

MONTHLY OPERATIONS REPORT

APRIL 1984

Throughout the report period, the Oyster Creek Station remained shutdown for the Maintenance and Refueling Outage.

Refueling of the reactor vessel core commenced on April 15, 1984. Refueling continued to April 26, 1984, at which time channeling of thirty-three fuel assemblies had to be completed. Channeling of the fuel was completed on April 30, 1984 and refueling of the reactor restarted on May 1, 1984. At the end of the report period a total of 104 fuel assemblies were loaded in the reactor core (18.5%). During the first half of April channel and clip installation on irradiated and new fuel bundles were the major activities in progress on the 119' level. Also, recoupling of 74 CRDs was completed prior to the start of refueling the reactor.

The start of reactor refueling was delayed by a number of problems, the major one being the resolution of a fast start problem with diesel generator No. 1. The diesel generator experienced governor and relay problems from April 2, 1984 to April 14, 1984. The diesel generator was removed from service for testing and troubleshooting a number of times during this period until the fast start problems were corrected.

Testing of Cleanup System valves replaced and/or repaired during the outage delayed placing the Cleanup System and CRD System in service the early part of April. However, due to the problems encountered during this time period with diesel generator No. 1, these activities did not delay refueling.

Other delays encountered were due to SRM operability problems/rod block problems (hydraulic accumulator switch problems) and manpower restrictions required by the refueling procedures and NRC regulations. On April 15, 1984, a trip of substation breaker S-145 required securing the Cleanup and CRD Systems. This caused a short delay in the resumption of refueling the next morning.

The overhaul of both CRD pumps was completed during the report period. "B" pump was completed on April 2, 1984 and "A" pump on April 30, 1984. A functional test of "A" CRD pump is still outstanding.

The cleaning of all hotwells and replacement of the remaining expansion joints on the main condenser were completed during the report period.

A tube leak in "B" fuel pool heat exchanger still has not been identified. The heat exchanger was taken out of service a number of times in an attempt to locate the leak. Each time it was placed back in service, an increase in RBCCW surge tank level was noted. The M&C Department was still working to identify the leak source.

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Air compressor No. 1 was out of service for both corrective and preventive maintenance on three different occasions during the report period (total out of service time approximately 13 days). Air compressor 1-2 was out of service for a three (3) day period to repair an oil leak. Air compressor 1-3 was inoperable for a one-day period for instrument calibration.

The prefilter and HEPA filters on Standby Gas Treatment System No. 1 were replaced.

The first cask shipment of spent CRD blades left the site on April 3, 1984. The empty cask was returned to the site the weekend of April 21, 1984. A subsequent cask shipment is pending the completion of reactor refueling (end of May).

Two cracked cells were discovered in the "B" station battery system (125VDC). New batteries are on order.

The inspection of the isolation condenser piping continued during the report period. A repair scope is being developed by the Tech. Functions Division.

The MSIVs failed leak rate testing and Plant Engineering suspects the inboard valves are being unseated during the test causing the unacceptable results. In an effort to correct the problem, Plant Engineering is presently developing a method to flood the main steam lines between the inboard valves and the main steam line plugs. The leak rate test will be performed after flooding of the main steam lines is completed.

Feedwater string "C" inlet valve V-2-9 still leaks after being repaired/rebuilt twice during the outage. Presently, no additional maintenance can be performed due to the Condensate System being in service (required for refueling).

Both demineralized water transfer pumps and No. 2 service water pump remained out of service for corrective maintenance at the end of the report period.

The reactor sample line from "B" reactor recirculation loop is plugged. Plant Engineering is tasked to determine the maximum hydro pressure that can be applied to unplug the line. To date, all attempts to unplug the line have been unsuccessful.

Testing of major plant power distribution cables was in progress at the end of the report period.

