

LASALLE NUCLEAR POWER STATION

UNIT 1

MONTHLY PERFORMANCE REPORT

APRIL 1984

COMMONWEALTH EDISON COMPANY

NRC DOCKET NO. 050-373

LICENSE NO. NPF-11

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I. INTRODUCTION

The LaSalle Nuclear Power Station is a Two Unit Facility Located in Marseilles, Illinois. Each Unit is a Boiling Water Reactor with a designed electrical output of 1078 MWe net. The Station is owned by Commonwealth Edison Company. The Architect/Engineer was Sargent & Lundy, and the primary construction contractor was Commonwealth Edison Company.

The condenser cooling method is a closed cycle cooling pond. Unit One is subject to License Number NPF-11, issued on April 17, 1982. The date of initial criticality was June 21, 1982. The unit commenced commercial generation of power on January 1, 1984. Unit Two is subject to license number NFP-18, issued on December 16, 1983. The date of initial criticality was March 10, 1984. The Unit is expected to commence commercial generation of power in August, '84.

This report was compiled by Aras R. Lintakas telephone number (815)357-6761, extension 324.

II. MONTHLY REPORT FOR UNIT ONE

A. SUMMARY OF OPERATING EXPERIENCE FOR UNIT ONE

April 1-14 The unit started the reporting period at 95% power with the generator synchronized to the grid. At 0200 hours on April 2, Reactor power was lowered to 75%. At 1500 hours on April 4, reactor power was raised to 97%. At 0230 hours on April 7, Reactor power was reduced to 75%. At 0710 on April 10, Reactor Power was raised to 97%. At 1500 hours, reactor power was lowered to 52%. On April 14 at 1905 hours, the reactor scrambled on low reactor water level. This was due to feedwater transients during testing of the "B" turbine driven reactor feedwater pump after maintenance. The reactor was critical for 331 hours and 5 minutes.

April 15-30 The Reactor went critical at 1430 hours on April 15. On April 16 at 0525 hours, the turbine has synchronized to the Grid. At 0800 hours on April 16, Power was increased to 29%. At 1530 hours on April 16, Reactor power was increased to 53%. At 1500 hours, reactor power was increased to 81%. On April 21 at 0320, reactor power was increased to 97%. The reactor was critical for 369 hours and 30 minutes.

B. PLANT OR PROCEDURE CHANGES, TESTS, EXPERIMENTS AND SAFETY RELATED MAINTENANCE.

1. Amendments to facility license or Technical Specification.

There were no amendments to the facility license or Technical Specification.

2. Facility or procedure changes requiring NRC approval.

There were no facility or procedure changes requiring NRC approval.

3. Tests and Experiments requiring NRC approval.

There were no tests or experiments requiring NRC approval.

4. Corrective maintenance of safety related equipment.

The following table (Table 1) presents a summary of safety-related maintenance completed on Unit One during the reporting period. The headings indicated in this summary include: Work Request numbers, LER numbers, Component Name, Cause of Malfunction, Results and Effects on Safe Operation, and Corrective Action.

TABLE 1
CORRECTIVE MAINTENANCE OF
SAFETY RELATED EQUIPMENT

WORK REQUEST	LER	COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS ON SAFE OPERATION	CORRECTIVE ACTION
35477		SBGT WRGM	Broken BNC Connector on Low Range Selector	Inaccurate indication on low range	Resoldered wire to connector and tightened in place.
35616		A" VE compressor OVE04CA	Coupling failure	Compressor inoperable	Reassembled coupling Verified operation.
34947	84-021	OXY125B Ammonia detector.	Incorrect wiring	Correct dampers will not isolate on high ammonia detection.	Corrected wiring and checked for proper contact operation.
34929	84-020	1D18-K609D VR RAD monitor	Relay Board Damaged	"D" VR Rad Monitor in tripped condition	Replaced relay board
34607	84-019	1E31-R620 RWCU leak detection	Spurious failure of summer card	Inaccurate indication on diff. flow meter.	Calibrated per LIS-RT-101
34107	84-018	Various MCC's	Butt splice inspection	Inspection to verify correct work performed.	Inspection performed by E.M. Dept. QC & QA. Verification & Document Review.
30858		OVE03CB	Abnormal wear of fan and motor.	None "A" train still operable.	Replaced fan and motor.
35285		OD18-N512 OD18-N513	Bad Mid & High Range SBGT Gas monitor detectors	No SBGT Wide Range Gas monitor indication	Replaced with new detectors recalibrated per LCP-820-24
35364		OPI-VC134	Dirty oil screen. Compress or trips.	Compressor OVC05CA trips on low oil pressure.	Cleaned oil screen

