

OPERATING DATA REPORT

DOCKET NO. 50-315
 DATE 5-2-84
 COMPLETED BY W.T. Gillet
 TELEPHONE 616-465-590

OPERATING STATUS

1. Unit Name: Donald C. Cook 1
2. Reporting Period: April 1984
3. Licensed Thermal Power (MWt): 3250
4. Nameplate Rating (Gross MWe): 1152
5. Design Electrical Rating (Net MWe): 1030
6. Maximum Dependable Capacity (Gross MWe): 1056
7. Maximum Dependable Capacity (Net MWe): 1020
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:

Notes

9. Power Level To Which Restricted, If Any (Net MWe): _____
10. Reasons For Restrictions, If Any: _____

	This Month	Yr.-to-Date	Cumulative
11. Hours In Reporting Period	719	2,903	81,791
12. Number Of Hours Reactor Was Critical	649.8	2,656.9	60,274.9
13. Reactor Reserve Shutdown Hours	0	0	463
14. Hours Generator On-Line	640.3	2,633.2	58,976.9
15. Unit Reserve Shutdown Hours	0	0	321
16. Gross Thermal Energy Generated (MWH)	1,855,391	7,818,348	172,162,404
17. Gross Electrical Energy Generated (MWH)	608,060	2,570,100	56,496,390
18. Net Electrical Energy Generated (MWE)	584,248	2,472,778	54,353,118
19. Unit Service Factor	89.1	90.7	74
20. Unit Availability Factor	89.1	90.7	74
21. Unit Capacity Factor (Using MDC Net)	79.7	83.5	66.9
22. Unit Capacity Factor (Using DER Net)	78.9	82.7	64.1
23. Unit Forced Outage Rate	10.9	9.3	7.9

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
 Shutdown Schedule September 1, 1984 for surveillance testing.

25. If Shut Down At End Of Report Period, Estimated Date of Startup: _____

25. Units In Test Status (Prior to Commercial Operation):	Forecast	Achieved
INITIAL CRITICALITY	_____	_____
INITIAL ELECTRICITY	_____	_____
COMMERCIAL OPERATION	_____	_____

B406070042 B40430
 PDR ADOCK 05000315
 R PDR

IE 24
 11

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-315

UNIT 1

DATE 5-2-84

COMPLETED BY W.T. Gillett

TELEPHONE 616-465-5901

MONTH April 1984

DAY	AVERAGE DAILY POWER LEVEL (MWE-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWE-Net)
1	<u>713</u>	17	<u>1032</u>
2	<u>1032</u>	18	<u>1022</u>
3	<u>803</u>	19	<u>865</u>
4	<u>819</u>	20	<u>1034</u>
5	<u>1032</u>	21	<u>1031</u>
6	<u>845</u>	22	<u>1031</u>
7	<u>610</u>	23	<u>1032</u>
8	<u>916</u>	24	<u>1032</u>
9	<u>224</u>	25	<u>1025</u>
10	<u>-</u>	26	<u>982</u>
11	<u>-</u>	27	<u>871</u>
12	<u>126</u>	28	<u>812</u>
13	<u>740</u>	29	<u>811</u>
14	<u>1032</u>	30	<u>811</u>
15	<u>1035</u>	31	<u>-</u>
16	<u>1028</u>		

INSTRUCTIONS

On this format list the average daily unit power level in MWE-Net for each day in the reporting month. Compute to the nearest whole number.

UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-315
UNIT NAME D.C. Cook - Unit 1
DATE 5-7-84
COMPLETED BY B.A. Svensson
TELEPHONE 616/465-5901
PAGE 1 of 3

REPORT MONTH APRIL, 1984

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	Licensee Event Report #	System Code ⁴	Component Code ⁵	Cause & Corrective Action to Prevent Recurrence
217 Cont'd	840331	F	0	B	4	N.A.	HH	HTEXCH	Reactor power further reduced from 70% to 58% to remove the west main feed pump from service for tube leak check. One tube was plugged. Reactor power returned to 100% on 840401.
218	840403	F	0	B	4	N.A.	HH	HTEXCH	Reactor power reduced to 55% to remove the west main feed pump from service to check the feed pump turbine condenser for tube leaks. One tube was plugged. Reactor power was returned to 100% on 840404.
219	840406	F	0	B	4	N.A.	HH	HTEXCH	Reactor power reduced to 56% to check the west main feed pump turbine condenser for tube leaks. One tube was plugged. A power increase was started on 840407. At 77% additional leakage was indicated in the west main feed pump turbine condenser and

