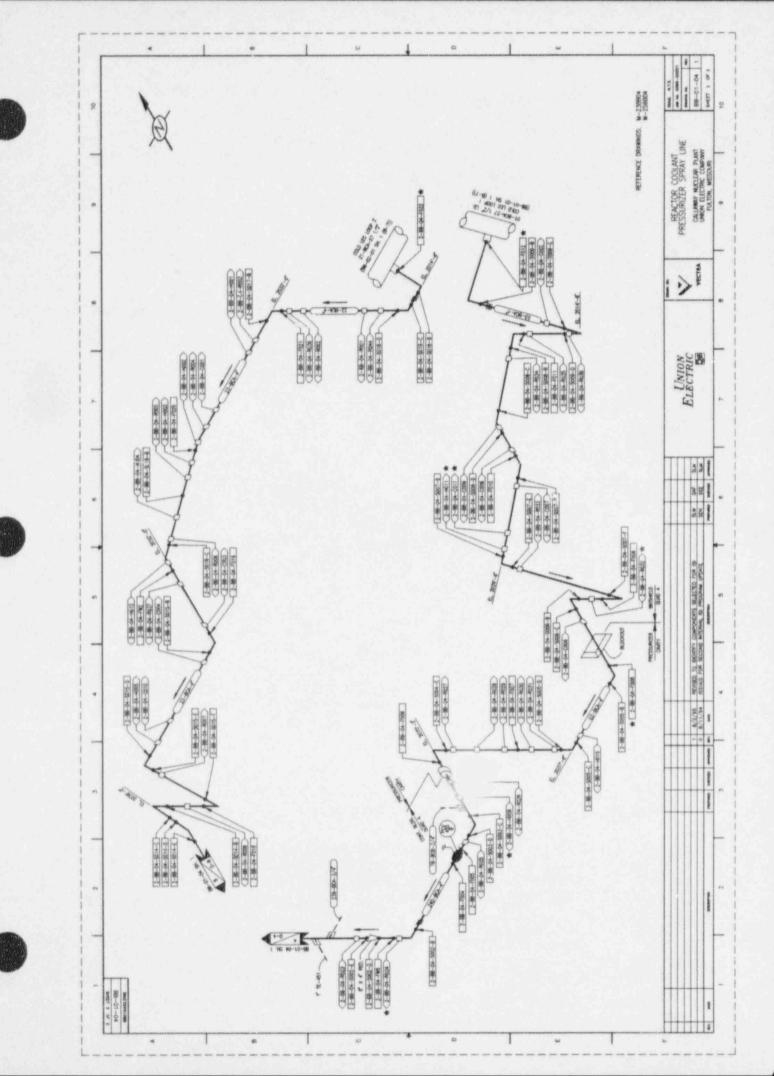


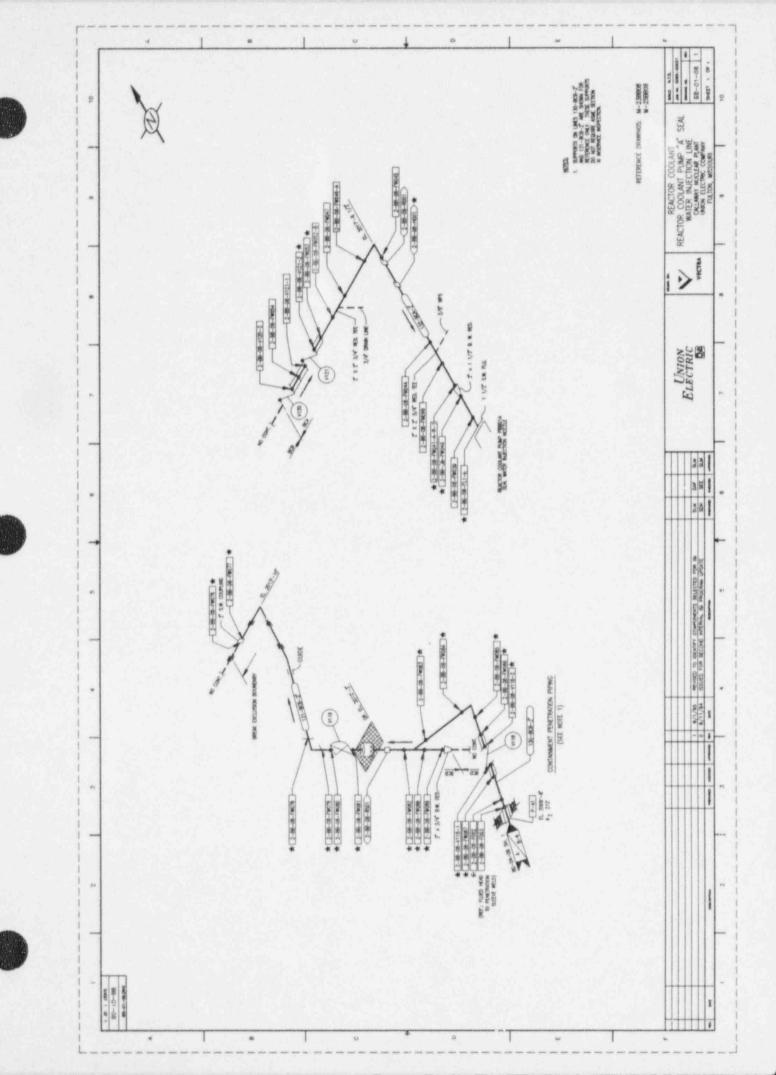


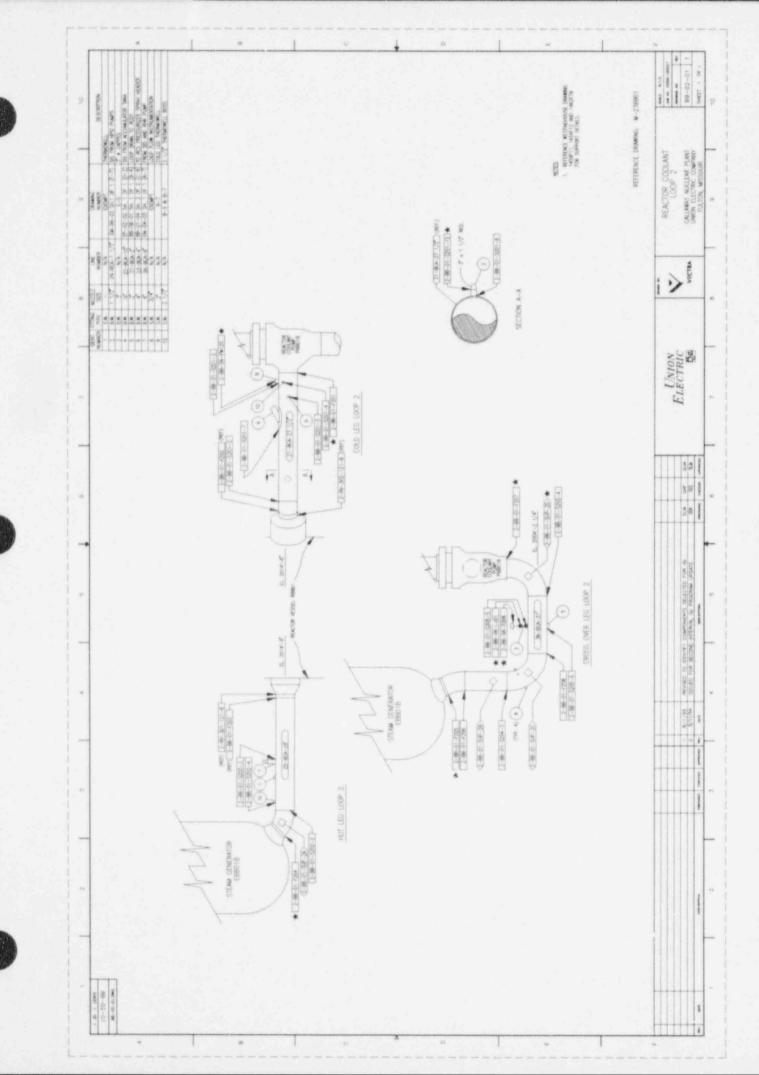


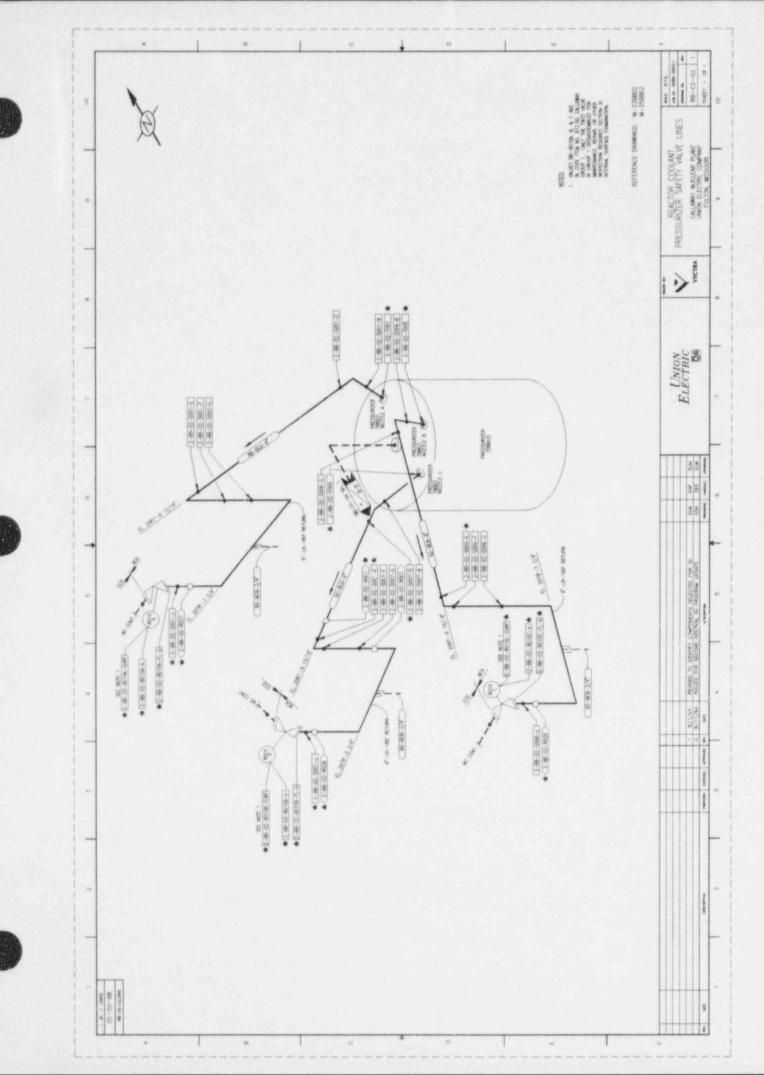


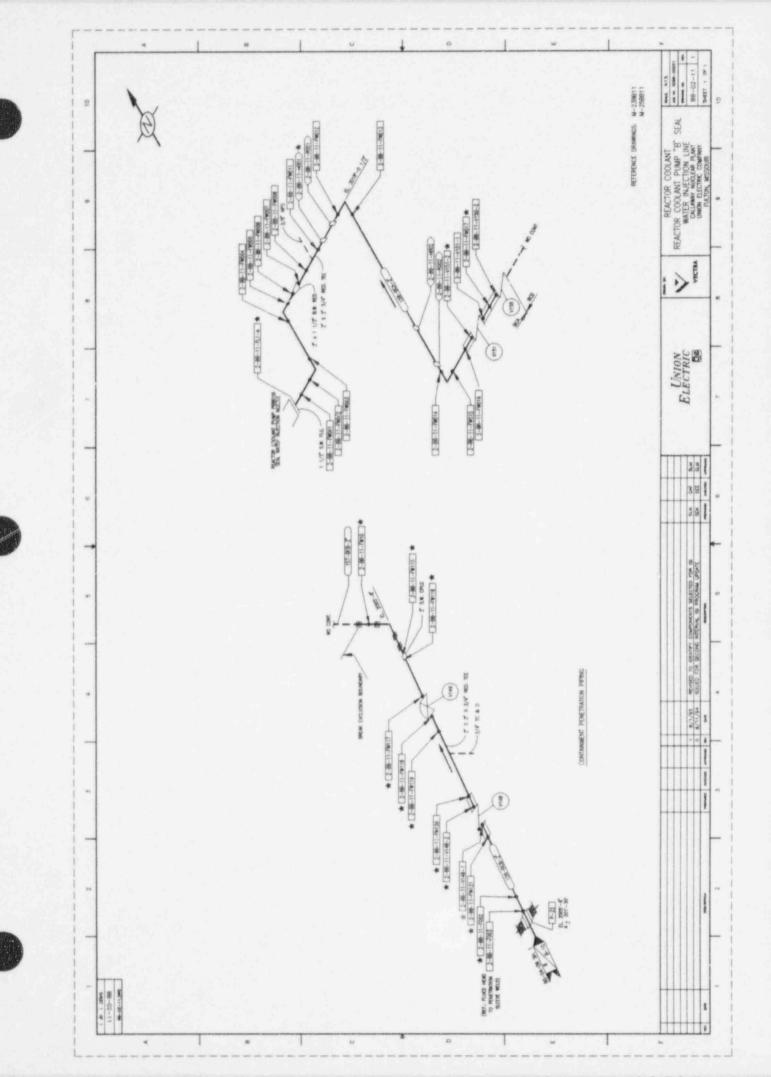


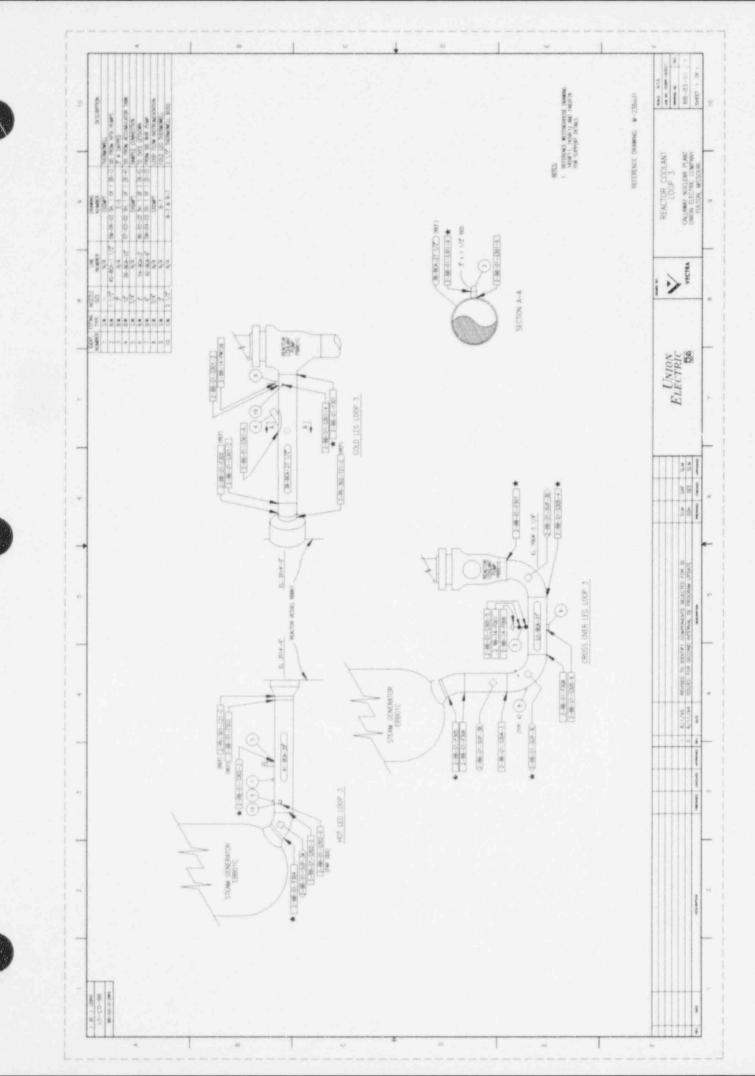


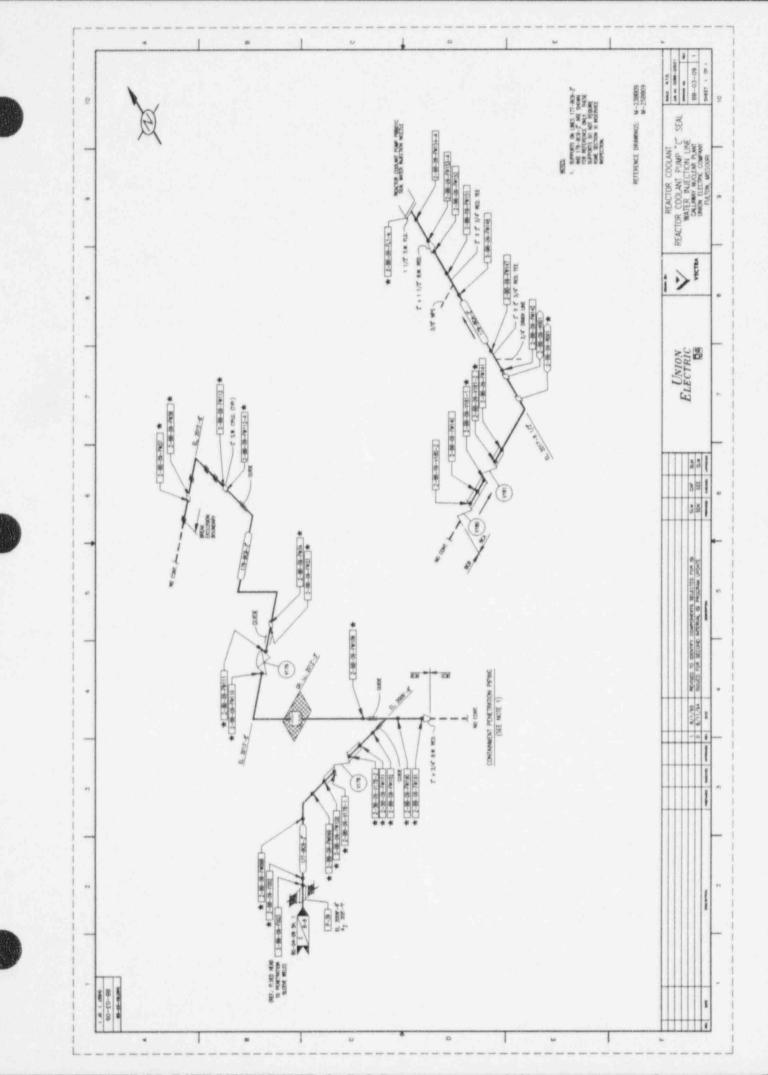


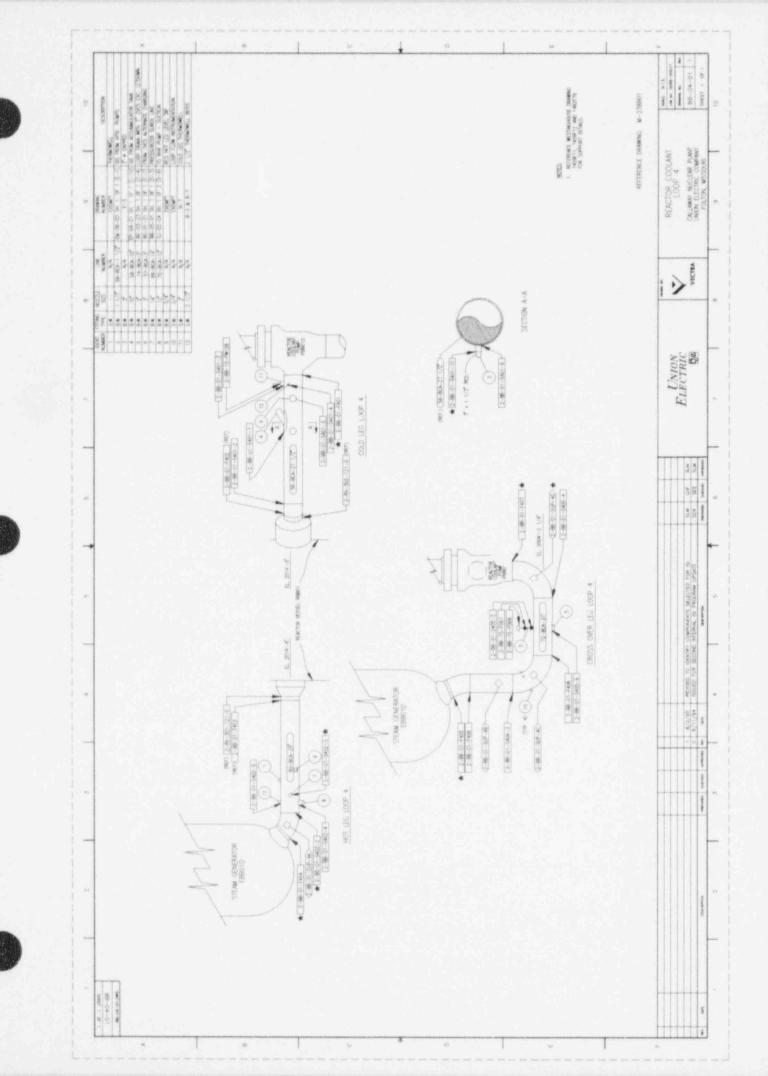


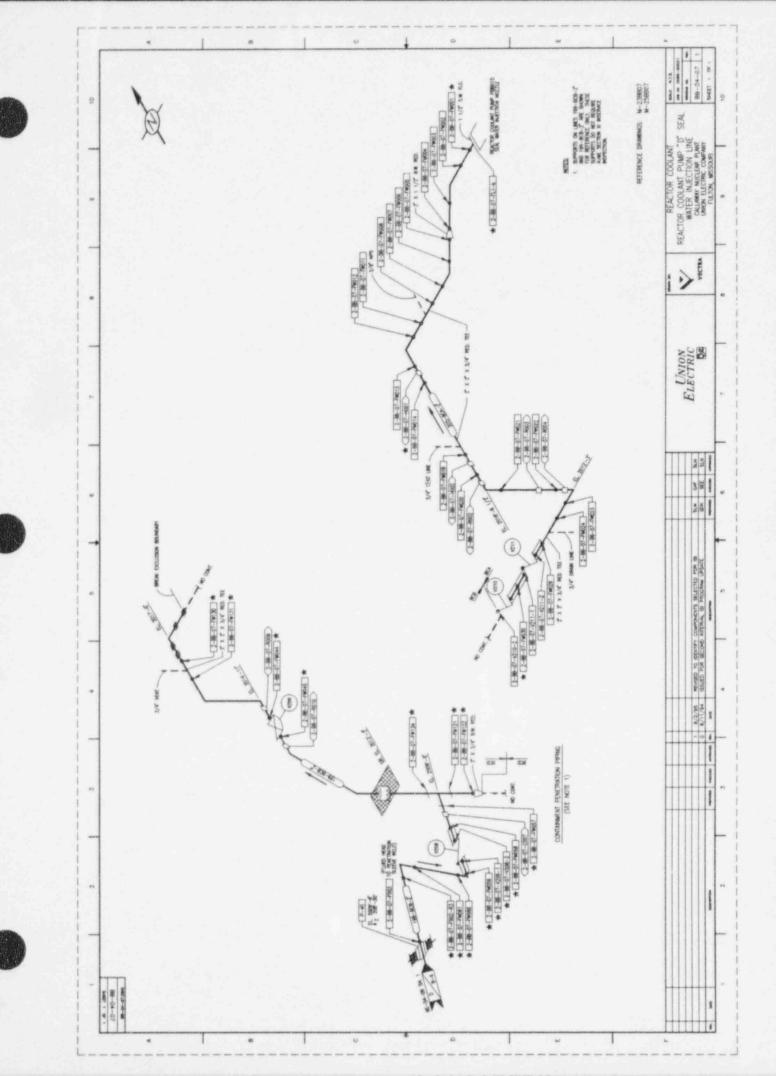


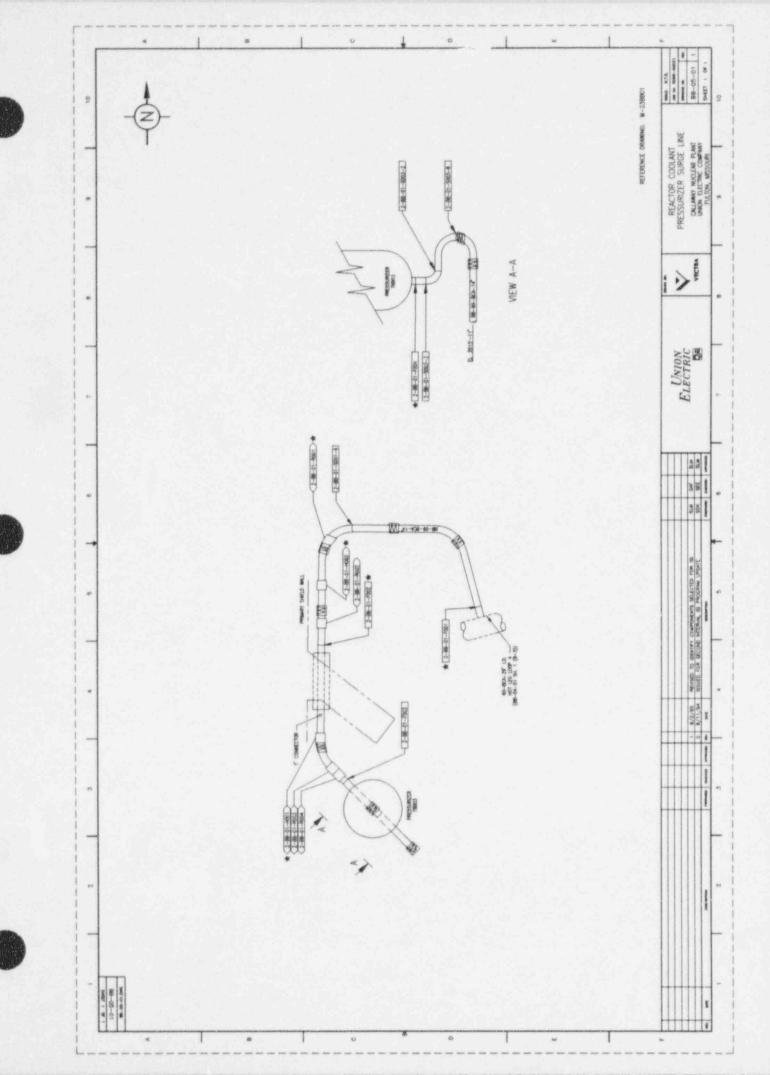


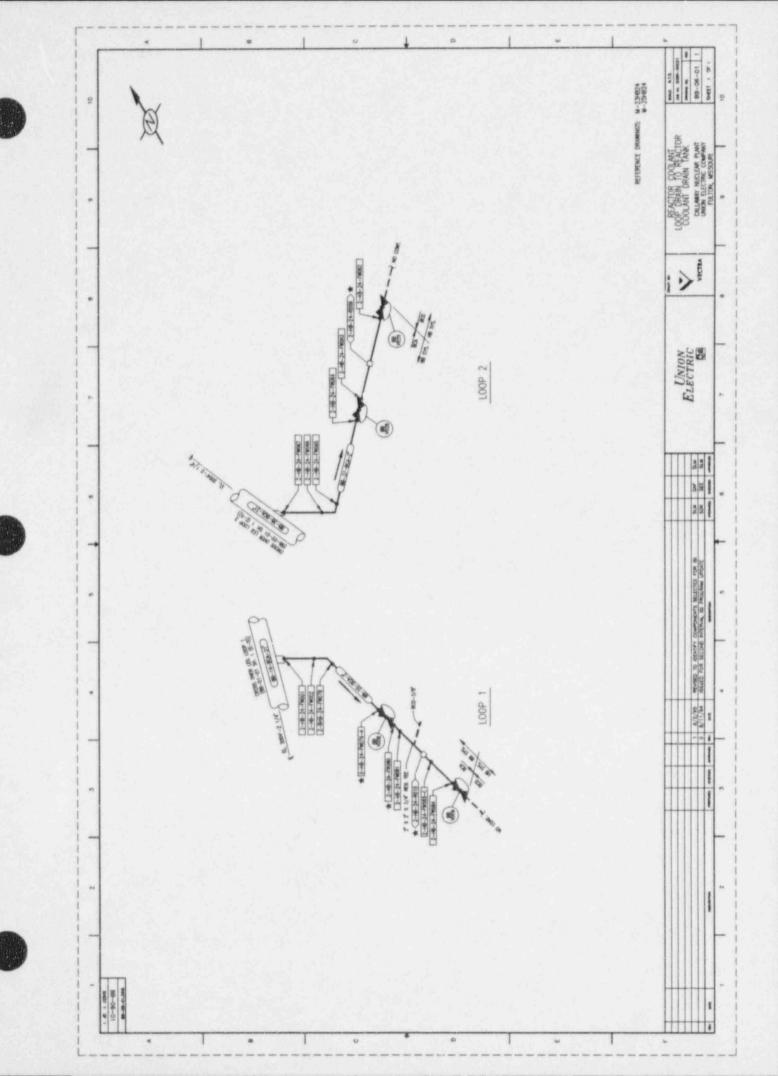


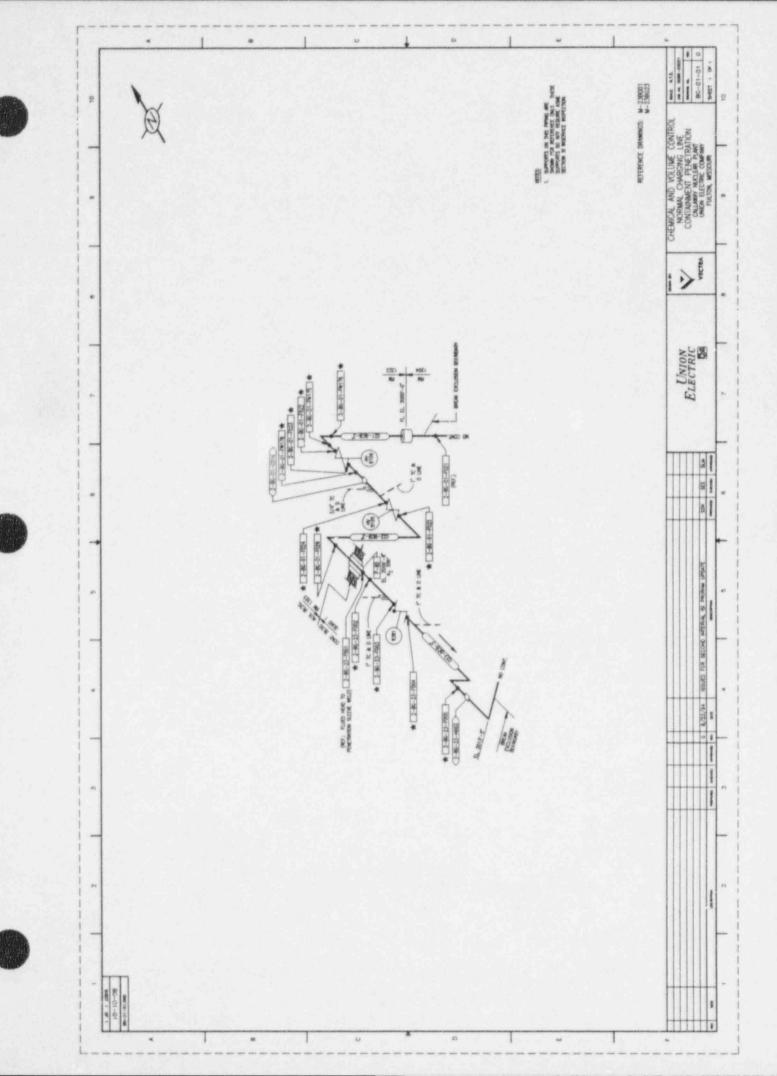


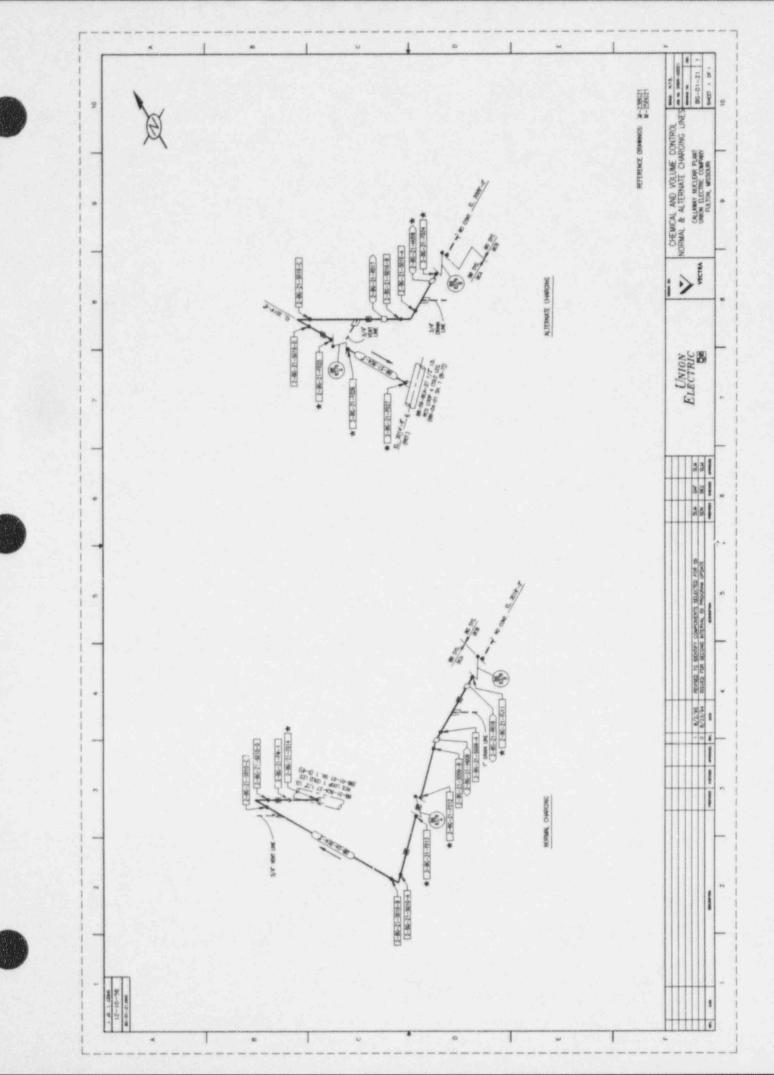


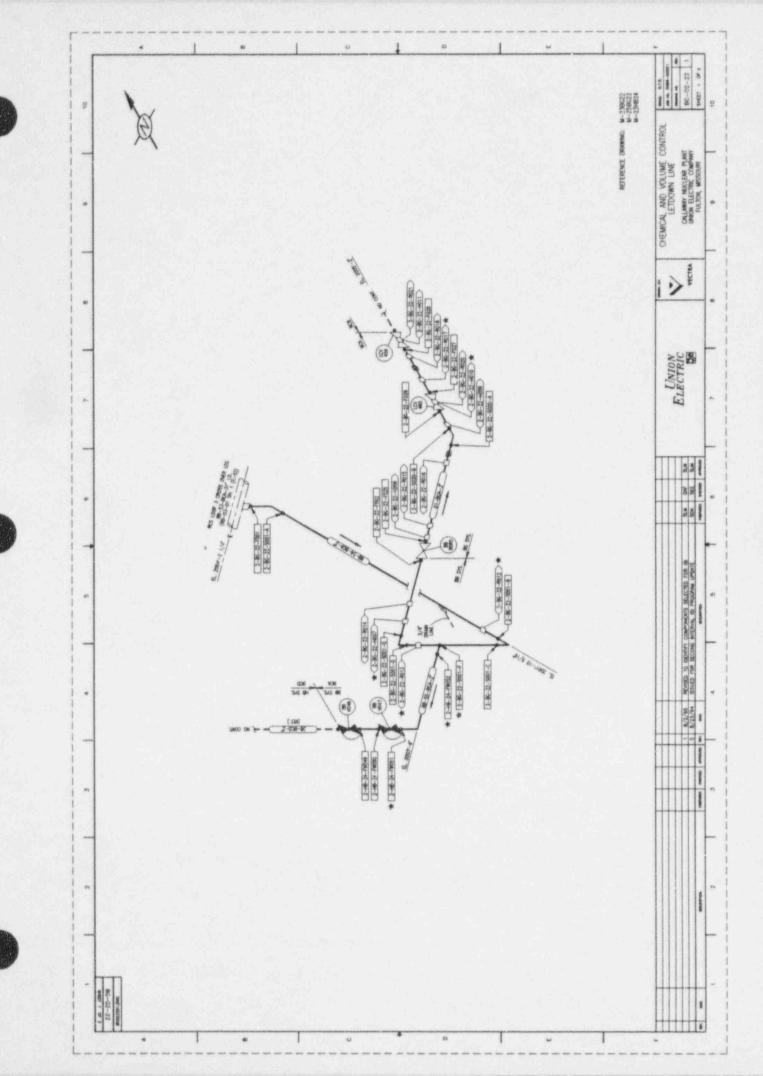




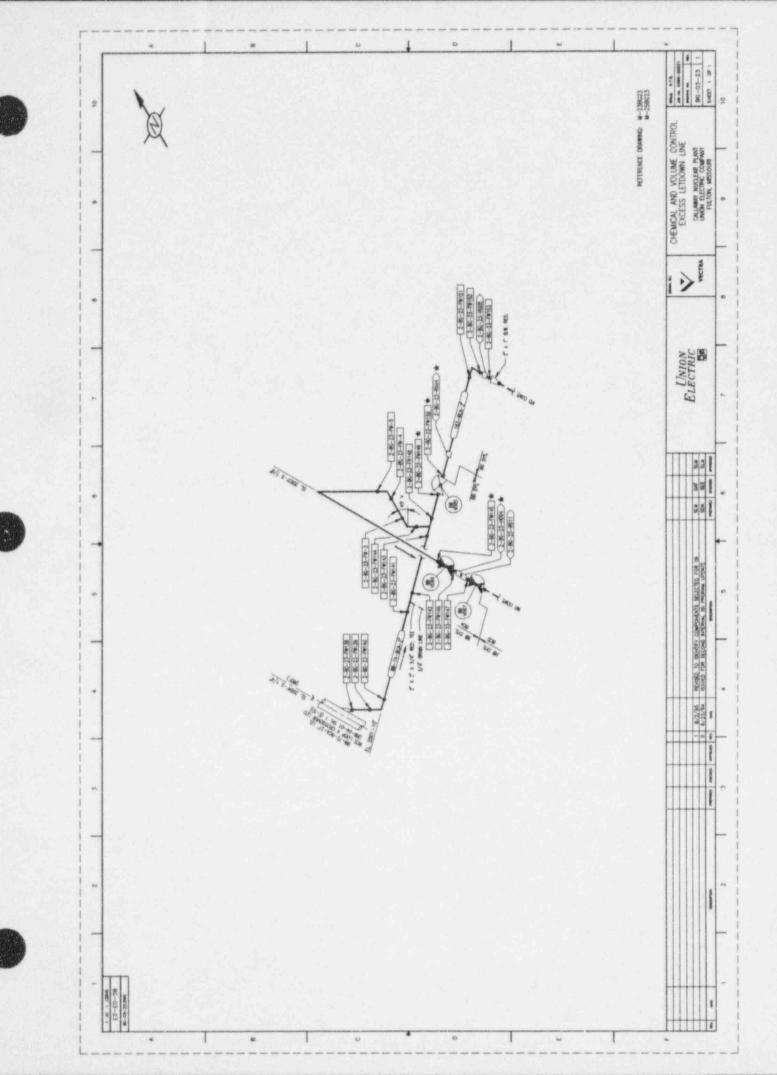


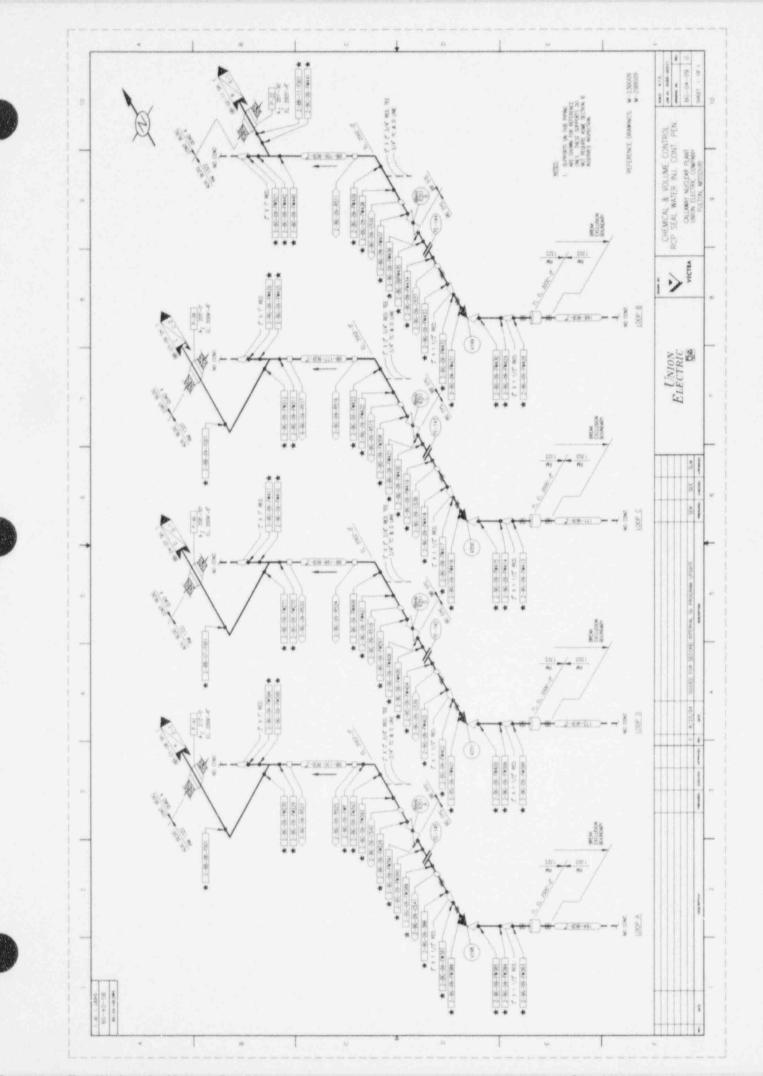


