

OPERATING DATA REPORT

DOCKET NO: 50-368  
 DATE: April, 1984  
 COMPLETED BY: L.S. Bramlett  
 TELEPHONE: 501-964-3145

OPERATING STATUS

1. Unit Name: Arkansas Nuclear One - Unit 2
2. Reporting Period: April 1-30, 1984
3. Licensed Thermal Power (Mwt): 2815
4. Nameplate Rating (Gross MWe): 942.57
5. Design Electrical Rating (Net MWe): 912
6. Maximum Dependable Capacity (Gross MWe): 897
7. Maximum Dependable Capacity (Net MWe): 858
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons: \_\_\_\_\_
9. Power Level To Which Restricted. If Any (Net MWe): None
10. Reasons For Restrictions. If Any: \_\_\_\_\_

	MONTH	YR-TO-DATE	CUMULATIVE
11. Hours in Reporting Period ....	719.0	2,903.0	35,927.0
12. Number of Hours Reactor was Critical .....	719.0	2,230.5	23,857.6
13. Reactor Reserve Shutdown Hours .....	0.0	0.0	1,430.1
14. Hours Generator On-Line .....	719.0	2,091.0	23,041.3
15. Unit Reserve Shutdown Hours ..	0.0	0.0	75.0
16. Gross Thermal Energy Generated (MWH) .....	1,998,149.0	5,011,980.0	57,561,520.0
17. Gross Electrical Energy Generated (MWH) .....	671,350.0	1,670,090.0	18,687,041.0
18. Net Electrical Energy Generated (MWH) .....	642,604.0	1,589,649.0	17,795,989.0
19. Unit Service Factor .....	100.0	72.0	64.1
20. Unit Availability Factor .....	100.0	72.0	64.3
21. Unit Capacity Factor (Using MDC Net) .....	104.2	63.8	57.7
22. Unit Capacity Factor (Using DER Net) .....	98.0	60.0	54.3
23. Unit Forced Outage Rate .....	0.0	2.8	19.1
24. Shutdown: Scheduled Over Next 6 Months (Type, Date, and Duration of Each): <u>None</u>			
25. If Shut Down At End of Report Period. Estimated Date of Startup: _____			
26. Units in Test Status (Prior to Commercial Operation):			

Forecast                      Achieved

INITIAL CRITICALITY \_\_\_\_\_  
 INITIAL ELECTRICITY \_\_\_\_\_  
 COMMERCIAL OPERATION \_\_\_\_\_

*JE 24*  
*1/1*

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO: 50-368  
UNIT: Two  
DATE: April, 1984  
COMPLETED BY: L. Brawlett  
TELEPHONE: 501-964-3145

MONTH April, 1984

DAY            AVERAGE DAILY POWER LEVEL  
                  (MWe-Net)

1	888
2	890
3	889
4	892
5	893
6	891
7	889
8	888
9	889
10	888
11	887
12	887
13	889
14	891
15	893
16	892
17	891
18	893
19	891
20	895
21	897
22	904
23	901
24	899
25	900
26	898
27	899
28	904
29	898
30	906
31	

INSTRUCTION

On this format, list the average daily unit power level in MWe-Net for each day in reporting month. Compute to the nearest whole megawatt.

NRC MONTHLY OPERATING REPORT

OPERATING SUMMARY

APRIL 1984

UNIT 2

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The unit began the month at 98% power due to Core Protection Calculator (CPC) limits. On April 20th, power was increased to 100% after adjustments were made to the CPC's.

UNIT SHUTDOWNS AND POWER REDUCTIONS  
REPORT FOR APRIL, 1984

DOCKET NO	50-368
UNIT NAME	ANO - Unit 2
DATE	May 15, 1984
COMPLETED BY	L. Bramlett
TELEPHONE	501-964-3145

<u>No.</u>	<u>Date</u>	<u>Type</u> <sup>1</sup>	<u>Duration</u> (Hours)	<u>Reason</u> <sup>2</sup>	<u>Method of</u> <u>Shutting</u> <u>Down Reactor</u> <sup>3</sup>	<u>Licensee</u> <u>Event</u> <u>Report #</u>	<u>System</u> <u>Code</u> <sup>4</sup>	<u>Component</u> <u>Code</u> <sup>5</sup>	<u>Cause &amp; Corrective</u> <u>Action to</u> <u>Prevent Recurrence</u>
None	N/A	N/A	N/A	N/A	N/A	N/A	N/A	N/A	N/A

1  
F: Forced  
S: Scheduled

2  
Reason:  
A-Equipment Failure (Explain)  
B-Maintenance or Test  
C-Refueling  
D-Regulatory Restriction  
E-Operator Training &  
License Examination  
F-Administrative  
G-Operational Error (Explain)  
G-Other (Explain)

3  
Method:  
1-Manual  
2-Manual Scram.  
3-Automatic Scram.  
4-Continuation  
5-Load Reduction  
9-Other

4  
Exhibit G - Instructions  
for Preparation of Data  
Entry Sheets for Licensee  
Event Report (LER) File (NUREG-  
0161)  
5  
Exhibit I - Same Source

DATE: April, 1984

REFUELING INFORMATION

1. Name of facility: Arkansas Nuclear One - Unit 2
2. Scheduled date for next refueling shutdown. June, 1985
3. Scheduled date for restart following refueling. August, 1985

4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment? If answer is yes, what, in general, will there be? If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload (Ref. 10 CFR Section 50.59)?

Yes, some proposed software changes to the Core Protection Calculators are being considered.

5. Scheduled date(s) for submitting proposed licensing action and supporting information. March, 1985
6. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures.

Burnable poison rods may be used in reload fuel.

7. The number of fuel assemblies (a) in the core and (b) in the spent fuel storage pool. a) 177 b) 168
8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies.  
present 988 increase size by 0

9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity.

DATE: 2003



ARKANSAS POWER & LIGHT COMPANY  
POST OFFICE BOX 551 LITTLE ROCK, ARKANSAS 72203 (501) 371-4000

May 15, 1984

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Mr. Harold S. Bassett, Director  
Division of Data Automation  
and Management Information  
Office of Resource Management  
U. S. Nuclear Regulatory Commission  
Washington, D. C. 20555

SUBJECT: Arkansas Nuclear One - Unit 2  
Docket No. 50-368  
License No. NPF-6  
Monthly Operating Report  
(File: 2-0520.1)

Gentlemen:

Attached is the NRC Monthly Operating Report for April 1984 for Arkansas Nuclear One - Unit 2.

Very truly yours,

John R. Marshall  
Manager, Licensing

JRM: SAB: ac

Attachment

cc: Mr. John T. Collins  
Regional Administrator  
U. S. Nuclear Regulatory Commission  
Region IV  
611 Ryan Plaza Drive, Suite 1000  
Arlington, TX 76011

Mr. Richard C. DeYoung  
Office of Inspection and Enforcement  
U. S. Nuclear Regulatory Commission  
Washington, DC 20555

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