

Tennessee Valley Authority, Post Office Box 2000, Soddy-Daisy, Tennessee, 37379

August 21, 1995

U.S. Nuclear Regulatory Commission ATTN: Document Control Desk Washington, D.C. 20555

Gentlemen:

In the Matter of Tennessee Valley Authority Ducket Nos. 50-327

50-328

SEQUOYAH NUCLEAR PLANT (SQN) - ADDITIONAL INFORMATION FOR TECHNICAL SPECIFICATION (TS) CHANGE 94-15, "POSTACCIDENT SAMPLING (PAS)"

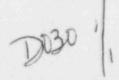
Reference: TVA letter to NRC dated April 6, 1995, "Sequoyah Nuclear Plant (SQN) - Technical Specification (TS) Change 94-15, 'Postaccident Sampling

(PAS)"

By the referenced letter, TVA submitted TS Change 94-15 that proposed to delete License Condition 2.C.(23)F of License No. DPR-77 for Unit 1 and 2.C.(16)g of License No. DPR-79 for Unit 2. These license conditions are associated with operation of SQN's PAS System.

Recent discussions between SQN's Licensing staff and NRC indicated that clarification was needed within SQN's Final Safety Analysis Report (FSAR) concerning the start of the response times for taking samples. Specifically, NRC indicated that the PAS System must have the capability of taking samples within the specified response time following the accident rather than following the onset of core stability. Accordingly, TVA is resubmitting Enclosure 5 to the above referenced letter in its entirety to reflect this clarification. Note that the justification for SQN's response time in the revised PAS Program (provided in Enclosure 4 of TVA's referenced letter) remains unaffected by the change in capability described above. SQN's PAS Program will be established to ensure the appropriate sampling response time capability exists and will be executed in accordance with good sampling practices. The additional information contained in this submittal does not affect the significant hazards evaluation previously provided in the referenced letter.

Enclosed are the revised FSAR pages associated with SQN's revised PAS Program.



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Please direct questions concerning this issue to D. V. Goodin at (615) 843-7734.

Sincerely,

R. H. Shell

Manager

SQN Site Licensing

R. H. Shell

Enclosure cc (Enclosure):

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ENCLOSURE

FINAL SAFETY EVALUATION REPORT
CHANGES ASSOCIATED WITH SEQUOYAH
NUCLEAR PLANTS REVISED POSTACCIDENT
SAMPLING PROGRAM