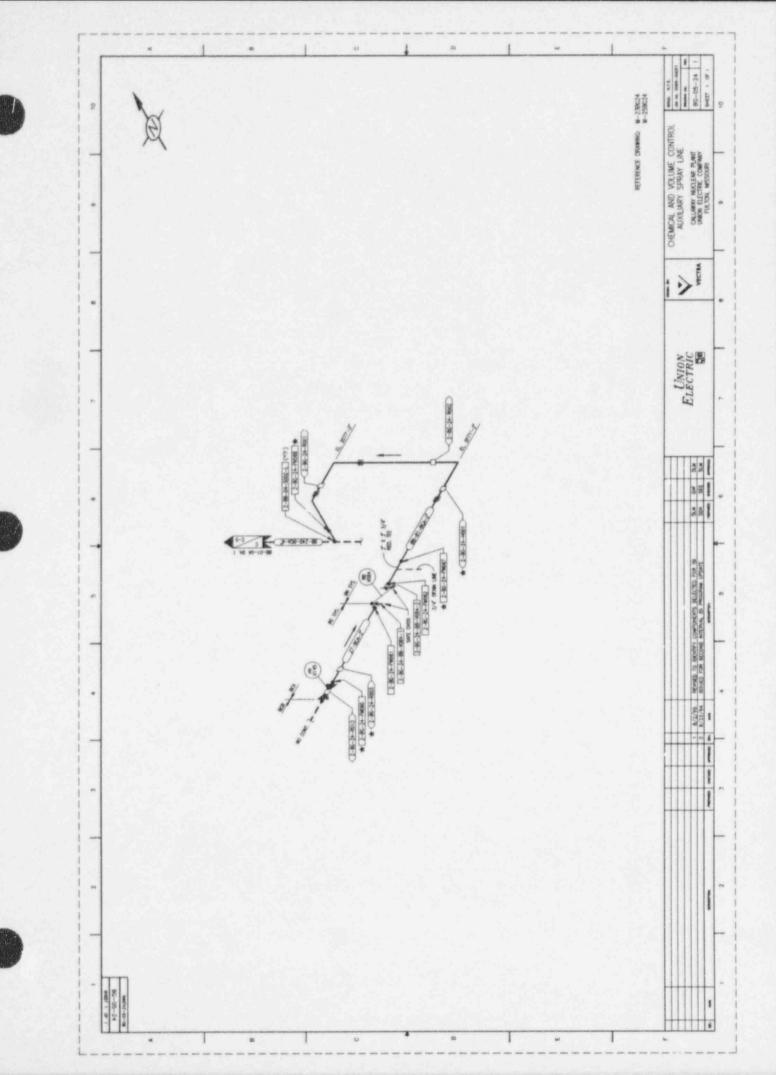


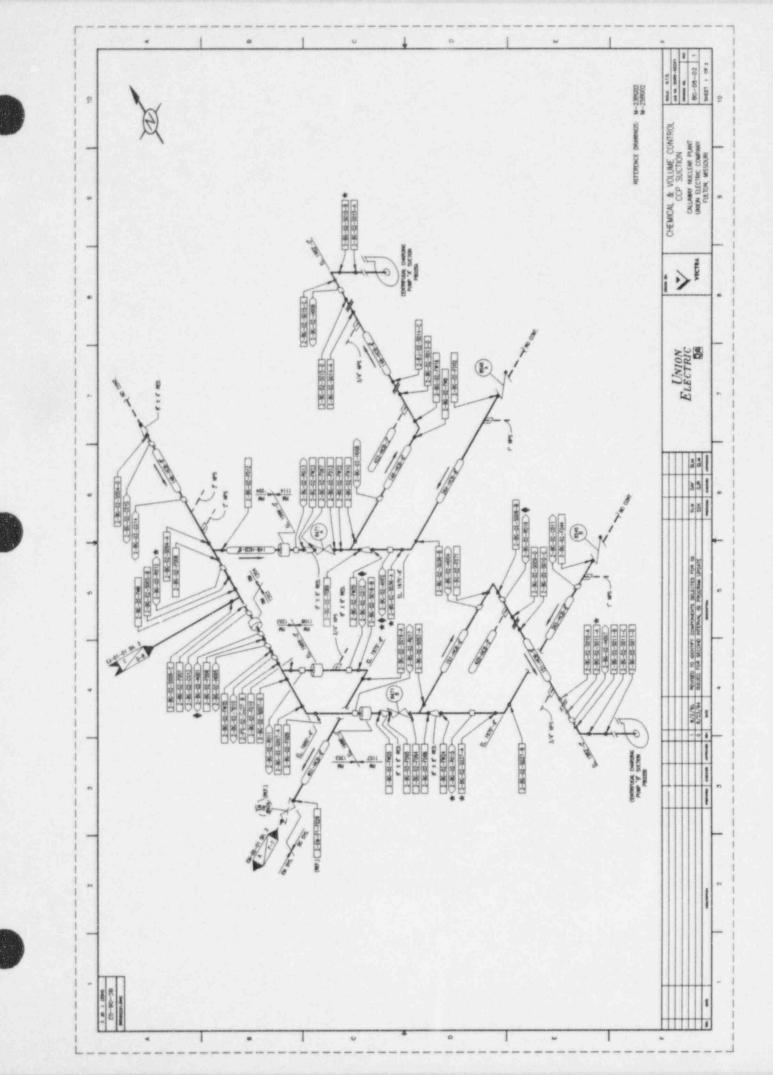


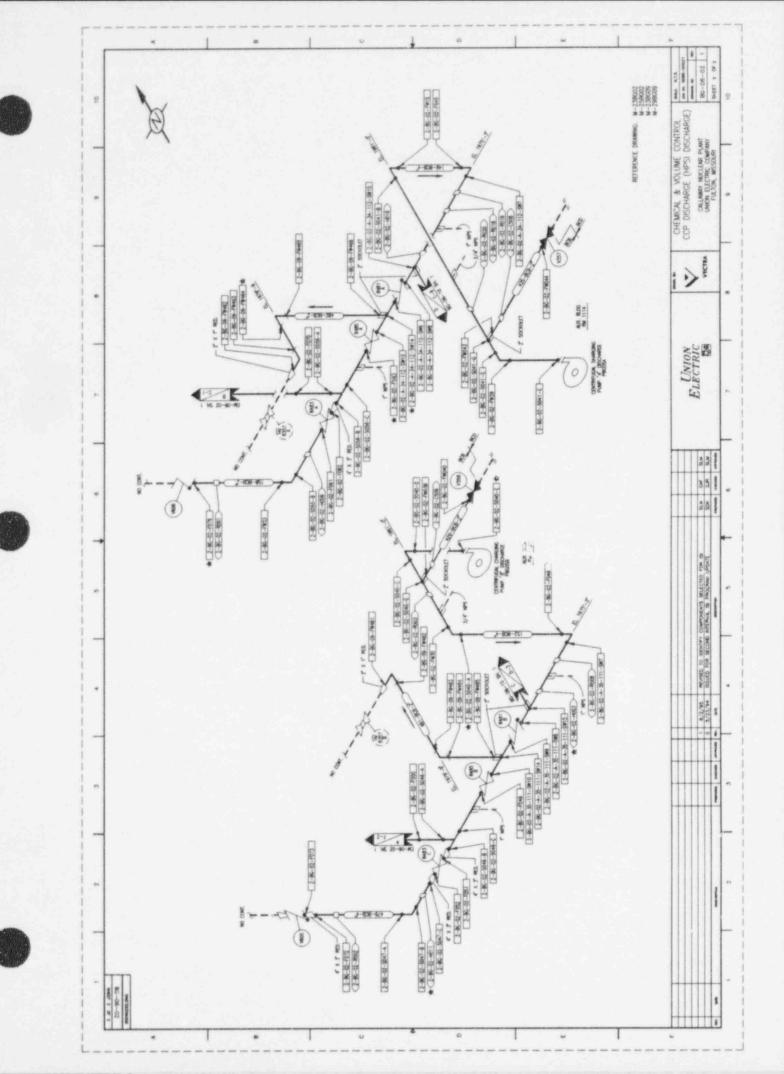


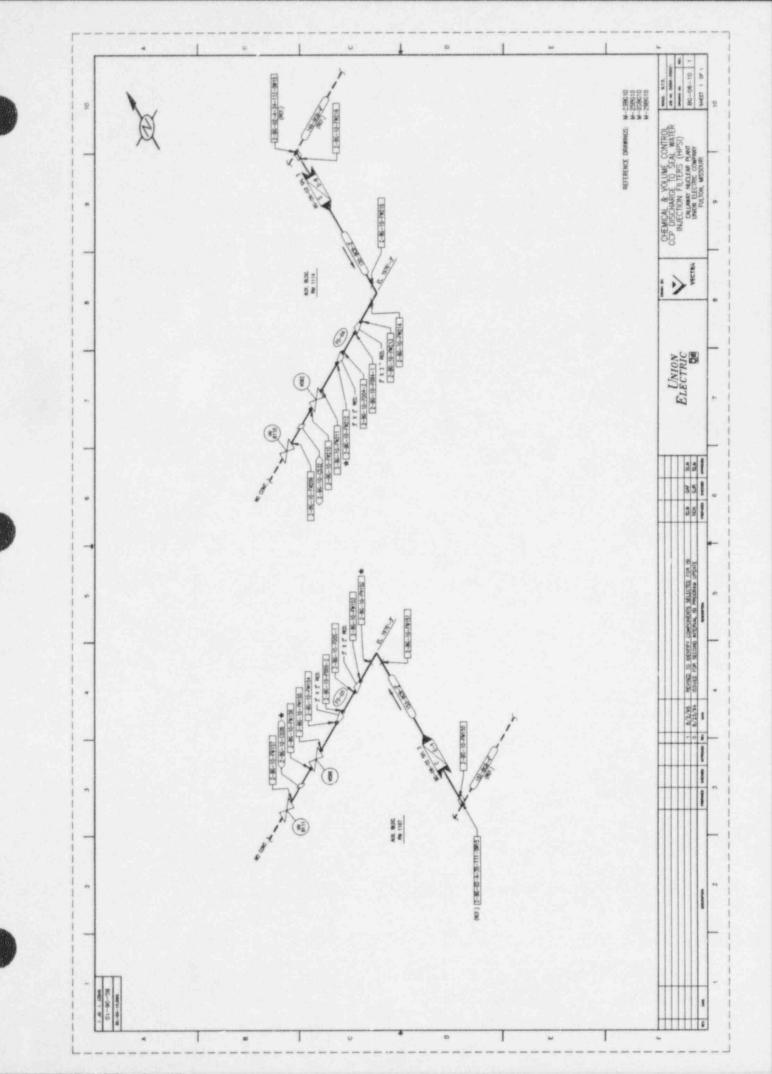


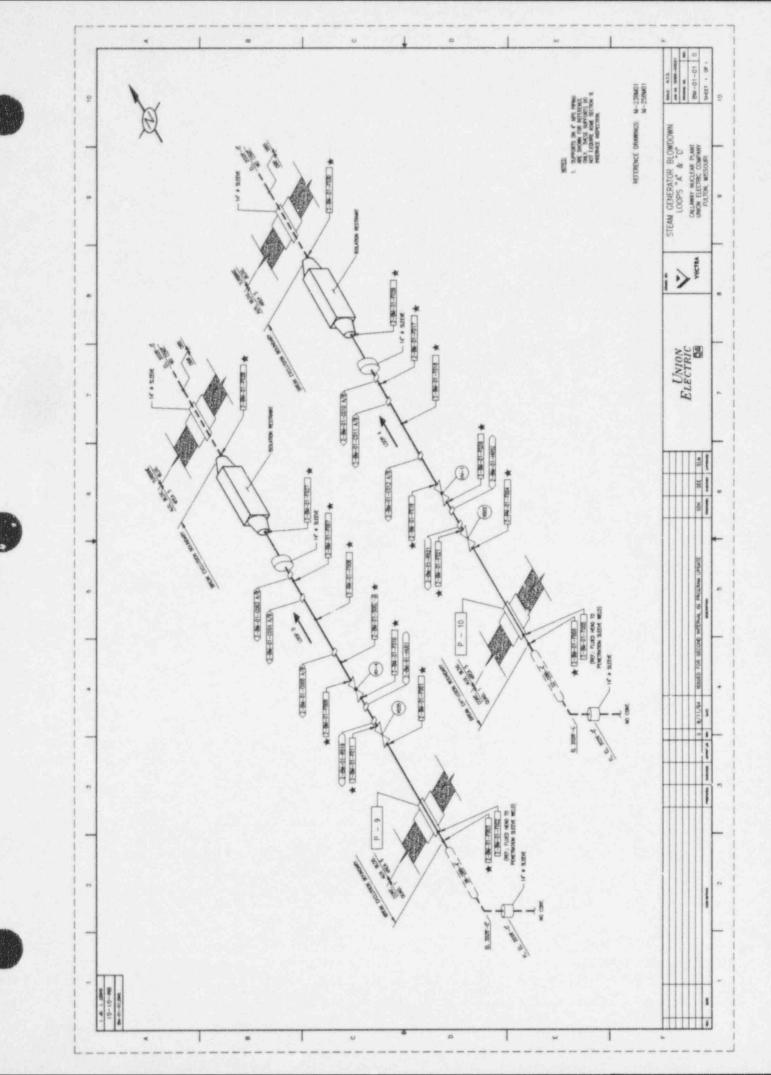


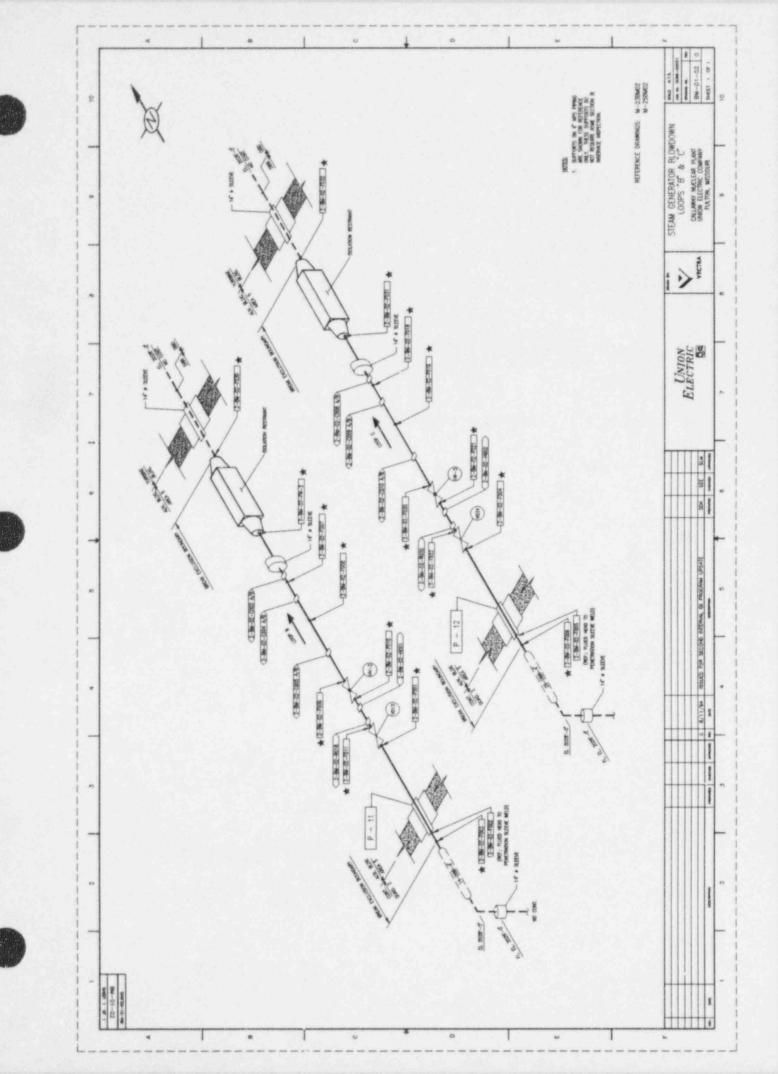


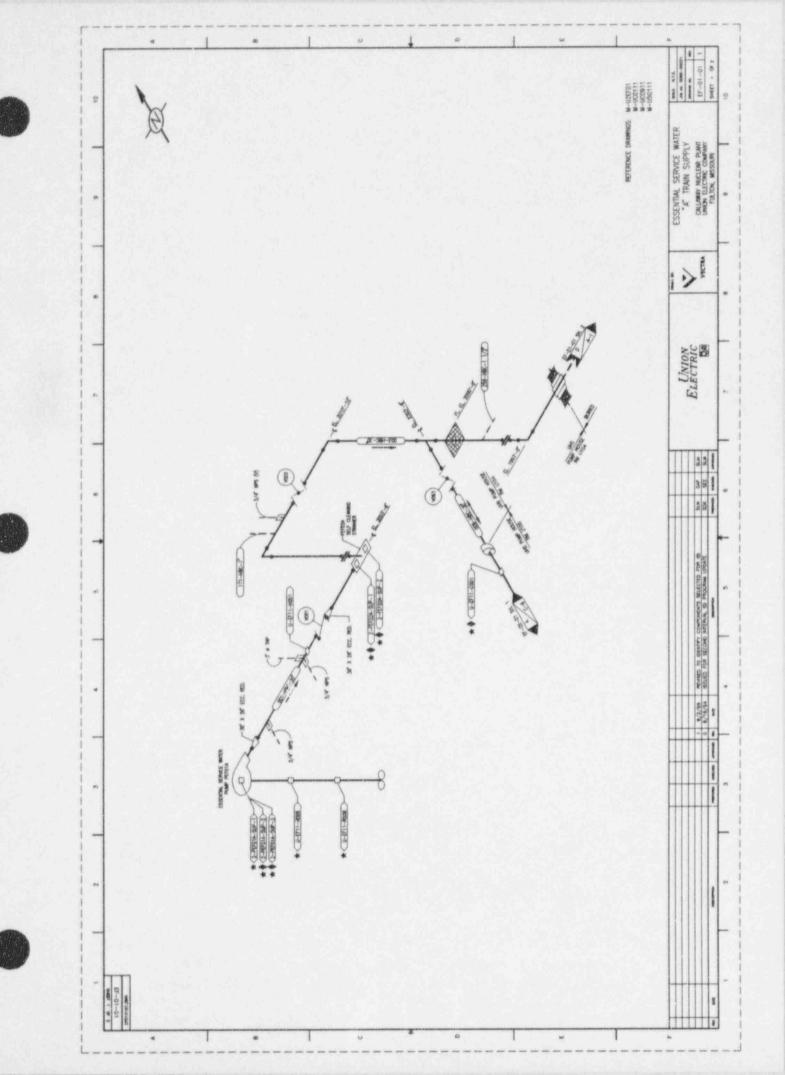


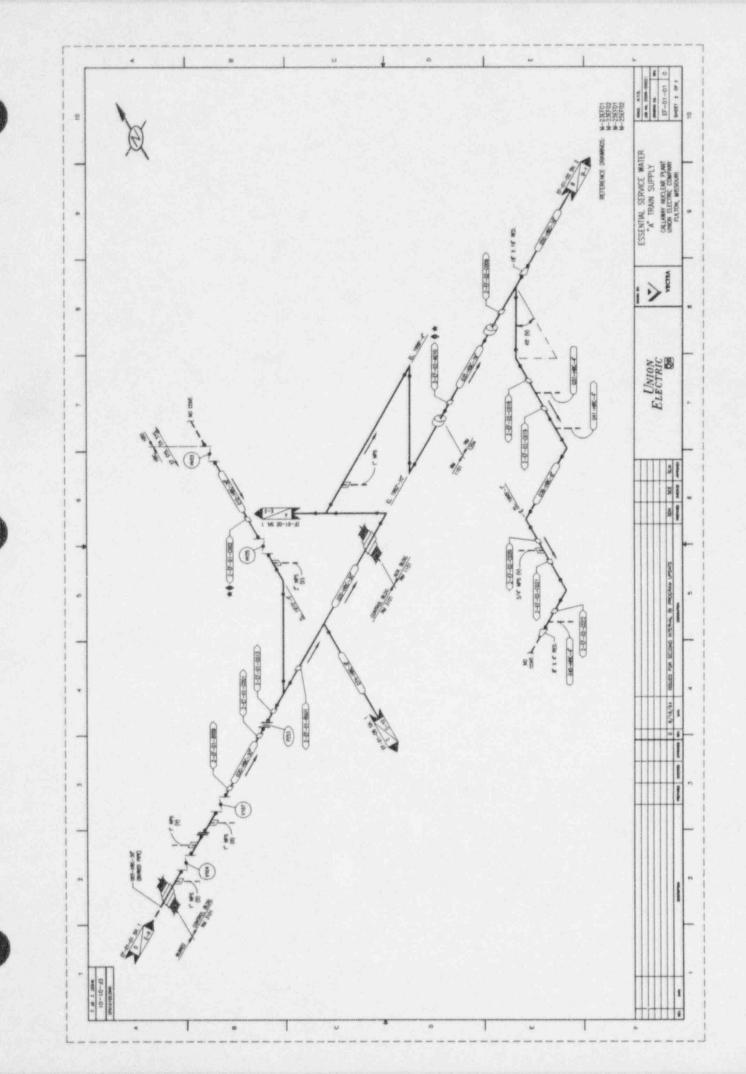


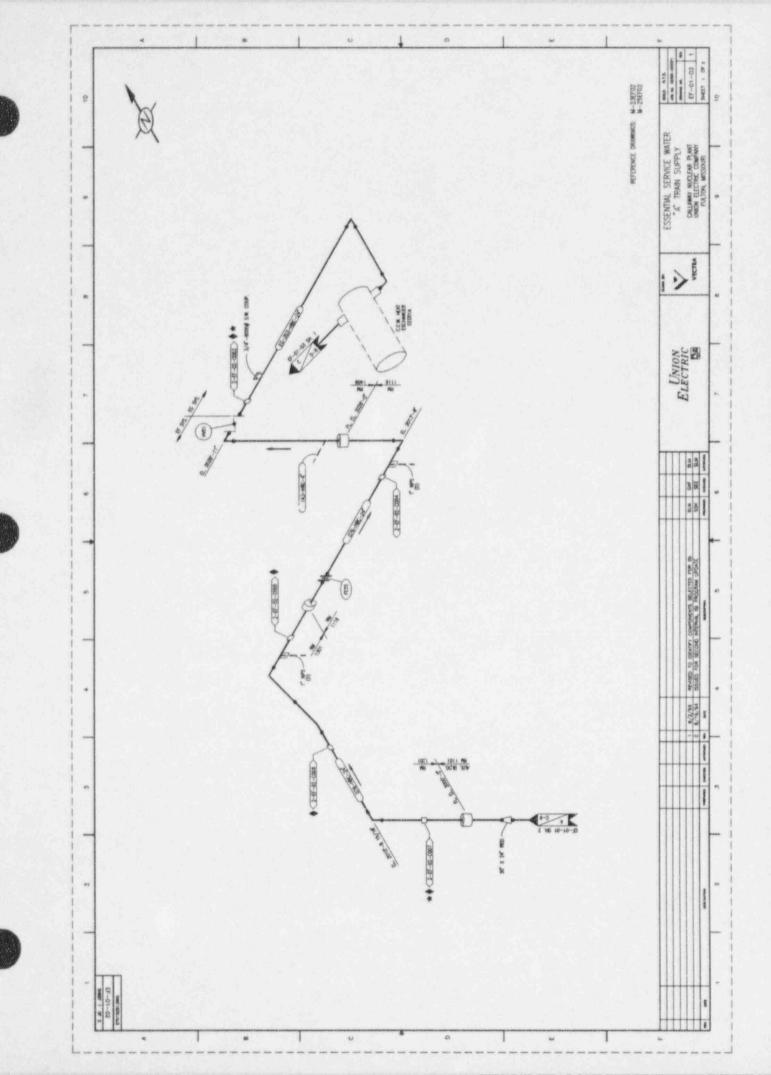


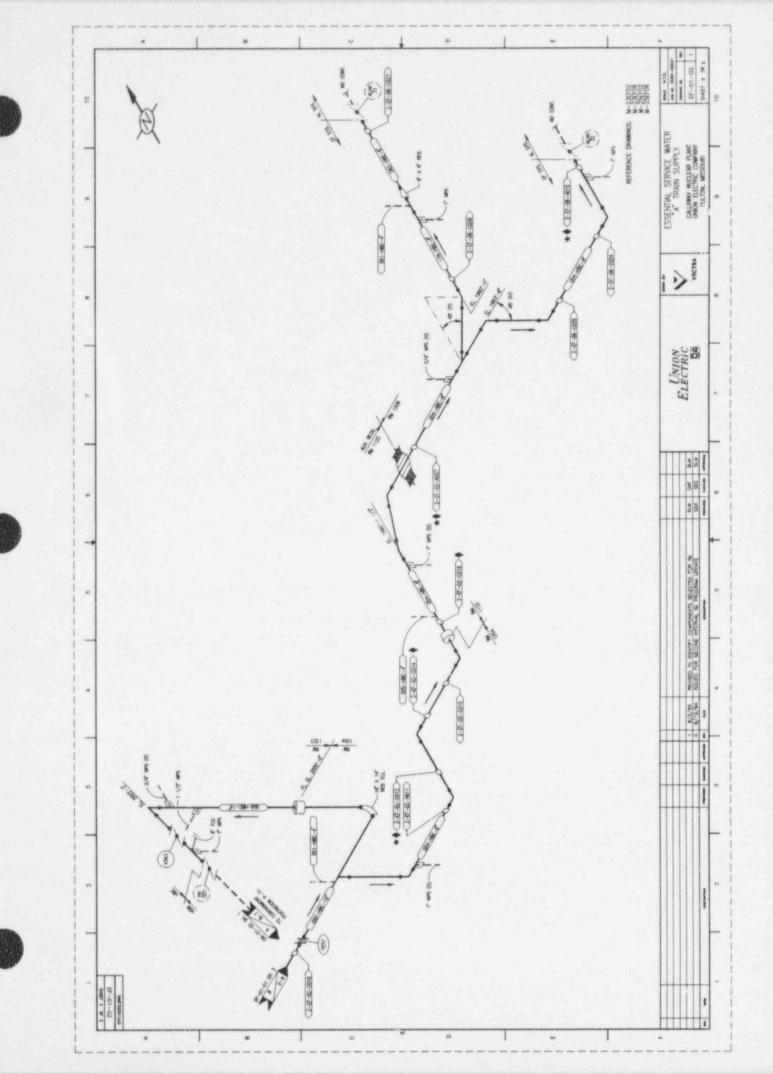


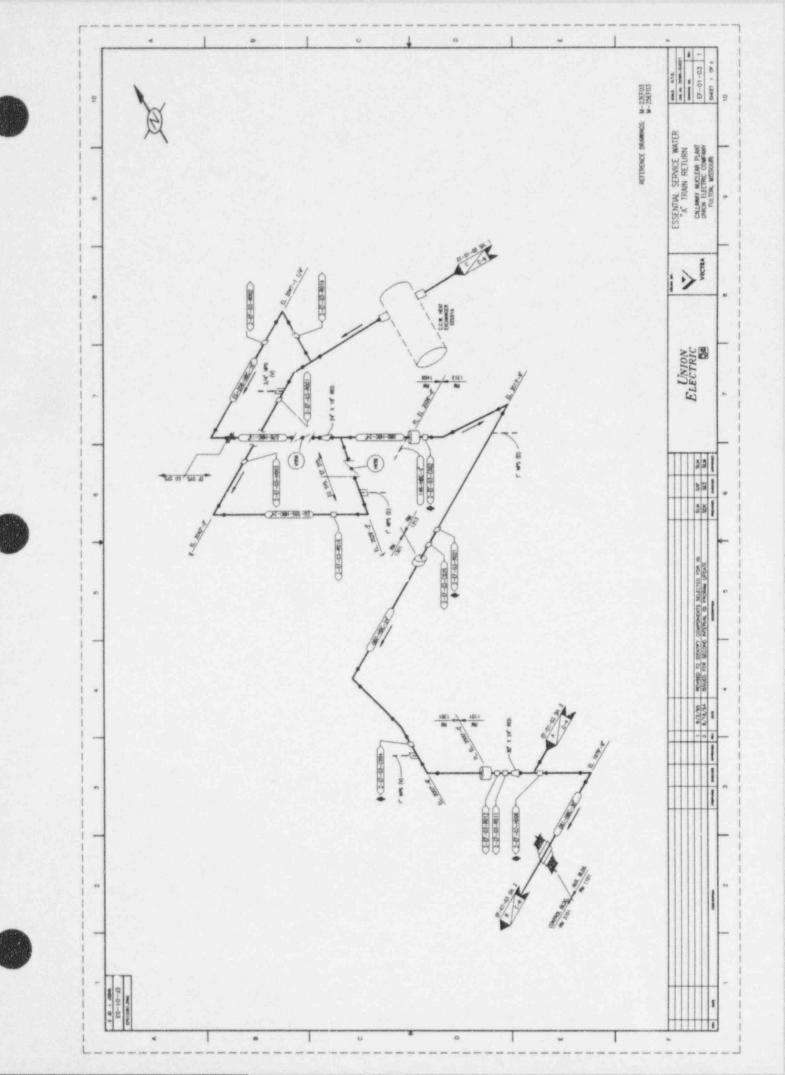


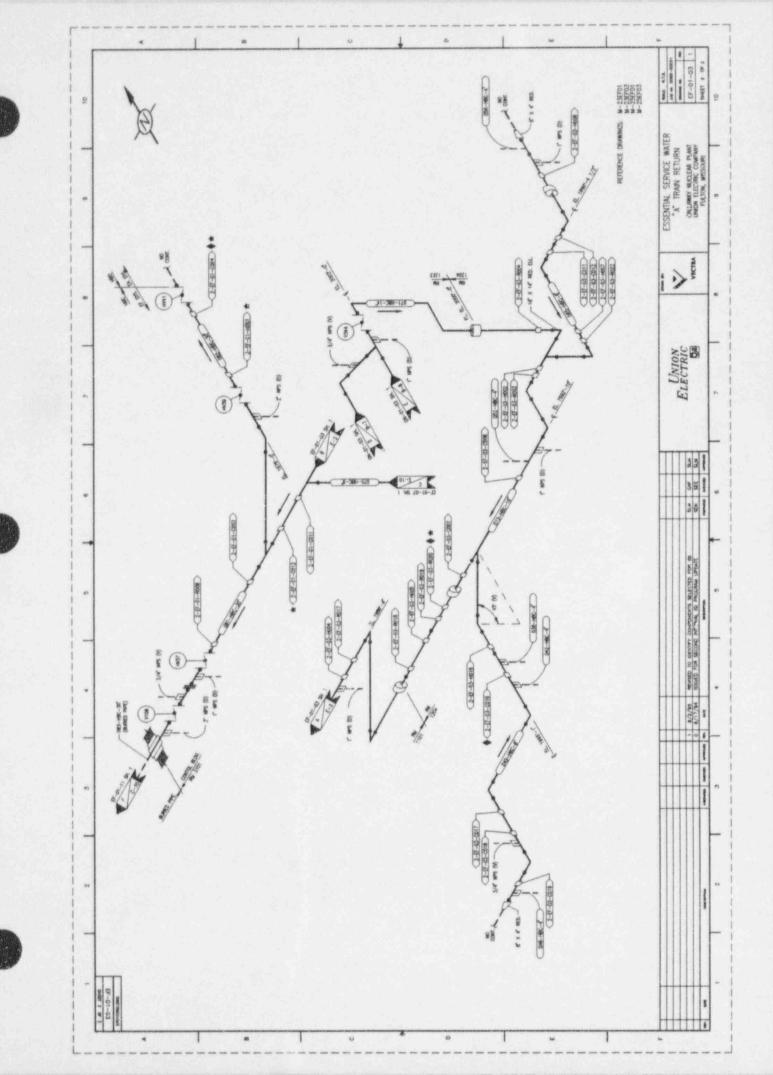


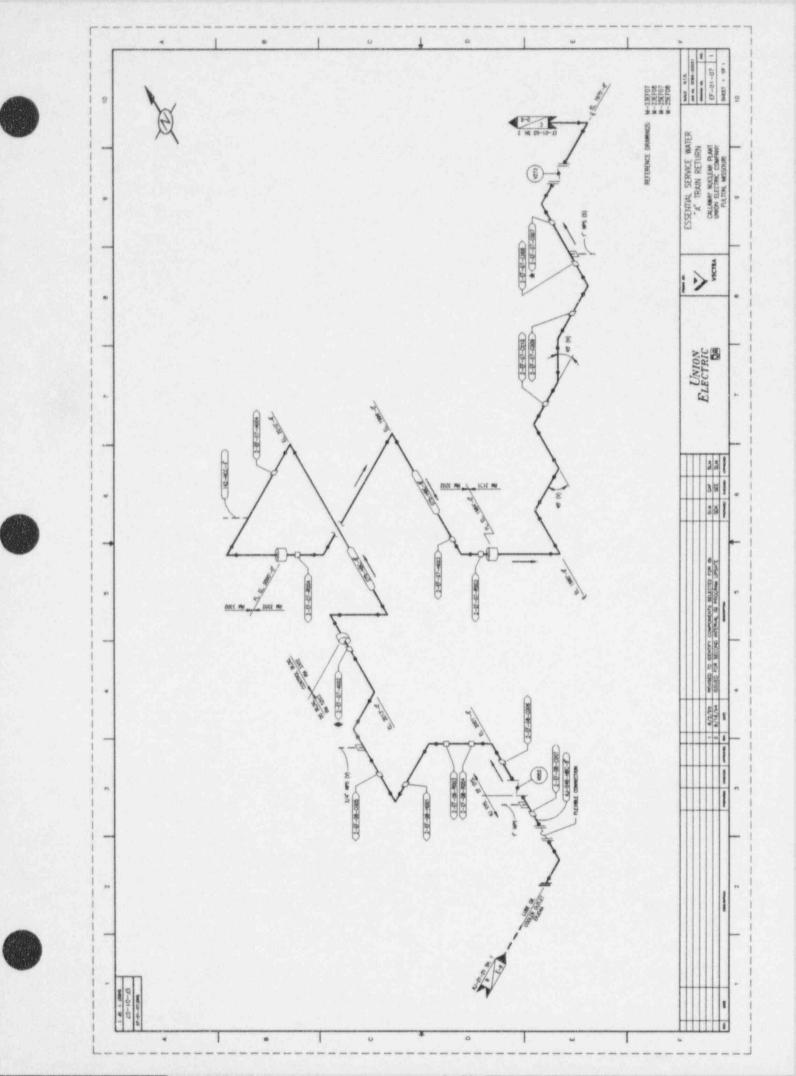


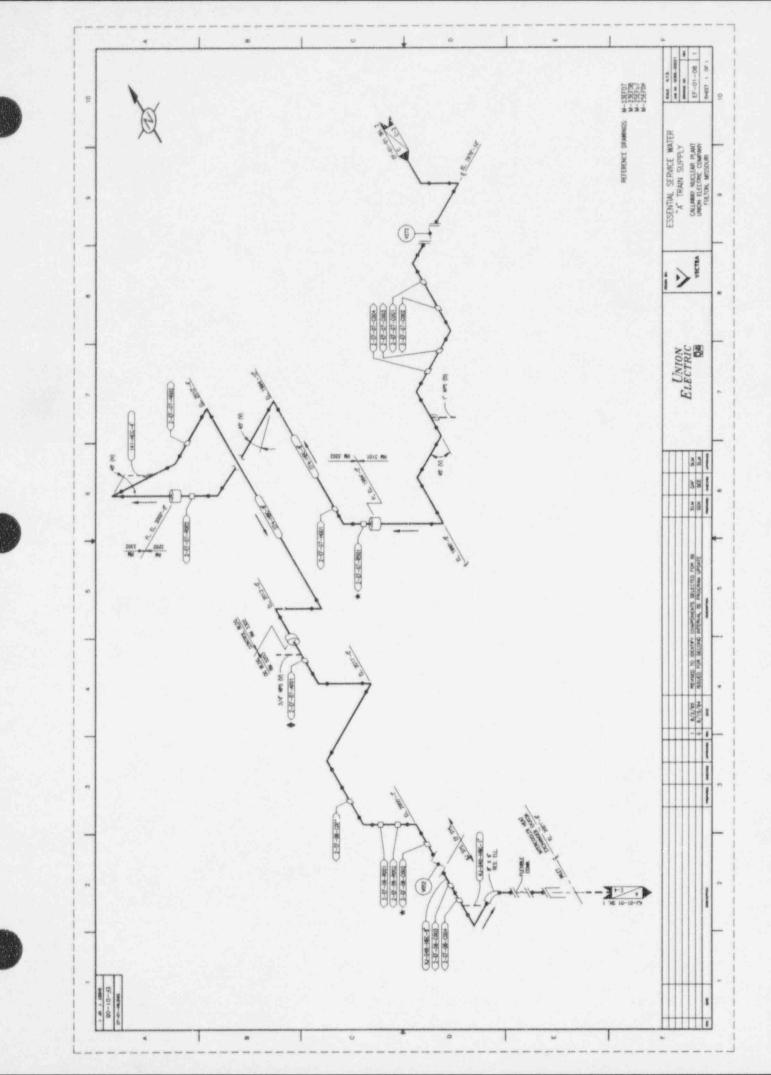


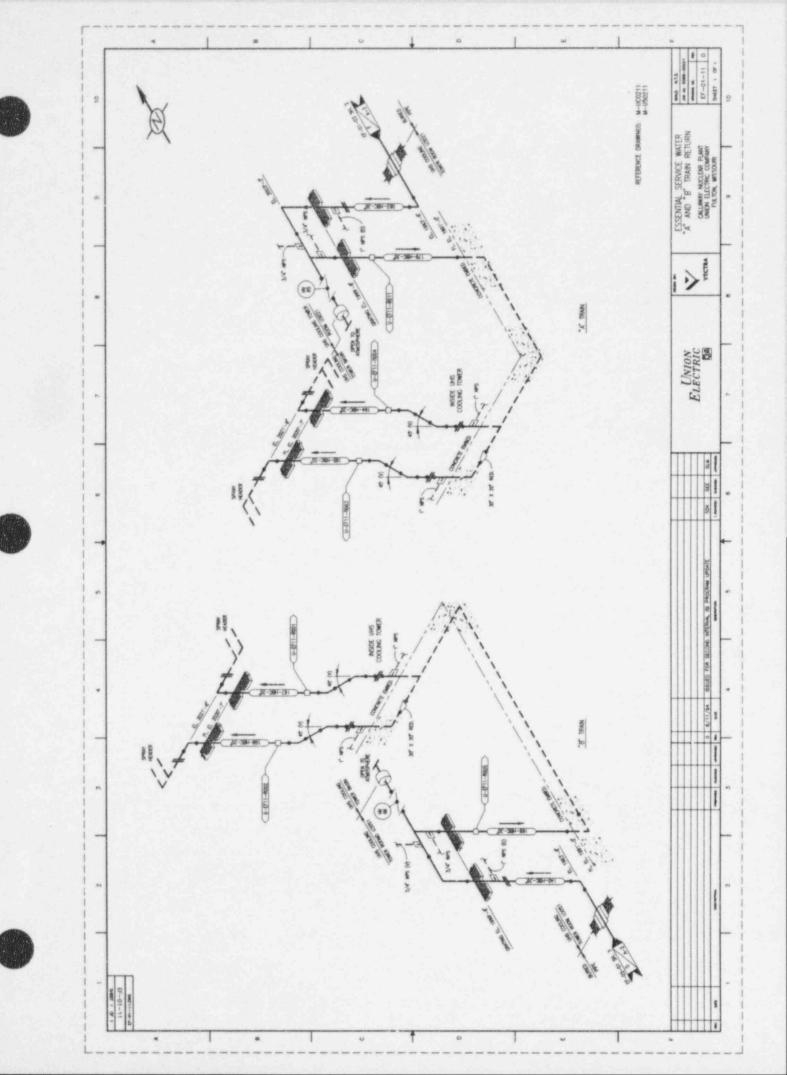


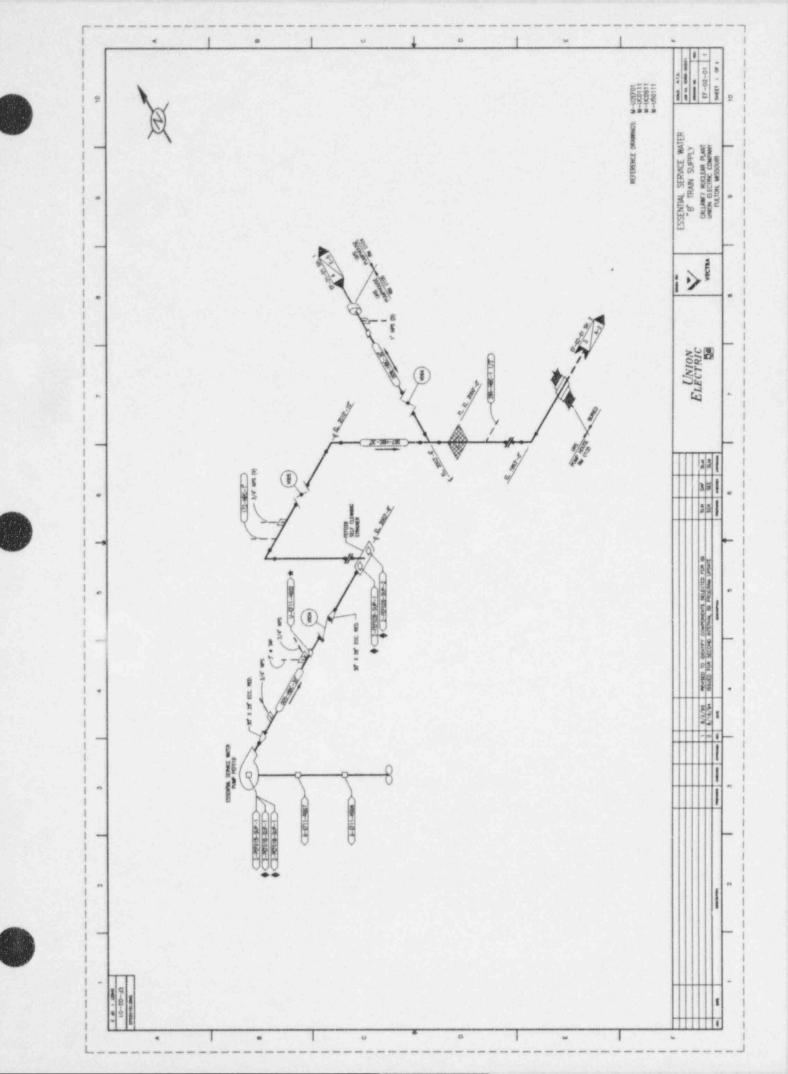


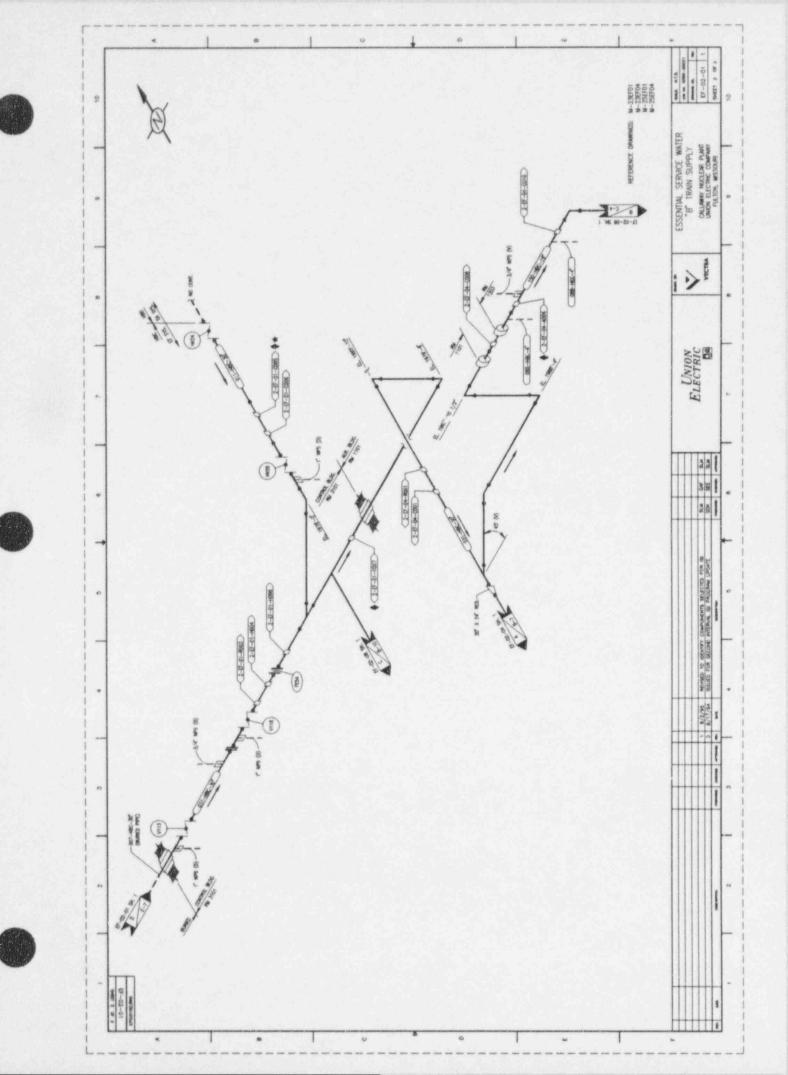