C. LICENSEE EVENT REPORTS

The following is a tabular summary of all licensee event reports for LaSalle Nuclear Power Station, Unit One, occurring during the reporting period, April 1 through April 30, 1984. This information is provided pursuant to the reportable occurrence reporting requirements as set forth in section 6.6.B.1 and 6.6.B.2 of the Technical Specifications.

<u>Licensee Event Report Number</u>	<u>Date</u>	<u>Title of Occurrence</u>
84-018-00	3/10/84	Control Cable Butt Splice failure.
84-019-00	3/20/84	RWCU Differential flow transmitter out of calibration.
84-020-00	3/27/84	Group IV PCIS isolation
84-021-00	3/27/84	Control Room HVAC System Ammonia Detector INOP.

D. DATA TABULATIONS

The following data tabulations are presented in this report:

1. Operating Data Report
2. Average Daily Unit Power Level
3. Unit Shutdowns and Power Reductions

1. OPERATING DATA REPORT

DOCKET NO. 050-373
UNIT LaSalle One
DATE April 15, 1984
COMPLETED BY Aras R. Lintakas
TELEPHONE (815)357-6761

OPERATING STATUS

1. REPORTING PERIOD: April 1984 GROSS HOURS IN REPORTING PERIOD: 719
 2. CURRENTLY AUTHORIZED POWER LEVEL (Mwt): 3323 MAX DEPEND CAPACITY (MWe-Net): 1036 DESIGN ELECTRICAL RATING (MWe-Net): 1078
 3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net): N/A
 4. REASONS FOR RESTRICTION (IF ANY):
- | | THIS MONTH | YR TO DATE | CUMULATIVE |
|---|----------------|----------------|----------------|
| 5. NUMBER OF HOURS REACTOR WAS CRITICAL | <u>699.6</u> | <u>1858.1</u> | <u>1858.1</u> |
| 6. REACTOR RESERVE SHUTDOWN HOURS | <u>19.4</u> | <u>1012.0</u> | <u>1012.0</u> |
| 7. HOURS GENERATOR ON LINE | <u>684.7</u> | <u>1720.3</u> | <u>1720.3</u> |
| 8. UNIT RESERVE SHUTDOWN HOURS | <u>0.0</u> | <u>1.0</u> | <u>1.0</u> |
| 9. GROSS THERMAL ENERGY GENERATED (MWH) | <u>2065440</u> | <u>4447241</u> | <u>4447241</u> |
| 10. GROSS ELEC. ENERGY GENERATED (MWH) | <u>692699</u> | <u>1451965</u> | <u>1451965</u> |
| 11. NET ELEC. ENERGY GENERATED (MWH) | <u>663674</u> | <u>1368536</u> | <u>1368536</u> |
| 12. REACTOR SERVICE FACTOR | <u>97.3%</u> | <u>64.0%</u> | <u>64.0%</u> |
| 13. REACTOR AVAILABILITY FACTOR | <u>100%</u> | <u>98.9%</u> | <u>98.9%</u> |
| 14. UNIT SERVICE FACTOR | <u>95.2%</u> | <u>59.3%</u> | <u>59.3%</u> |
| 15. UNIT AVAILABILITY FACTOR | <u>95.2%</u> | <u>59.3%</u> | <u>59.3%</u> |
| 16. UNIT CAPACITY FACTOR (USING MDC) | <u>89.1%</u> | <u>45.5%</u> | <u>45.5%</u> |
| 17. UNIT CAPACITY FACTOR (USING DESIGN MWe) | <u>85.6%</u> | <u>43.7%</u> | <u>43.7%</u> |
| 18. UNIT FORCED OUTAGE RATE | <u>4.8%</u> | <u>34.8%</u> | <u>34.8%</u> |
19. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH) SEPTEMBER 1984, TWO WEEK SHUTDOWN FOR SCHEDULED SURVEILLANCES.
 20. IF SHUT DOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP: N/A
 21. UNITS IN TEST STATUS (PRIOR TO COMMERCIAL OPERATION):

