



THE CLEVELAND ELECTRIC ILLUMINATING COMPANY

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MURRAY R. EDELMAN
VICE PRESIDENT
NUCLEAR

May 31, 1984

Mr. James G. Keppler
Regional Administrator, Region III
Office of Inspection and Enforcement
U.S. Nuclear Regulatory Commission
799 Roosevelt Road
Glen Ellyn, Illinois 60137

RE: Perry Nuclear Power Plant
Docket Nos. 50-440; 50-441
Drywell Equipment Hatch Covers
[RDC 105 (84)]

Dear Mr. Keppler:

This letter serves as an interim report pursuant to 10CFR50.55(e) concerning the Units I and II Drywell Equipment Hatch Covers which were fabricated and supplied under procurement specification SP669 by Newport News Industrial Corporation, Newport News, Virginia (NNIV). Mr. J. W. McCormick-Barger of your office was first notified on May 4, 1984, by Mr. P. Martin of The Cleveland Electric Illuminating Company that this problem was being evaluated per our Deviation Analysis Report 182.

This report contains a description of the deficiency, corrective action, and the planned date for our next/final report.

Description of the Deficiency

During a review of the vendor (NNIV) documentation package for the Unit I Drywell Equipment Hatch Cover it was noted that approximately 100 welds attaching stiffeners to the cover plate received only a 10% random magnetic particle examination (MT) although fabrication drawings required a 100% MT of these welds. Nonconformance Report (NR) #CQC-3259 was written identifying the condition. Review of the documentation for the Unit II assembly showed this also received a 10% random MT in lieu of the required 100%. Nonconformance Report #TAS-082 was written to document this condition for Unit II.

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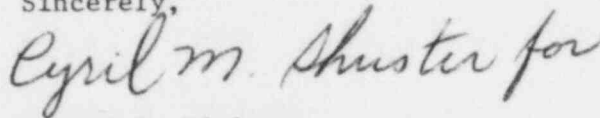
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Correction Action

The design/fabricator of the hatch cover assemblies, NNIV, is currently evaluating the conditions identified on the above referenced NRs. This evaluation includes a review of the design criteria utilized on these assemblies to verify that the specified 100% MT is required or if a random 10% MT, as performed, is adequate to ensure structural integrity of the assemblies under all design load conditions. We will inform you of the results of this evaluation and any recommended corrective action in our next report which will be submitted by August 31, 1984.

Please call if there are any additional questions.

Sincerely,



Murray R. Edelman
Vice President
Nuclear Group

MRE:pab

cc: Mr. J. A. Grobe
USNRC, Site Office

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