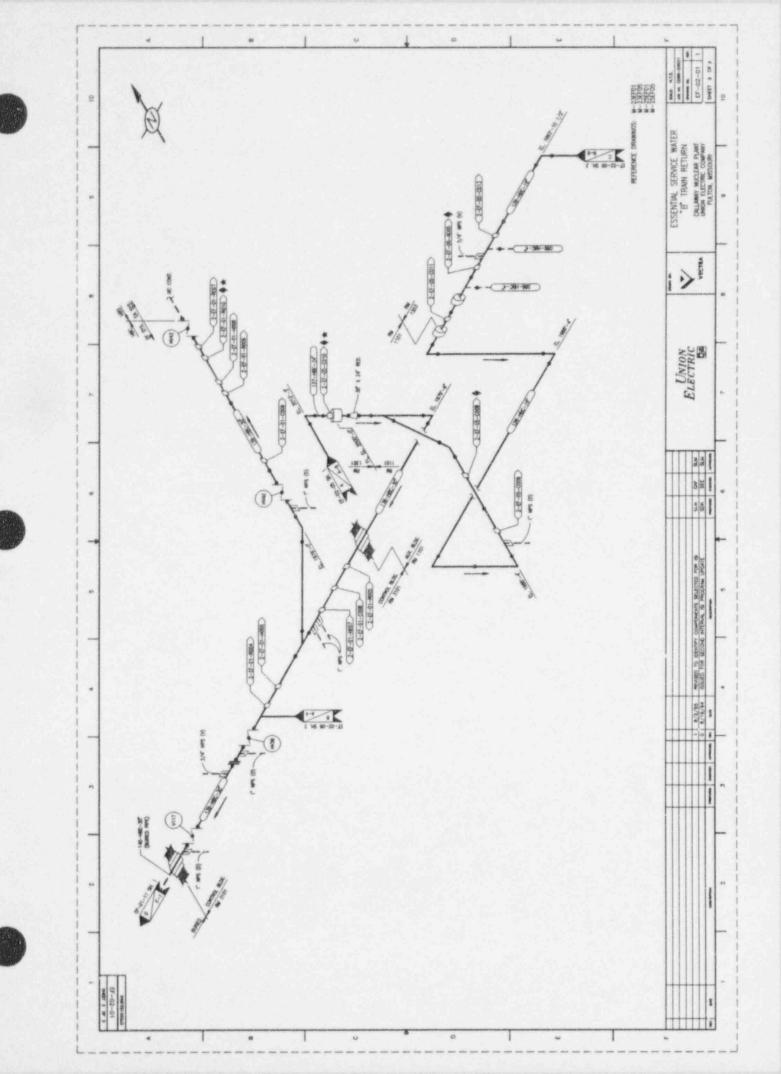


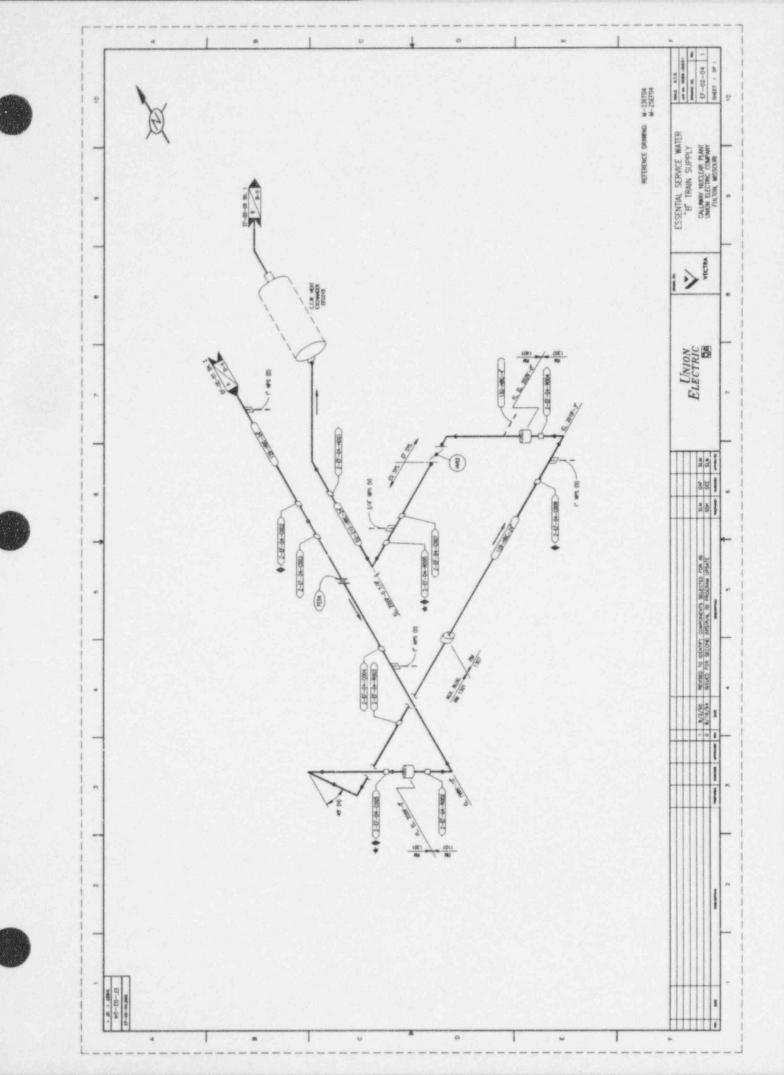


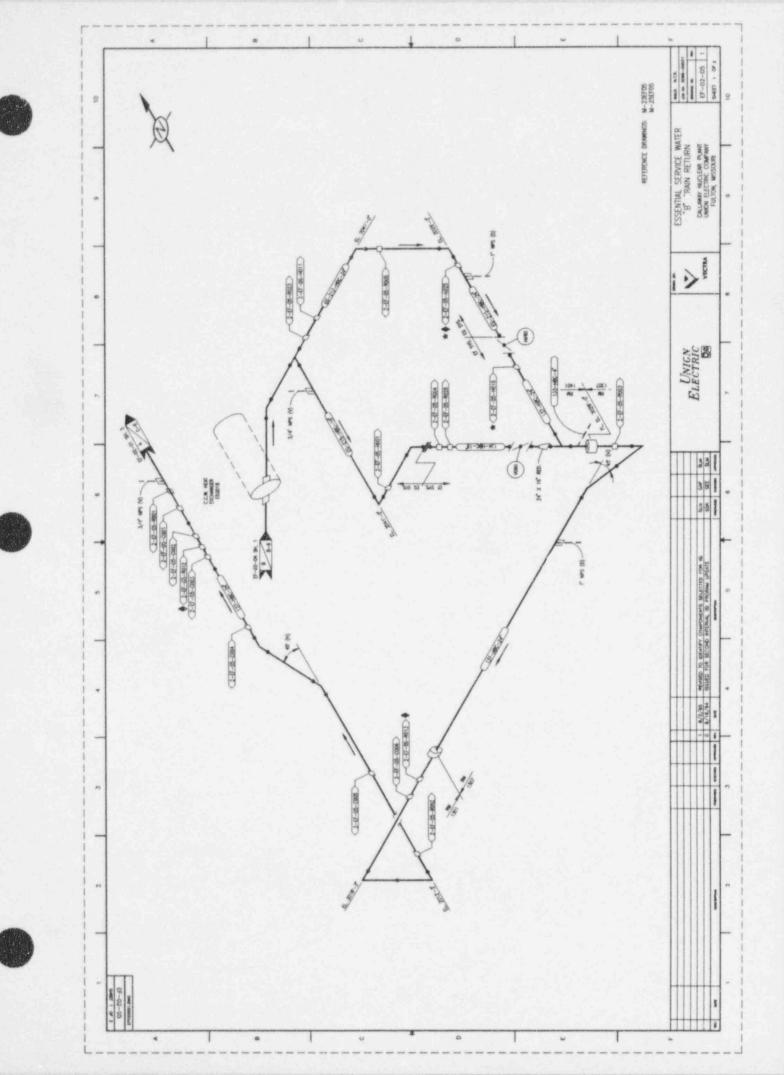


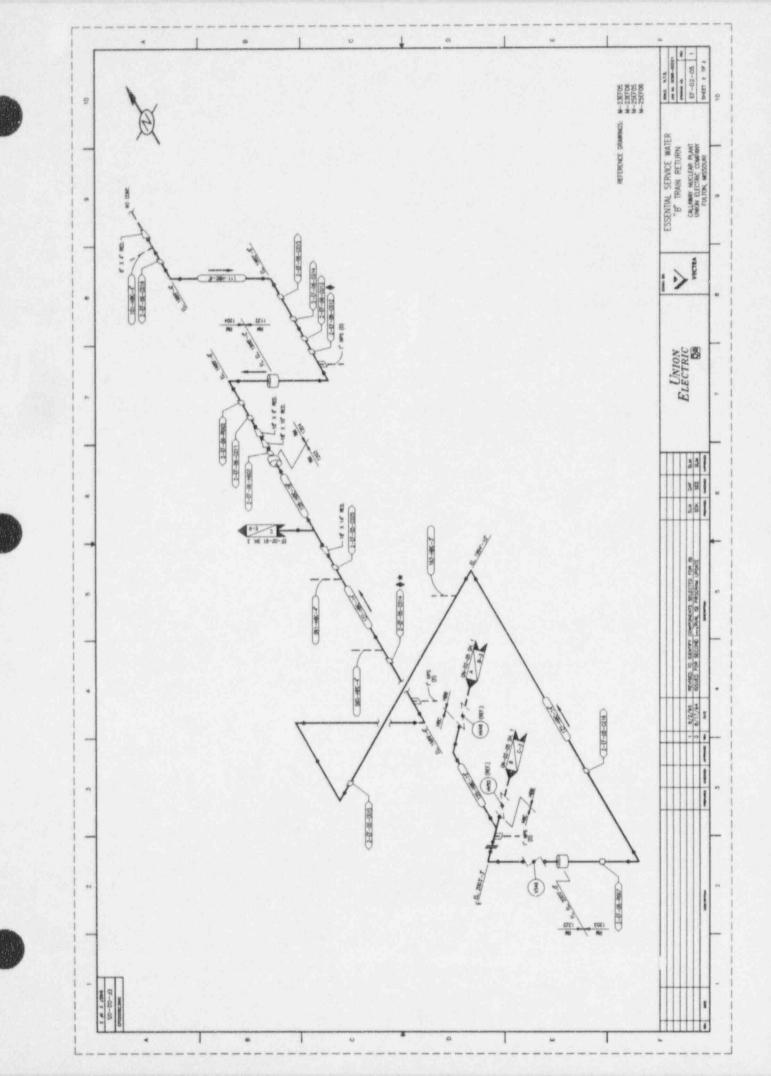


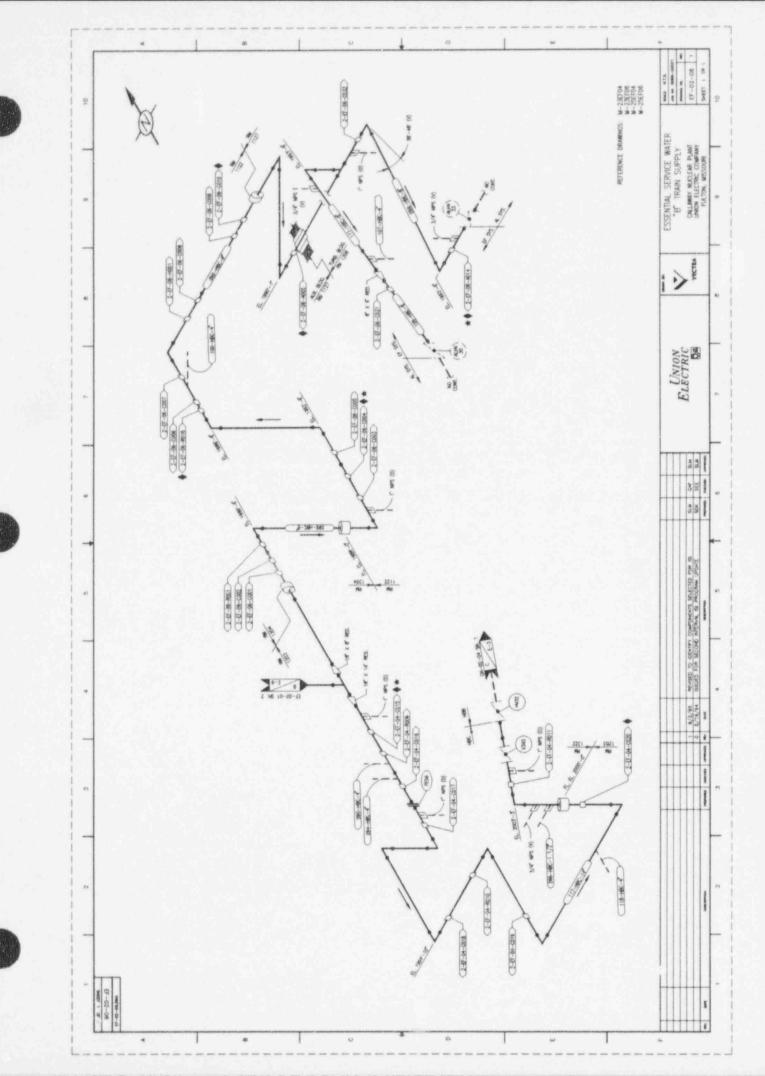


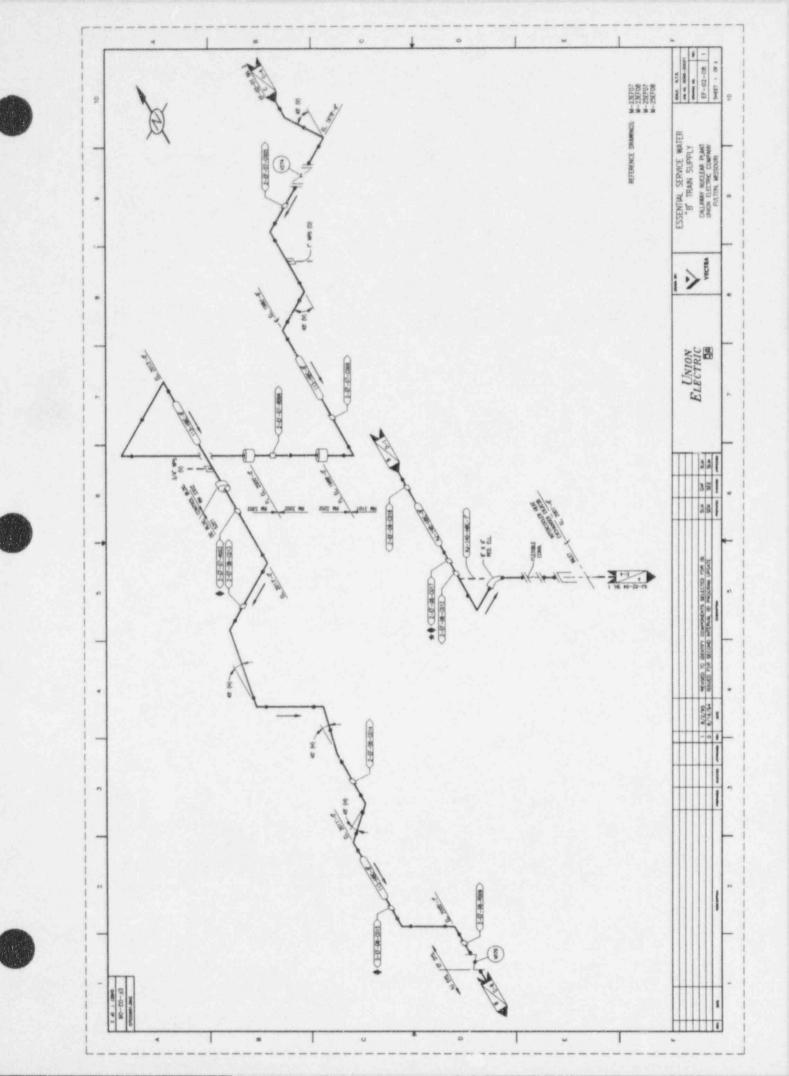


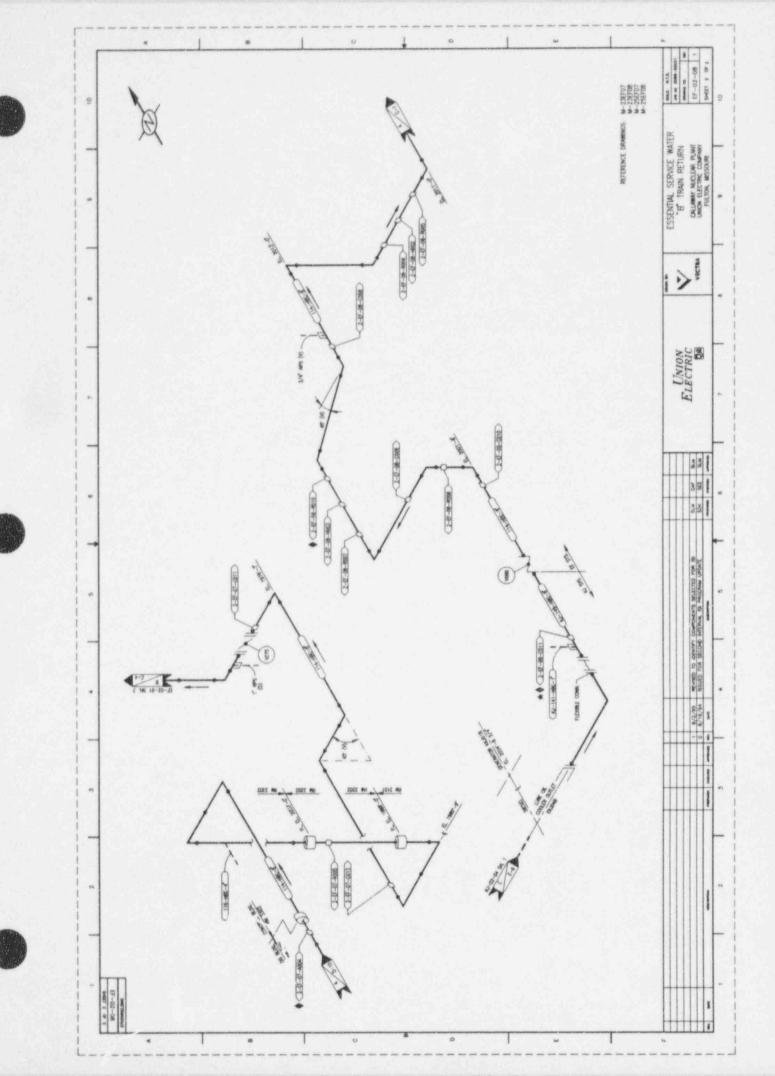


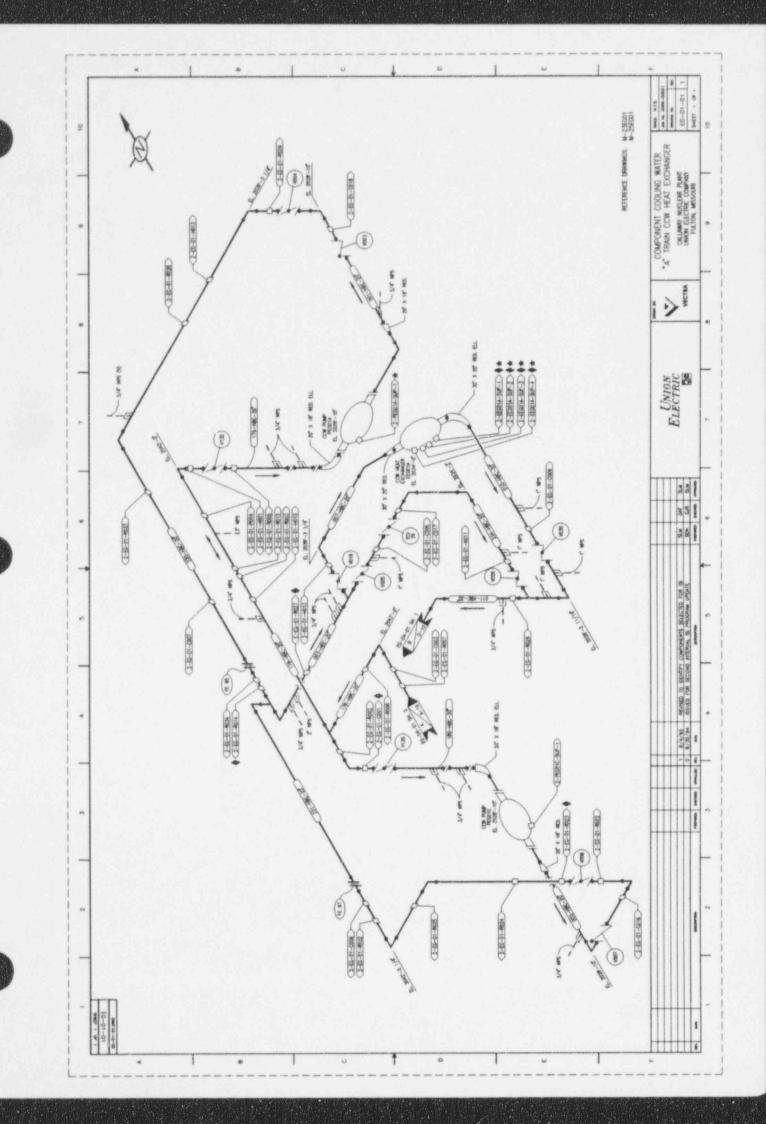


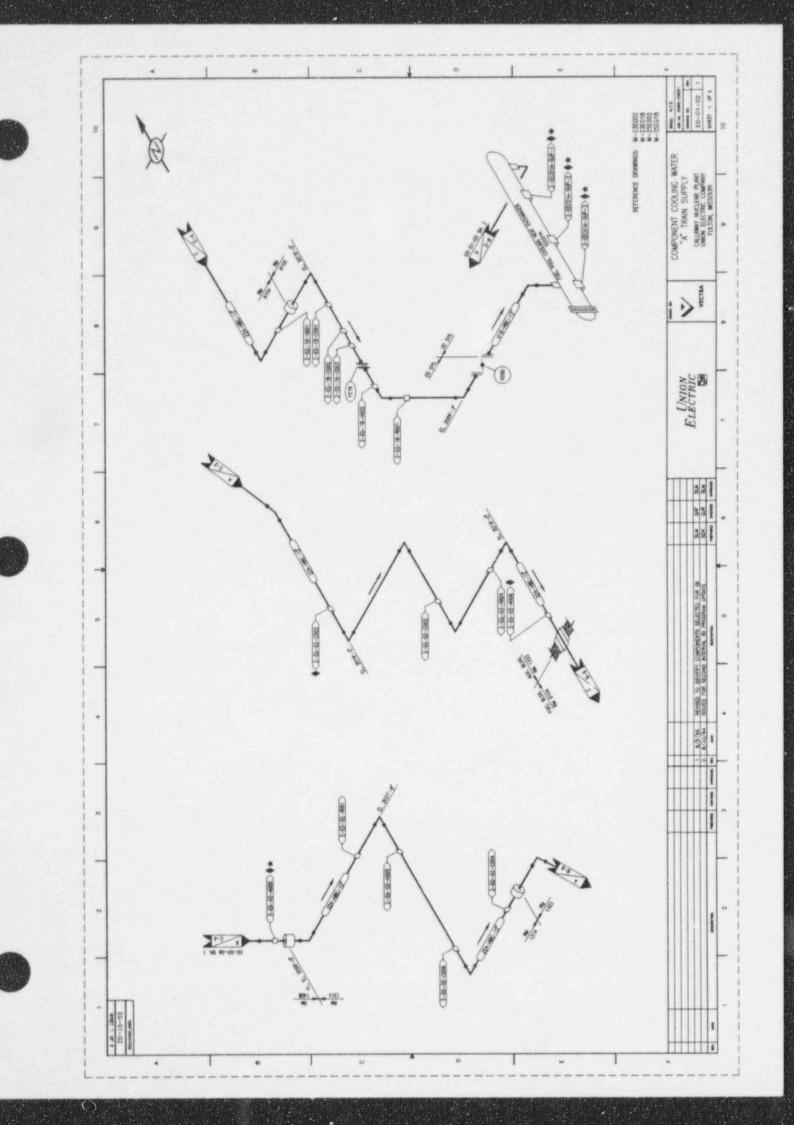




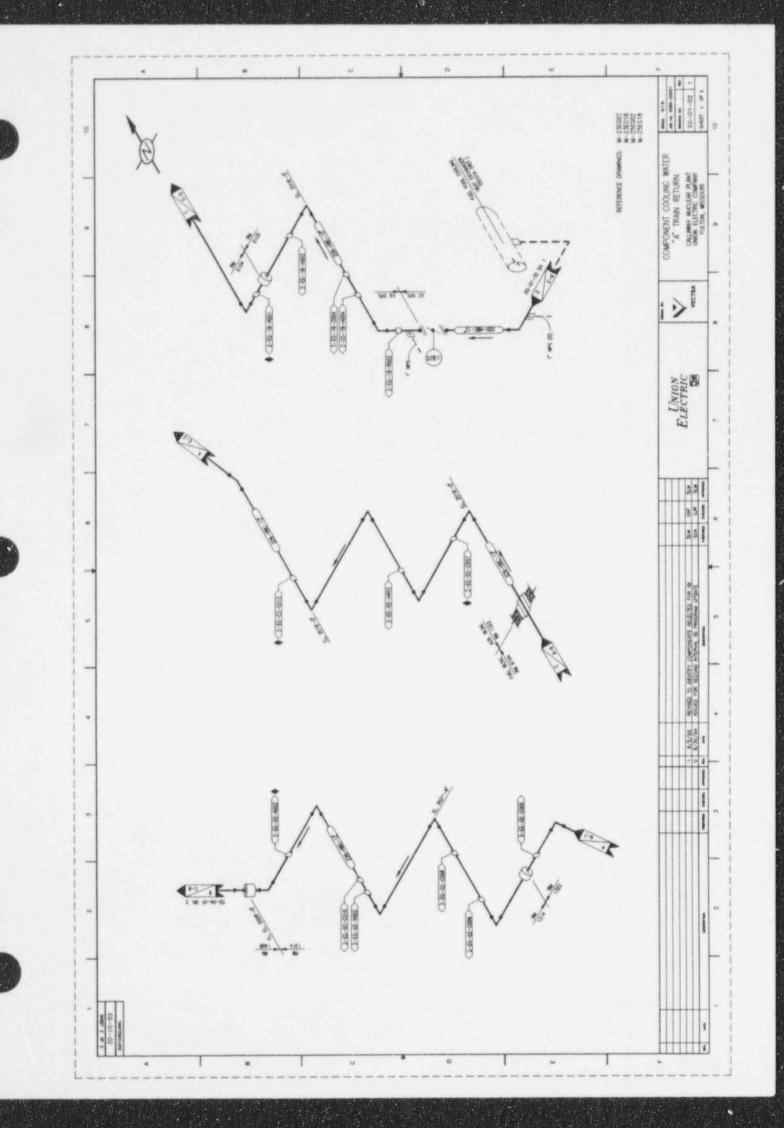


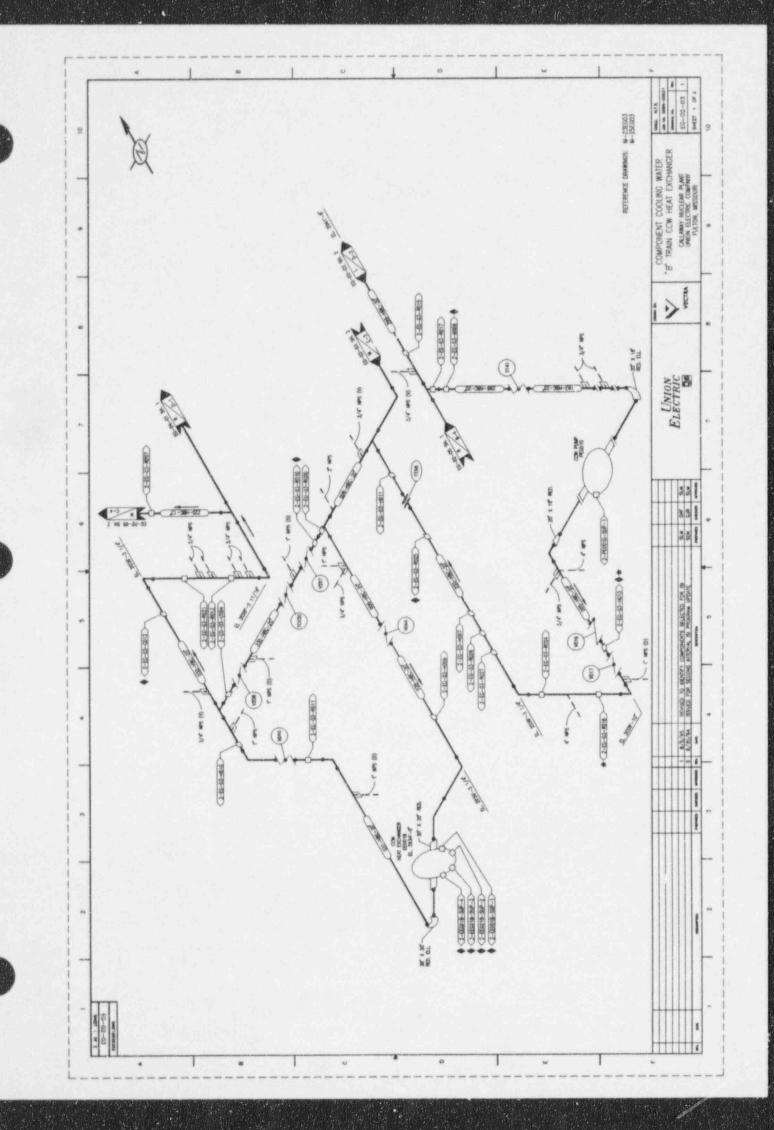


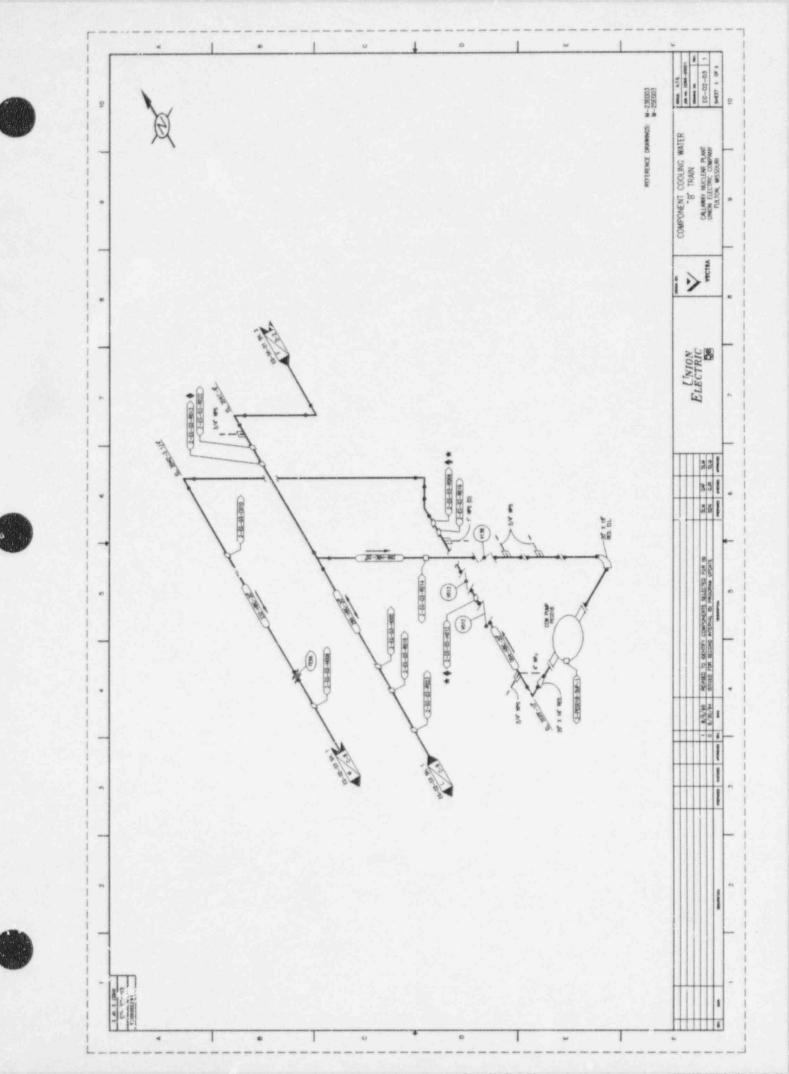


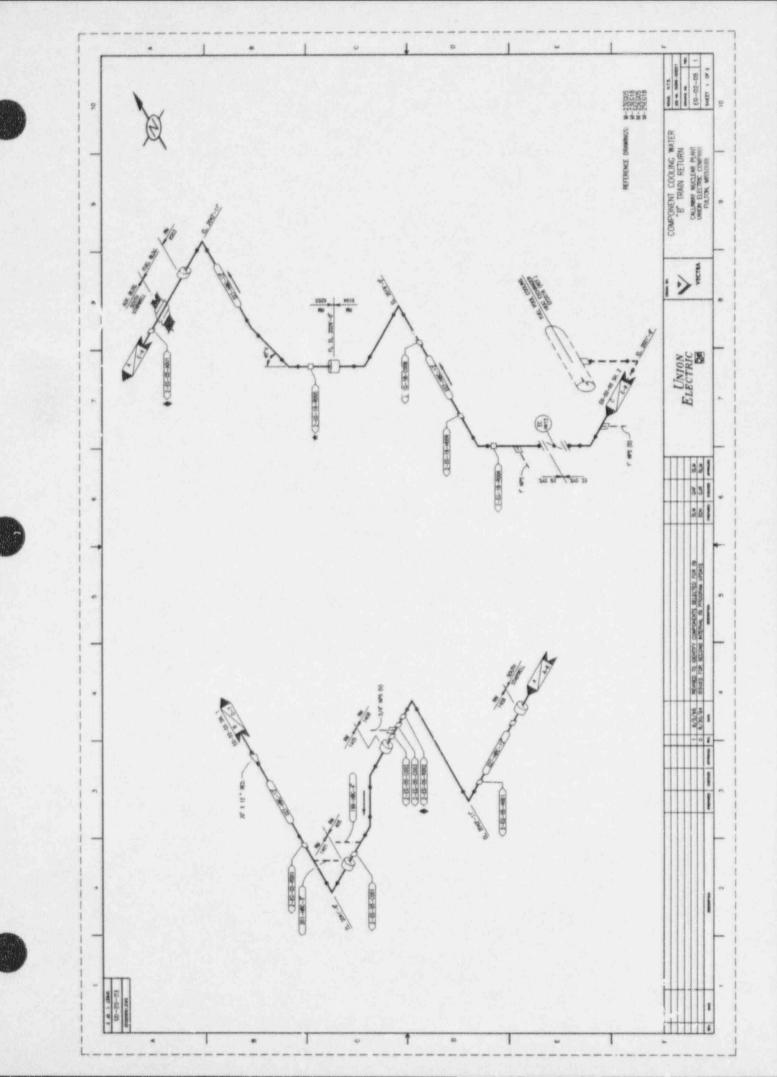


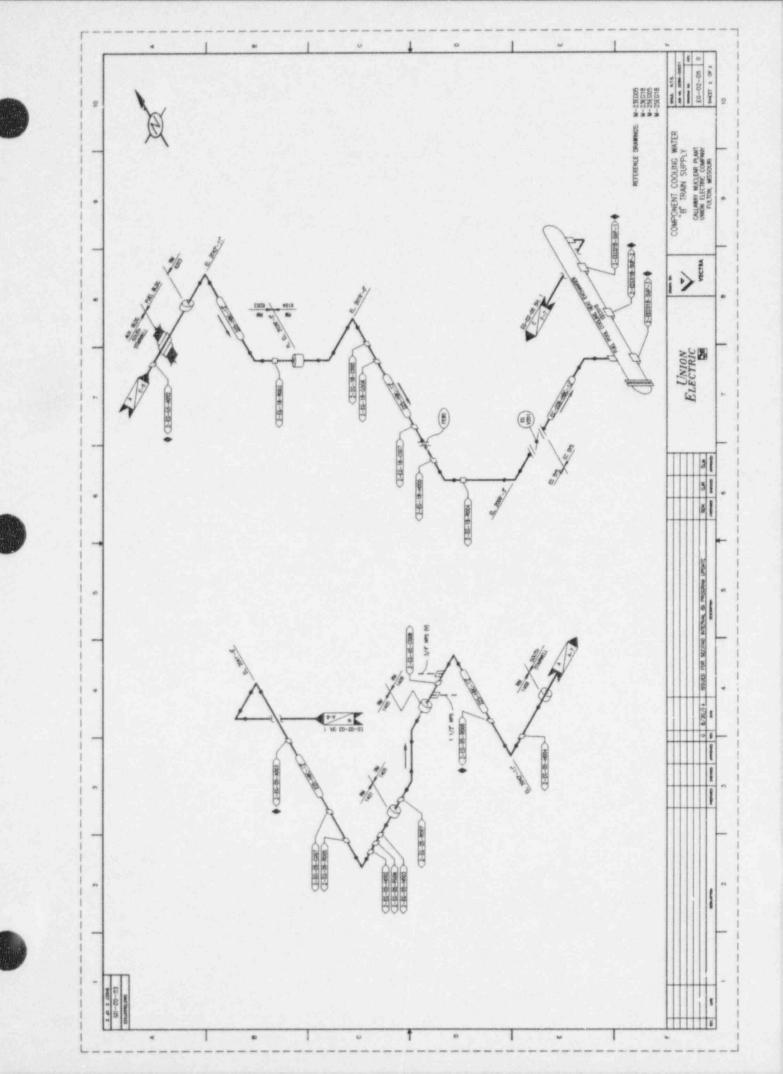
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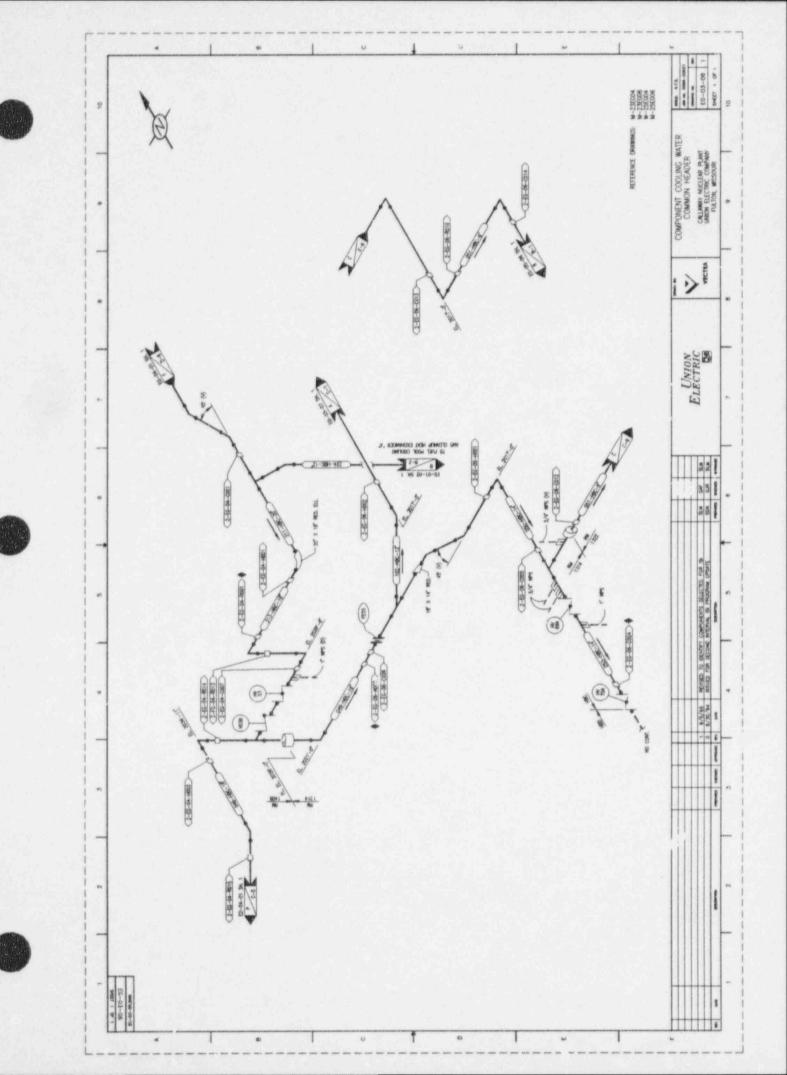