	FORECAST	ACHIEVED
INITIAL CRITICALITY	<u> </u>	<u>6/21/82</u>
INITIAL ELECTRICITY	<u> </u>	<u>9/04/82</u>
COMMERCIAL OPERATION	<u> </u>	<u>1/1/84</u>

2. AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO: 050-373
UNIT: LASALLE ONE
DATE: APRIL 15, 1984
COMPLETED BY: Aras R. Lintakas
TELEPHONE: (815) 357-6761

MONTH: April 1984

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

1. _____	967	17. _____	732
2. _____	848	18. _____	818
3. _____	969	19. _____	876
4. _____	1039	20. _____	1027
5. _____	1077	21. _____	1041
6. _____	1085	22. _____	949
7. _____	813	23. _____	1063
8. _____	897	24. _____	1082
9. _____	1035	25. _____	1075
10. _____	1070	26. _____	1056
11. _____	1077	27. _____	1064
12. _____	1078	28. _____	1057
13. _____	1073	29. _____	978
14. _____	475	30. _____	1071
15. _____	0	31. _____	N/A
16. _____	319		

INSTRUCTIONS

On this form list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt. These figures will be used to plot a graph for each reporting month. Note that when maximum dependable capacity is used for the net electrical rating of the unit there may be occasions when the daily average power level exceeds the 100% line (or the restricted power level line.) In such cases the average daily unit power output sheet should be footnoted to explain the apparent anomaly.

LTP-300-7
Revision 3
March 1, 1983
9 (Final)

3. UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 050-374
UNIT NAME LaSalle Two
DATE May 10, 1984
COMPLETED BY Aras R. Lintakas
TELEPHONE (815)357-6761

REPORT MONTH APRIL 1984
CONTINUED

NO.	DATE	TYPE		DURATION (HOURS)	REASON (1)	METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER	CORRECTIVE ACTIONS/COMMENTS
		F: FORCED	S: SCHEDULED				
8	4/14/84	F		34.3	G(1)	3	Procedures were changed to more clearly define the operational setpoints

(1) The operator changed the setpoint of the reactor water level control below the operational level allowed by existing procedures for paralleling an additional feedwater pump. This resulted in a low reactor water level scram.

E. UNIQUE REPORTING REQUIREMENTS

1. Safety/Relief valve operations for Unit 1.

There were no relief valve operations for Unit One for this reporting period.

2. ECCS Systems Outages

The following outages were taken on ECCS Systems during the reporting period.

<u>OUTAGE NO.</u>	<u>EQUIPMENT</u>	<u>PURPOSE OF OUTAGE</u>
1-363-84	S/D Cooling Suction	Prevent S/D Cooling Initiation w/vents open.
1-381-84	1B D/G Air Comp.	Design evaluation
1-395-84	1A D/G	Inspect oil pump & strainer.

3. Off-Site Dose Calculation Manual

There were no changes to the off-site dose calculations manual during this reporting period.

4. Radioactive Waste Treatment Systems.

There were no changes to the Radioactive Waste Treatment System during this reporting period.

LASALLE NUCLEAR POWER STATION

UNIT 2

MONTHLY PERFORMANCE REPORT

APRIL 1984

COMMONWEALTH EDISON COMPANY

NRC DOCKET NO. 050-374

LICENSE NO. NPF-18

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 - 1. Safety/Relief Valve Operations
 - 2. ECCS System Outages
 - 3. Off-Site Dose Calculation Manual Changes
 - 4. Major Changes to Radioactive Waste Treatment System

I. INTRODUCTION

The LaSalle Nuclear Power Station is a Two Unit Facility Located in Marseilles, Illinois. Each Unit is a Boiling Water Reactor with a designed electrical output of 1078 MWe net. The Station is owned by Commonwealth Edison Company. The Architect/Engineer was Sargent & Lundy, and the primary construction contractor was Commonwealth Edison Company.

