UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

August 23, 1995

50-454/456

Mr. D. L. Farrar, Manager
Nuclear Regulatory Services
Commonwealth Edison Company
Executive Towers West III
1400 Opus Place, Suite 500
Downers Grove, Illinois 60515

SUBJECT: REQUEST FOR ADDITIONAL INFORMATION REGARDING THE STEAM GENERATOR TUBE VOLTAGE-BASED REPAIR CRITERIA (TAC NOS. M91671, M91672, M91673 AND M91674)

Dear Mr. Farrar:

In the course of our review of the Commonwealth Edison Company's (ComEd) pending request for license amendments submitted on July 7, 1995, regarding a revision to the technical specifications governing the steam generator (SG) tube voltage-based repair criteria for Byron Unit 1 and Braidwood Unit 1, we have identified a need for additional information. Your submittal of July 7, 1995, revised and superseded in its entirety, your original request for license amendments submitted on February 13, 1995. We have previously transmitted four requests for additional information (RAIs) in letters dated May 31, 1995, June 22, 1995, August 3, 1995, and August 11, 1995. This latest RAI was developed during our review of your submittal dated August 14, 1995, in response to our third RAI issued on August 3, 1995. The issues in this RAI were discussed briefly in the meeting held in Rockville, Maryland, on August 17, 1995, between members of the NRC staff and representatives of ComEd. For convenience, we are continuing the numbering in the same sequence we established in our prior RAIs on this matter.

The area of concern in the present RAI is related to the documentation of the thermal-hydraulic computer code TRANFLO as well as the comparisons cited in your letter dated August 14, 1995, between the results from TRANFLO and RELAP5.

As stated in our previous RAIs related to the pending license amendments, without timely and high quality technical resolution of the outstanding issues, it is unlikely that the staff will be able to reach a positive conclusion on your pending license amendments.

This requirement affects nine or fewer respondents and, therefore, is not subject to Office of Management and Budget review under P.L. 96-511.

If you have any questions on these matters, please contact M. D. Lynch at (301) 415-3023.

Sincerely,

Original signed by

M. D. Lynch, Senior Project Manager
Project Directorate III-2
Division of Reactor Projects III/IV
Office of Nuclear Reactor Regulation

Docket Nos. STN 50-454
and STN 50-456

Enclosure: RAI

cc w/encl: See next page

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*See previous concurrence

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REQUEST FOR ADDITIONAL INFORMATION

REGARDING THE PROPOSED REVISIONS TO THE TECHNICAL SPECIFICATIONS

RELATED TO THE STEAM GENERATOR TUBE VOLTAGE-BASED REPAIR CRITERIA

BYRON UNIT 1 AND BRAIDWOOD UNIT 1

DOCKET NOS. STN 50-454 AND STN 40-456

55. In your letter dated August 14, 1995, you state that the anticipated uncertainty in the tube support plate (TSP) hydrodynamic load analysis using the TRANFLO code, as submitted, ranges from 25 to 30 percent. List the elements of your analysis which contribute to this range of uncertainty, quantitatively indicate the relative contribution of each element, and explain the basis for each element listed.

56. Provide the results of the RELAP5 calculations cited in your letter dated August 14, 1995, including plotted time histories of the parameters used for your comparison to the TRANFLO results. As a minimum, provide the differential pressures for each TSP, the steam generator mass flow, and the steam pressure at the outlet nozzle.

57. Provide detailed information describing the RELAP model discussed in your letter dated August 14, 1995. As a minimum, this information should indicate: (1) which version of the RELAP code was used; (2) the code options used for the calculations; (3) a detailed noding plan; (4) a description of the model explaining the noding plan; (5) and the input deck (in hard-copy and electronic form) of the hot standby case for a postulated main steamline break (MSLB).