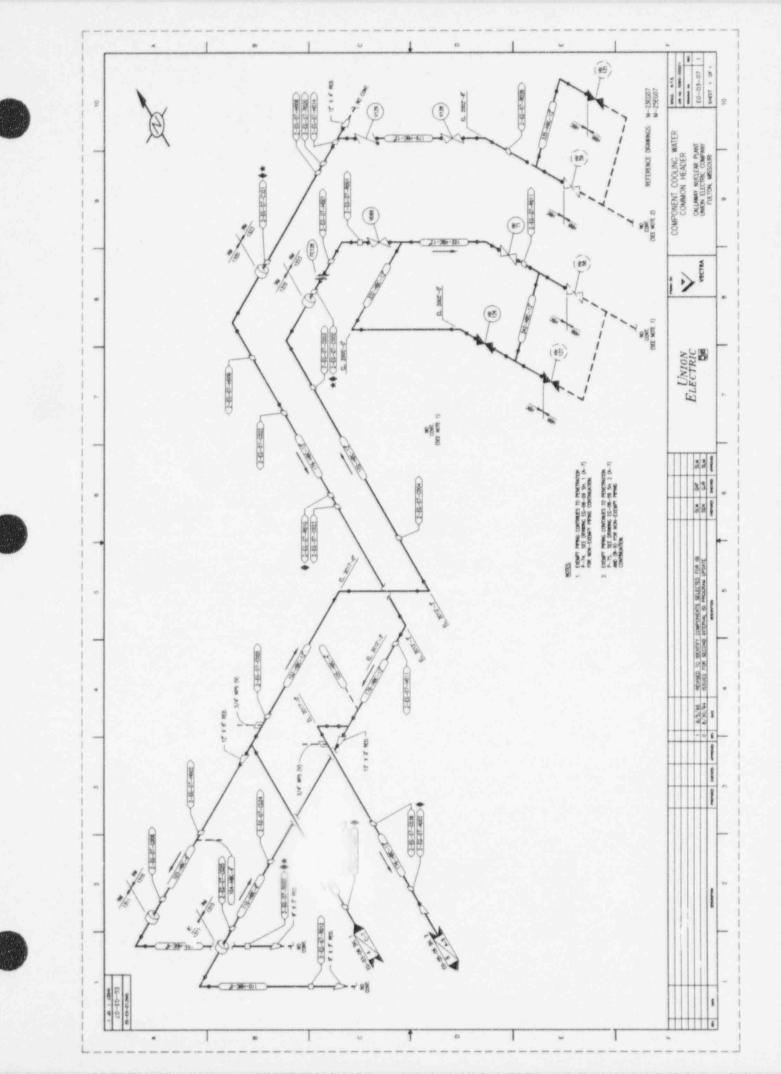


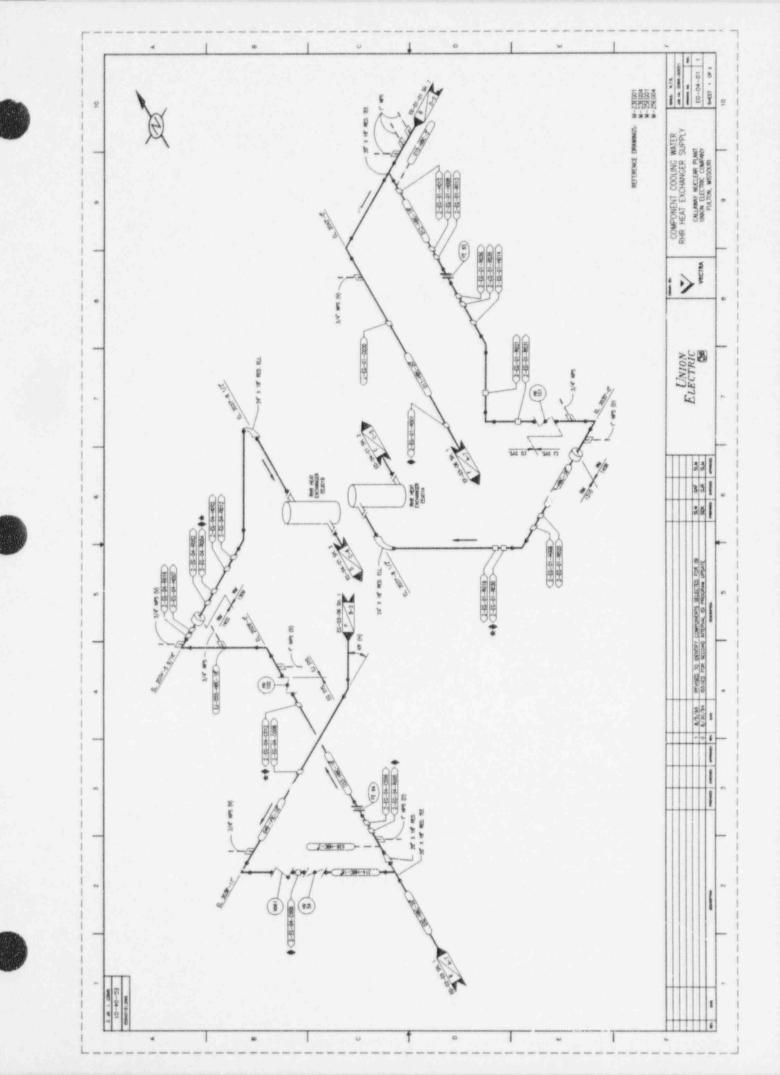


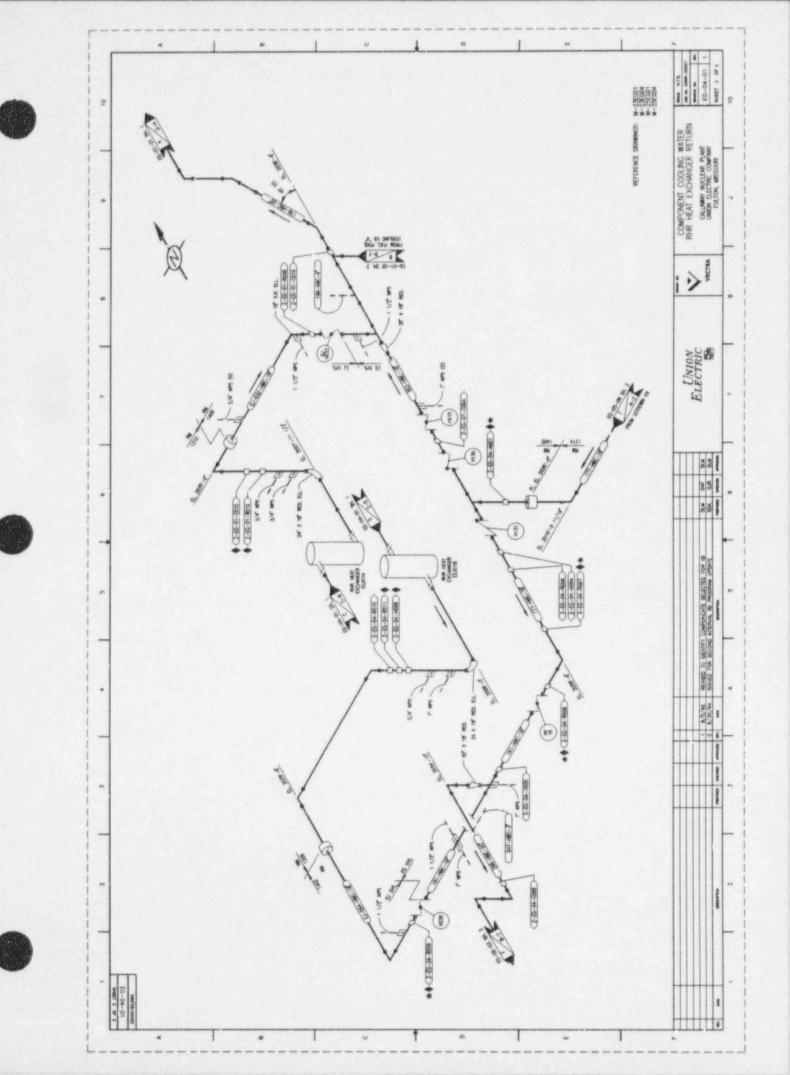


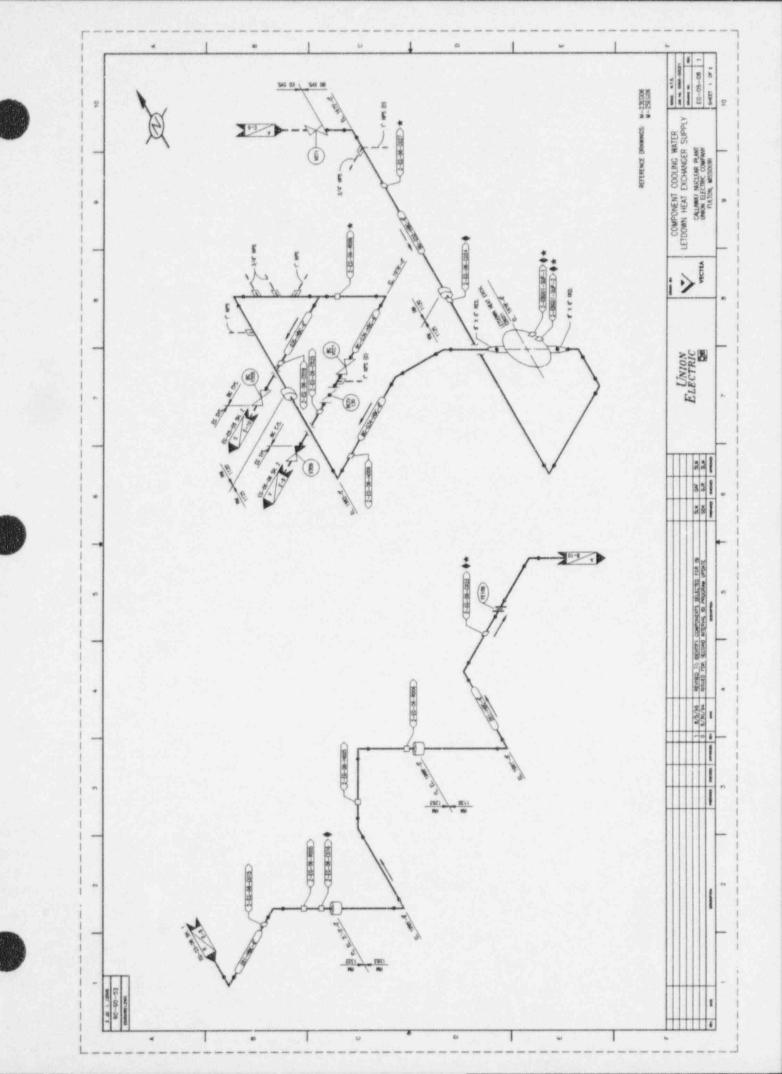


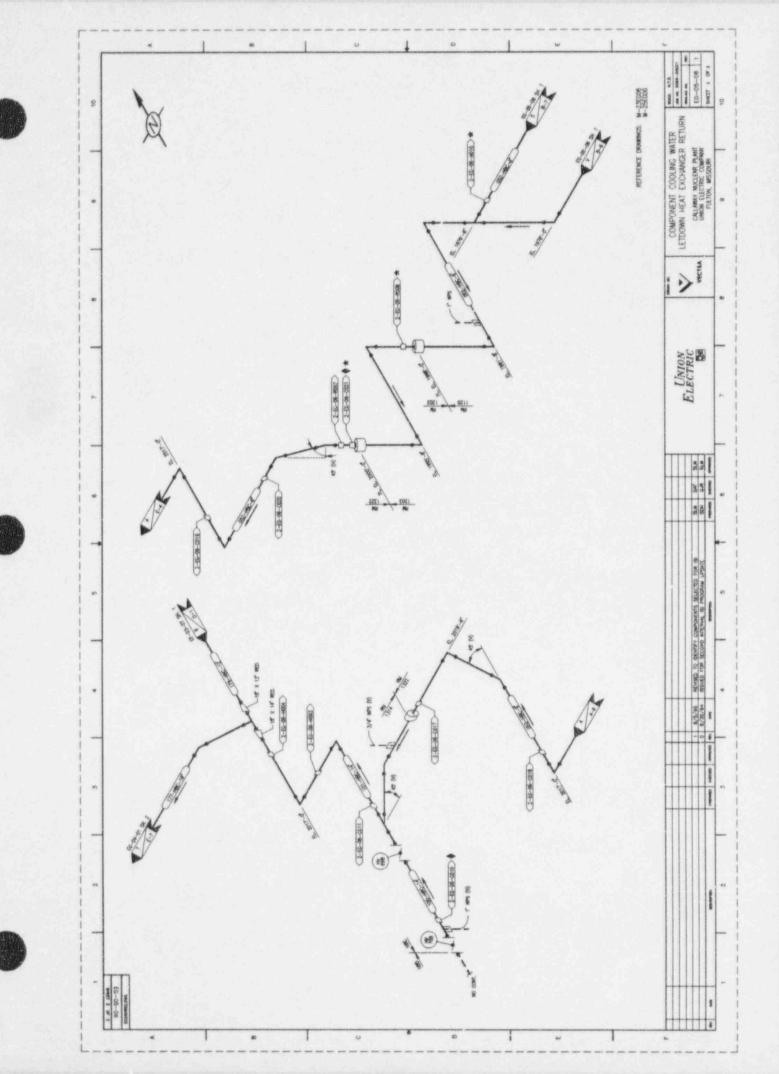


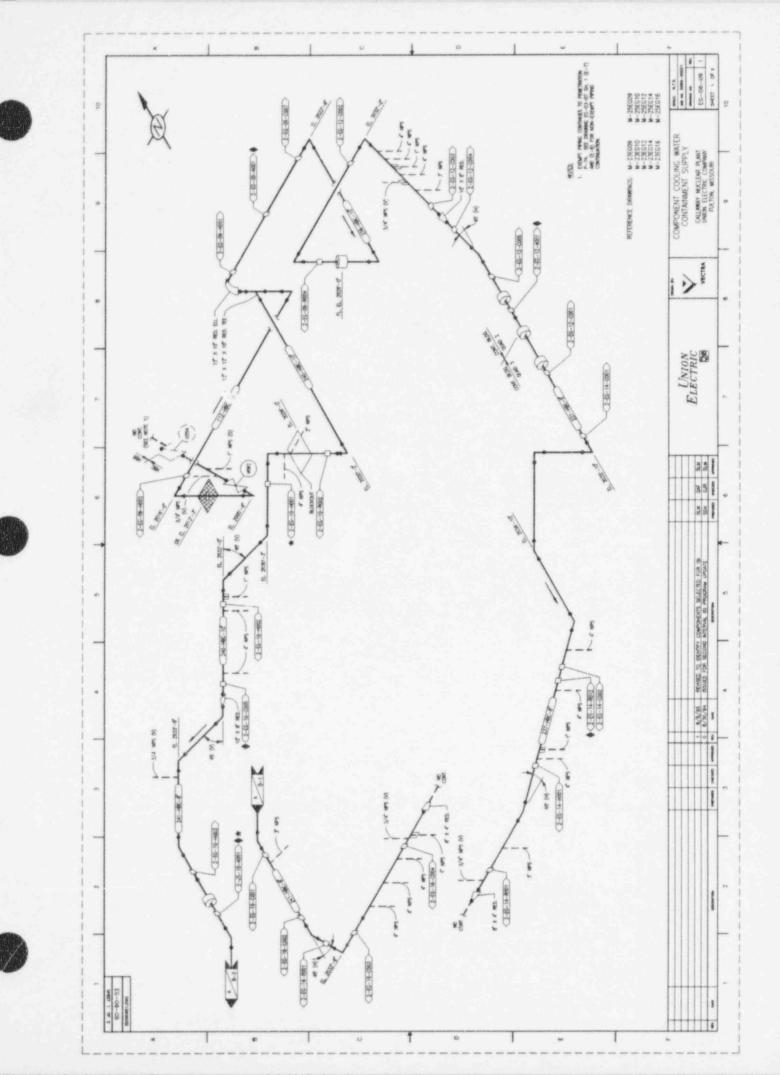


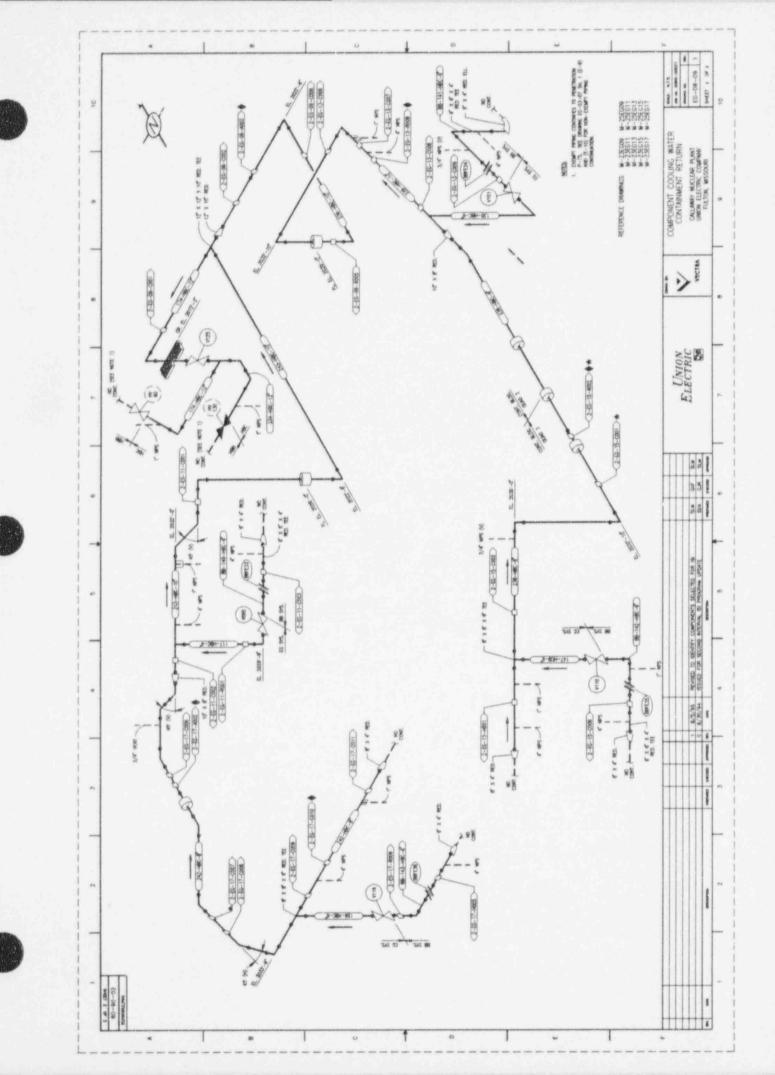


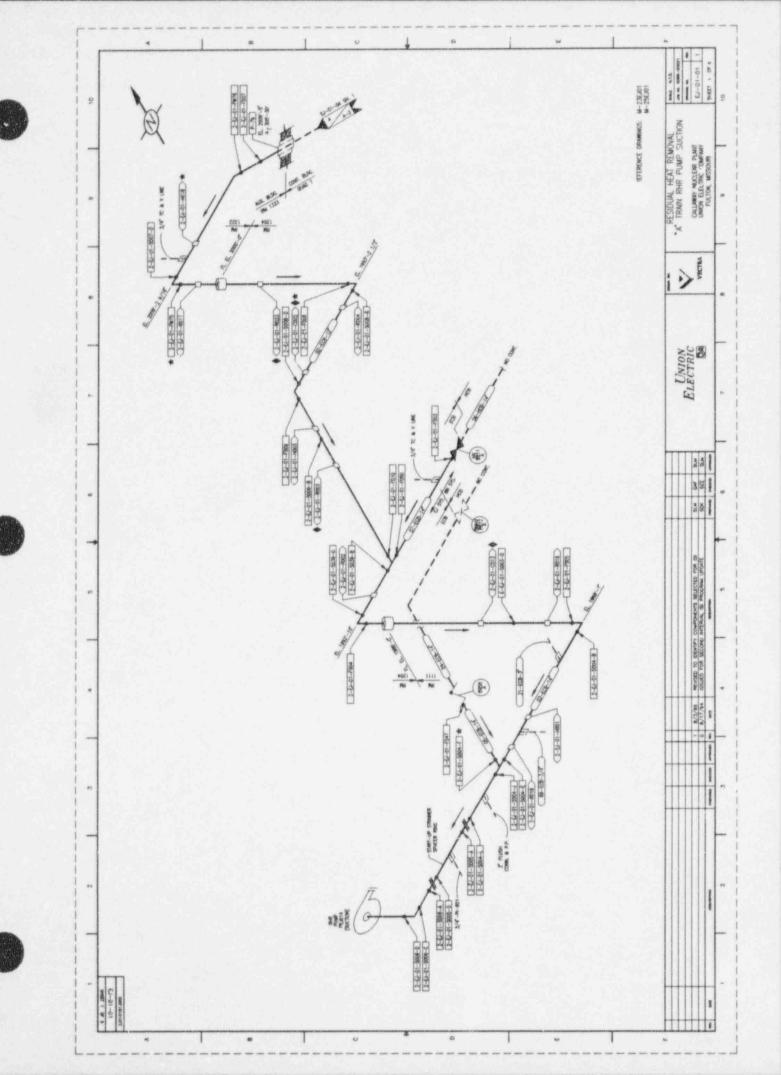


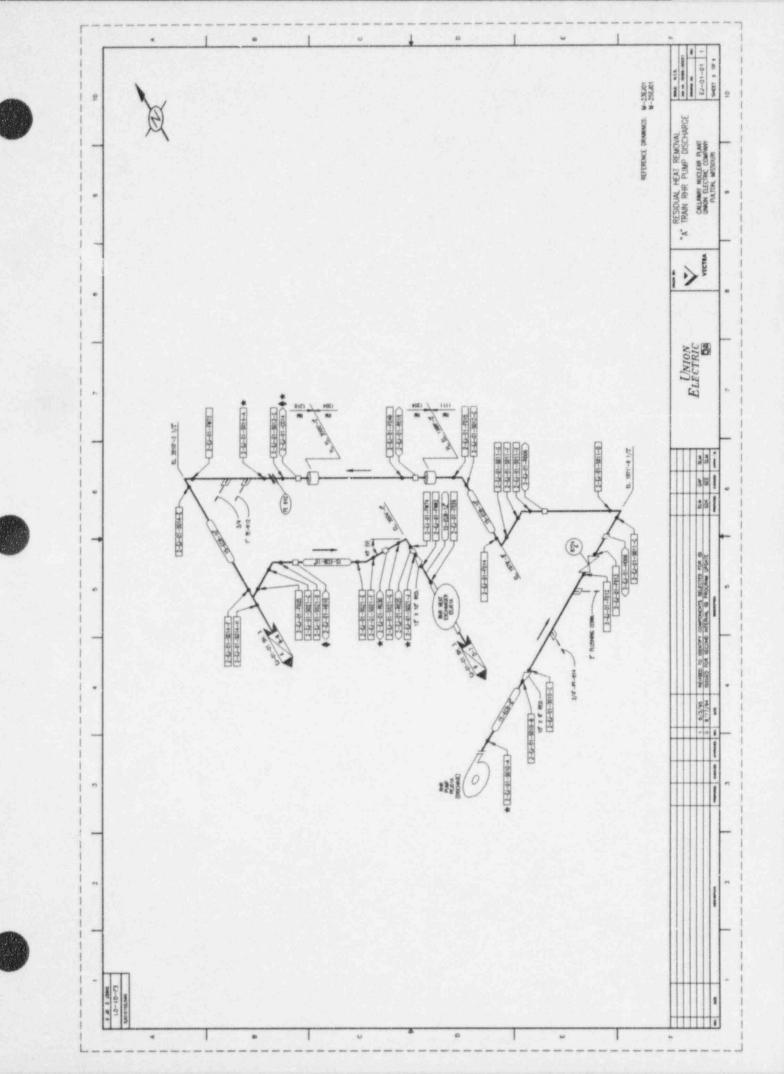


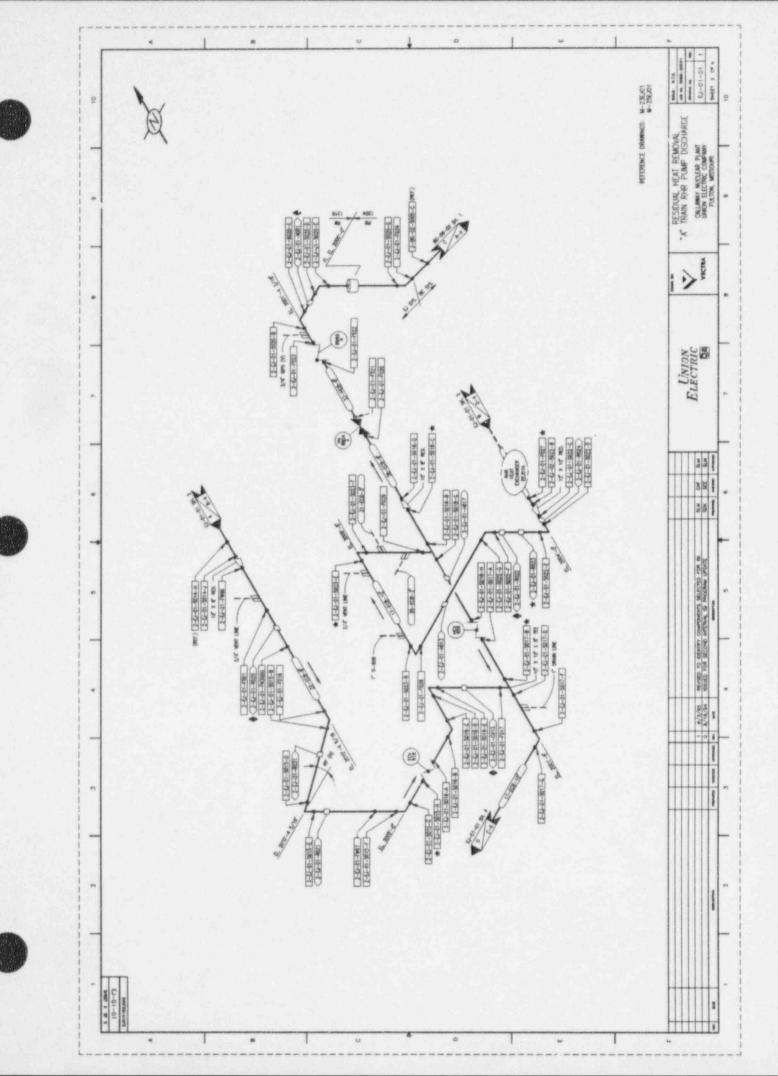


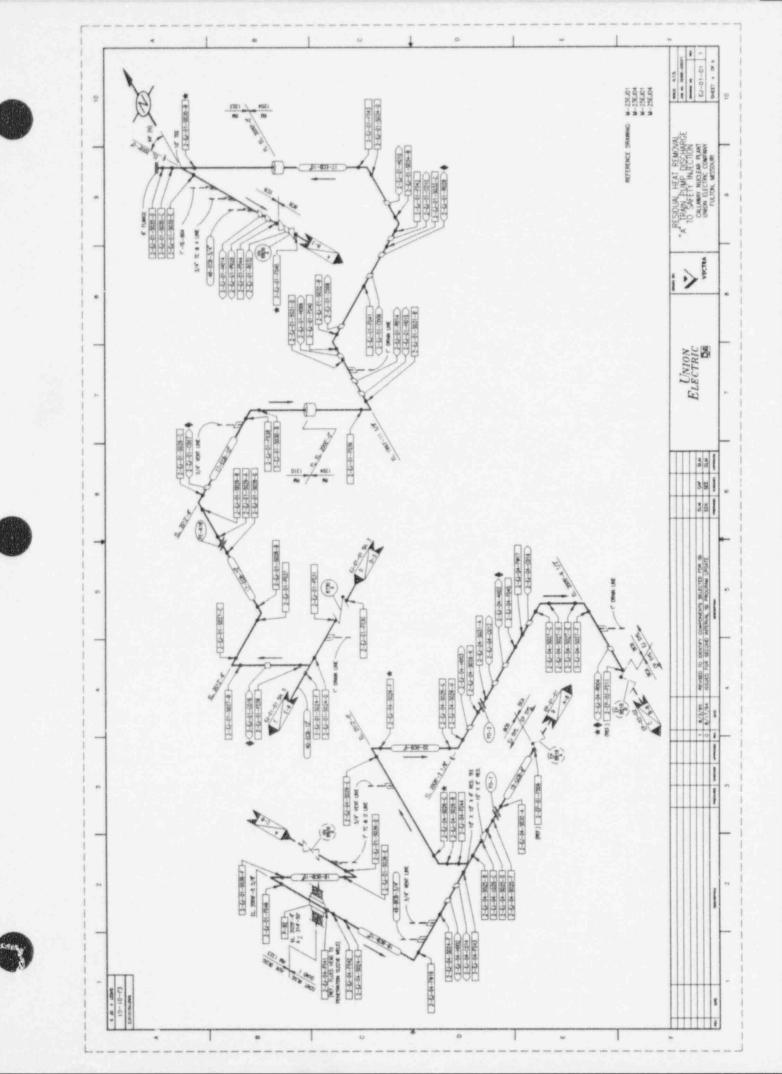


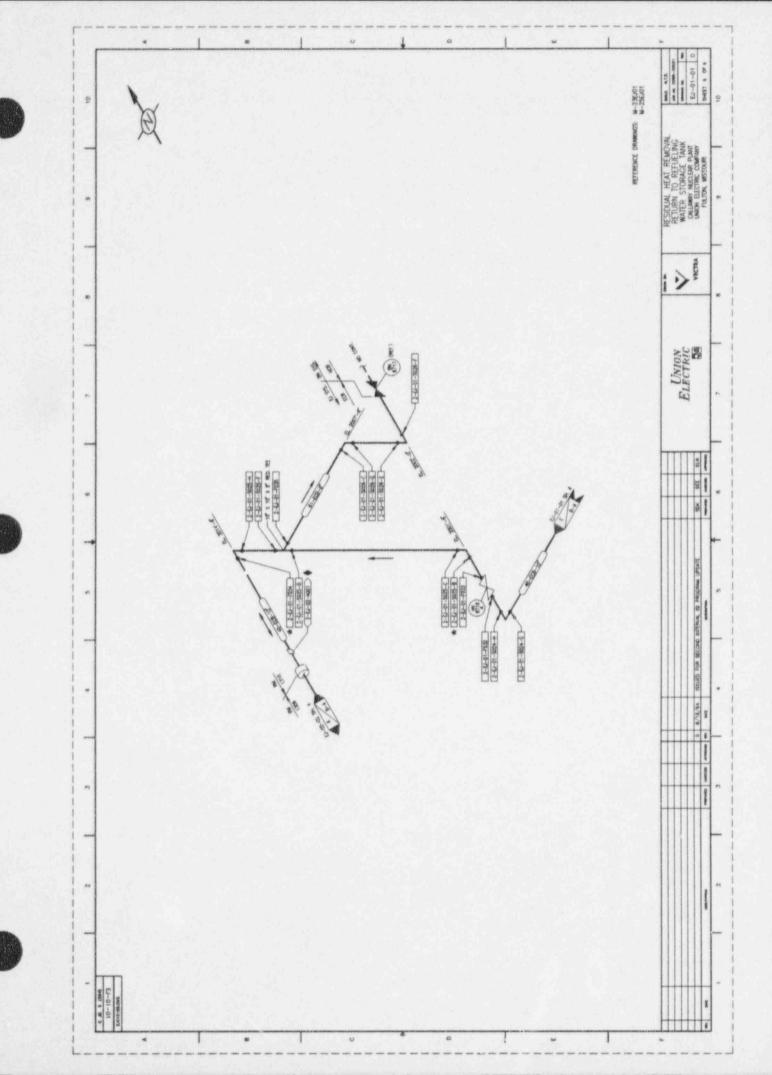


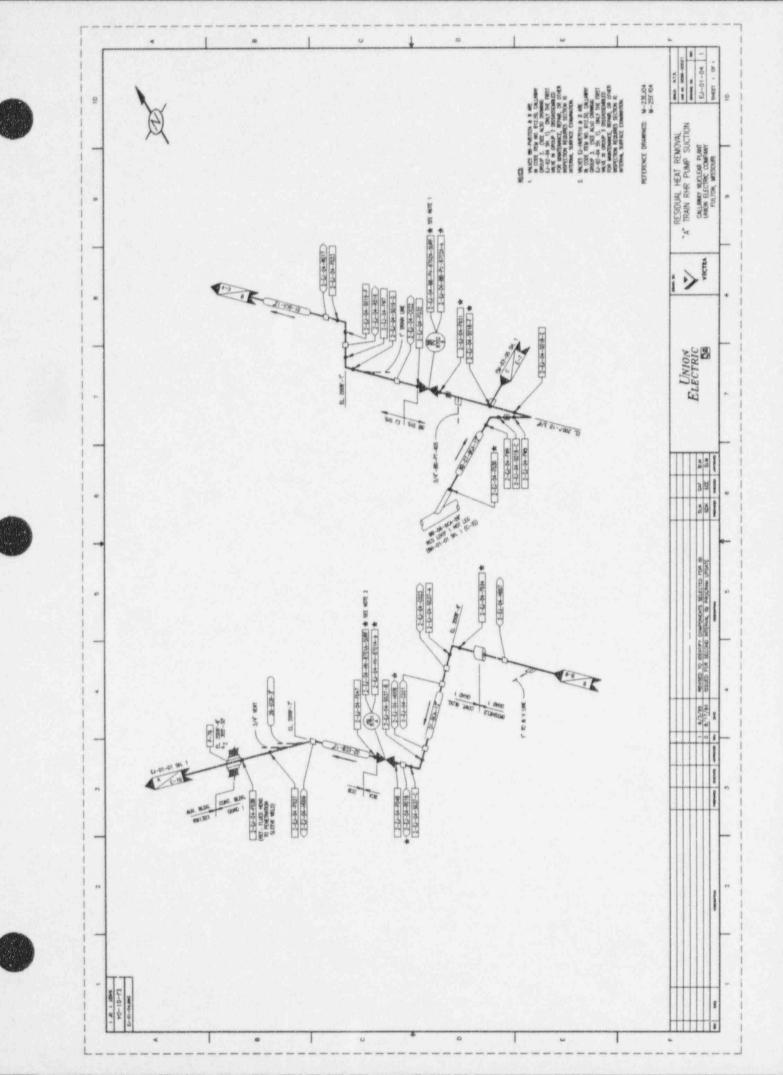


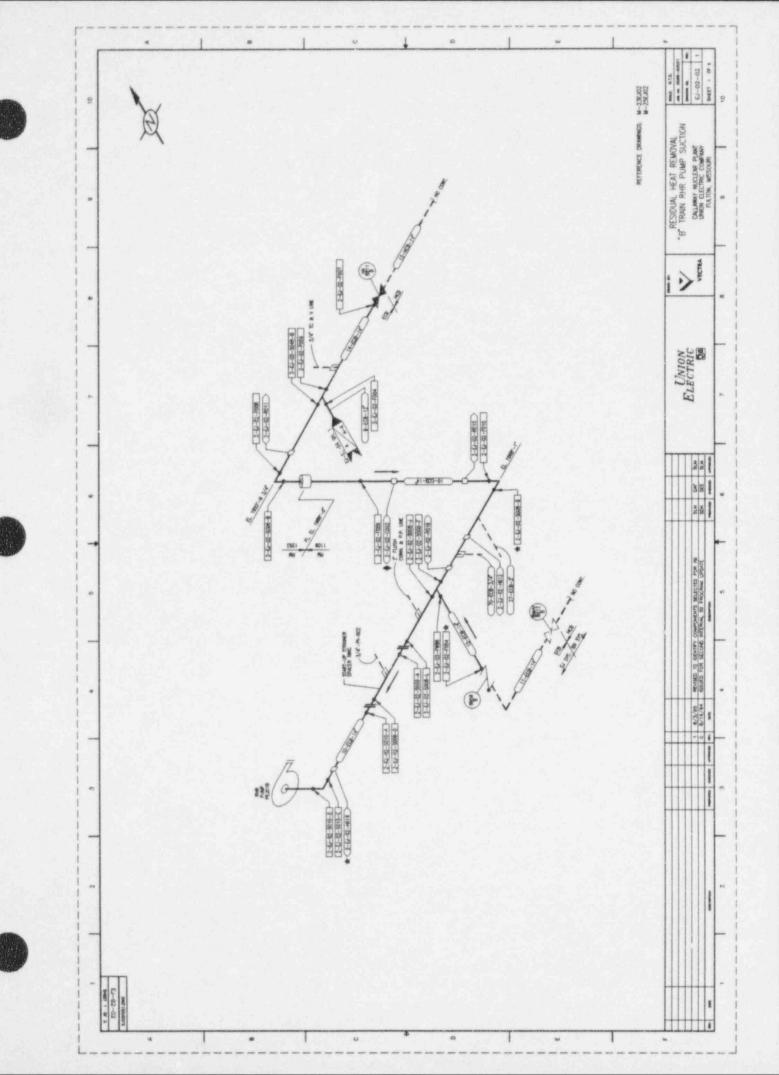


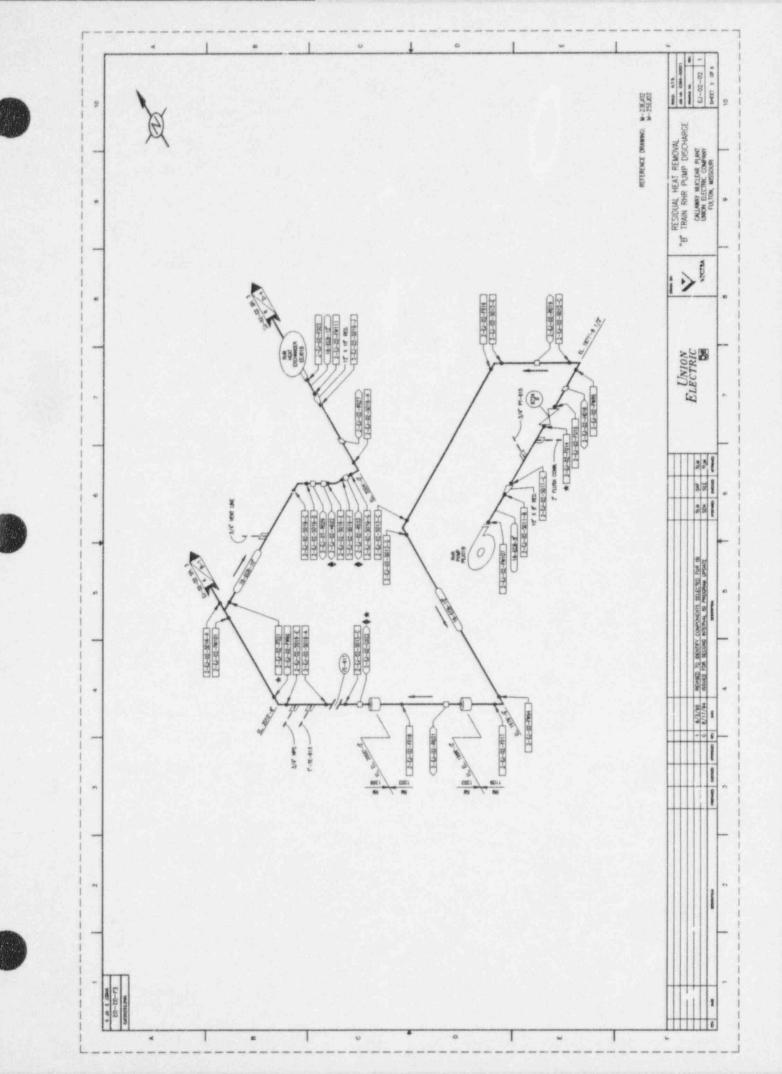


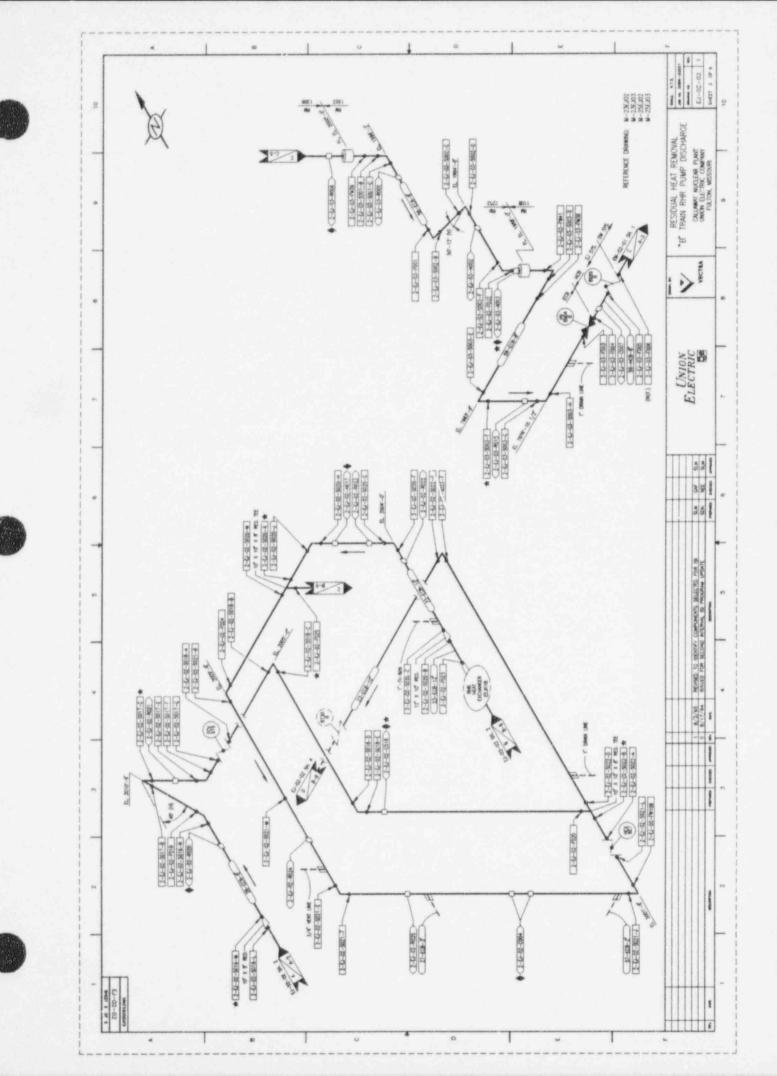


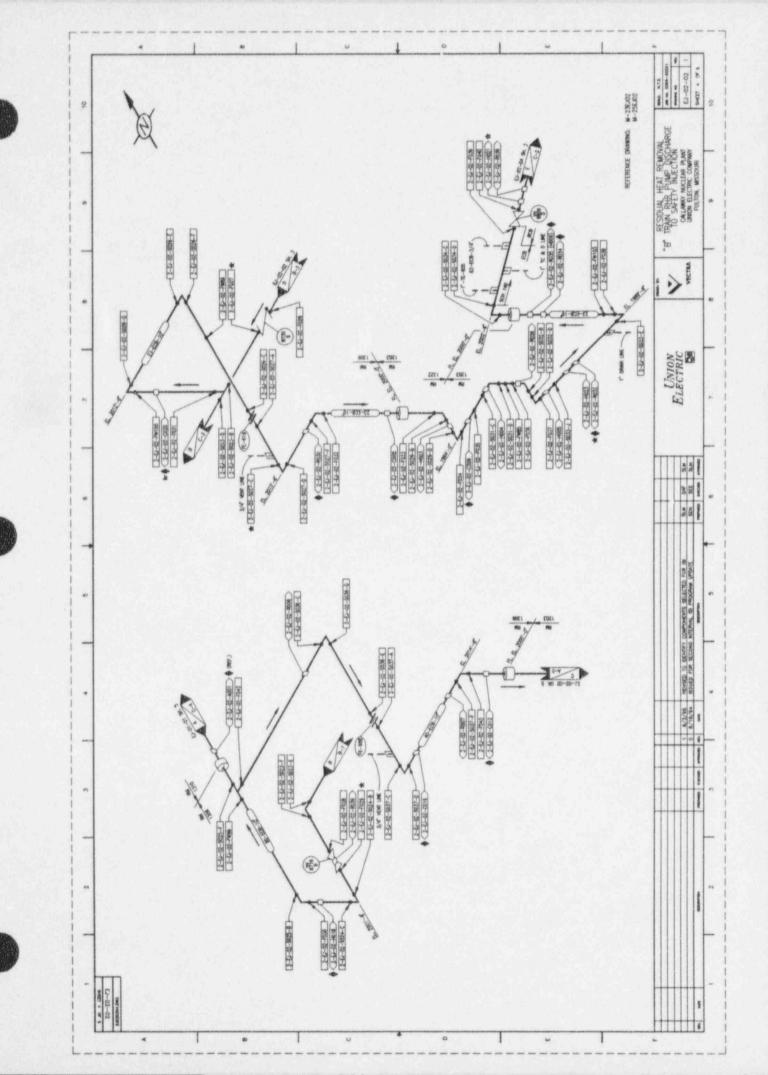


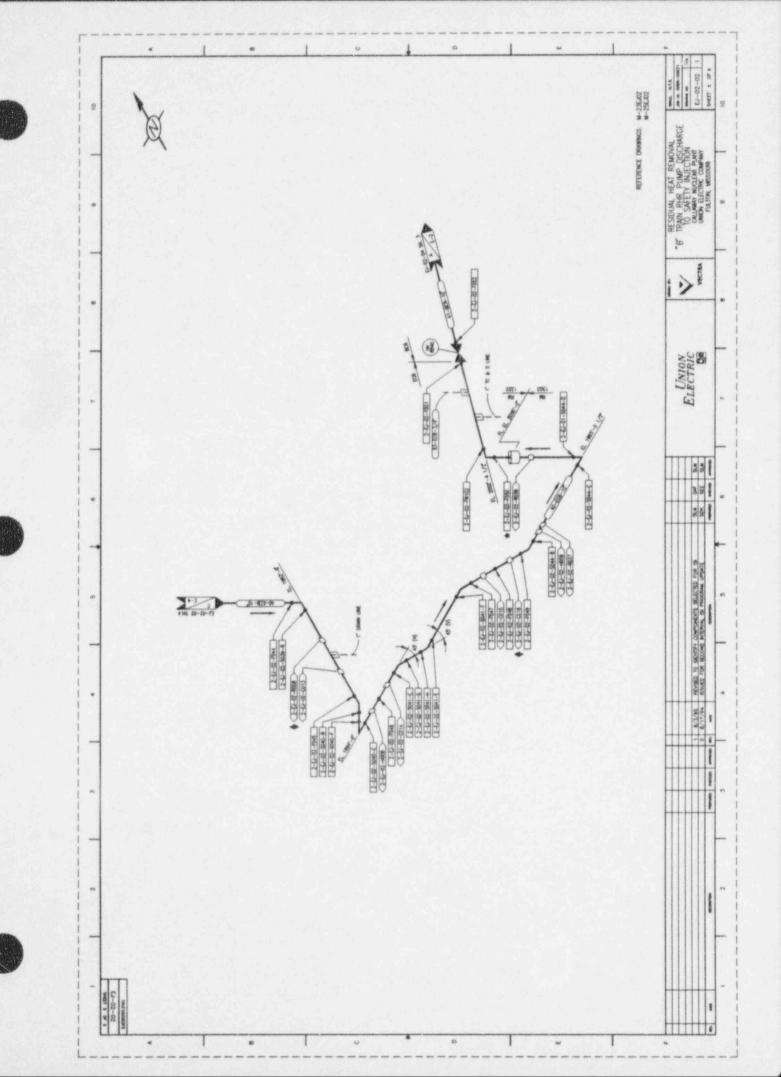