The condenser cooling method is a closed cycle cooling pond. Unit One is subject to License Number NPF-11, issued on April 17, 1982. The date of initial criticality was June 21, 1982. The unit commenced commercial generation of power on January 1, 1984. Unit Two is subject to license number NPF-18, issued on December 16, 1983. The date of initial criticality was March 10, 1984. The Unit is expected to commence commercial generation of power in August, '84.

This report was compiled by Aras R. Lintakas, telephone number (815)357-6761, extension 324.

II. MONTHLY REPORT FOR UNIT TWO

A. SUMMARY OF OPERATING EXPERIENCE FOR UNIT TWO

April 1-2 The Unit started the reporting period with the reactor critical at less than 1% power. At 0125 hours on April 2, the reactor was shutdown. The reactor was critical for 25 hours and 25 minutes.

April 2-14 The reactor went critical at 0900 hours on April 2. At 1700 hours on April 14, the reactor was shutdown. The reactor was critical for 320 hours and 0 minutes.

April 18-26 The reactor went critical at 0437 hours on April 18. The generator was synchronized to the grid at 1809 hours on April 20. At 1820 hours on April 20, the generator was removed from the grid. At 2050 hours on April 20, the generator was synchronized to the grid. At 2113 hours on April 20, the generator was removed from the grid for lockout verification. At 1730 hours on April 21, the generator was synchronized to the grid. At 1740 hours on April 21, the generator was removed from the grid. At 1045 hours on April 22, the generator was synchronized to the grid. On April 22 at 1730 hours, the generator was removed from the grid for testing. At 1815 hours on April 22, the generator was synchronized to the grid. At 1500 hours on April 24, Reactor power was increased to 18%. At 1620 hours on April 25, the generator was removed from the grid. On April 26 at 0505 hours, the reactor was manually scrammed due to problems with condensate suction strainers. The reactor was critical for 192 hours and 28 minutes.

April 28-30 The reactor went critical at 0115 hours on April 28. At 1130 hours on April 29, the generator was synchronized to the grid. AT 2147 hours on April 29, the generator was removed from the grid. On April 30 at 0430 hours, the generator was synchronized to the grid. At 1600 hours on April 30, reactor power was at 18%. The reactor was critical for 70 hours and 45 minutes.

B. PLANT OR PROCEDURE CHANGES, TESTS, EXPERIMENTS AND SAFETY RELATED MAINTENANCE.

1. Amendments to facility license or Technical Specifications.

There were no amendments to the facility license or Technical Specifications during the reporting period.

2. Facility or procedure changes requiring NRC approval.

There were no facility or procedure changes requiring NRC approval during the reporting period.

3. Tests and experiments requiring NRC approval.

The abstract to the residual heat removal system startup test, table 14.2-134, was revised. This was approved by telephone conversation with Mr. Richard Becker on April 16, 1984. (See Attached.)

4. Corrective Maintenance of Safety Related Equipment.

The following table (Table 1) presents a summary of safety-related maintenance completed on Unit One during the reporting period.

The headings indicated in this summary include: Work Request numbers, LER Numbers, Component Name, cause of malfunction, results and effects on safe operation, and corrective action.

April 17, 1984

TO: A. Bournia

SUBJECT: Commonwealth Edison Company
LaSalle County Station
FSAR Changes Relating to the Startup Test Program

REFERENCE: Telephone Conversation with Mr. Richard Becker
on April 16, 1984.

Per our telephone Conversation with Dick on April 16, 1984 we are informally transmitting the enclosed marked up changes to the LaSalle County Station FSAR. Please forward these changes to Dick for his use.

Those participating in the referenced telephone conversation included Robert Bishop, Administrative Assistant Superintendent - LaSalle Station and Mr. William Guldemond, Senior Resident Inspector for LaSalle - NRC Region III. These changes will be included in the next revision to the FSAR and have been tentatively agreed upon per this telephone conversation.