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 A Equipment Failure (Explain)
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 C Refueling
 D Regulatory Restriction
 E Operator Training & License Examination
 F Administrative
 G Operational Error (Explain)
 H Other (Explain)

³
 Method:
 1-Manual
 2-Manual Scram
 3-Automatic Scram
 4-Other (Explain)

⁴
 Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

⁵
 Exhibit I - Same Source

UNIT SHUTDOWNS AND POWER REDUCTIONS

INSTRUCTIONS

This report should describe all plant shutdowns during the report period. In addition, it should be the source of explanation of significant dips in average power levels. Each significant reduction in power level (greater than 20% reduction in average daily power level for the preceding 24 hours) should be noted, even though the unit may not have been shut down completely¹. For such reductions in power level, the duration should be listed as zero, the method of reduction should be listed as 4 (Other), and the Cause and Corrective Action to Prevent Recurrence column should explain. The Cause and Corrective Action to Prevent Recurrence column should be used to provide any needed explanation to fully describe the circumstances of the outage or power reduction.

NUMBER. This column should indicate the sequential number assigned to each shutdown or significant reduction in power for that calendar year. When a shutdown or significant power reduction begins in one report period and ends in another, an entry should be made for both report periods to be sure all shutdowns or significant power reductions are reported. Until a unit has achieved its first power generation, no number should be assigned to each entry.

DATE. This column should indicate the date of the start of each shutdown or significant power reduction. Report as year, month, and day. August 14, 1977 would be reported as 770814. When a shutdown or significant power reduction begins in one report period and ends in another, an entry should be made for both report periods to be sure all shutdowns or significant power reductions are reported.

TYPE. Use "F" or "S" to indicate either "Forced" or "Scheduled," respectively, for each shutdown or significant power reduction. Forced shutdowns include those required to be initiated by no later than the weekend following discovery of an off-normal condition. It is recognized that some judgment is required in categorizing shutdowns in this way. In general, a forced shutdown is one that would not have been completed in the absence of the condition for which corrective action was taken.

DURATION. Self-explanatory. When a shutdown extends beyond the end of a report period, count only the time to the end of the report period and pick up the ensuing down time in the following report periods. Report duration of outages rounded to the nearest tenth of an hour to facilitate summation. The sum of the total outage hours plus the hours the generator was on line should equal the gross hours in the reporting period.

REASON. Categorize by letter designation in accordance with the table appearing on the report form. If category H must be used, supply brief comments.

METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER. Categorize by number designation

¹Note that this differs from the Edison Electric Institute (EEI) definitions of "Forced Partial Outage" and "Scheduled Partial Outage." For these terms, EEI uses a change of 30 MW as the break point. For larger power reactors, 30 MW is too small a change to warrant explanation.

in accordance with the table appearing on the report form. If category 4 must be used, supply brief comments.

LICENSEE EVENT REPORT =. Reference the applicable reportable occurrence pertaining to the outage or power reduction. Enter the first four parts (event year, sequential report number, occurrence code and report type) of the five part designation as described in Item 17 of Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161). This information may not be immediately evident for all such shutdowns, of course, since further investigation may be required to ascertain whether or not a reportable occurrence was involved.) If the outage or power reduction will not result in a reportable occurrence, the positive indication of this lack of correlation should be noted as not applicable (N/A).

SYSTEM CODE. The system in which the outage or power reduction originated should be noted by the two digit code of Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161).

Systems that do not fit any existing code should be designated XX. The code ZZ should be used for those events where a system is not applicable.

COMPONENT CODE. Select the most appropriate component from Exhibit I - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161), using the following criteria:

- A. If a component failed, use the component directly involved.
- B. If not a component failure, use the related component, e.g., wrong valve operated through error; list valve as component.
- C. If a chain of failures occurs, the first component to malfunction should be listed. The sequence of events, including the other components which fail, should be described under the Cause and Corrective Action to Prevent Recurrence column.

Components that do not fit any existing code should be designated XXXXXX. The code ZZZZZZ should be used for events where a component designation is not applicable.

CAUSE & CORRECTIVE ACTION TO PREVENT RECURRENCE. Use the column in a narrative fashion to amplify or explain the circumstances of the shutdown or power reduction. The column should include the specific cause for each shutdown or significant power reduction and the immediate and contemplated long term corrective action taken, if appropriate. This column should also be used for a description of the major safety-related corrective maintenance performed during the outage or power reduction including an identification of the critical path activity and a report of any single release of radioactivity or single radiation exposure specifically associated with the outage which accounts for more than 10 percent of the allowable annual values.