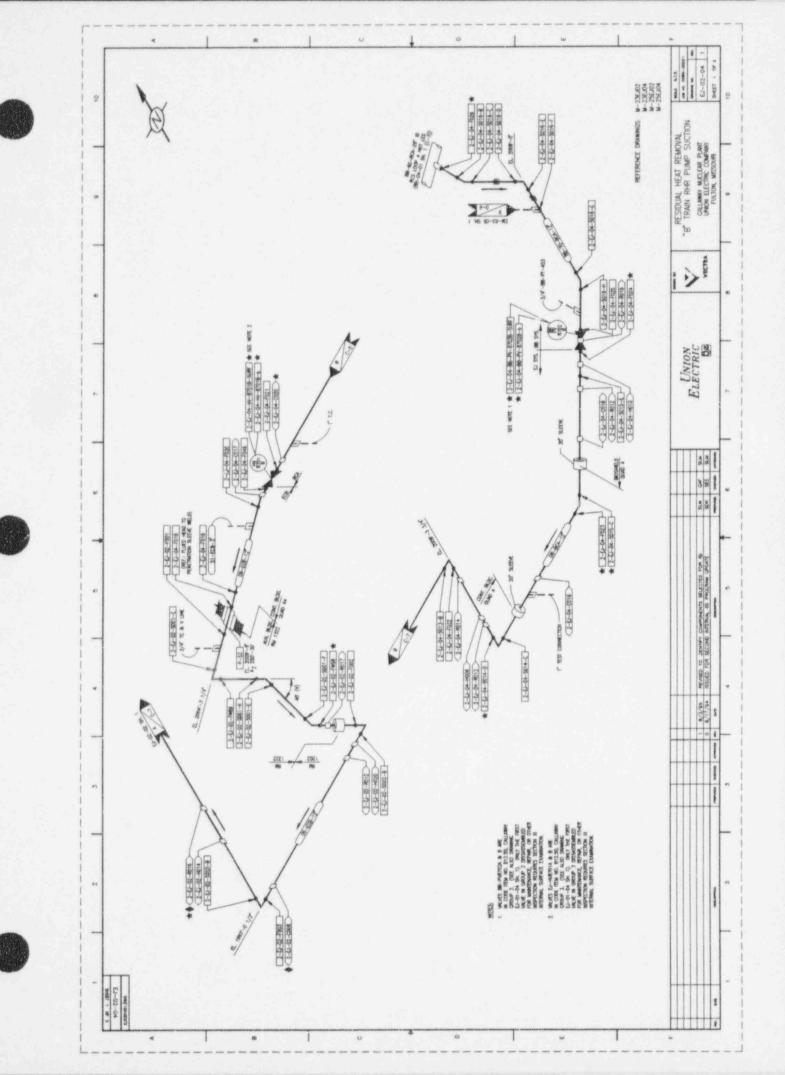


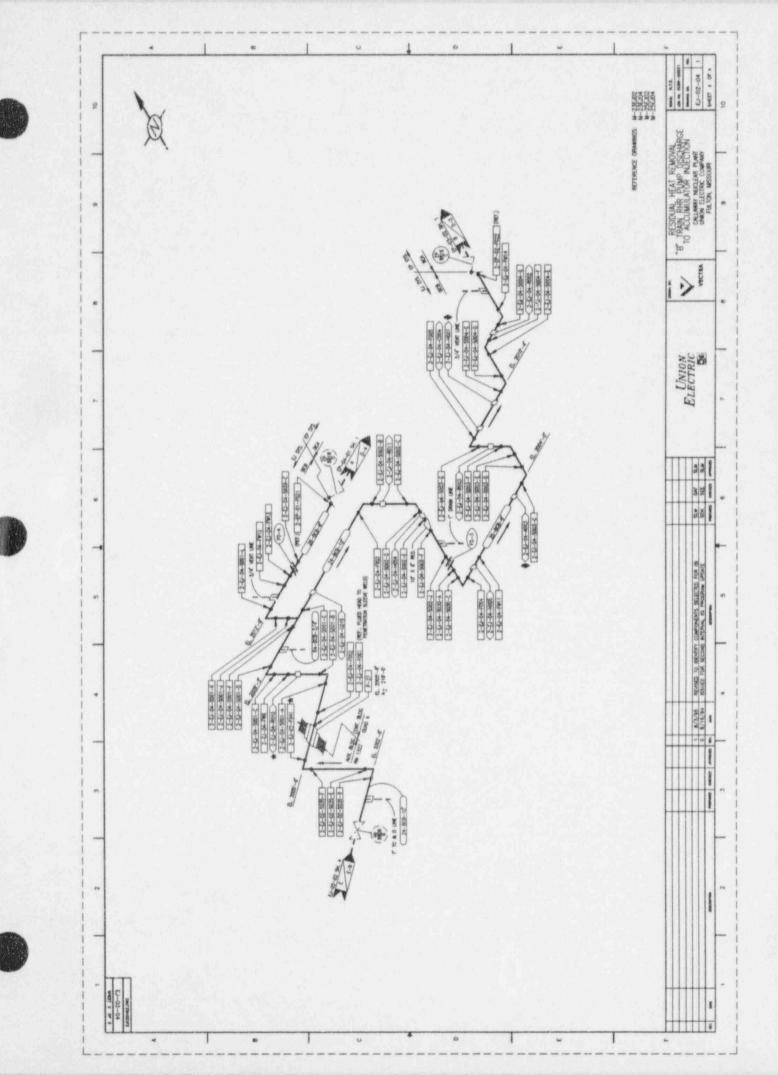


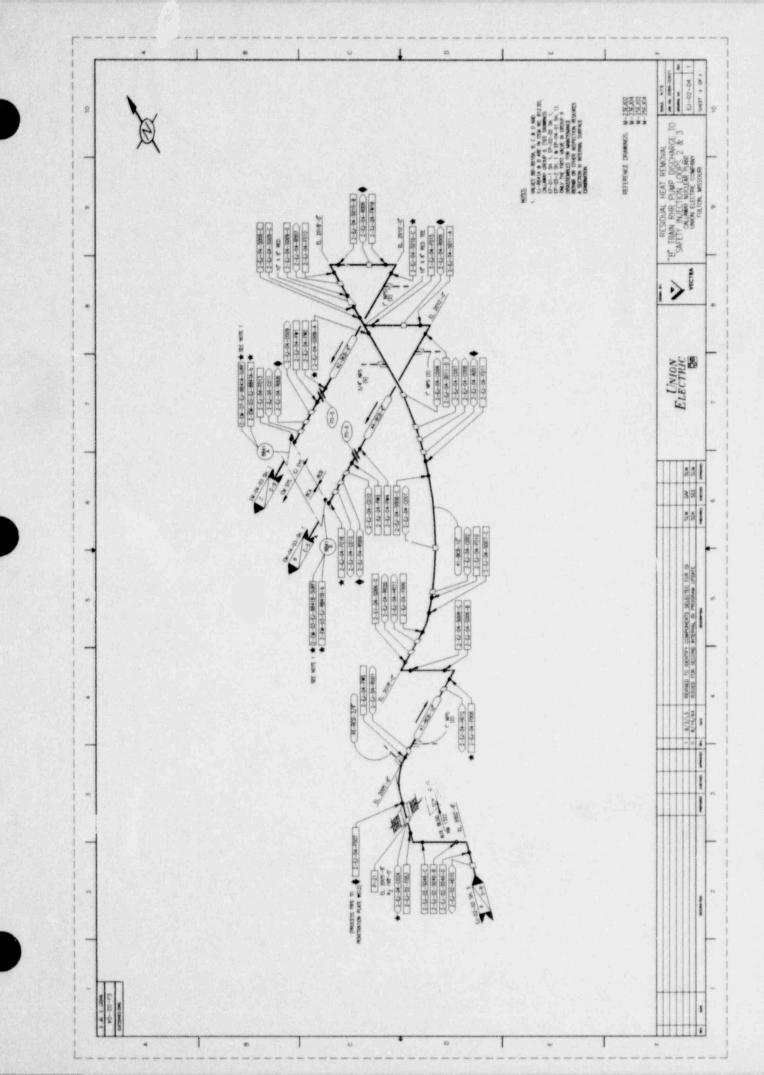


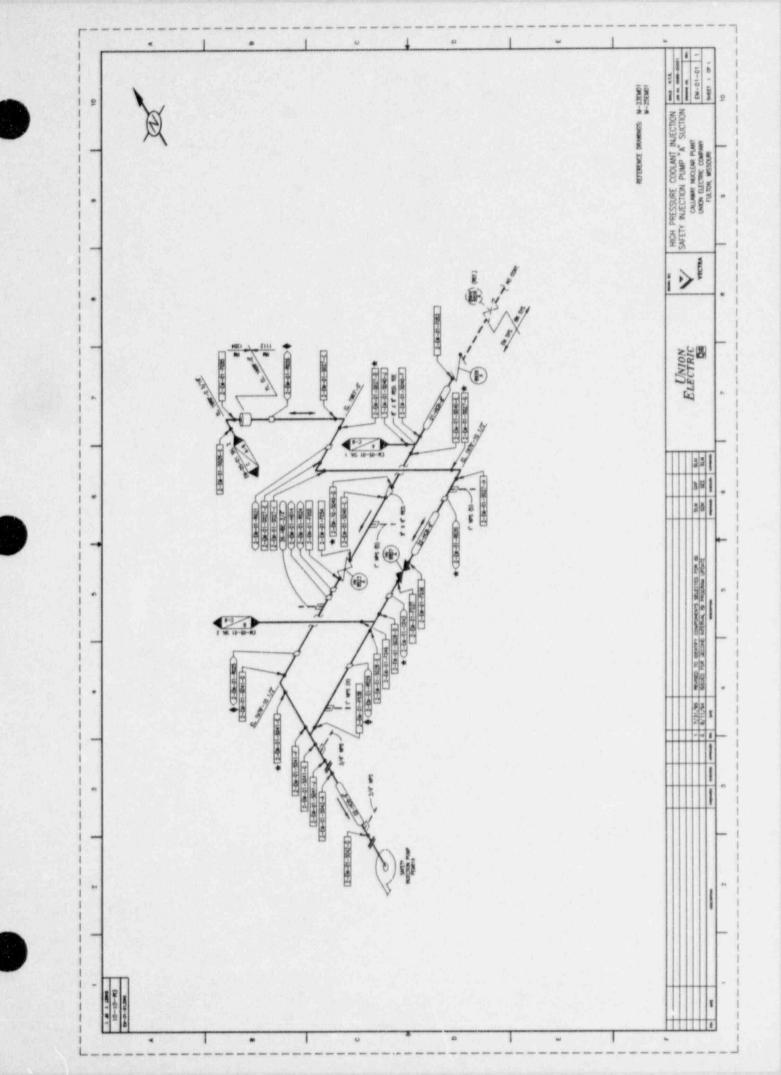


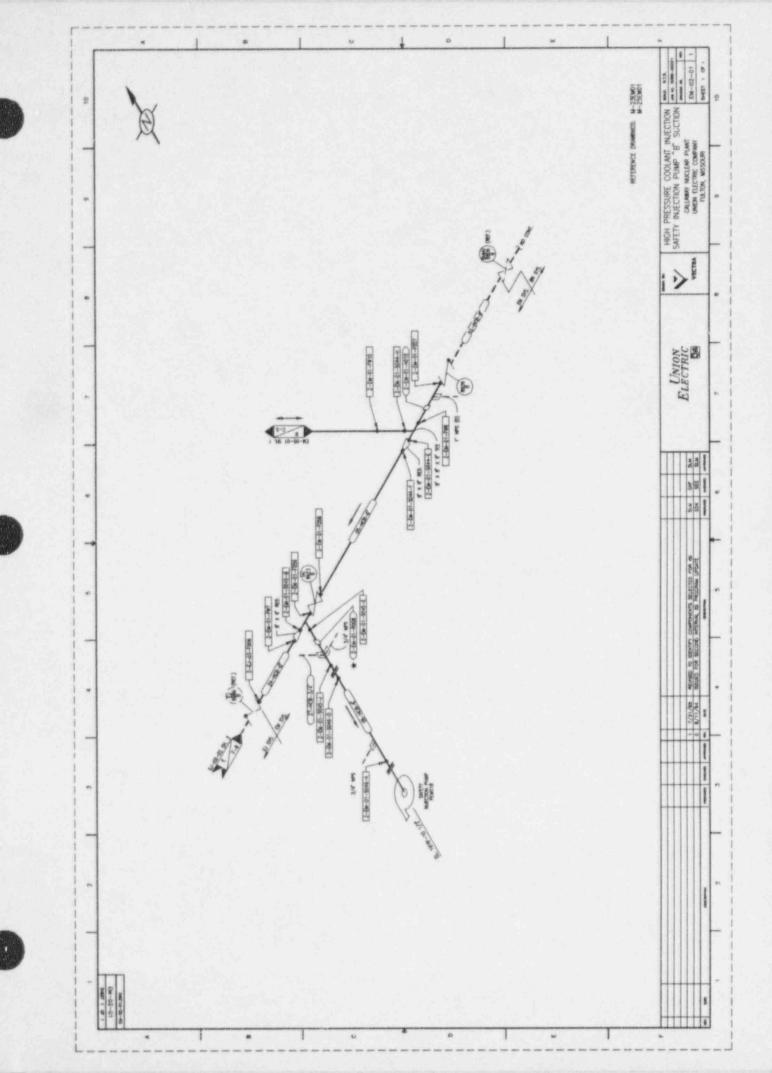


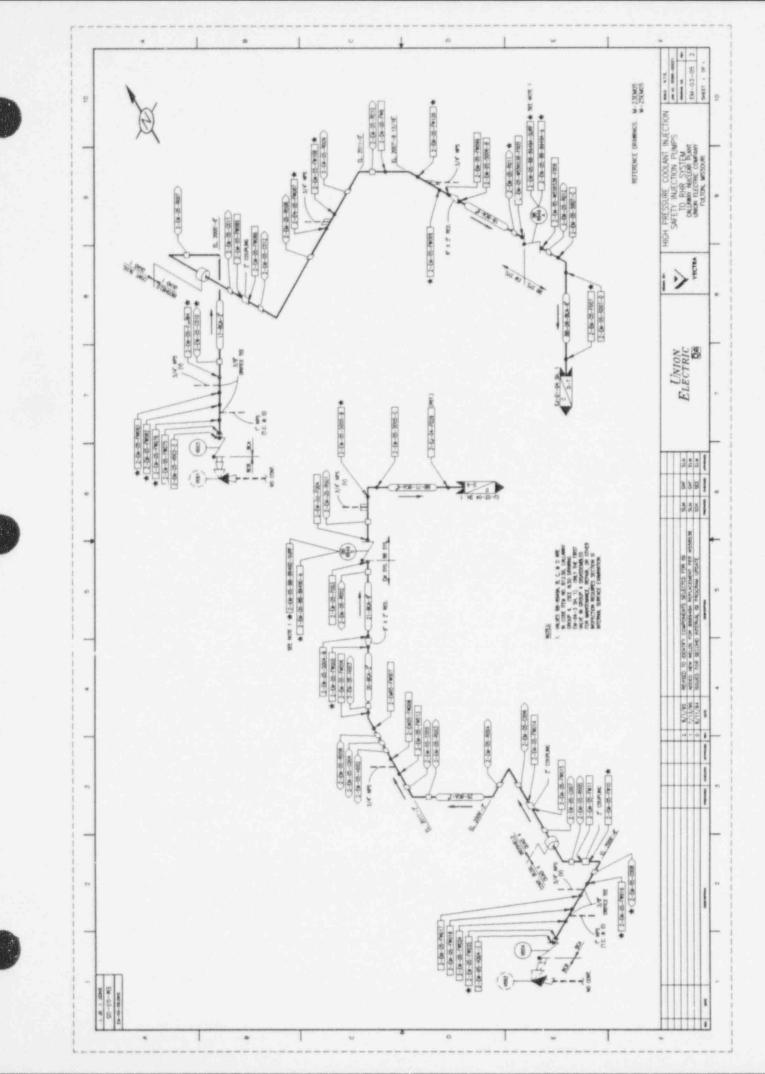


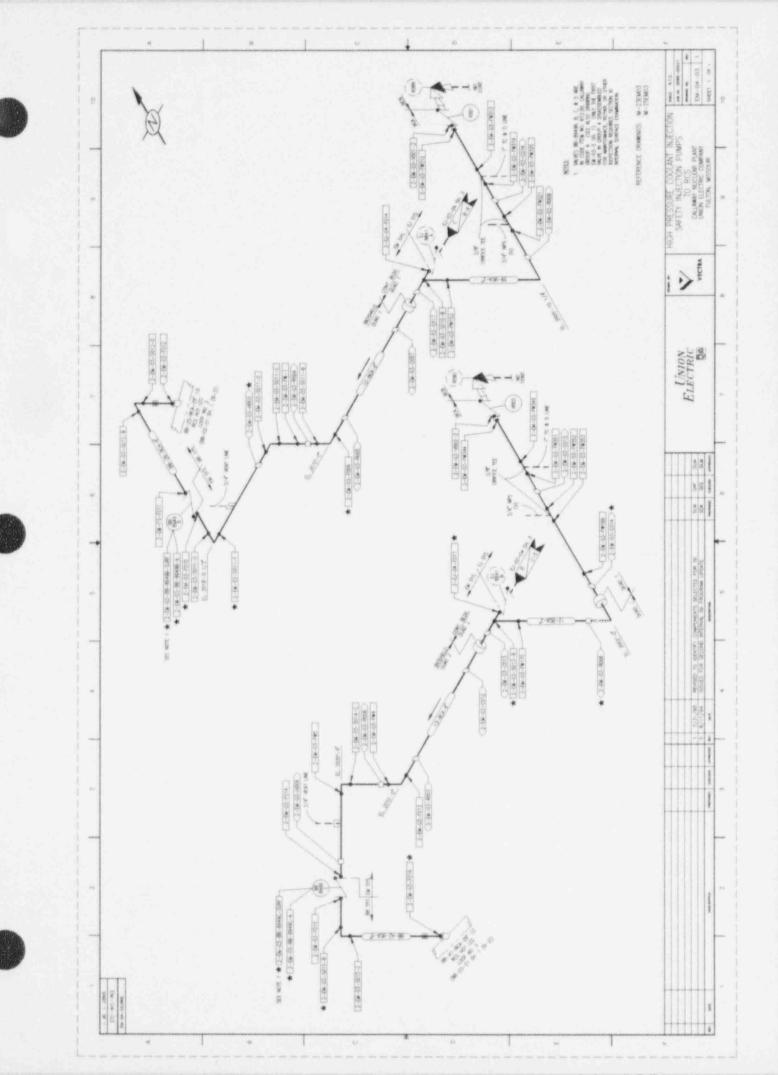


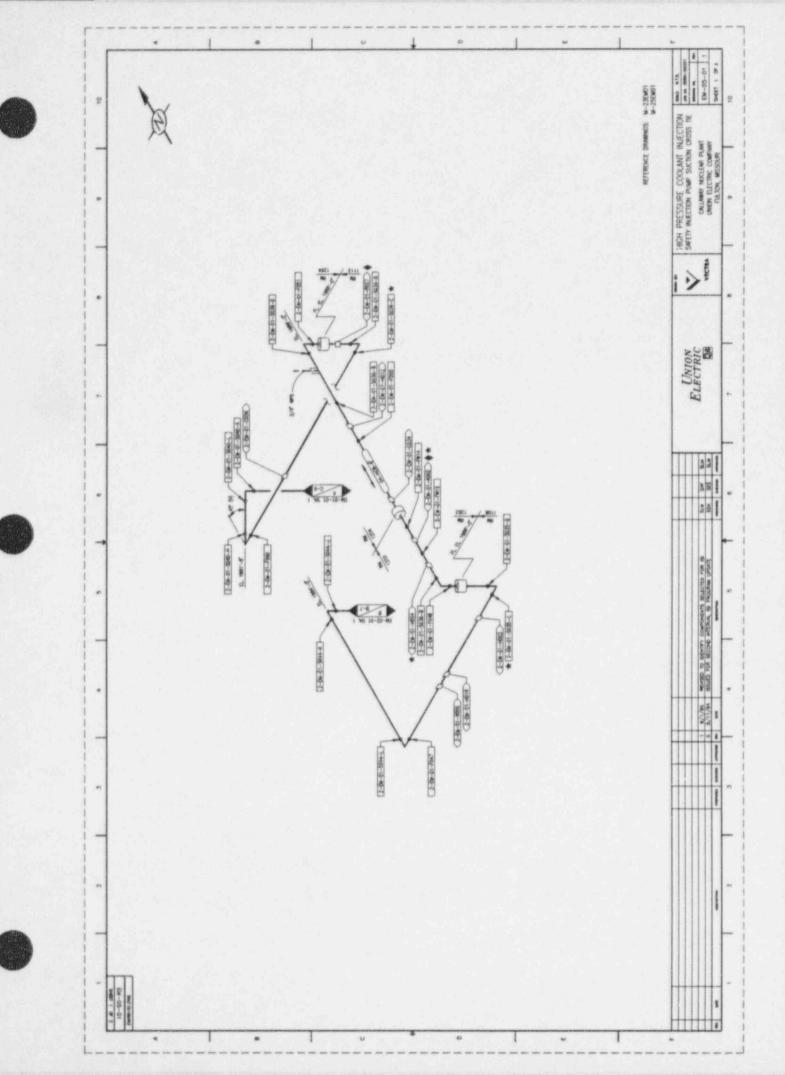


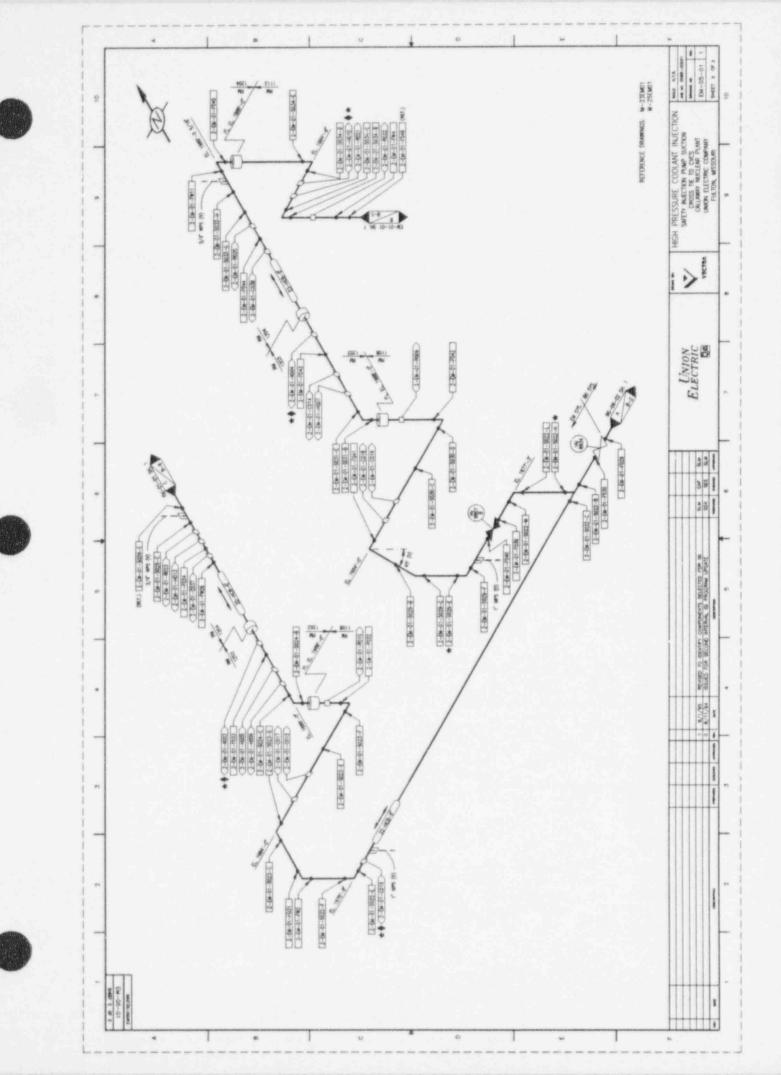


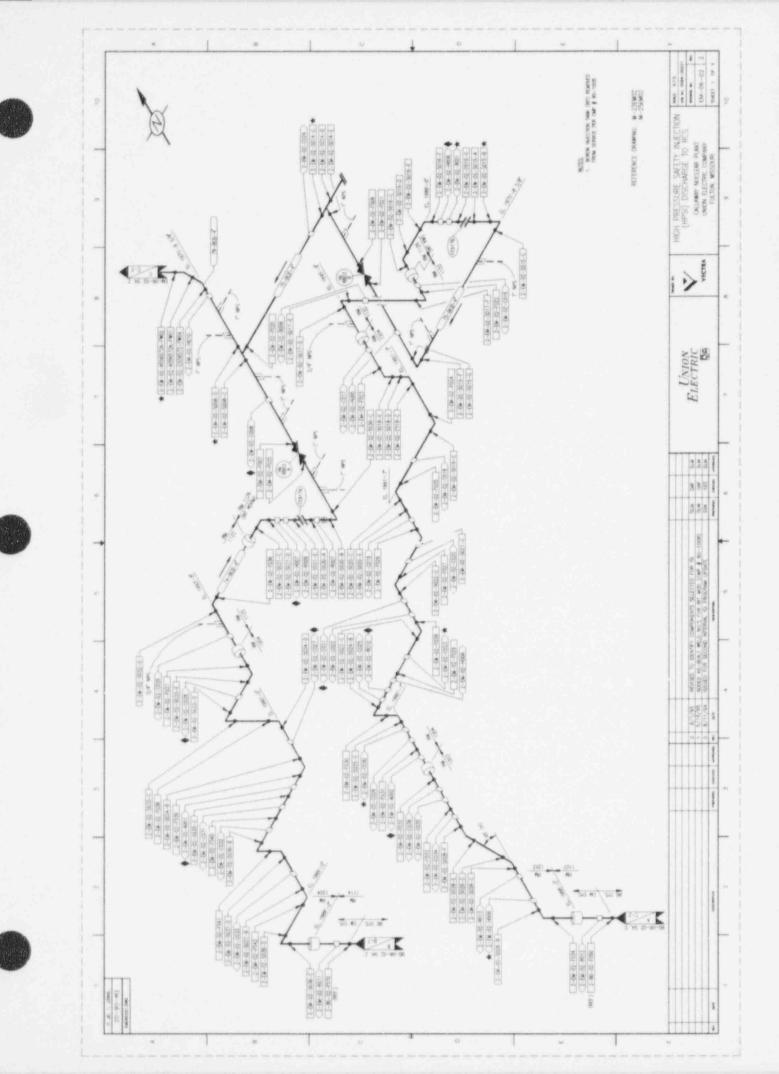


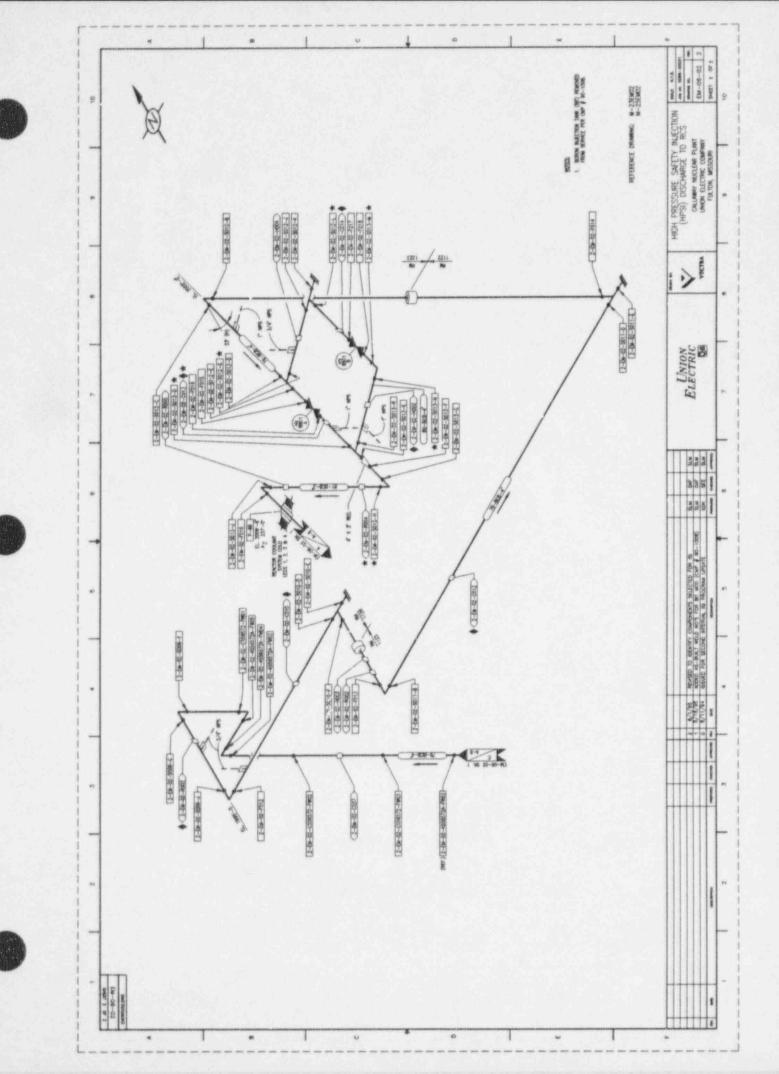


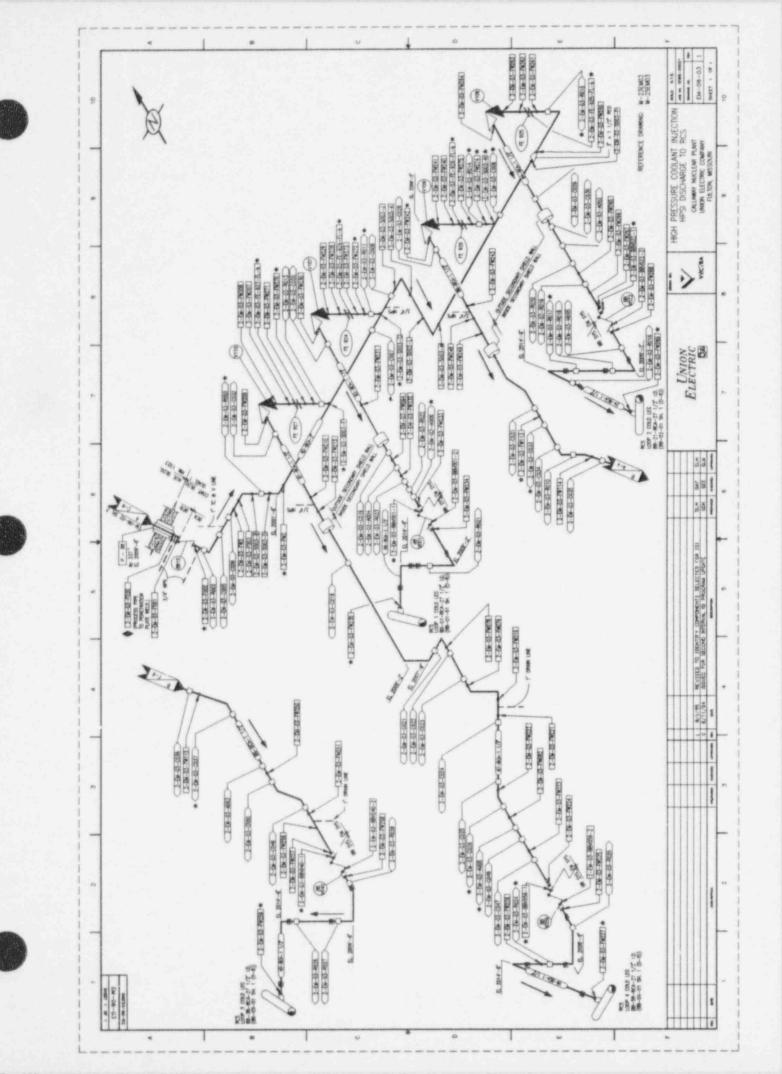


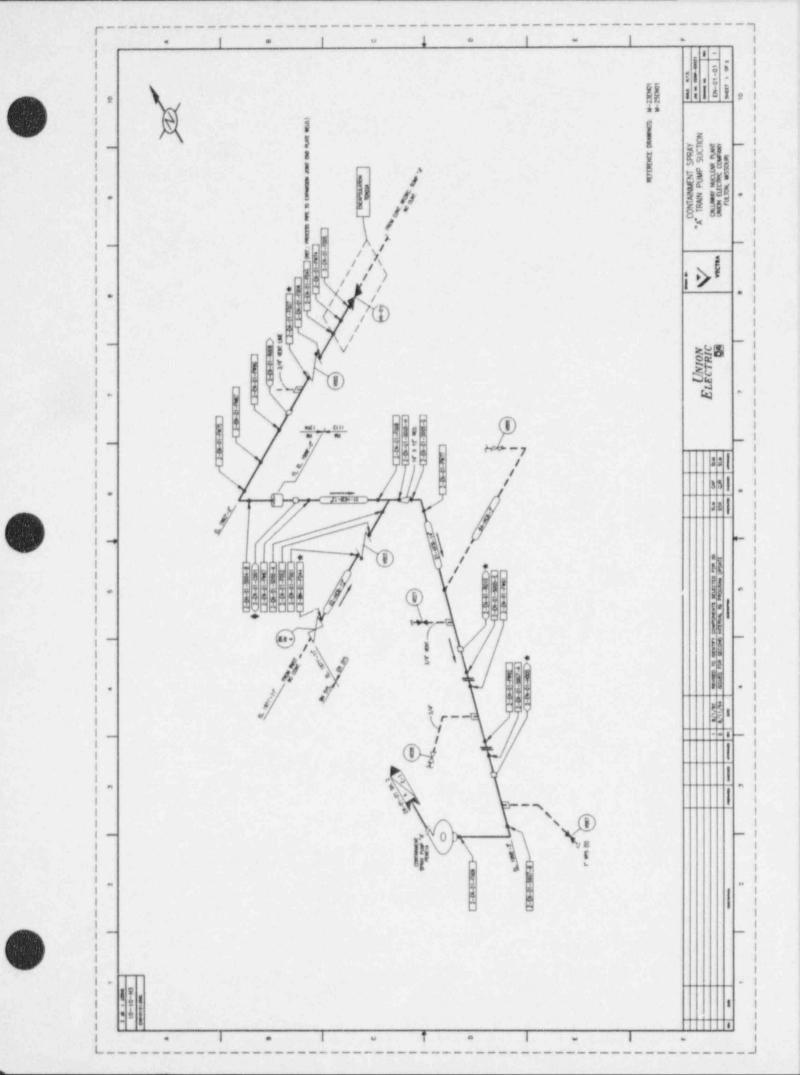


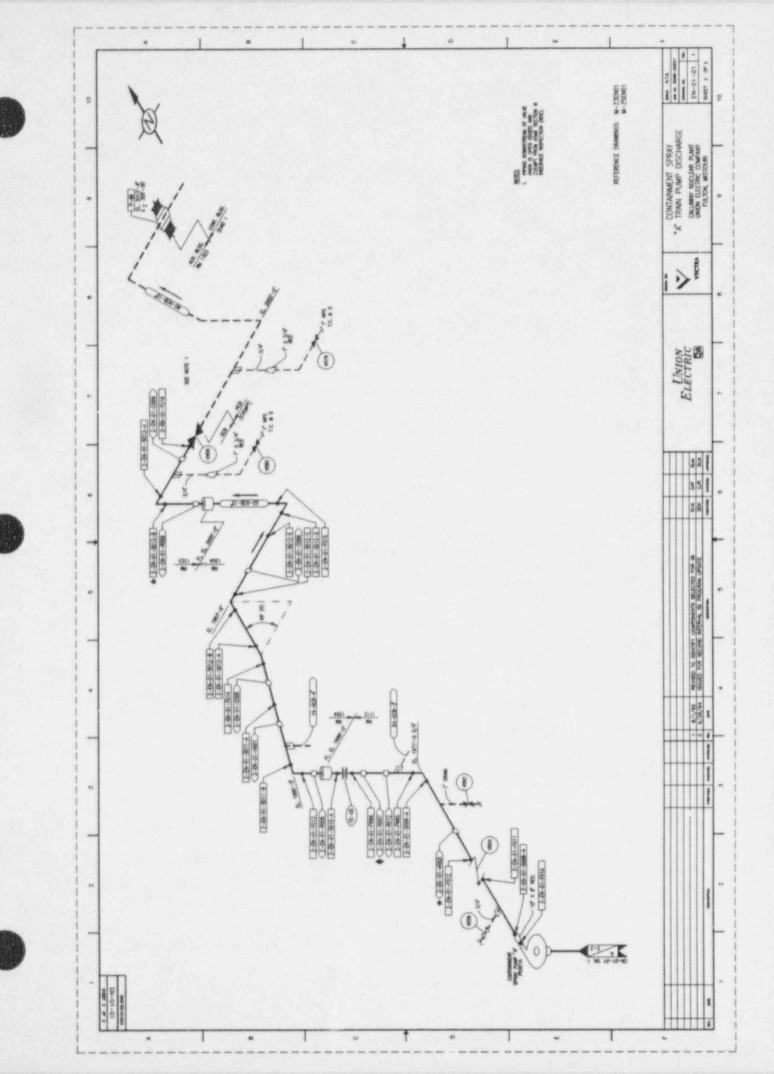


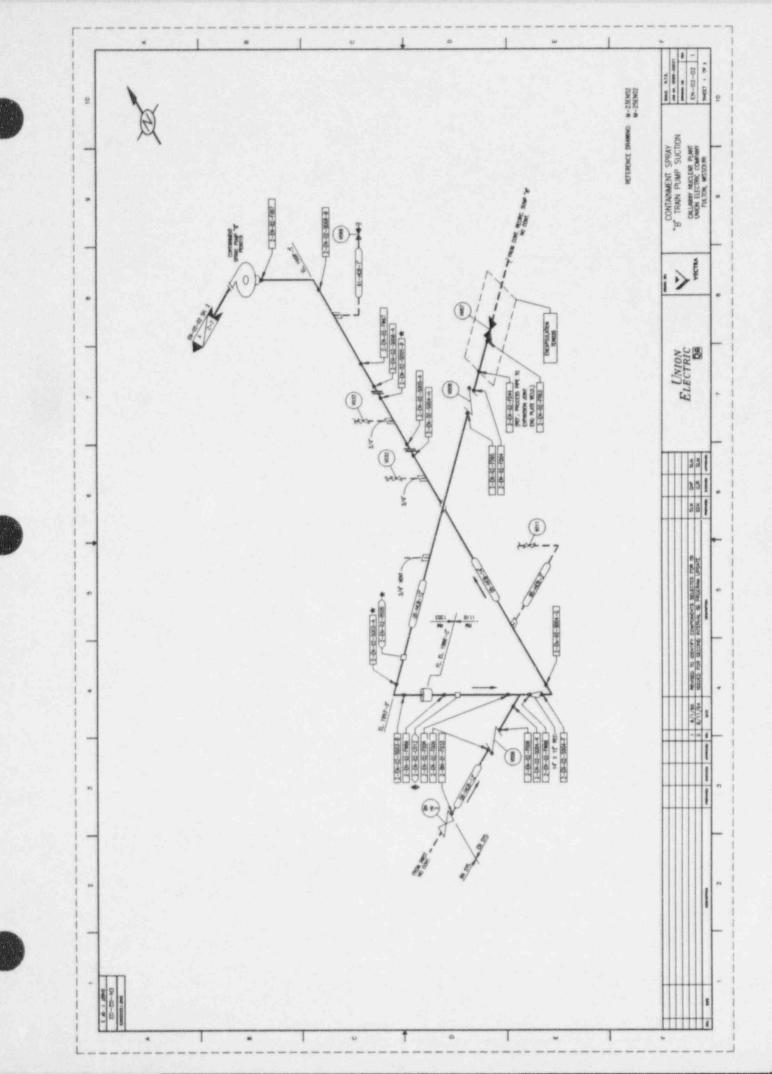


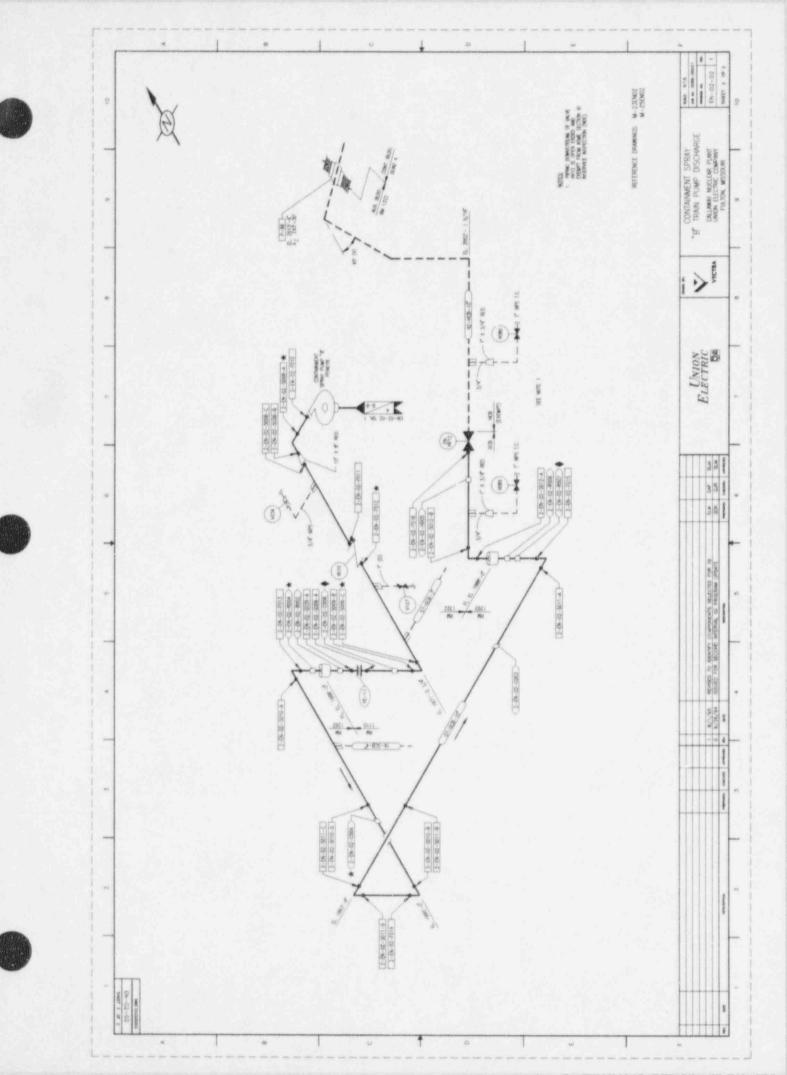


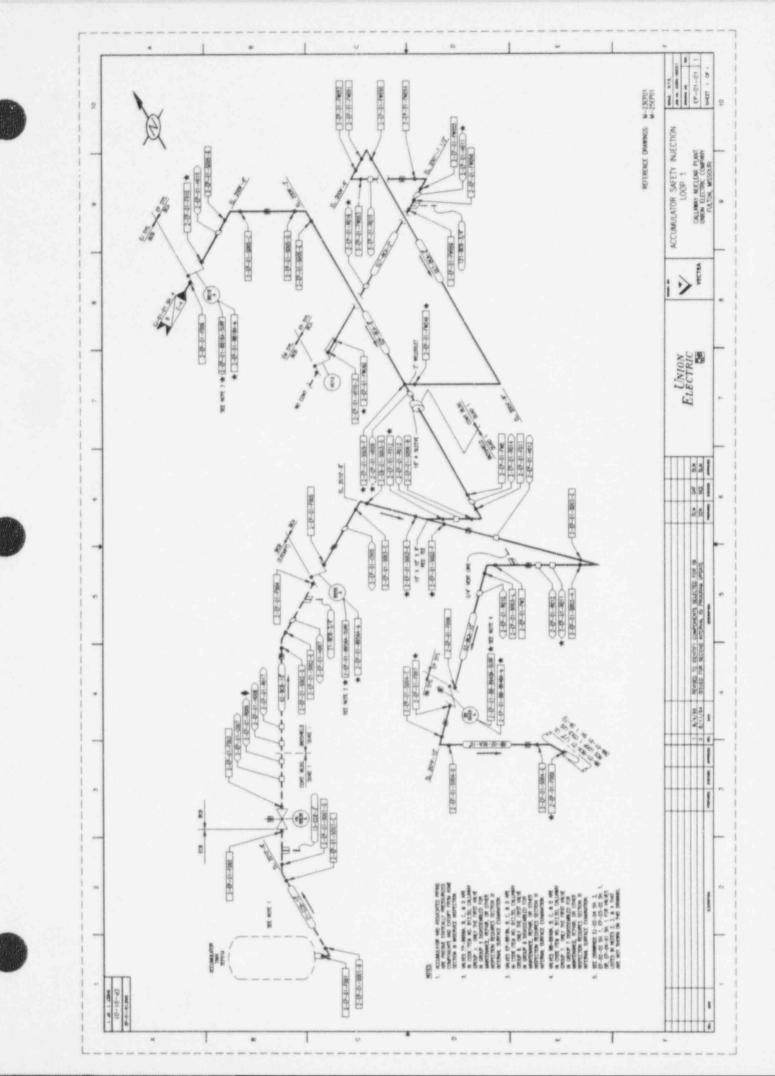