Please feel free to call if you have any immediate problems with the proposed changes. When your review has been completed, a conference call to confirm their acceptability is requested so that a formal change to the LaSalle County Station FSAR can be submitted.

G. J. Diederich
Superintendent
LaSalle County Station

GJD/crh

Attachment

CC: J. C. Renwick
R. D. Bishop
W. Guldemond
R. Becker

TABLE 14.2-134

RESIDUAL HEAT REMOVAL (RHR) STARTUP TESTPURPOSE

The purpose of this test is to demonstrate the ability of the Residual Heat Removal (RHR) system to remove residual and decay heat from the nuclear system so that refueling and nuclear system servicing can be performed, ~~and, in conjunction with the Reactor Core Isolation Cooling (RCIC) System, to condense steam while the reactor is isolated from the main condenser.~~

DESCRIPTION

~~With the reactor above 10% power, the condensing mode of the RHR system will be tuned, and capacities demonstrated. Condensing heat exchanger performance characteristics will be demonstrated. During the first suitable reactor cooldown, the shutdown cooling mode of the RHR system and the design heat removal capability of the RHR Heat Exchangers will be demonstrated.~~ *The heat removal capacity of the RHR heat exchangers will be demonstrated in the supplies or pool cooling mode.*

The decay heat load is insignificant early in the startup test program. Use of this mode with low core exposure could result in exceeding the 100° F/hr cooldown rate limitation of the vessel, ~~if both RHR heat exchangers are used simultaneously.~~ Late in the test program, after accumulating significant core exposure, a suitable demonstration can more adequately ~~verify~~ *ability to cooldown demonstrate the reactor.* the RHR heat exchangers

ACCEPTANCE CRITERIALevel 1

Not applicable

Level 2

The RHR system shall be capable of operating in the ~~steam condensing (with both one and two heat exchangers), suppression pool cooling and shutdown cooling mode (with either heat exchanger in operation) at the flow rates and temperature~~ *heat transfer* differentials indicated on the process diagrams. In the steam condensing mode, for small disturbances, each variable must have a decay ratio less than 0.25 throughout the expected operating range of each controller's expected operating range. *The RHR system shall be capable of operating in the steam condensing mode.*

TABLE 14.2-134 (Cont'd)

~~The time to place the RHR heat exchangers in the steam condensing mode with the RCIC using the heat exchanger condensate flow for suction shall be 1/2 hour or less.~~

The RHR system performance in the shutdown cooling mode shall ~~not be less than that indicated on the process diagram.~~ provide a controlled cooldown during shutdown.

INITIAL CONDITIONS

1. All construction tests and preoperational tests are completed and approved.
2. All system instrumentation installed and calibrated.
3. All systems must be operational as required to perform the test.

The RHR System heat exchangers shall meet the heat transfer of the process diagram for the suppression pool cooling mode.

TABLE 1
CORRECTIVE MAINTENANCE OF
SAFETY RELATED EQUIPMENT

WORK REQUEST	LER	COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS ON SAFE OPERATION	CORRECTIVE ACTION
34069		ZUR-CM027	Bad ΔP Cell and circuit card	Inaccurate Rx level Indication.	Replaced ΔP cell & circuit cart Performed calibration.
30999		RHR Serv. Water Outlet Stop 2E12-F068A	Packing Leak	Additional Rx Bldg. Equip. Drain Sump Input	Replaced Packing.
34995		RX Level Instr 1B21-N037D	Out of calibration	Inaccurate Rx level Indication	Recalibrated to in tolerance limits.
35107		"B" Post Loca O ₂ Monitor	Erratic Indication	No effect on operation	Performed LIS-PC-07
35139		Instr. N2 Bottle Bank check valve 2IN043.	Foreign material in check valve seat	Depressurized north ADS Bottle bank when IN compressor output was low	Disassembled check valve & removed foreign material.
35141		IRM "D"	Bad 15 VDC Regulator	IRM Indicates downscale.	Replaced 15 VDC regulator. Performed LIS-NR-02.
35155		2B21-F065A	Packing leak	Additional Rx Bldg. Equip. Drain Sump Input.	Added Packing and made adjustments per LMP-GM-01.
35165		Valve 2C11- F102 for CRD 34-35	Valve stuck in open position.	None- Control Rod 34-35 at "full in" position, & electrically disarmed.	Removed valve assembly and installed plug.