For long textual reports continue narrative on separate paper and reference the shutdown or power reduction for this narrative.

UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH APRIL, 1984

DOCKET NO. 50-315
 UNIT NAME D.C. Cook - Unit 1
 DATE 5-7-84
 COMPLETED BY B.A. Svensson
 TELEPHONE 616/465-5901
 PAGE 2 of 3

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	Licensee Event Report #	System Code ⁴	Component Code ⁵	Cause & Corrective Action to Prevent Recurrence
219 Cont'd.									
220	840409	F	78.7	A	1	84-004	BA	SHV	reactor power was again reduced to 55%. One additional tube was plugged. A steam side inspection did not reveal any problems that could be the cause for the high tube failure rate. Reactor power was returned to 100% on 840408. On 840406 at 0311 hours, the turbine driven auxiliary feed pump was declared inoperable due to the inability to trip the turbine trip and throttle valve. Repairs could not be accomplished within the 72-hours allowed by Tech. Spec. 3.7.1.2. The Unit was removed from service on 840409 and placed in Hot Shutdown, Mode 4, as required by Tech. Spec. 3.7.1.2. The valve was repaired and the Unit returned to service on 840412. 100% reactor power was

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 H Other (Explain)

³
 Method:
 1 Manual
 2 Manual Scram.
 3 Automatic Scram.
 4 Other (Explain)

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UNIT SHUTDOWNS AND POWER REDUCTIONS

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DURATION. Self-explanatory. When a shutdown extends beyond the end of a report period, count only the time to the end of the report period and pick up the ensuing down time in the following report periods. Report duration of outages rounded to the nearest tenth of an hour to facilitate summation. The sum of the total outage hours plus the hours the generator was on line should equal the gross hours in the reporting period.

REASON. Categorize by letter designation in accordance with the table appearing on the report form. If category H must be used, supply brief comments.

METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER. Categorize by number designation

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UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH APRIL, 1984

DOCKET NO. 50-315
 UNIT NAME D.C. Cook - Unit 1
 DATE 5-7-84
 COMPLETED BY B.A. Svensson
 TELEPHONE 616/465-5901
 PAGE 3 of 3

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	Licensee Event Report #	System Code ⁴	Component Code ⁵	Cause & Corrective Action to Prevent Recurrence
220 Cont'd.									reached on 840413.
221	840418	F	0	F	4	N.A.	ZZ	ZZZZZZ	Reactor power reduced to 80% as required by Plant Secondary Chemistry Specifications due to high dissolved oxygen in the condensate. Reactor power was returned to 100% on 840419.
222	840427	F	0	F	4	N.A.	ZZ	ZZZZZZ	Reactor power was again reduced to 80% due to high dissolved oxygen in the condensate. Reactor power remained at 80% at the end of the month. The source of air inleakage has been isolated and reactor power was returned to 100% on 840502.

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³
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Components that do not fit any existing code should be designated XXXXYY. The code ZZZZZZ should be used for events where a component designation is not applicable.

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For long textual reports continue narrative on separate paper and reference the shutdown or power reduction for this narrative.

Docket No.: 50-315
Unit Name: D. C. Cook Unit 1
Completed By: G. J. Peak
Telephone: (616) 465-5901
Date: 5/7/84
Page: 1 of 2

MONTHLY OPERATING ACTIVITIES - APRIL 1984

Highlights:

The Unit entered the reporting period in Mode 1 with the reactor at 58% of rated thermal power due to the West Main Feedpump being out of service for a condenser tube leak check. The Unit had to be removed from service on 4/9/84 due to an inoperable turbine driven auxiliary feedwater pump. The cause of the inoperability was a trip and throttle valve which would not trip the turbine. The turbine driven auxiliary feedwater pump was subsequently declared operable on 4/11/84 and the Unit was returned to service on 4/12/84. Major power reductions occurred to remove the West Main Feedpump from service for condenser tube leak checks, and due to Unit exceeding secondary side chemistry specifications. As the reporting period came to an end, the Unit was operating at 80% of rated thermal power due to secondary side oxygen concentration being out of specification.

Total electrical generation for the month was 608,060 MWH.

