

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) SAN ONOFRE NUCLEAR GENERATING STATION, UNIT 2	DOCKET NUMBER (2) 0 5 0 0 0 3 6 1	PAGE (3) 1 OF 0 2
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TITLE (4)
WASTE GAS PROCESSING SYSTEM VALVE FAILURE

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)		
MONTH	DAY	YEAR	YEAR	SEQ. NUMBER	REV. NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)
0 5	0 2	8 4	8 4	0 2 1 7	0 1 0	0 6	0 1 1	8 4	SONGS UNIT 3		0 5 0 0 0 3 6 1 2
0 5	0 2	8 4	8 4	0 2 1 7	0 1 0	0 6	0 1 1	8 4			0 5 0 0 0 3 6 1 2

OPERATING MODE (9) **1**

POWER LEVEL (10) **1 1 0 0**

THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)

<input type="checkbox"/> 20.402(b)	<input type="checkbox"/> 20.405(c)	<input type="checkbox"/> 50.73(a)(2)(iv)	<input type="checkbox"/> 73.71(b)
<input type="checkbox"/> 20.405(a)(1)(i)	<input type="checkbox"/> 50.36(c)(1)	<input checked="" type="checkbox"/> 50.73(a)(2)(v)	<input type="checkbox"/> 73.71(c)
<input type="checkbox"/> 20.405(a)(1)(ii)	<input type="checkbox"/> 50.36(c)(2)	<input type="checkbox"/> 50.73(a)(2)(vii)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)
<input type="checkbox"/> 20.405(a)(1)(iii)	<input type="checkbox"/> 50.73(a)(2)(i)	<input checked="" type="checkbox"/> 50.73(a)(2)(viii)(A)	
<input type="checkbox"/> 20.405(a)(1)(iv)	<input type="checkbox"/> 50.73(a)(2)(ii)	<input type="checkbox"/> 50.73(a)(2)(viii)(B)	
<input type="checkbox"/> 20.405(a)(1)(v)	<input type="checkbox"/> 50.73(a)(2)(iii)	<input type="checkbox"/> 50.73(a)(2)(x)	

LICENSEE CONTACT FOR THIS LER (12)

NAME	TELEPHONE NUMBER
J. G. HAYNES, STATION MANAGER	7 1 1 4 4 9 2 1 7 7 0 0

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC
B	W/E	V	K/O	8/5	N				

SUPPLEMENTAL REPORT EXPECTED (14)

YES (If yes, complete EXPECTED SUBMISSION DATE) NO

EXPECTED SUBMISSION DATE (15)

MONTH	DAY	YEAR

Abstract (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On 5/2/84, at 0114, with Units 2 and 3 in Mode 1 at 100% power, backleakage of flow from the Waste Gas Compressor (WGC) C010 through discharge check valve SA1902MU003 into standby WGC C011 resulted in a release via the WGC C011 rupture disc. The waste gas released through the rupture disc was routed by the Radwaste HVAC System to the Plant Vent Stack causing Plant Vent Monitor 2RE-7865 to alarm. At 0129 an Unusual Event was declared per Tab A1-1 of the Emergency Plan because the alarm on 2RE-7865 continued for more than 15 minutes and the release had not been terminated. The release was isolated at 0136 by manually valving out WGC C011 which terminated the Unusual Event. The release was calculated to have been approximately 107 curies of Xe-133. The concentration in unrestricted areas, when averaged over one hour, was 7.14E-7 microcuries per cubic centimeter (2.38 times the applicable concentration in Appendix B, Table II of 10 CFR 20 in unrestricted areas, when averaged over one hour).

Corrective action taken was replacement of the rupture disc. Additional corrective action will be to repair check valve SA1902MU003 during the next cold shutdown of both Units 2 and 3. Also, the need for a design modification is being evaluated.

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PDR ADOCK 05000361
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LICENSEE EVENT REPORT (LER)
TEXT CONTINUATION

FACILITY NAME (1) SAN ONOFRE NUCLEAR GENERATING STATION, UNIT 2	DOCKET NUMBER (2) 0 5 0 0 0 3 6 1	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQ. NUMBER	REV. NUMBER			
		8 4	- 0 2 7	- 0 0	0 2	OF	0 2

TEXT (If more space is required, use additional NRC Form 366A's) (17)

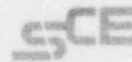
On 5/2/84, at 0114, with Units 2 and 3 in Mode 1 at 100% power, backleakage of flow from the Waste Gas Compressor (WGC) C010 (EIIS System Code WE) through discharge check valve SA1902MU003 (EIIS Component Code V) into standby WGC C011 resulted in a release via the WGC C011 rupture disc. The waste gas released through the rupture disc was routed by the Radwaste HVAC System (EIIS System Code VH) to the Plant Vent Stack causing Plant Vent Monitor 2RE-7865 (EIIS System Code IL) to alarm. At 0129 an Unusual Event was declared per Tab A1-1 of the Emergency Plan because the alarm on 2RE-7865 continued for more than 15 minutes and the release had not been terminated. The release was isolated at 0136 by manually valving out WGC C011 which terminated the Unusual Event.

The release was calculated to have been approximately 107 curies of Xe-133. The concentration in unrestricted areas, when averaged over one hour, was 7.14E-7 microcuries per cubic centimeter (2.38 times the applicable concentration in Appendix B, Table II of 10 CFR 20 in unrestricted areas, when averaged over one hour). In accordance with the action statement of Technical Specification Limiting Condition for Operation (LCO) 3.11.2.1, actions were immediately initiated to terminate the release.

This event is similar to that reported in LER 82-113 (Docket No. 50-361) where the corrective action consisted of replacing the rupture disc. For this occurrence, in addition to replacing the rupture disc, additional corrective action will be to repair check valve SA1902MU003 during the next cold shutdown of both Units 2 and 3. Also, the need for a design modification is being evaluated to replace both WGC C010 and C011 rupture discs by relief valves piped to either the compressor discharge or surge tank. This action would eliminate the open system release path and preclude recurrence.

There are no credible circumstances that would have increased the severity of this event.

Southern California Edison Company



SAN ONOFRE NUCLEAR GENERATING STATION

P.O. BOX 128

SAN CLEMENTE, CALIFORNIA 92672

J. G. HAYNES
STATION MANAGER

TELEPHONE
(714) 492-7700

June 1, 1984

U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555

Subject: Docket No. 50-361
30-Day Report
Licensee Event Report No. 84-027
San Onofre Nuclear Generating Station, Units 2 and 3

Pursuant to 10 CFR 50.73(a)(2)(v) and 50.73(a)(2)(viii), this submittal provides the required 30-day written Licensee Event Report (LER) for an occurrence involving the waste gas processing system. Since this occurrence involved a shared system between Units 2 and 3, a single report is enclosed per NUREG-1022. The health and safety of plant personnel or the public were not affected by this event.

If you require any additional information, please so advise.

Sincerely,

Enclosure: LER No. 84-027

cc: A. E. Chaffee (USNRC Resident Inspector, Units 1, 2 and 3)
J. P. Stewart (USNRC Resident Inspector, Units 2 and 3)

J. B. Martin (Regional Administrator, NRC Region V)

Institute of Nuclear Power Operations (INPO)

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