TABLE 1 Cont'd

LTP-300-7
Revision 3
March 1, 1983
5CORRECTIVE MAINTENANCE OF
SAFETY RELATED EQUIPMENT

WORK REQUEST	LER	COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS ON SAFE OPERATION	CORRECTIVE ACTION
35289		HPCS Water leg pump.	Abnormal wear of motor bearings.	HPCS Inoperable	Replaced motor
35334		MSIV limit switches	Limit switch settings changed due to "Hot" conditions.	Incorrect valve position indication.	Adjusted limit switches per LKS-MS-101.
35371		IRM C	Detector will not insert	None, all other IRM's operable	Readjusted "full out" and "full in" limit switches.
35441		2B21-F019	Valve will not "seal in" the close direction.	Valve will not fully close	Replaced faulty torque switch.
35978		2B21-R884A wide range level recorder	Recorder indicates upscale.	Incorrect level indication.	Performed & recalibrated Per LIS-NB-202
35400		2B D/G	Leaking fuel oil to injector line	None, D/G still operable.	Torqued fuel oil stud nut to 40 ft/lbs.

C. LICENSEE EVENT REPORTS

The following is a tabular summary of all licensee event reports for LaSalle Nuclear Power Station, Unit Two, occurring during the reporting period, April 1 through April 30, 1984. This information is provided pursuant to the reportable occurrence reporting requirements as set forth in section 6.6.B.1 and 6.6.B.2 of the Technical Specifications.

<u>Licensee Event Report Number</u>	<u>Date</u>	<u>Title of Occurrence</u>
84-010-00	3/13/84	RWCU Isolation on High Pump Room Differential Temperature.
84-011-00	3/16/84	Reactor vessel Water Level high switches out of calibration.

D. DATA TABULATIONS

The following data tabulations are presented in this report:

1. Operating Data Report
2. Average Daily Unit Power Level
3. Unit Shutdowns and Power Reductions

1. OPERATING DATA REPORT

DOCKET NO. 050-374
UNIT LaSalle Two
DATE May 10, 1984
COMPLETED BY Aras R. Lintakas
TELEPHONE (815)357-6761

OPERATING STATUS

1. REPORTING PERIOD: April 1984 GROSS HOURS IN REPORTING PERIOD: 719
 2. CURRENTLY AUTHORIZED POWER LEVEL (Mwt): 3323 MAX DEPEND CAPACITY (MWe-Net): 1036 DESIGN ELECTRICAL RATING (MWe-Net): 1078
 3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net): N/A
 4. REASONS FOR RESTRICTION (IF ANY):
- | | THIS MONTH | YR TO DATE | CUMULATIVE |
|---|---------------|---------------|---------------|
| 5. NUMBER OF HOURS REACTOR WAS CRITICAL | <u>583.6</u> | <u>708.1</u> | <u>708.1</u> |
| 6. REACTOR RESERVE SHUTDOWN HOURS | <u>135.4</u> | <u>530.7</u> | <u>530.7</u> |
| 7. HOURS GENERATOR ON LINE | <u>107.4</u> | <u>107.4</u> | <u>107.4</u> |
| 8. UNIT RESERVE SHUTDOWN HOURS | <u>0.0</u> | <u>0.0</u> | <u>0.0</u> |
| 9. GROSS THERMAL ENERGY GENERATED (MWH) | <u>143472</u> | <u>151750</u> | <u>151750</u> |
| 10. GROSS ELEC. ENERGY GENERATED (MWH) | <u>16683</u> | <u>16683</u> | <u>16683</u> |
| 11. NET ELEC. ENERGY GENERATED (MWH) | <u>14216</u> | <u>14216</u> | <u>14216</u> |
| 12. REACTOR SERVICE FACTOR | <u>N/A</u> | <u>N/A</u> | <u>N/A</u> |
| 13. REACTOR AVAILABILITY FACTOR | <u>N/A</u> | <u>N/A</u> | <u>N/A</u> |
| 14. UNIT SERVICE FACTOR | <u>N/A</u> | <u>N/A</u> | <u>N/A</u> |
| 15. UNIT AVAILABILITY FACTOR | <u>N/A</u> | <u>N/A</u> | <u>N/A</u> |
| 16. UNIT CAPACITY FACTOR (USING MDC) | <u>N/A</u> | <u>N/A</u> | <u>N/A</u> |
| 17. UNIT CAPACITY FACTOR (USING DESIGN MWe) | <u>N/A</u> | <u>N/A</u> | <u>N/A</u> |
| 18. UNIT FORCED OUTAGE RATE | <u>N/A</u> | <u>N/A</u> | <u>N/A</u> |
19. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH)
 20. IF SHUT DOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP: N/A
 21. UNITS IN TEST STATUS (PRIOR TO COMMERCIAL OPERATION):
- | | FORECAST | ACHIEVED |
|----------------------|-------------------|-------------------|
| INITIAL CRITICALITY | <u> </u> | <u>3/10/84</u> |
| INITIAL ELECTRICITY | <u> </u> | <u>4/20/84</u> |
| COMMERCIAL OPERATION | <u>Aug. 84</u> | <u> </u> |

2. AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO: 050-374
UNIT: LASALLE TWO
DATE: May 10, 1984
COMPLETED BY: Aras R. Lintakas
TELEPHONE: (815) 357-6761
MONTH: April 1984

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

1. _____	0.0	17. _____	0.0
2. _____	0.0	18. _____	0.0
3. _____	0.0	19. _____	0.0
4. _____	0.0	20. _____	0.0
5. _____	0.0	21. _____	3
6. _____	0.0	22. _____	49
7. _____	0.0	23. _____	144
8. _____	0.0	24. _____	147
9. _____	0.0	25. _____	97
10. _____	0.0	26. _____	0
11. _____	0.0	27. _____	0
12. _____	0.0	28. _____	0
13. _____	0.0	29. _____	49
14. _____	0.0	30. _____	137
15. _____	0.0	31. _____	N/A
16. _____	0.0		

INSTRUCTIONS

On this form list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt. These figures will be used to plot a graph for each reporting month. Note that when maximum dependable capacity is used for the net electrical rating of the unit there may be occasions when the daily average power level exceeds the 100% line (or the restricted power level line.) In such cases the average daily unit power output sheet should be footnoted to explain the apparent anomaly.

LTP-300-7
Revision 3
March 1, 1983
9 (Final)

3. UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH APRIL 1984

DOCKET NO. 050-373
UNIT NAME LaSalle Two
DATE May 10, 1984
COMPLETED BY Aras R. Lintakas
TELEPHONE (815)357-6761

NO.	DATE	TYPE		DURATION (HOURS)	REASON (1)	METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER	CORRECTIVE ACTIONS/COMMENTS
		F: FORCED	S: SCHEDULED				
1	4/2/84		S	7.6	B	1	Normal Unit Shutdown.
2	4/14/84		S	83.6	B	1	Normal reactor shutdown to repair motor driven reactor feed pump dis- charge drain line.
Generator was synchronized to the grid for the first time on April 20, 1984 at 1809 hours.							
3	4/20/84	F		2.5	A	1	Instrumentation repaired
4	4/20/84		S	20.3	B	1	Verification of Generator Lockout.
5	4/21/84	F		17.1	A	1	High vibration on bearing #5.
6	4/22/84		S	.8	B	1	Overspeed testing.

LTP-300-7
Revision 3
March 1, 1983
9 (Final)

3. UNIT SHUTDOWNS AND POWER REDUCTIONS (CONT'D)

DOCKET NO. 050-373
UNIT NAME LaSalle Two
DATE May 10, 1984
COMPLETED BY Aras R. Lintakas
TELEPHONE (815)357-6761

REPORT MONTH APRIL 1984
CONTINUED

NO.	DATE	TYPE	DURATION (HOURS)	REASON (1)	METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER	CORRECTIVE ACTIONS/COMMENTS
		F: FORCED S: SCHEDULED				
7	4/25/84	F	91.1	A	1	Defective Generator thrust Bearing thermocouple
8	4/29/84		6.7	A	1	Leak in EHC #1 control valve fixed.