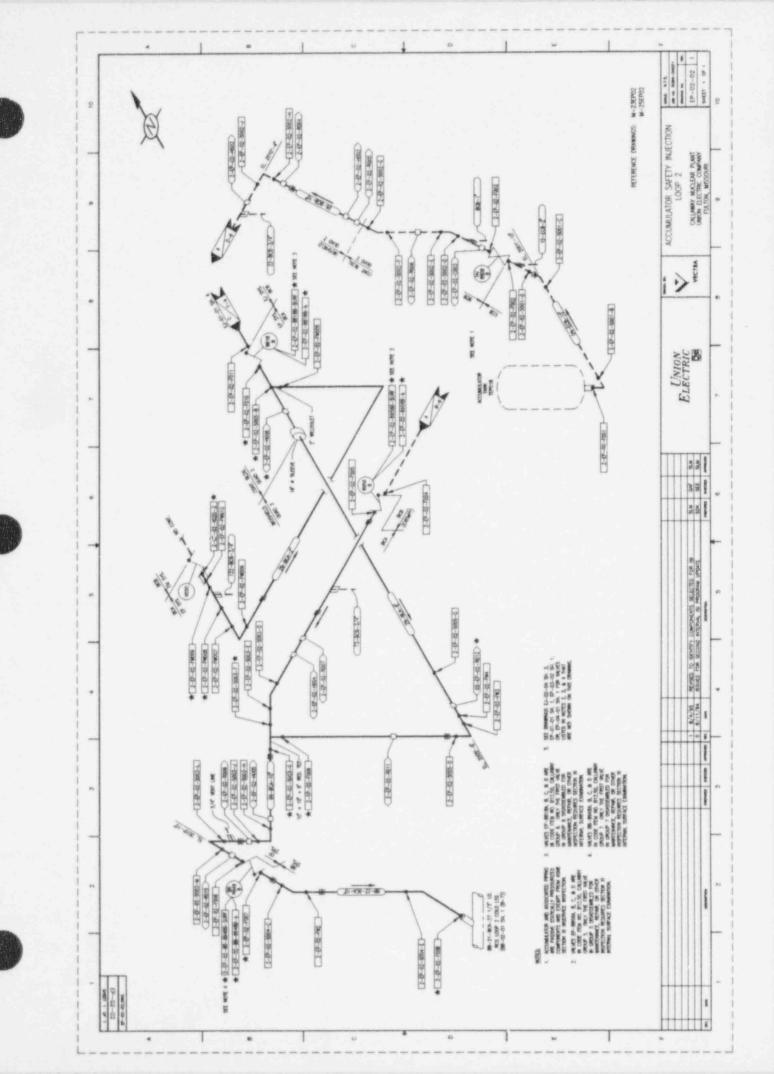


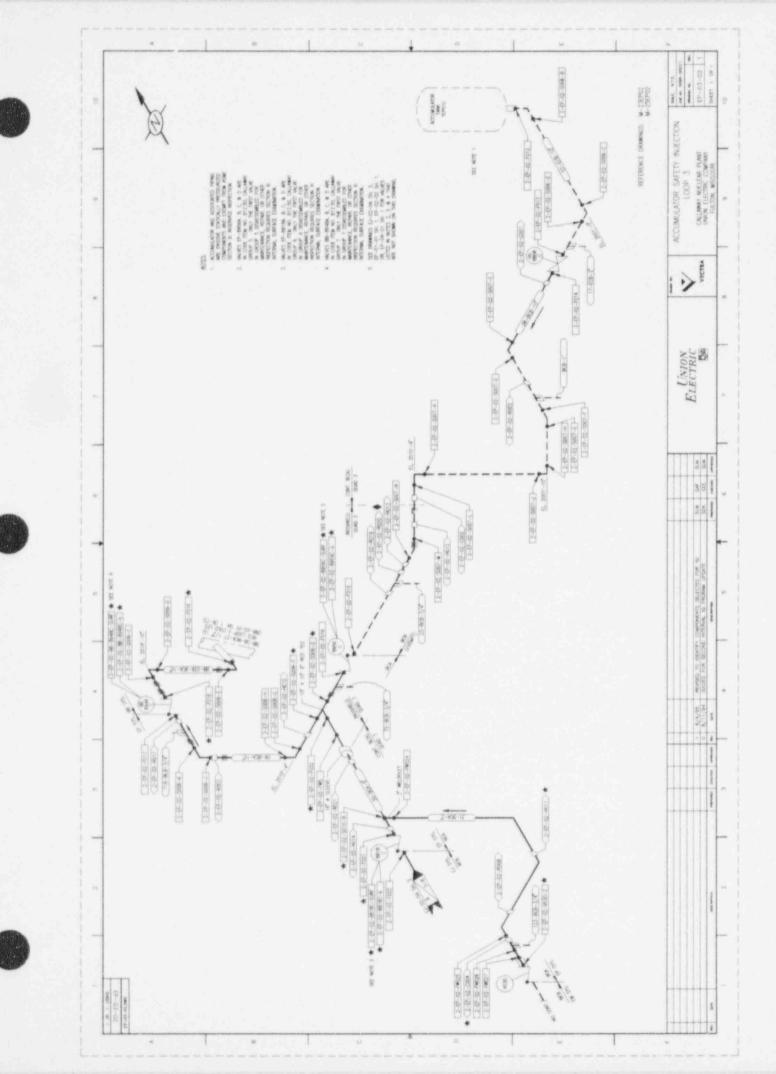


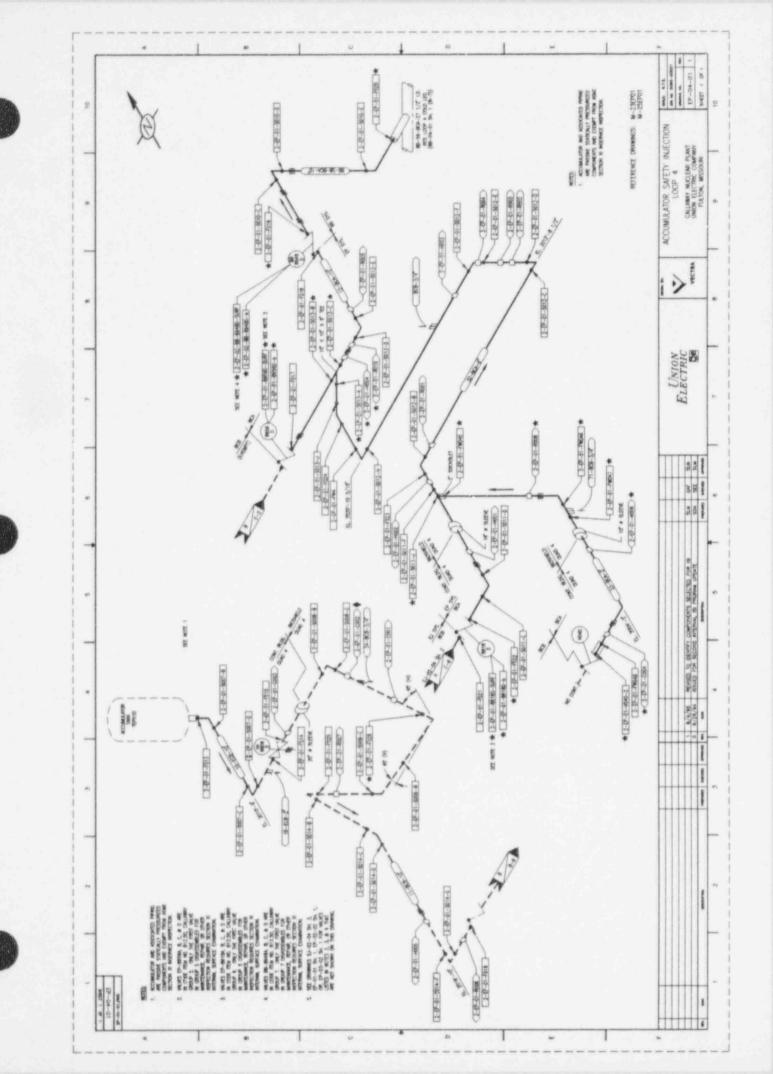


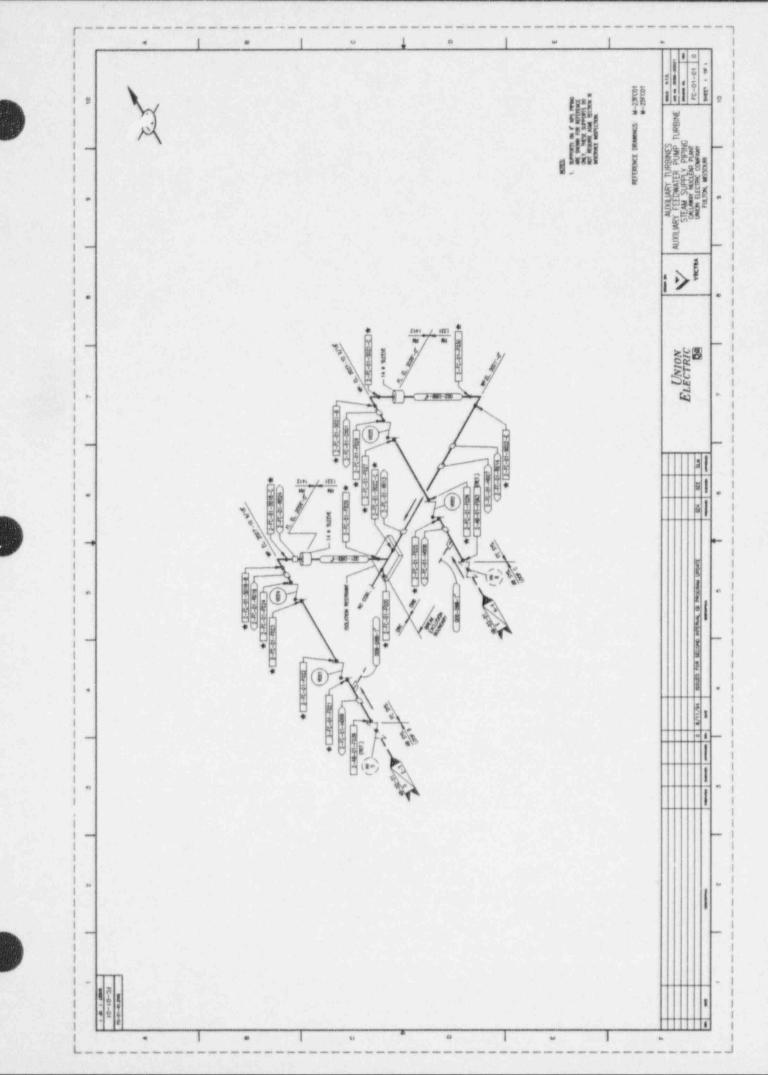


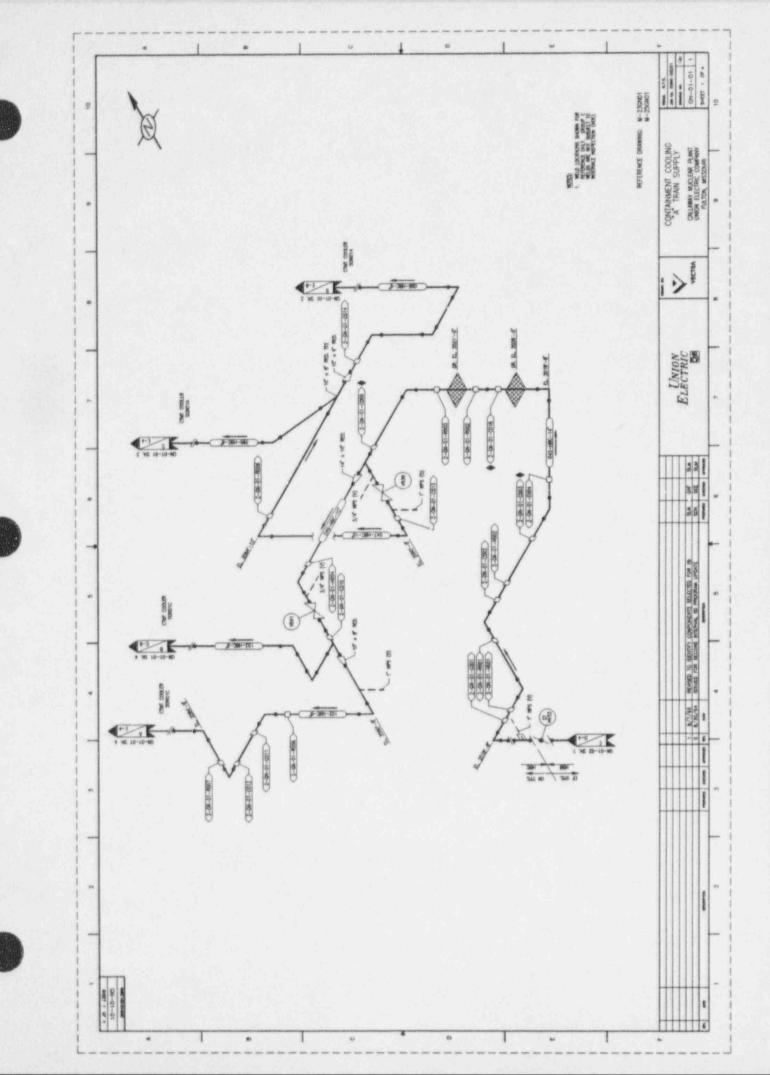


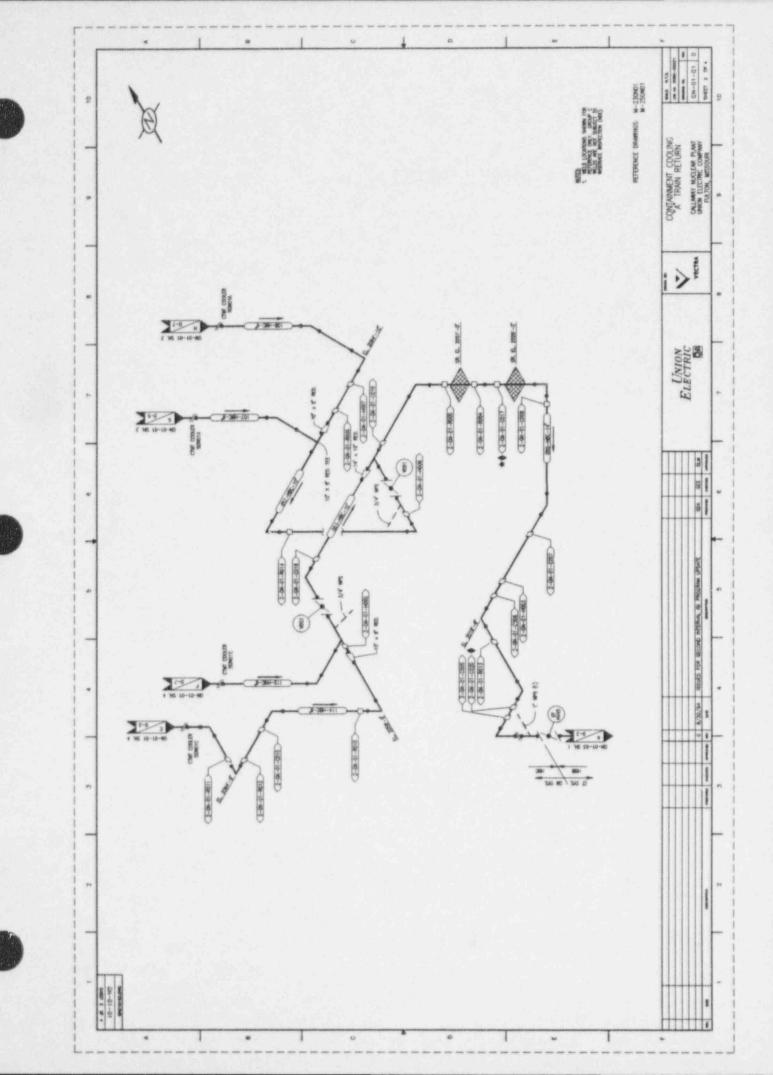


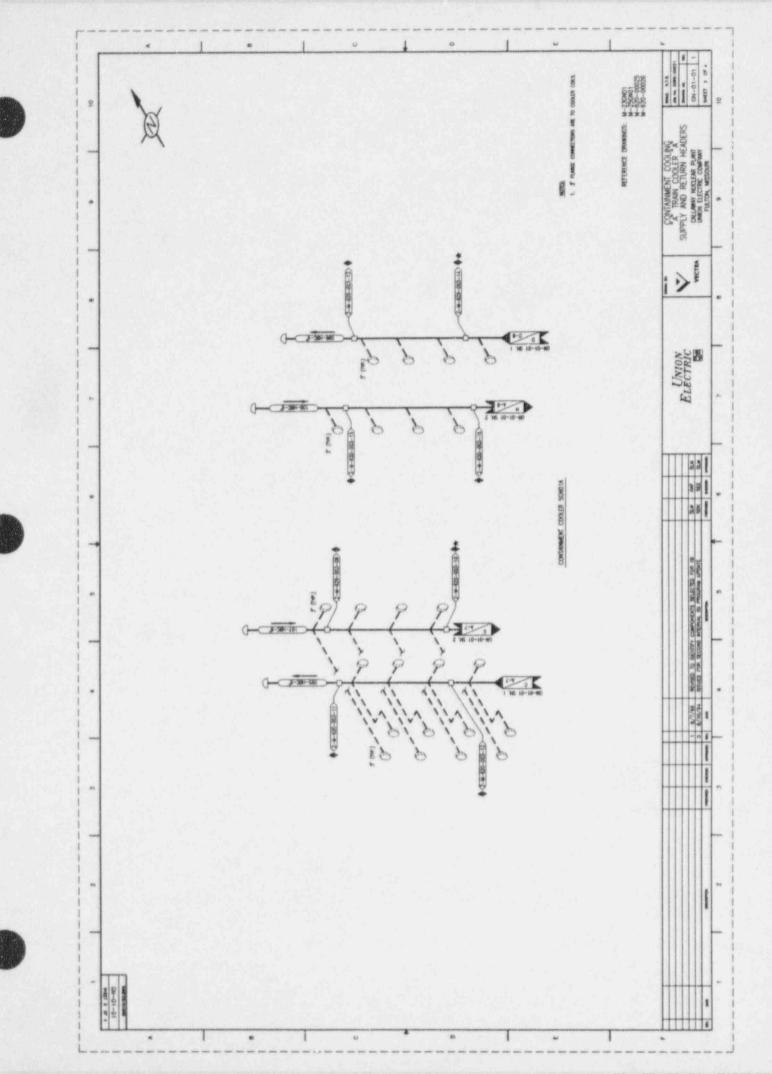


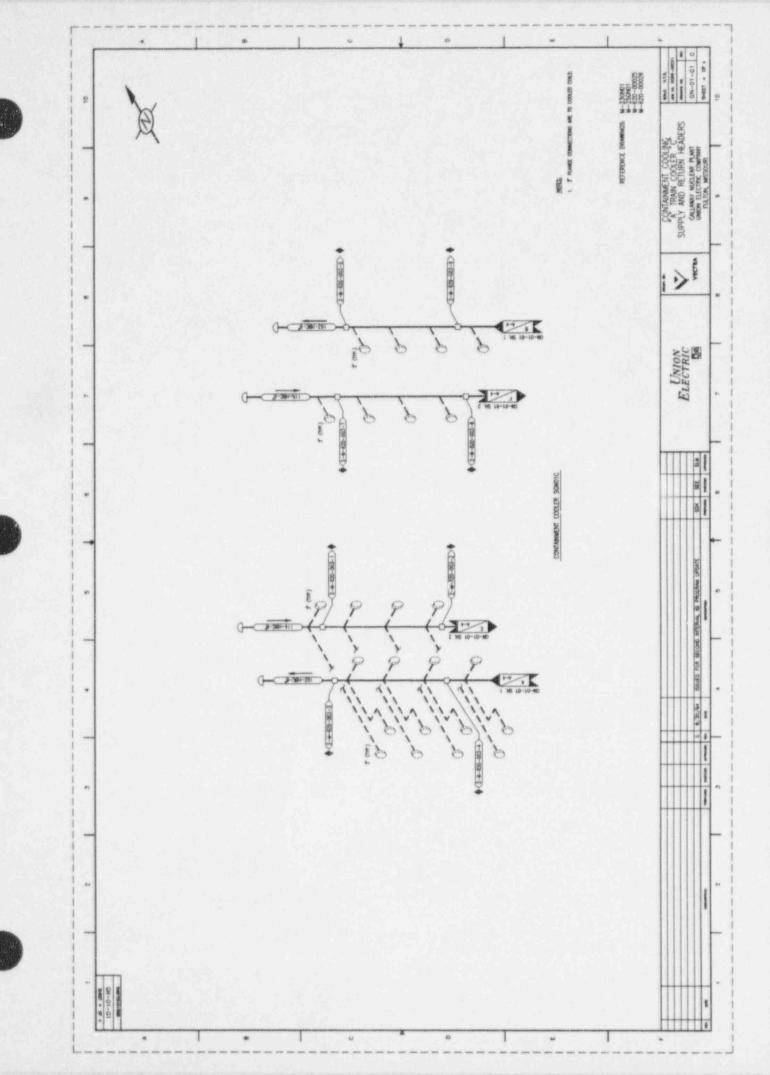


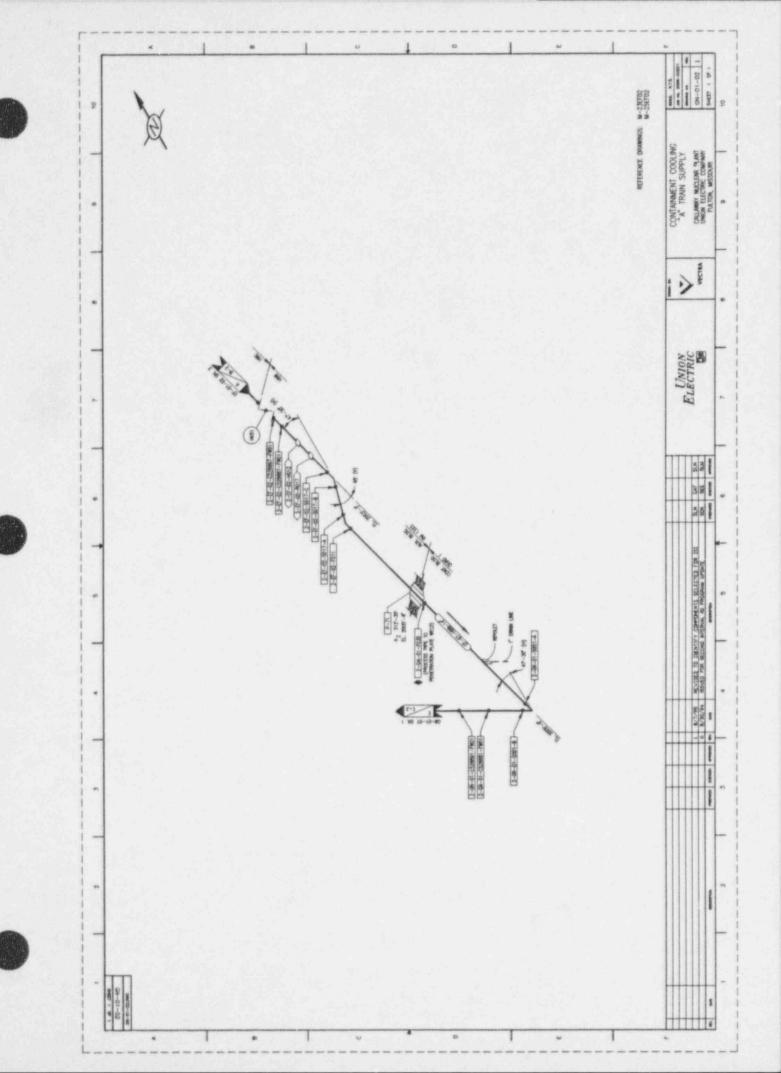


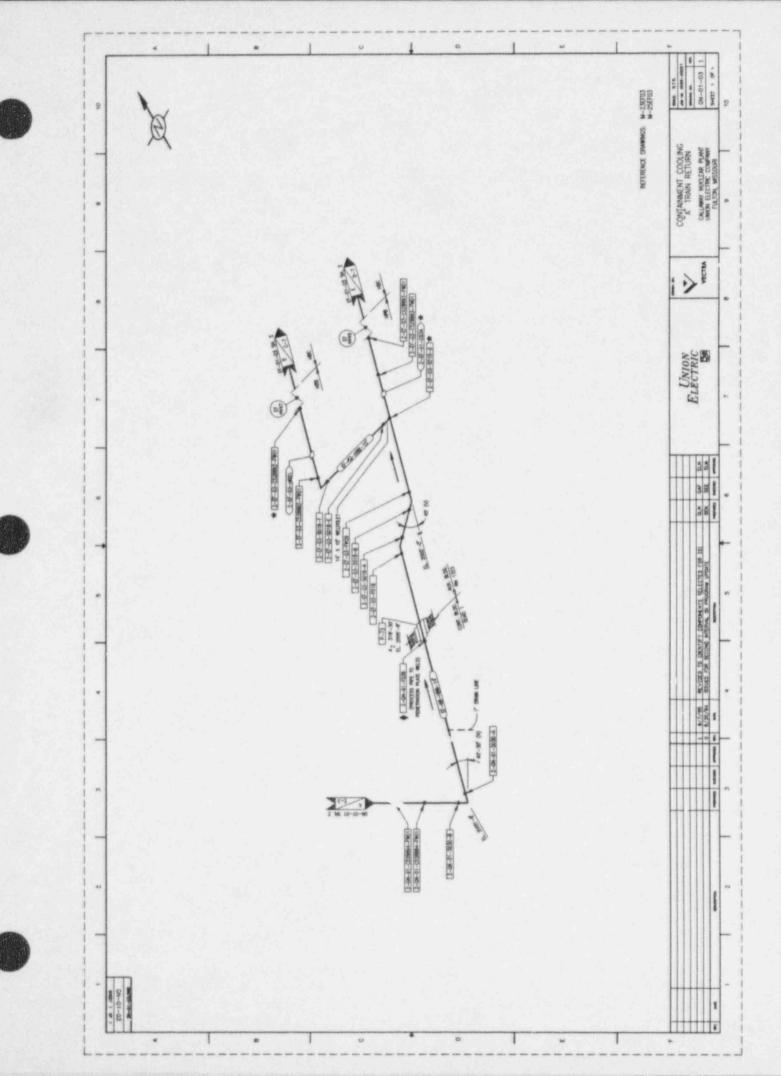


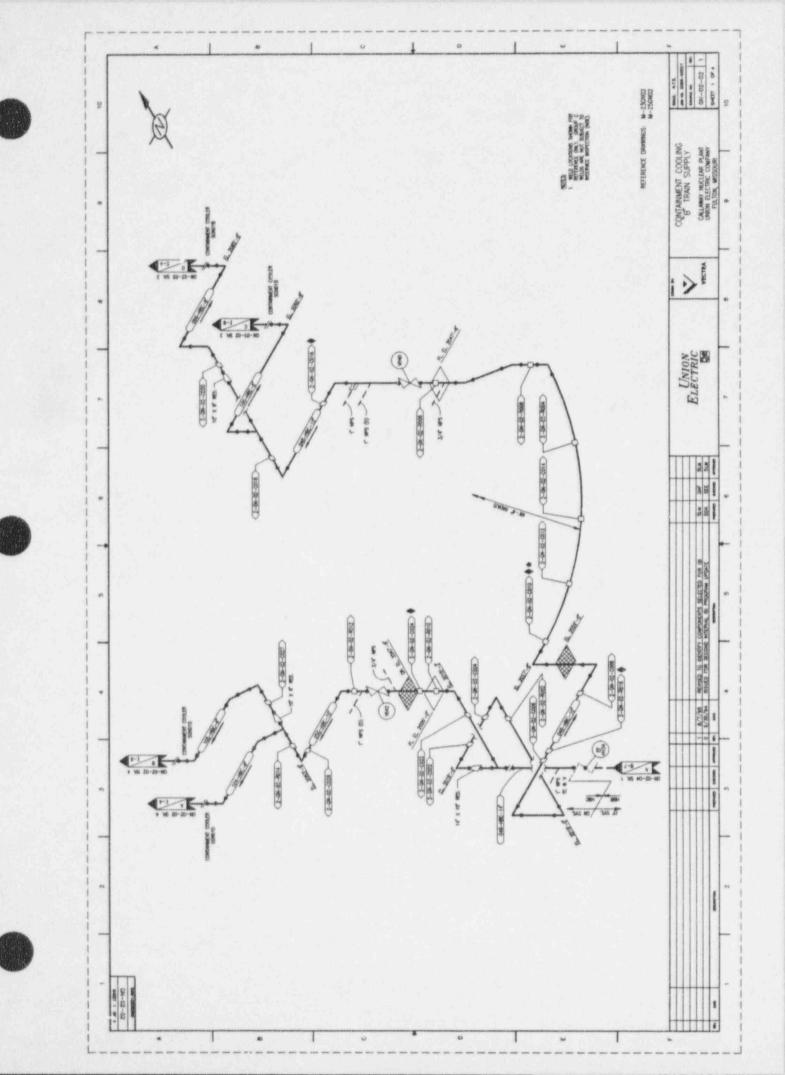


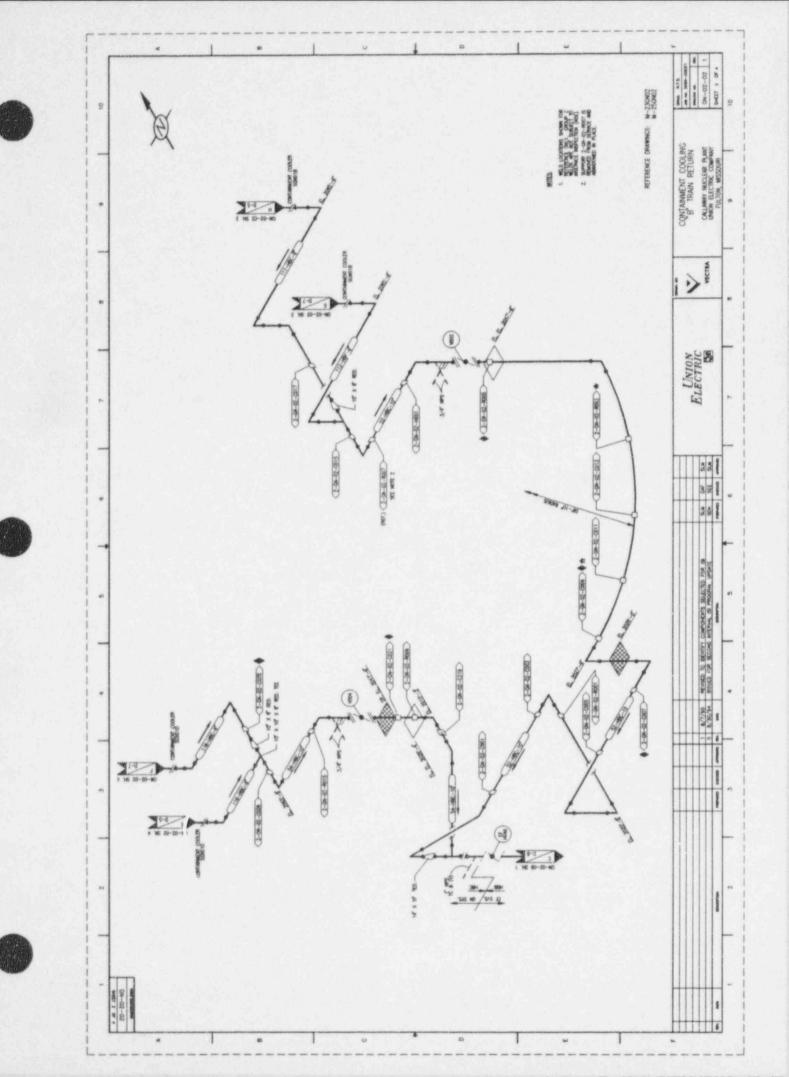


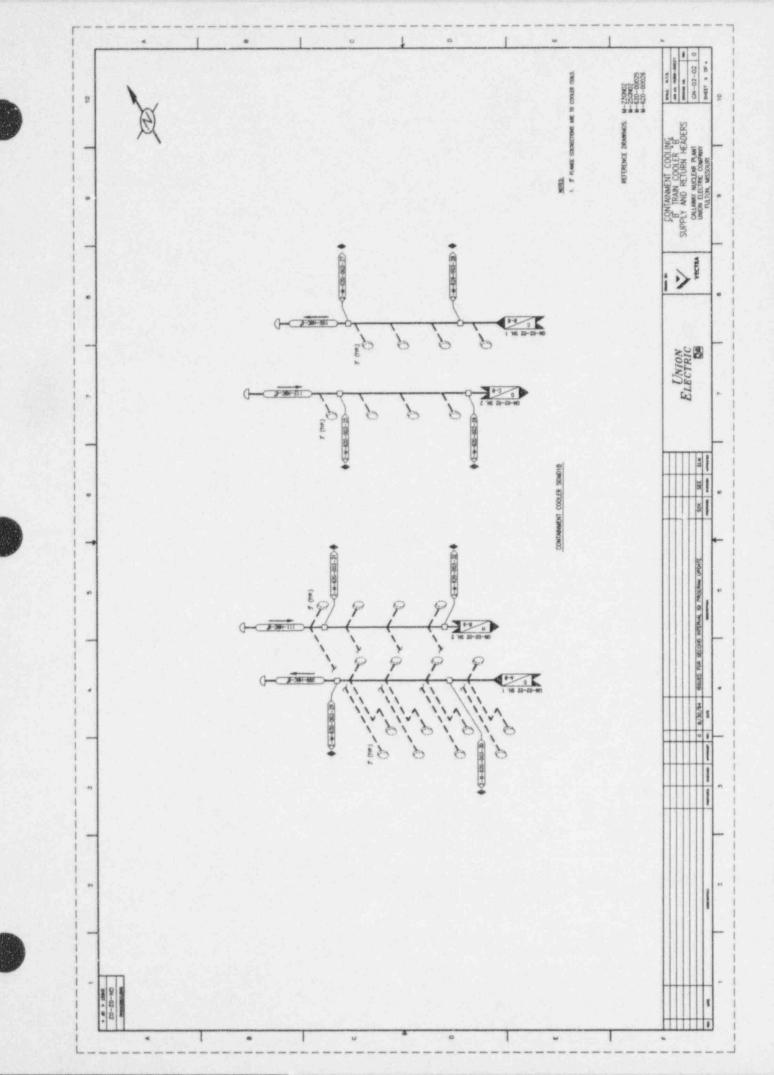


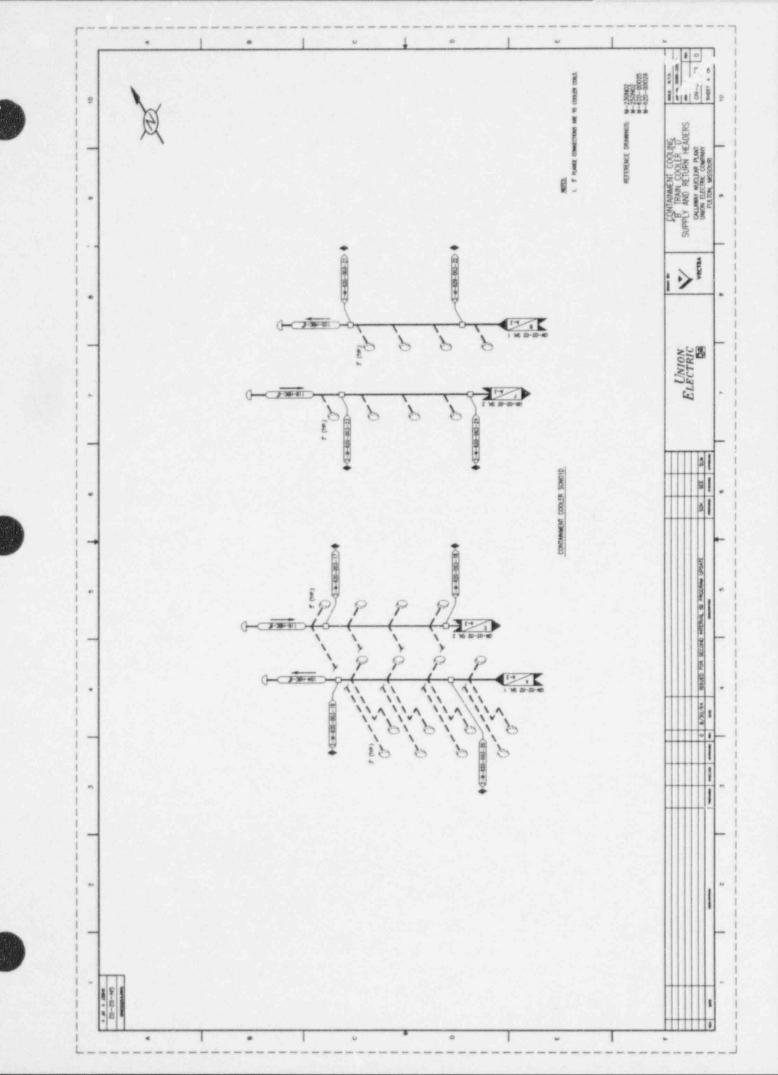


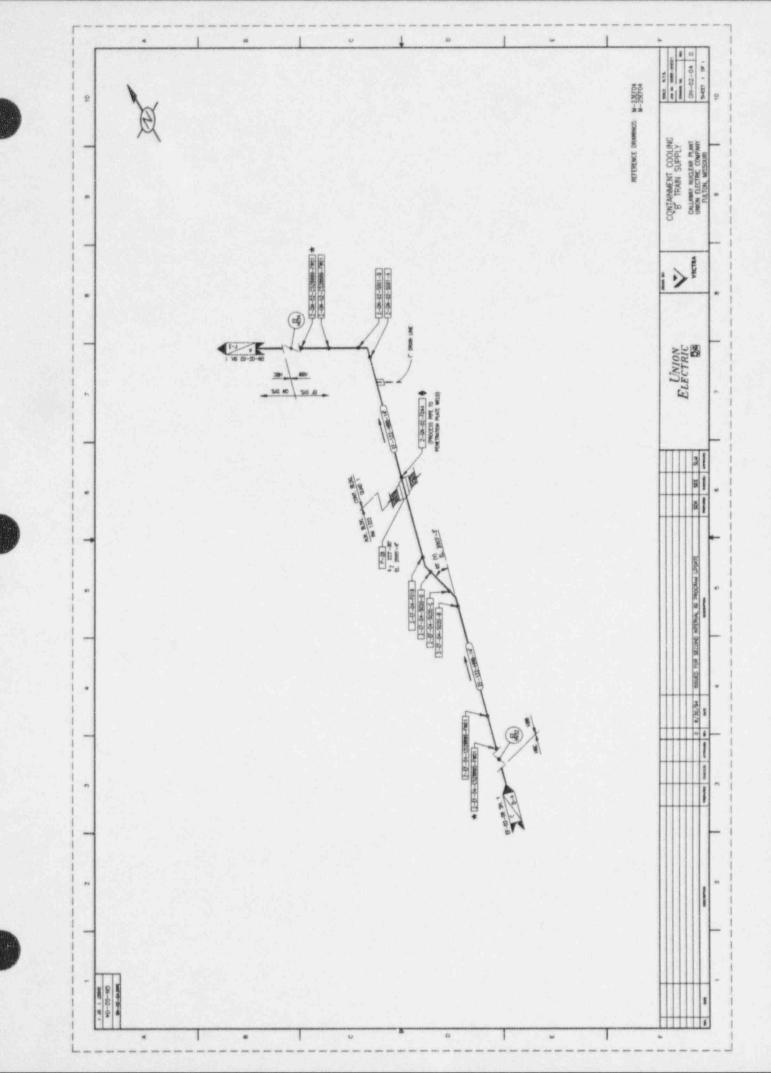


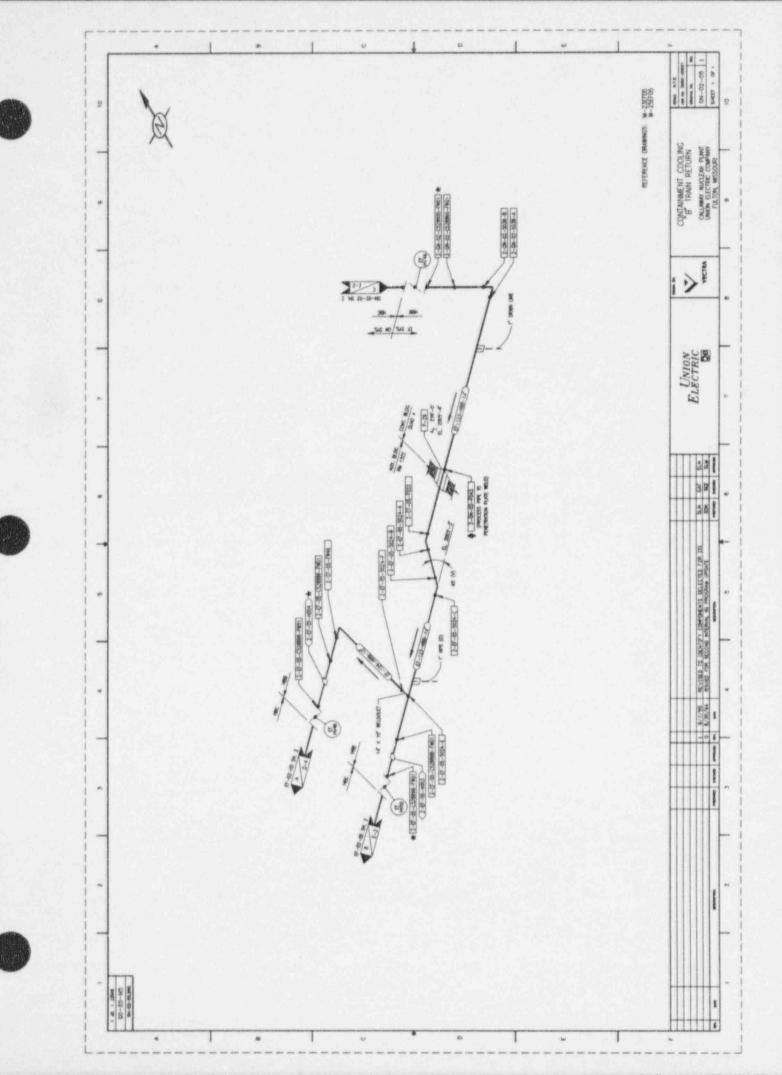


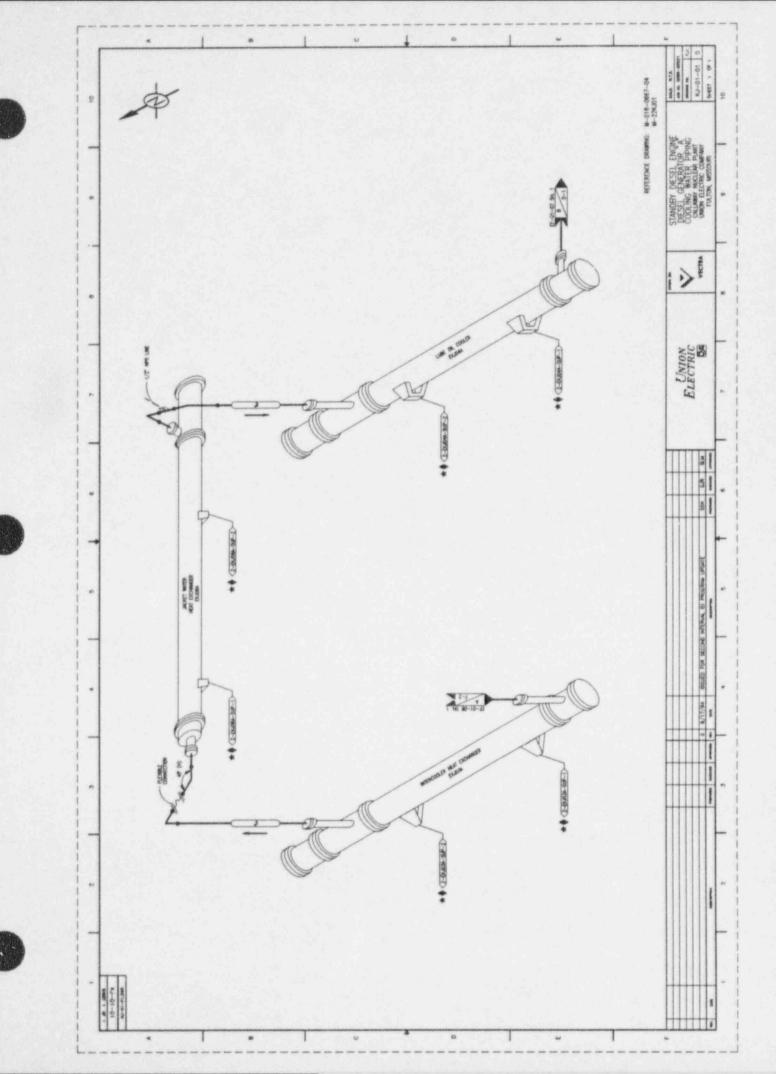


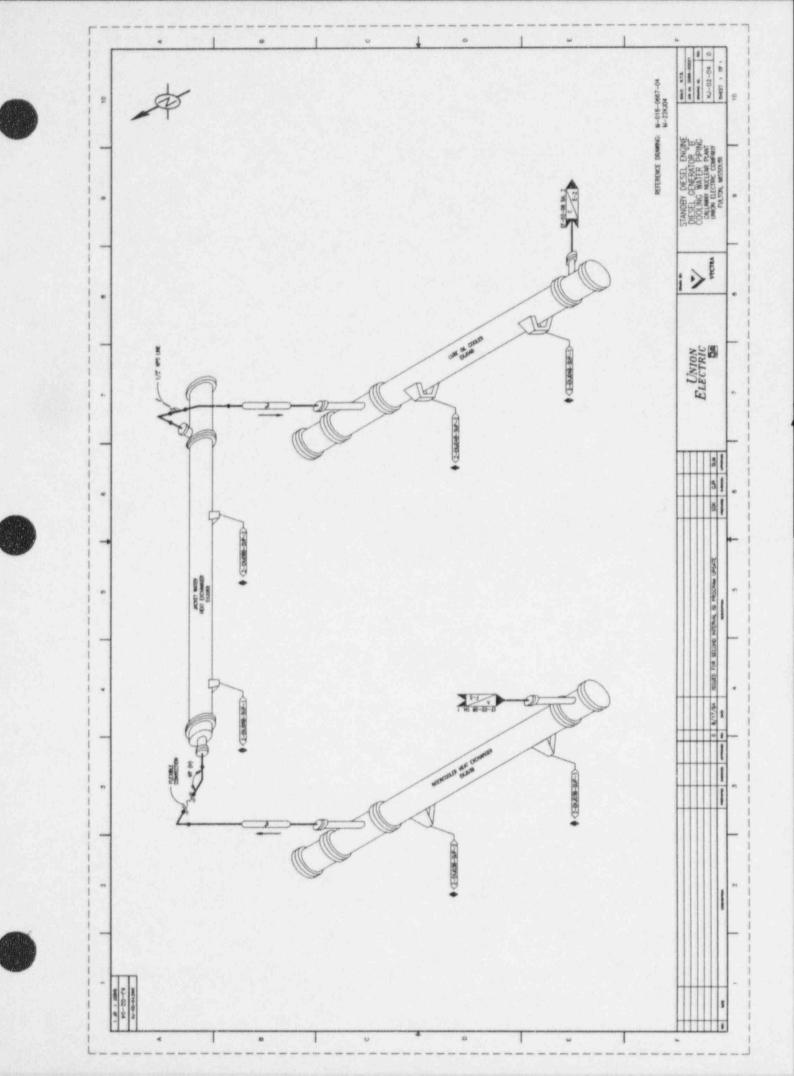