Summary:

- 4/1/84 Power was reduced to 58% at 0000 hours to remove the West Main Feedpump from service for a condenser tube leak check. Power was returned to 100% at 2345 hours.
- 4/3/84 Power was reduced to 55% at 1725 hours to remove the West Main Feedpump from service for a condenser tube leak check.
- 4/4/84 Power was returned to 100% at 2030 hours.
- 4/6/84 Power was reduced to 56% at 1855 hours to remove the West Main Feedpump from service for a condenser tube leak check. The turbine driven auxiliary feedwater pump was declared inoperable at 0311 hours because the turbine could not be tripped by use of the trip push button.
- 4/8/84 Power was returned to 100% at 1326 hours.
- 4/9/84 The Unit was removed from service due to an inoperable turbine driven auxiliary feedwater pump. The Main Turbine was tripped at 0700 hours. The reactor trip breakers were opened at 0728 hours. Mode 4 was entered at 1455 hours.
- 4/10/84 A heat up of the Reactor Coolant System began at 0240 hours and Mode 3 was entered at 0312 hours.
- 4/11/84 The turbine driven auxiliary feedwater pump was declared operable at 0530 hours.

Docket No.: 50-315
Unit Name: D. C. Cook Unit 1
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Page: 2 of 2

- 4/12/84 The reactor was critical at 0443 hours. Mode 1 was entered at 1151 hours. The Main Turbine was rolled at 1241 hours. The Main Generator was paralleled at 1345 hours.
- 4/19/84 Power was reduced to 80% at 0300 hours due to secondary side oxygen concentration being out of specification. The Unit was returned to 100% power at 2205 hours.
- 4/25/84 A power reduction to 80% started at 2025 hours due to secondary side oxygen concentration being out of specification.
- 4/26/84 Secondary side oxygen concentration was in specification at 0300 hours and the power reduction stopped at 81.5%. Power was returned to 100% at 1000 hours.
- 4/27/84 Power was reduced to 80% at 1030 hours due to secondary side oxygen concentration being out of specification.

The Control Room Cable Vault Halon System remains inoperable as of 1400 hours on 4/5/83. The backup CO₂ System for the Control Room Cable Vault remains operable.

DOCKET NO.	<u>50 - 315</u>
UNIT NAME	<u>D. C. Cook - Unit No. 1</u>
DATE	<u>5-7-84</u>
COMPLETED BY	<u>B. A. Svensson</u>
TELEPHONE	<u>(616) 465-5901</u>
PAGE	<u>1 of 1</u>

MAJOR SAFETY-RELATED MAINTENANCE

APRIL, 1984

- M-1 A new gasket was installed to repair a leaking orifice flange located between the cold leg RTD manifold and valve, #RC-180L4.
- M-2 The Unit 1 turbine driven auxiliary feed pump trip and throttle valve would not trip closed. The valve was disassembled and a new plug and pilot valve/stem were installed. The stem pilot screw was adjusted. Installed a union on the lower leakoff line and capped both ends of the lower bonnet leakoff line. Adjusted, cleaned and lubricated the valve operator. Functional testing was performed following repairs and the valve was returned to service.
- C&I-1 Emergency diesel generator LCD pneumatic tachometer circuitry failed. A replacement bridge and potentiometer network were installed. The tach was adjusted to read 514 RPM at synchronous speed during a test run of the generator.
- C&I-2 Source range nuclear instrument channel N32 indicator was at 10 CPS with the channel de-energized. The triaxial connector cable was found to have "ground loops" between its shields. When a new cable was installed, N32 reading returned to zero.



INDIANA & MICHIGAN ELECTRIC COMPANY

Donald C. Cook Nuclear Plant
P.O. Box 458, Bridgman, Michigan 49106

May 7, 1984

Director, Office Of Management Information
and Program Control
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

Gentlemen:

Pursuant to the requirements of Donald C. Cook Nuclear Plant Unit 1
Technical Specification 6.9.1.6, the attached Monthly Operating
Report for the Month of April, 1984 is submitted.

Sincerely,


W. G. Smith, Jr.
Plant Manager

WGS:ab

Attachments

cc: J. E. Dolan
M. P. Alexich
R. W. Jurgensen
NRC Region III
E. R. Swanson
R. O. Bruggee (NSAC)
R. C. Callen
S. J. Mierzwa
R. F. Kroeger
B. H. Bennett
J. D. Huebner
J. H. Wennigan
A. F. Kozlowski
R. F. Hering
J. F. Stietzel
PNSRC File
INPO Records Center

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