E. UNIQUE REPORTING REQUIREMENTS

1. Safety/Relief Valve Operations for Unit 2.

<u>DATE</u>	<u>VALVES ACTUATED</u>	<u>NO & TYPE ACTUATIONS</u>	<u>PLANT CONDITION</u>	<u>DESCRIPTION OF EVENT</u>
4/11/84	2B21-F013A	5 Manual	902 PSIG	LST 83-350
4/11/84	2B21-F013B	4 Manual	902 PSIG	LST 83-350
4/11/84	2B21-F013C	4 Manual	902 PSIG	LST 83-350
4/11/84	2B21-F013D	4 Manual	902 PSIG	LST 83-350
4/11/84	2B21-F013E	4 Manual	902 PSIG	LST 83-350
4/11/84	2B21-F013F	4 Manual	902 PSIG	LST 83-350
4/11/84	2B21-F013G	4 Manual	902 PSIG	LST 83-350
4/11/84	2B21-F013H	4 Manual	902 PSIG	LST 83-350
4/11/84	2B21-F013J	4 Manual	902 PSIG	LST 83-350
4/11/84	2B21-F013K	4 Manual	902 PSIG	LST 83-350
4/11/84	2B21-F013L	4 Manual	902 PSIG	LST 83-350
4/11/84	2B21-F013M	4 Manual	902 PSIG	LST 83-350
4/11/84	2B21-F013N	4 Manual	902 PSIG	LST 83-350
4/11/84	2B21-F013P	4 Manual	902 PSIG	LST 83-350
4/11/84	2B21-F013R	4 Manual	902 PSIG	LST 83-350
4/11/84	2B21-F013S	4 Manual	902 PSIG	LST 83-350
4/11/84	2B21-F013U	4 Manual	902 PSIG	LST 83-350
4/11/84	2B21-F013V	4 Manual	902 PSIG	LST 83-350

2. ECCS Systems Outages

The following outages were taken on ECCS Systems during the reporting period.

<u>OUTAGE NO.</u>	<u>EQUIPMENT</u>	<u>PURPOSE OF OUTAGE</u>
2-482-84	HPCS Water Leg Pump	Replace motor bearings
2-492-84	2E12-F040A	Adjust limits
2- 3-84	2A D/G Air Comp.	Remove/Repair Motor
2-522-84	2B D/G	Repair Fuel Leak
2-533-84	RHR "A" HX Discharge valve air supply	
2-544-84	2A D/G	Replace A/C Oil Circ Pump
2-545-84	2A D/G B Starting A/C	Repair
2-546-84	2A D/G A Starting A/C	Repair
2-549-84	2E12-F040A	Repair Leakage.
2-570-84	RHR "A" HX Upstream vent to S/P	
2-576-84	RHR "A" HX downstream vent to S/P.	Per T.S. 3.6.3. will not close fully with M.O.

3. Off-Site Dose Calculation Manual

There were no changes to the off-site dose calculations manual during this reporting period.

4. Radioactive Waste Treatment Systems.

There were no changes to the Radioactive Waste Treatment System during this reporting period.



Commonwealth Edison
LaSalle County Nuclear Station
Rural Route #1, Box 220
Marseilles, Illinois 61341
Telephone 815/357-6761

May 10, 1984

Director, Office of Management Information
and Program Control
United States Nuclear Regulatory Commission
Washington, D.C. 20555

ATTN: Document Control Desk

Gentlemen:

Enclosed for your information is the monthly performance report covering
LaSalle County Nuclear Power Station for the period covering April 1
through April 30, 1984.

Very truly yours,

CE Dargent

for
G. J. Diederich
Superintendent
LaSalle County Station

GJD/ARL/bej

Enclosure

xc: J. G. Keppler, NRC, Region III
NRC Resident Inspector LaSalle
Gary Wright, Ill. Dept. of Nuclear Safety
D. P. Galle, CECO
D. L. Farrar, CECO
INPO Records Center
Ron A. Johnson, PIP Coordinator SNED
W. R. Jackson, GE Resident

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