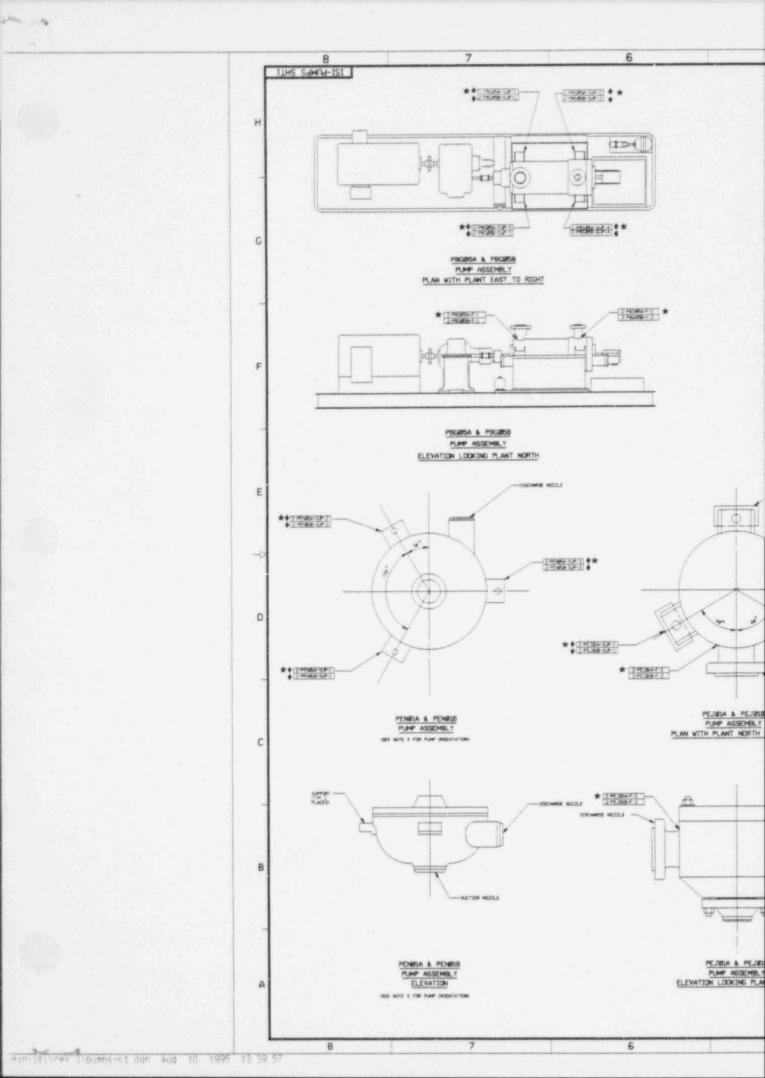


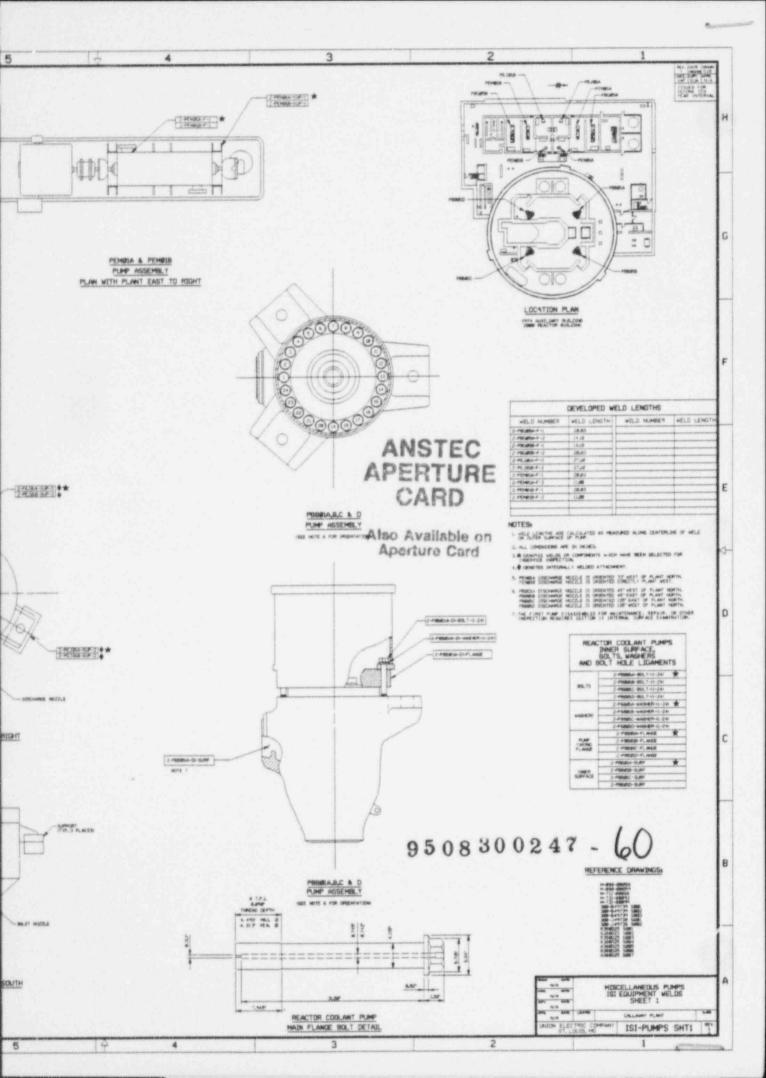


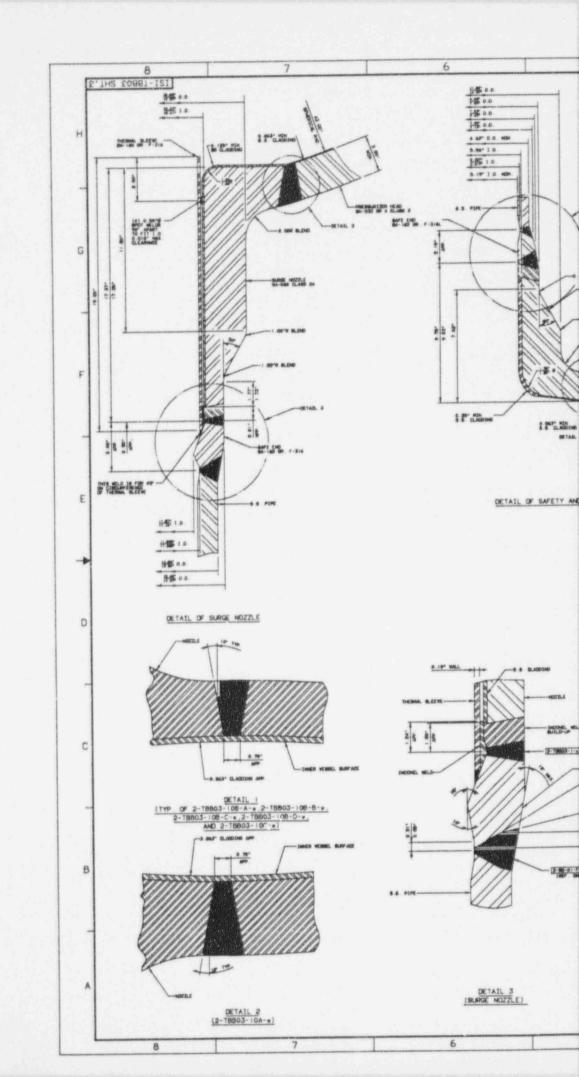
Additional Information Second Ten-Year Interval Inservice Inspection Plan Callaway Nuclear Power Plant, Unit 1 Docket No. 50-483

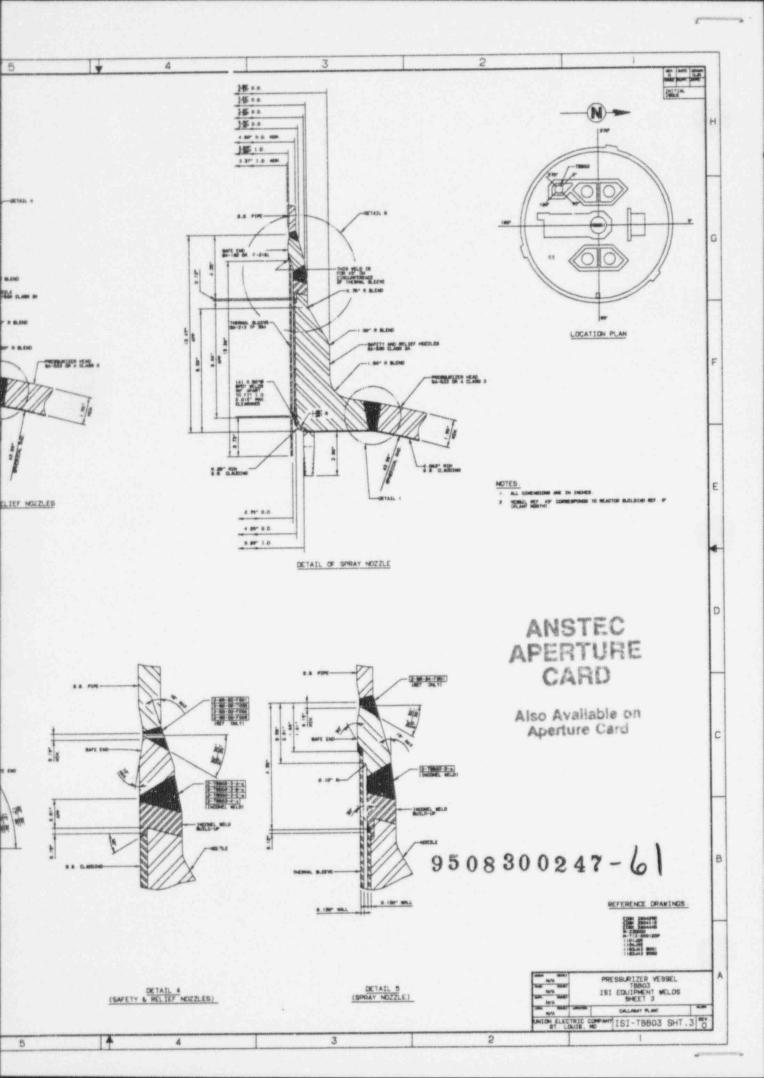
ATTACHMENT B-3

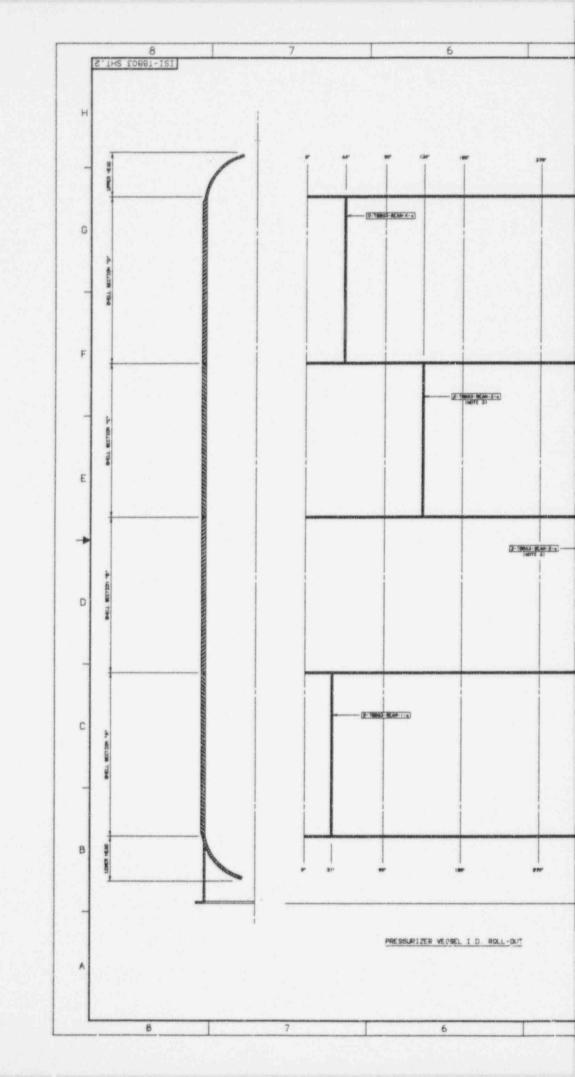
EQUIPMENT DETAIL DRAWINGS

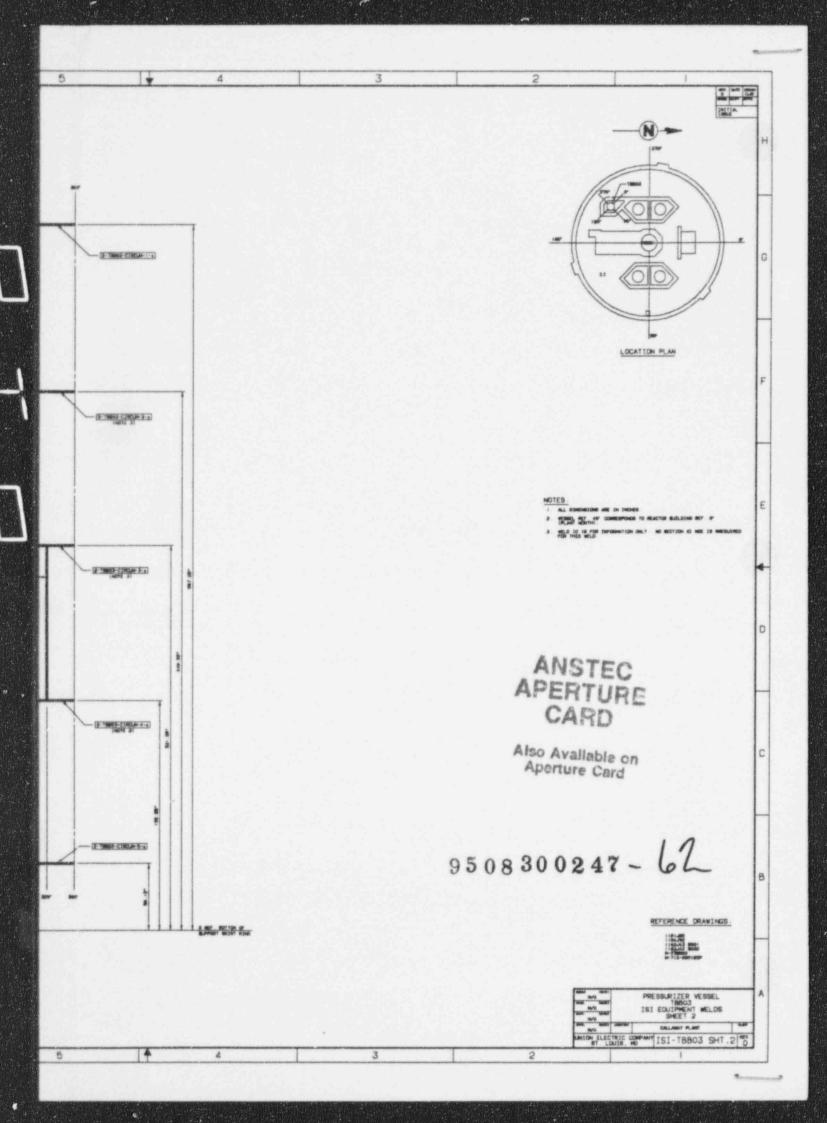




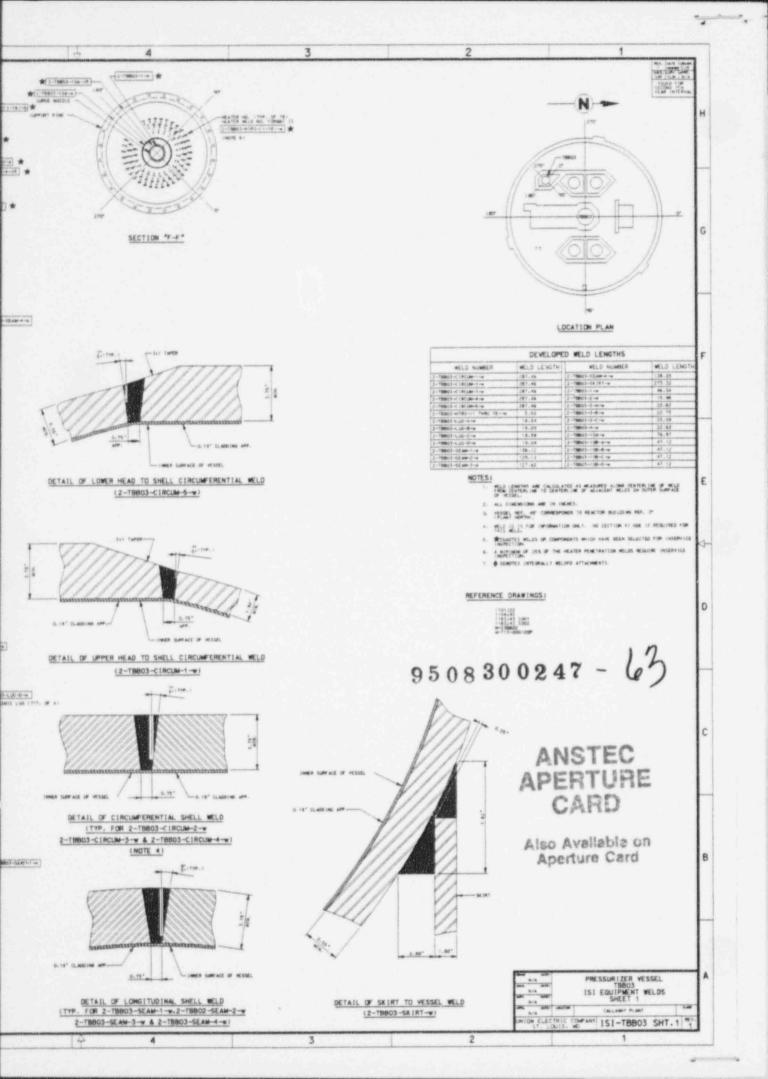




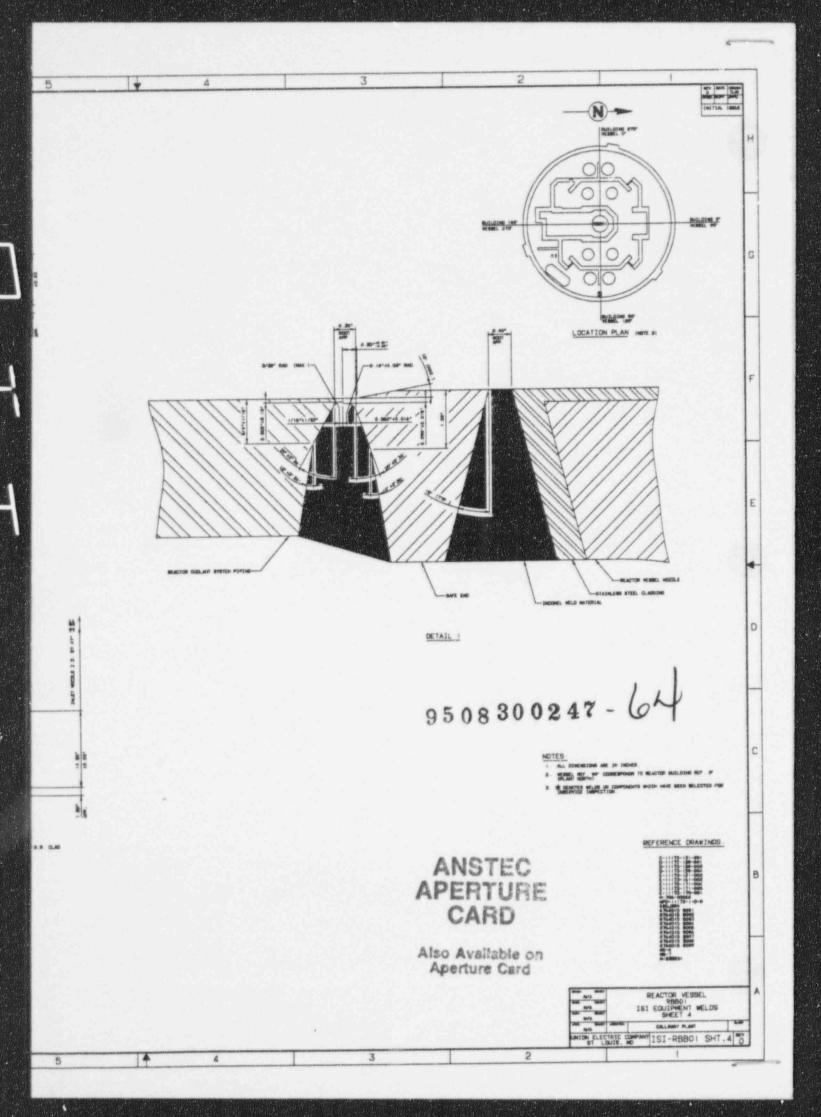


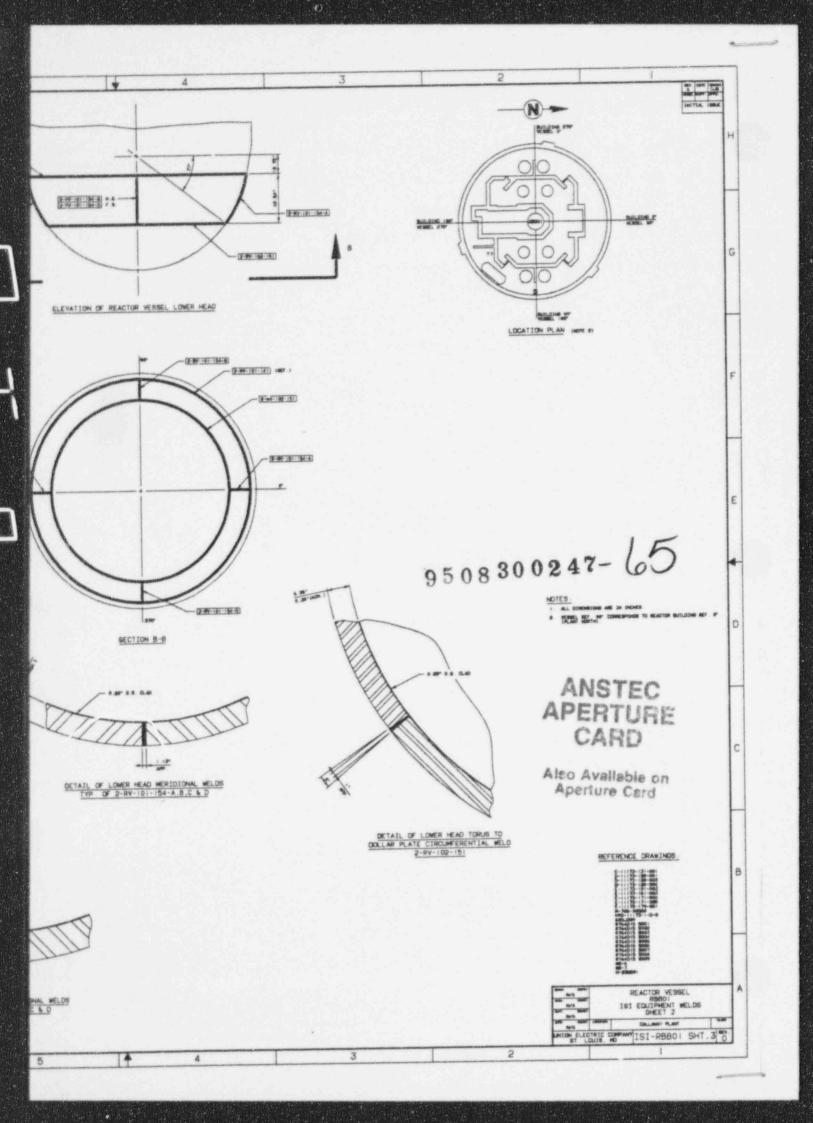


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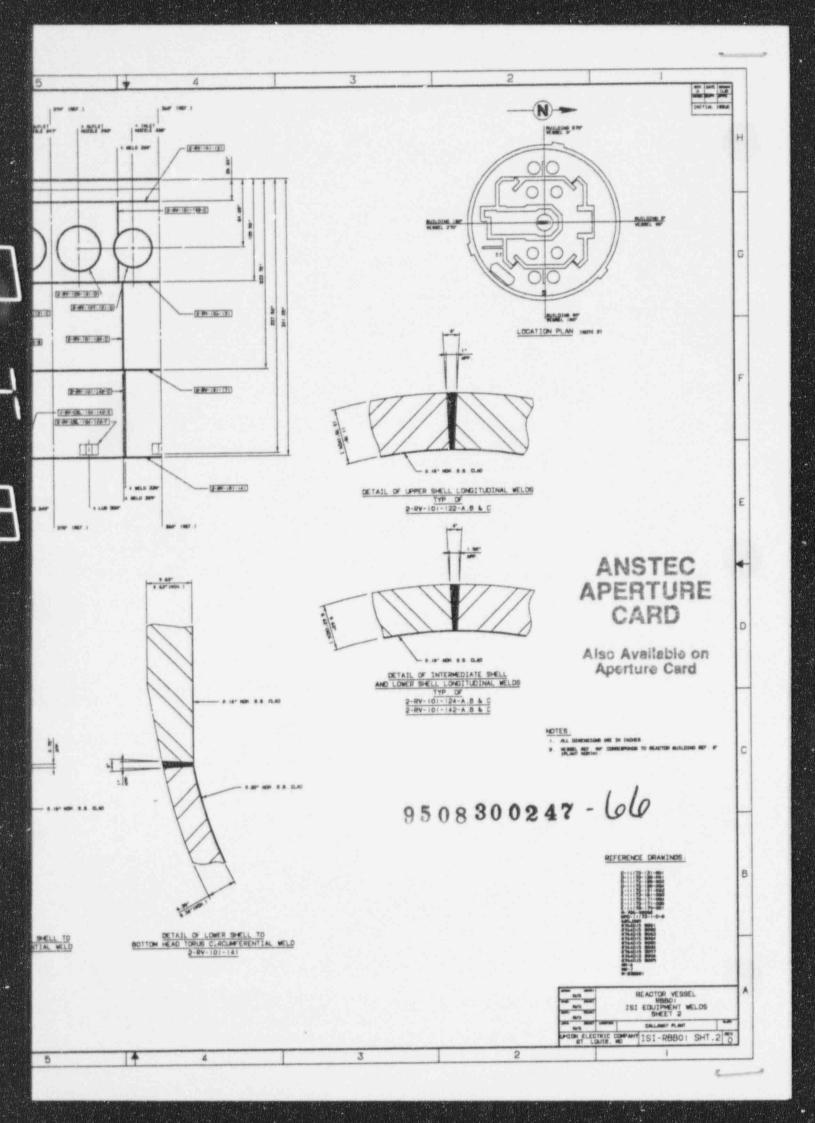
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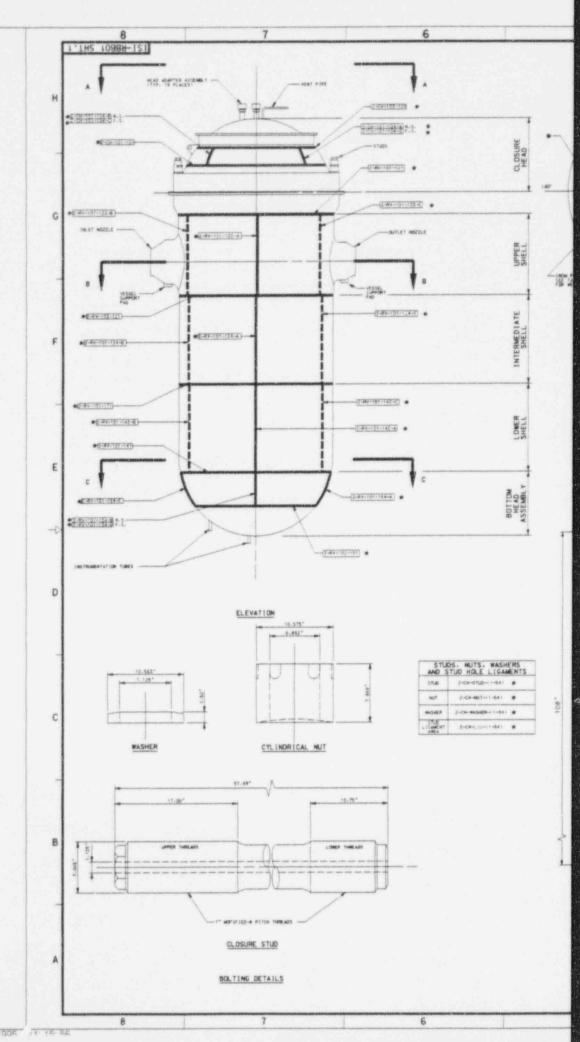




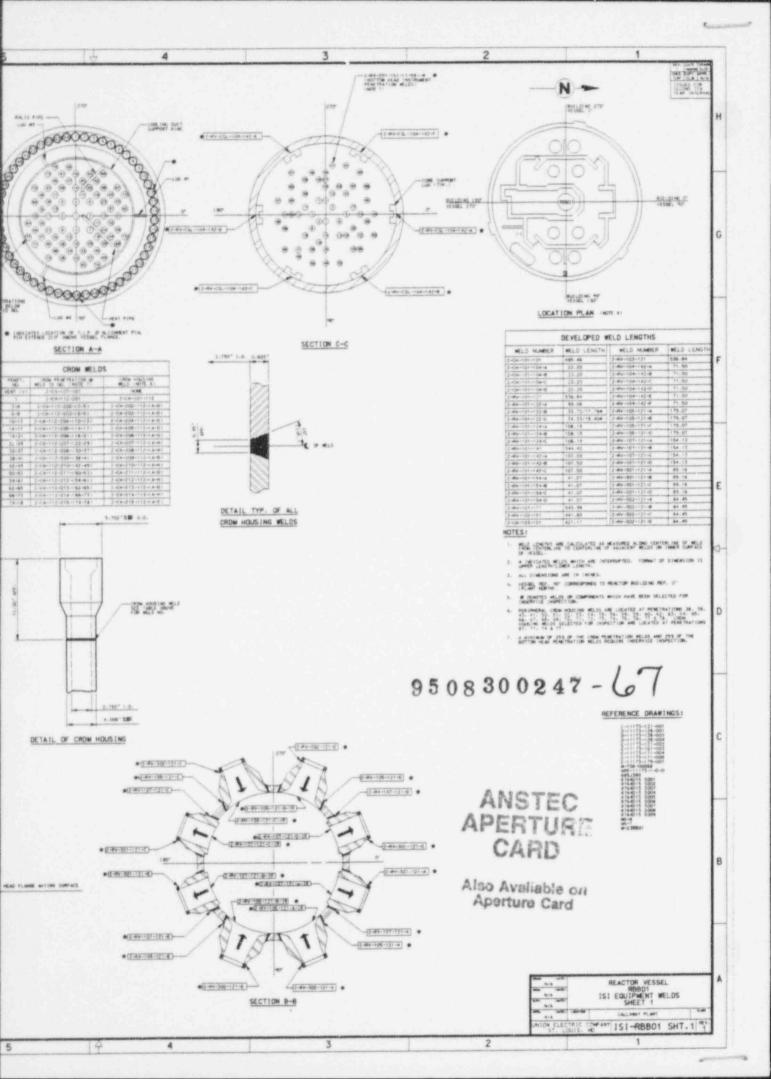
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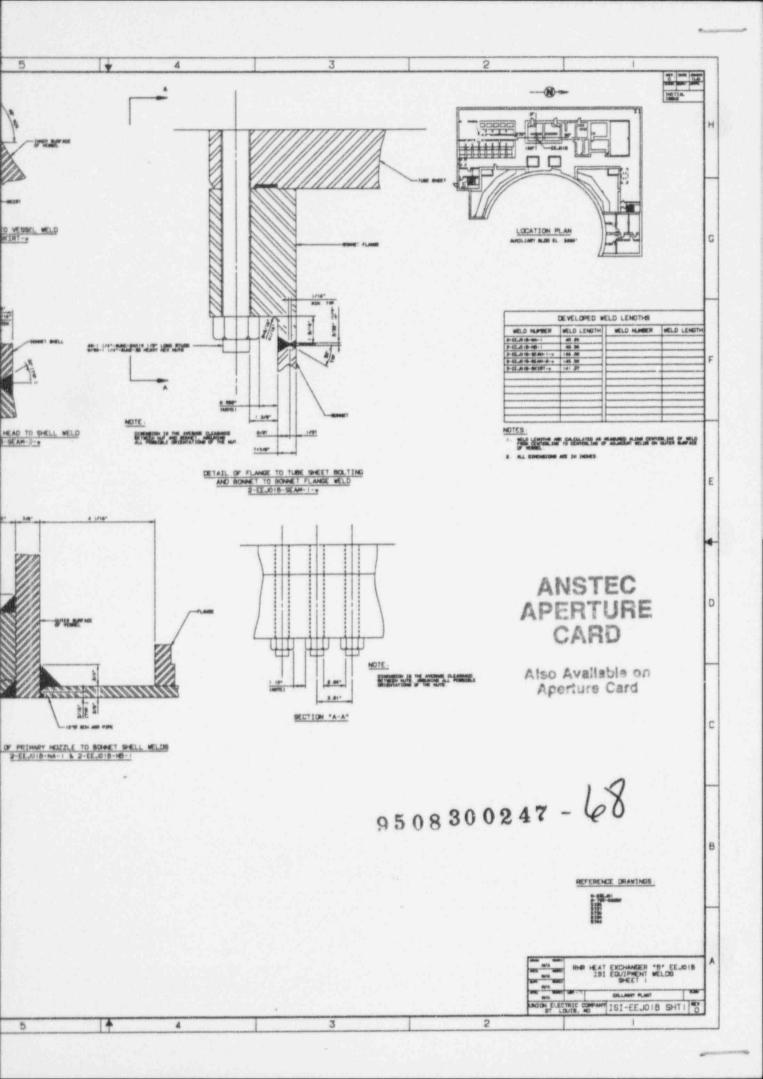




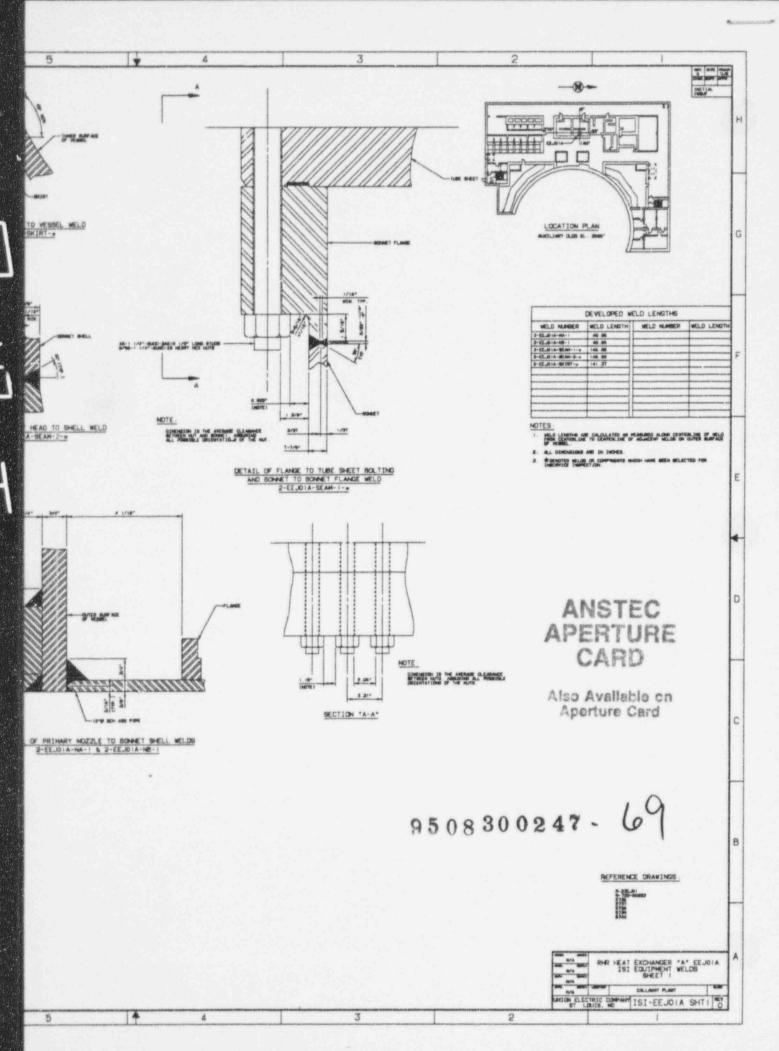
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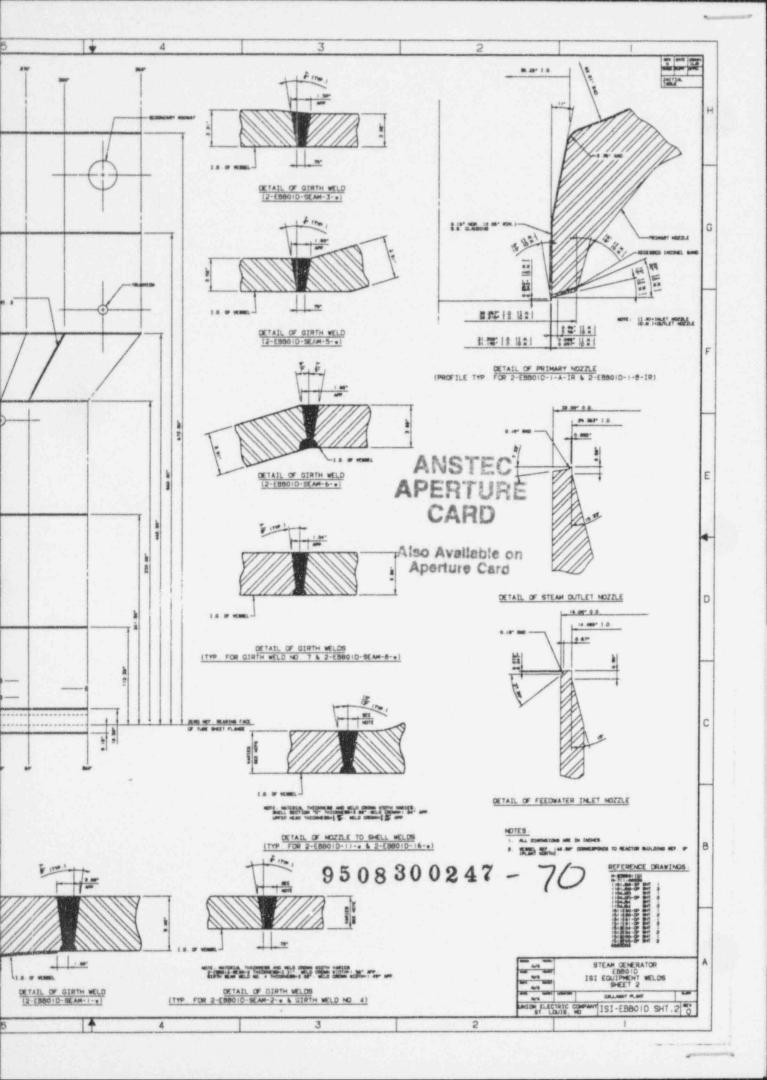


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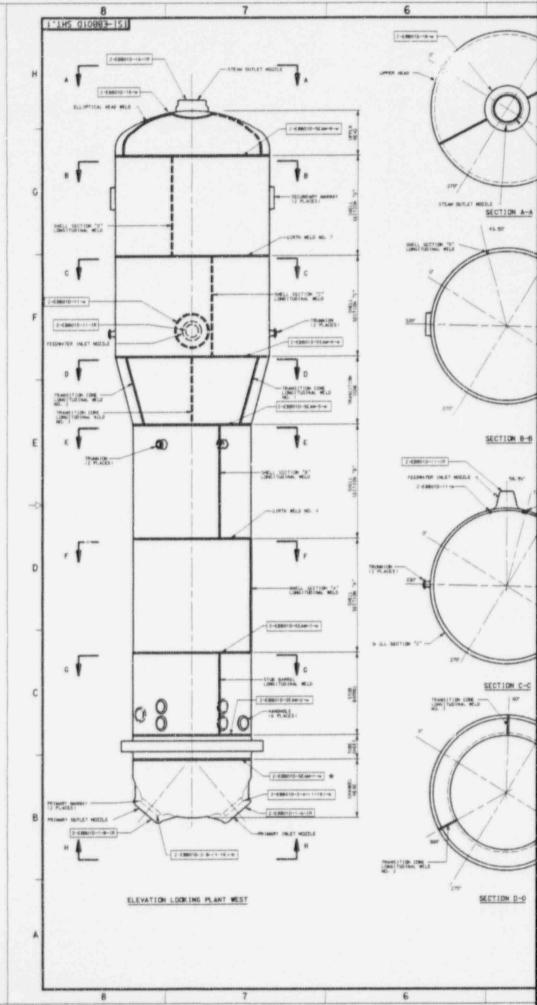


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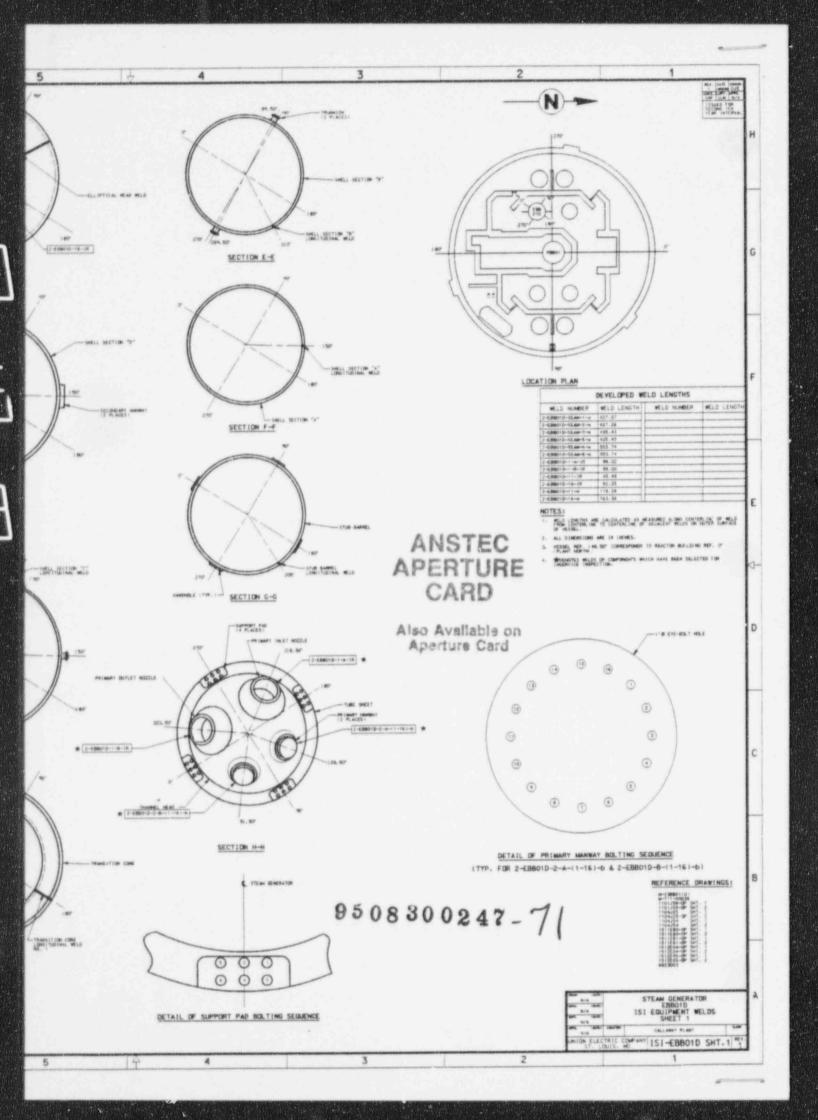
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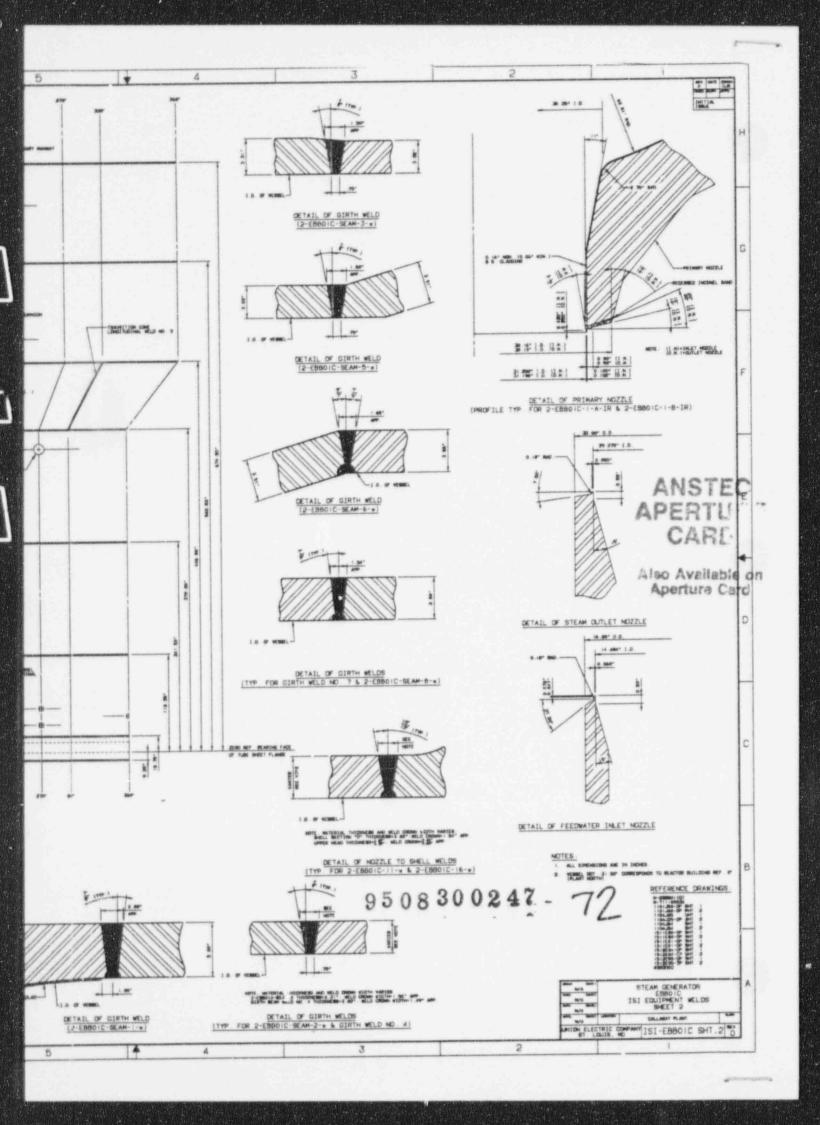
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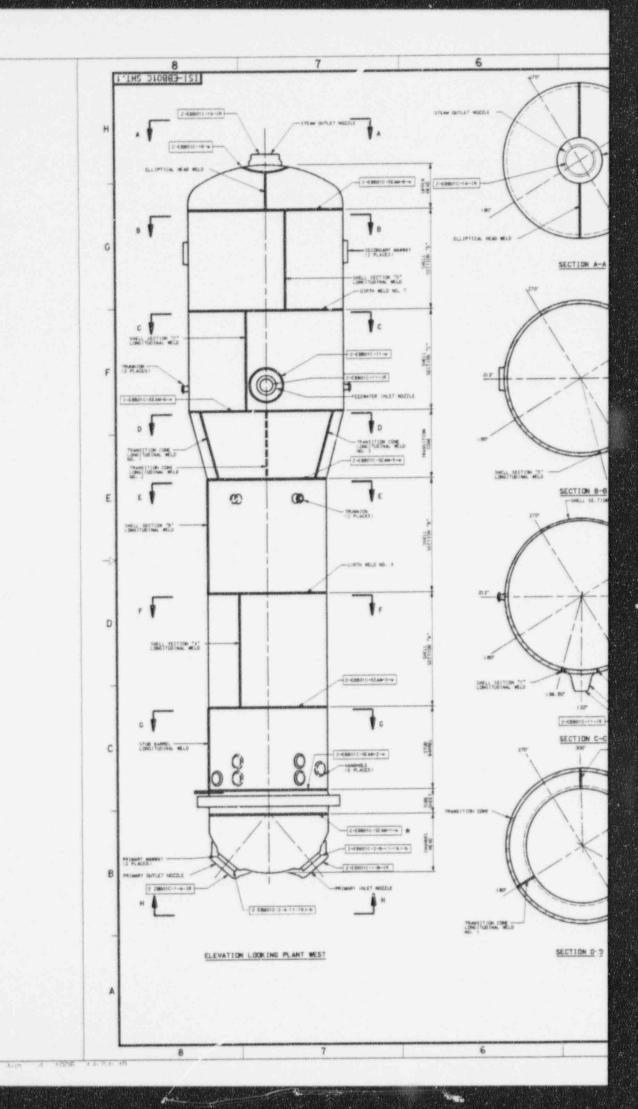
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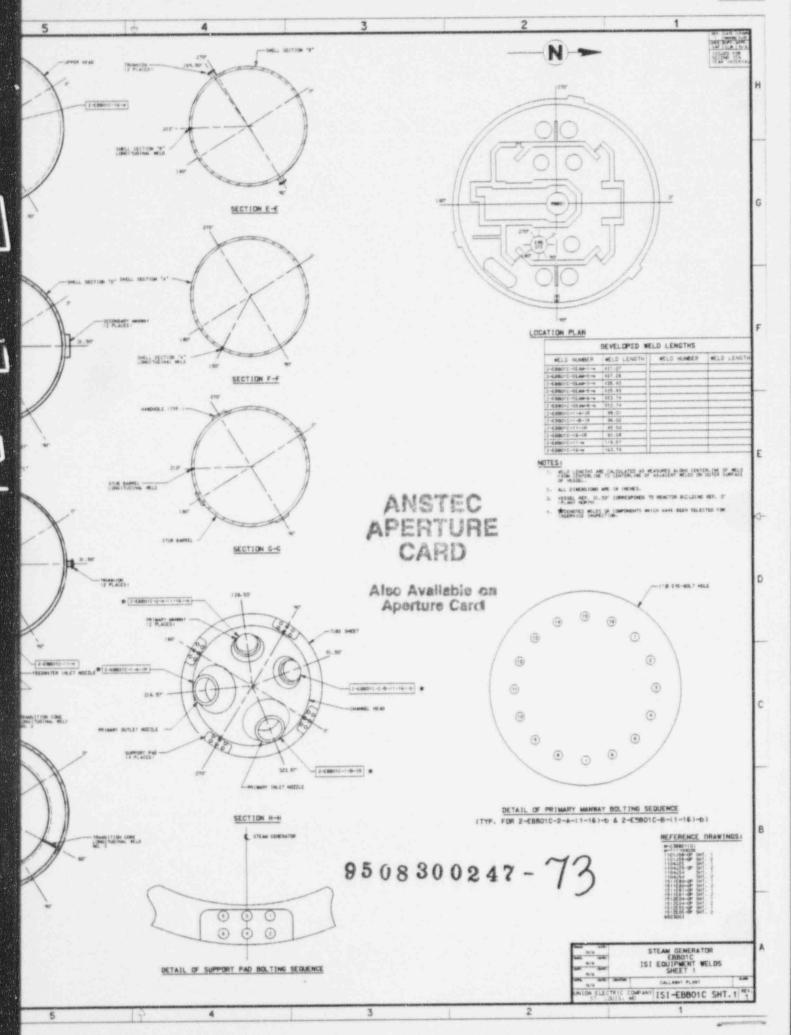
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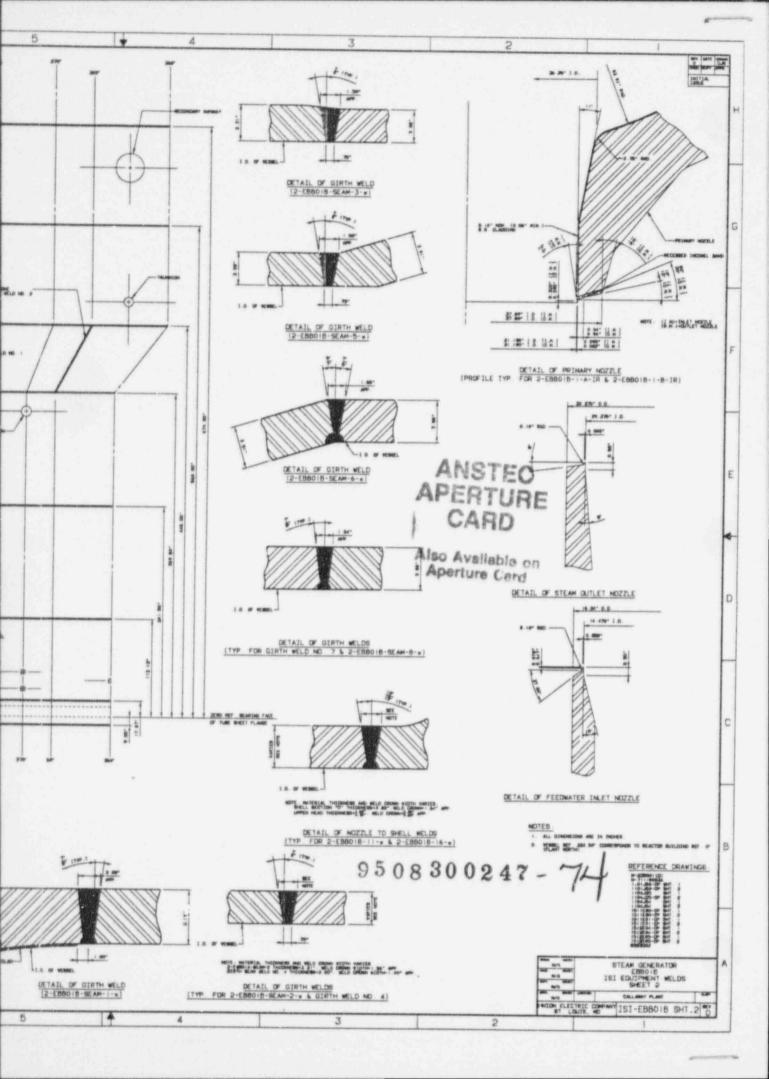
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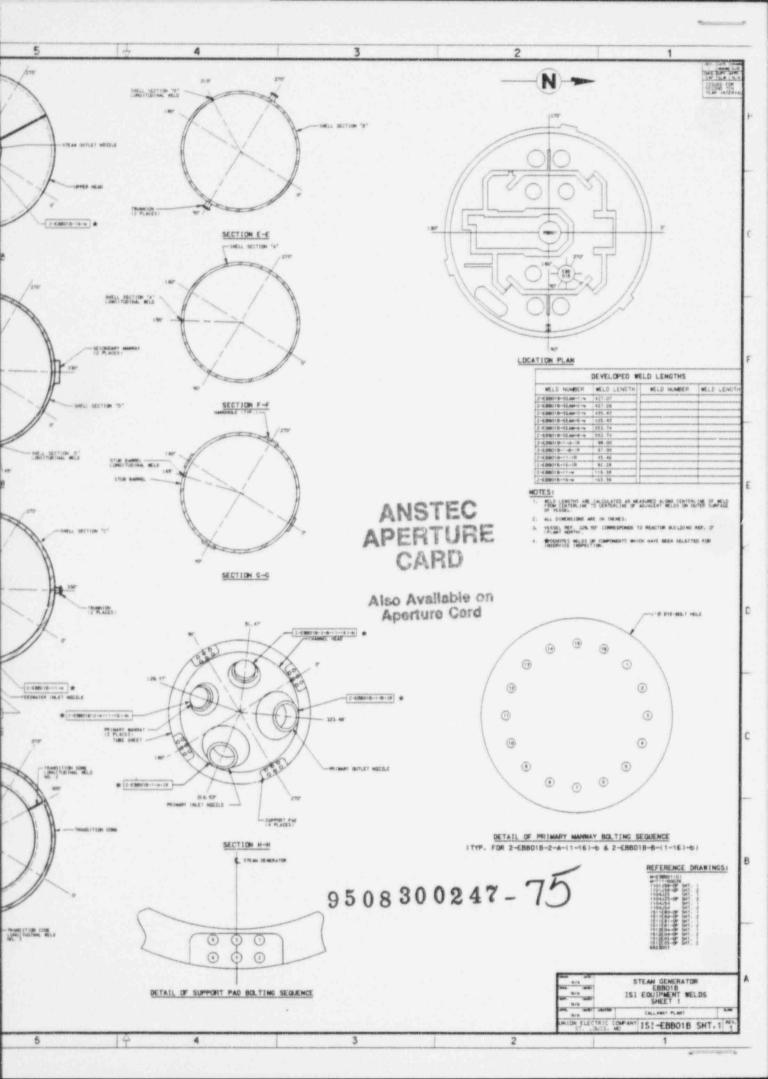


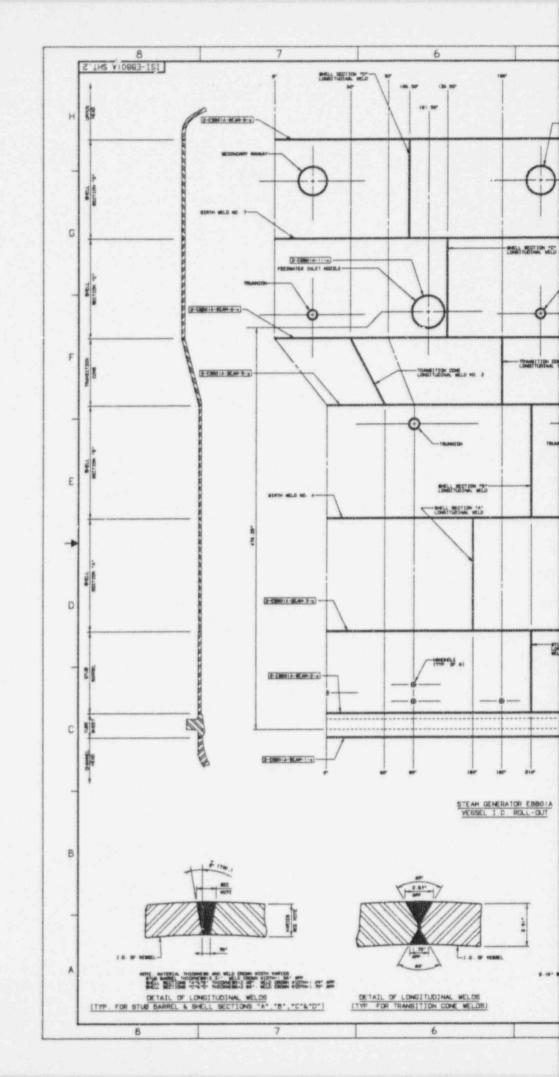




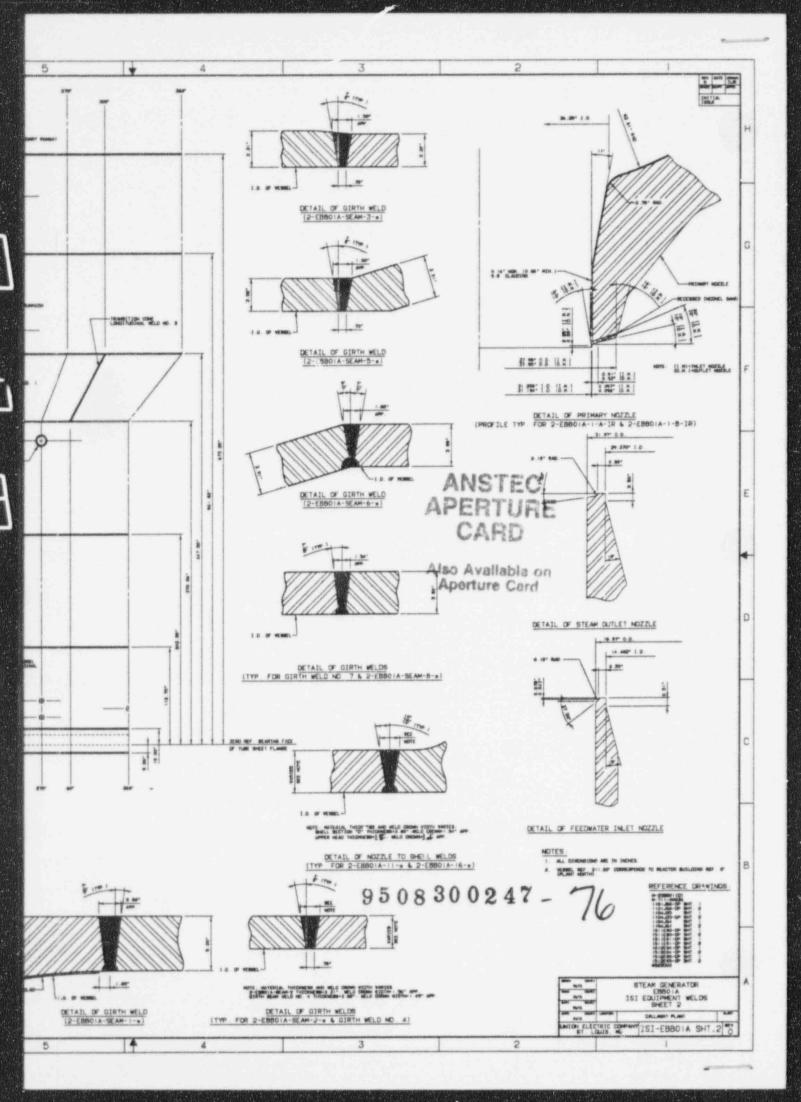
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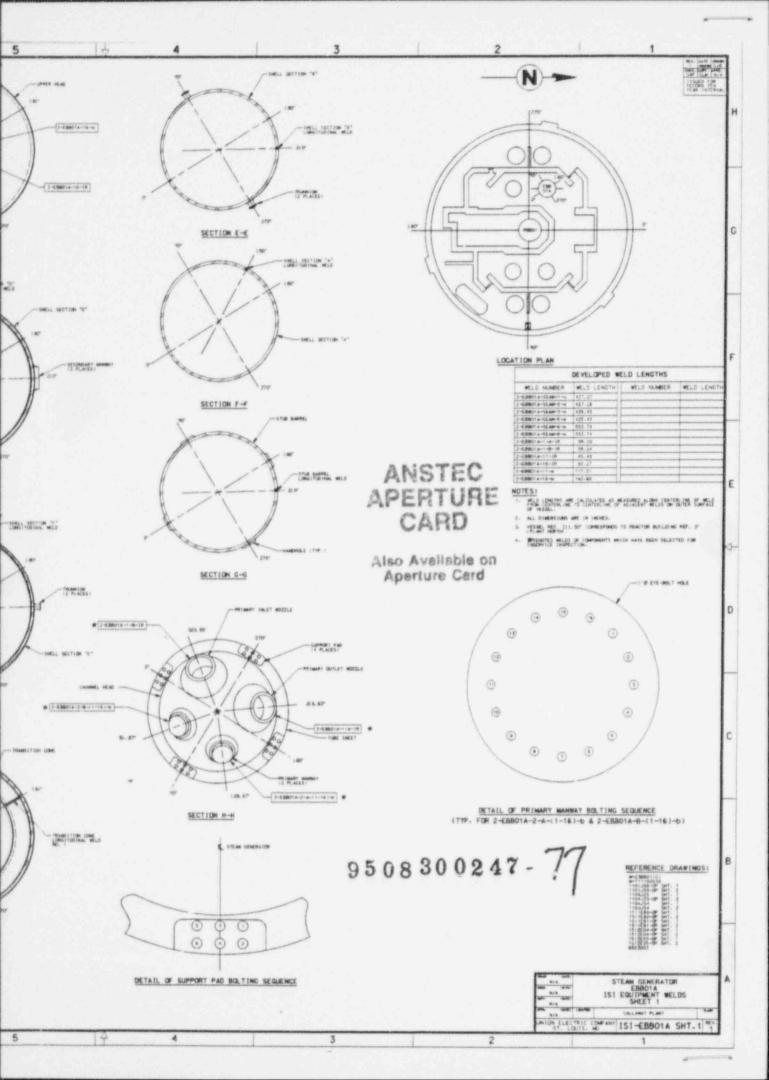


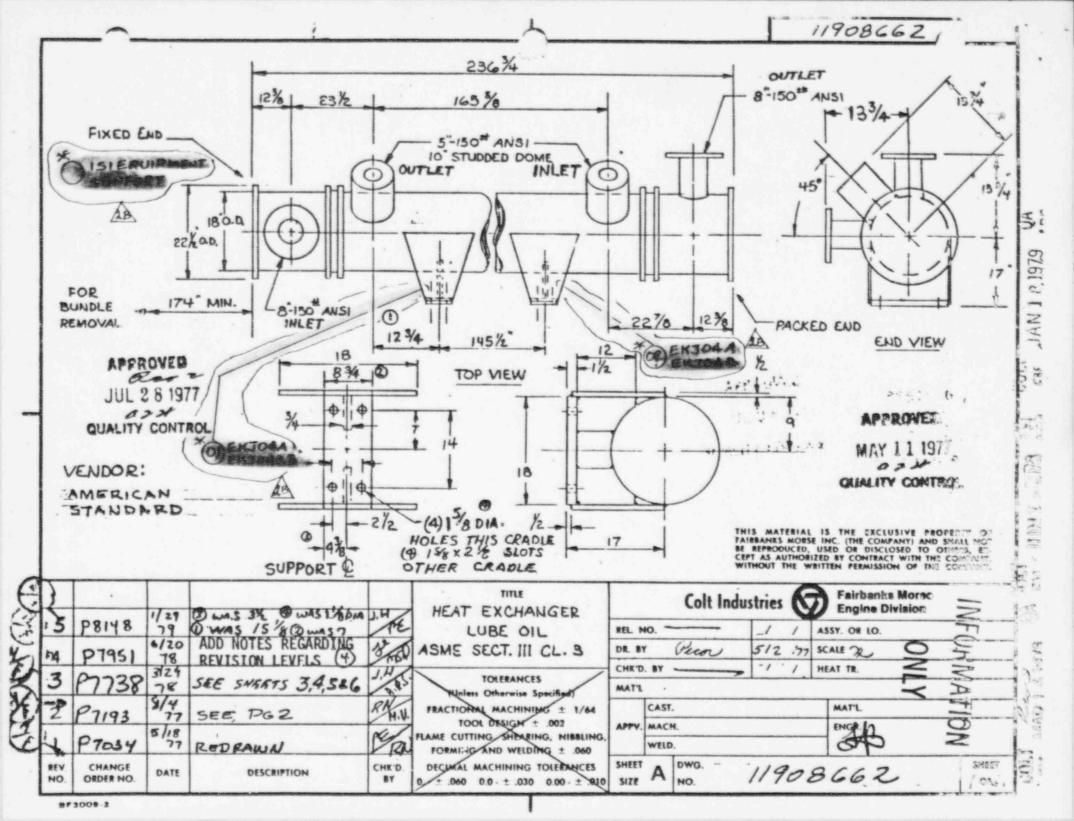


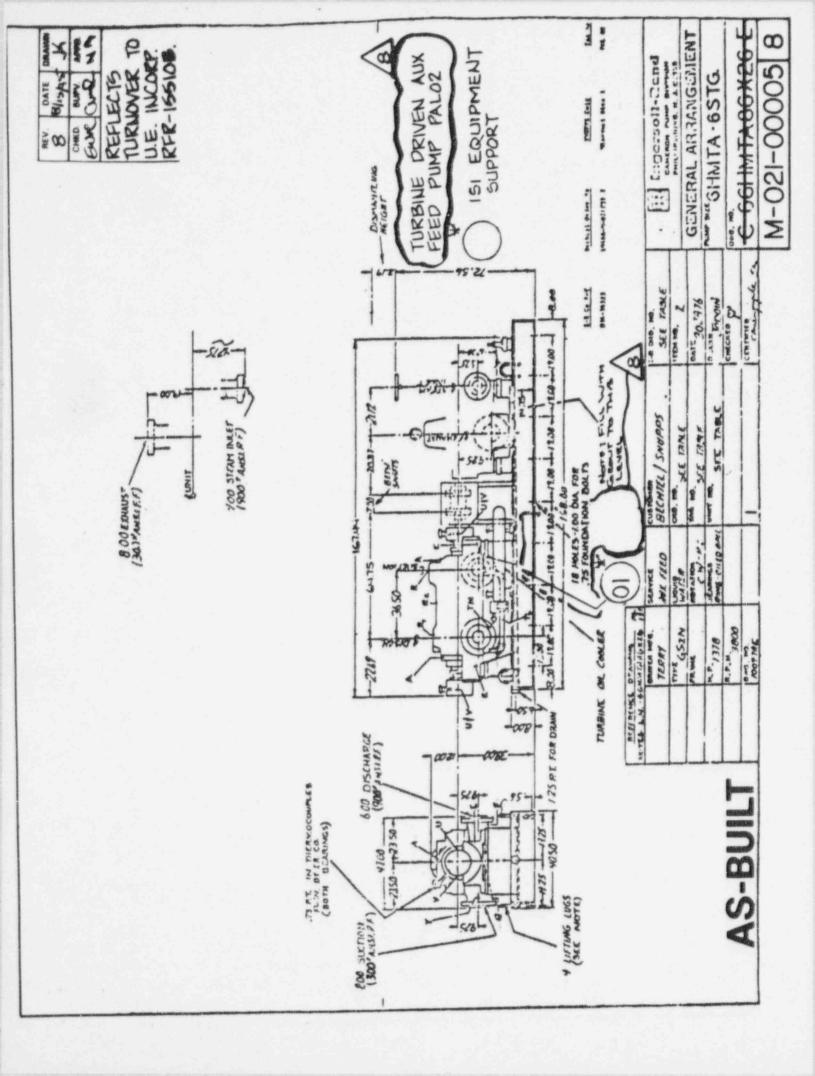


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ATTACHMENT B-4

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ATTACHMENT C

REACTOR VESSEL SUPPORT DETAIL DRAWINGS (REFERENCE RELIEF REQUEST ISI-03)

Additional Information Second Ten-Year Interval Inservice Inspection Plan Callaway Nuclear Power Plant, Unit 1 Docket No. 50-483

ATTACHMENT B-4

COMPONENT/EQUIPMENT SUPPORT DRAWINGS

RELIEF REQUEST NUMBER, ISI-03

(PAGE 3 OF 3)