The following Licensee Event Reports were submitted during April 1984:

Reportable Occurrence No. 50-219/84-001

On March 2, 1984, diesel generator (DG) #2 was started and allowed to run approximately 4 hours for post-maintenance testing. The DG fuel oil tank level after testing was above the Technical Specification limit of 14,500 gallons. Early on March 3, 1984, another load test was performed on DG #2 for approximately 1 hour. The oil tank level by the level gauge was below the Technical Specification limit; however, by visual check inside the tank, the level appeared to be above the Technical Specification limit. On the next shift the level was checked again and found to be below the Technical Specification limit. The apparent cause of the occurrence is attributed to operator error for not sufficiently following-up on the low level indication. The immediate corrective action was to transfer oil to the tank to bring the level above the Technical Specification limit. Future solutions to be evaluated include the following:

- 1) Utilize a level instrument which is not sensitive to changes in specific gravity, or one which compensates for changes in specific gravity.
- 2) Reduce the Technical Specifications limit to a value which does not compromise the design basis capacity, yet which allows more operational flexibility than there is at present.
- 3) Replace the existing tank with a larger tank.
- 4) Instruct operators to transfer oil as required by the indication on the level gauge.

Reportable Occurrence No. 50-219/84-002

On March 2, 1984, the unit sub-station (USS) circuit breaker to Motor Control Center (MCC) 1B32 failed to trip when the undervoltage (UV) device was de-energized. Again, on March 7, 1984, the circuit breakers to the shutdown cooling pump (SDC) NU02B and the building exhaust fan EF-1-6 failed to trip within the specified time limit in the procedure when their UV devices were de-energized. These events in themselves are not considered to be reportable, but were reported under 10 CFR 50.73 as they may be an indication of a potential generic problem. The cause of occurrence on the MCC 1B32 circuit breaker was attributed to high torque on the trip shaft bearings due to hardened lubricant. The cause of occurrence on the SDC pump NU02B and EF-1-6 circuit breakers was attributed to instrument drift of the static time delay UV tripping device. The immediate corrective action was to perform preventive maintenance on the circuit breakers. The circuit breakers were tested for operability three (3) times and were returned

to service. The longer term corrective actions are the same as those listed in GPU Nuclear's response to IE Bulletin 83-08, namely, PM procedure revision, investigation into modifying the control circuits and more frequent checks of the UV trip mechanisms.

UNIT SHUTDOWN AND POWER REDUCTIONS

REPORT MONTH April 1984

DOCKET NO. 50-219
 UNIT NAME Oyster Creek
 DATE 5-2-84
 COMPLETED BY R. Baran
 TELEPHONE 971-4640

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	Licensee Event Report #	System Code ⁴	Component Code ⁵	Cause & Corrective Action to Prevent Recurrence
31	2-11-83	S	10656	C	1	N/A	ZZ	ZZZZZZ	Start of the 1983 Refueling and Maintenance Outage.

¹
 F: Forced
 S: Scheduled

²
 Reason:
 A-Equipment Failure (Explain)
 B-Maintenance of Test
 C-Refueling
 D-Regulatory Restriction
 E-Operator Training & License Examination
 F-Administrative
 G-Operational Error (Explain)
 H-Other (Explain)

³
 Method:
 1-Manual
 2-Manual Scram.
 3-Automatic Scram.
 4-Other (Explain)

⁴
 Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

⁵
 Exhibit I - Same Source

OPERATING DATA REPORT
 OPERATING STATUS

1. DOCKET: 50-219
2. REPORTING PERIOD: April, 1984
3. UTILITY CONTACT: JOSEPH R. MOLNAR 609-971-4699
4. LICENSED THERMAL POWER (MWt): 1930
5. NAMEPLATE RATING (GROSS MWe): $687.5 \times 0.8 = 550$
6. DESIGN ELECTRICAL RATING (NET MWe): 650
7. MAXIMUM DEPENDABLE CAPACITY (GROSS MWe): 650
8. MAXIMUM DEPENDABLE CAPACITY (NET MWe): 620
9. IF CHANGES OCCUR ABOVE SINCE LAST REPORT, GIVE REASONS: NONE
10. POWER LEVEL TO WHICH RESTRICTED, IF ANY (NET MWe): N/A
11. REASON FOR RESTRICTION, IF ANY: NONE

	<u>MONTH</u>	<u>YEAR</u>	<u>CUMULATIVE</u>
12. REPORT PERIOD HRS	720.0	2904.0	125832.0
13. HOURS RX CRITICAL	0.0	0.0	84623.9
14. RX RESERVE SHUTDOWN HRS	0.0	0.0	468.2
15. HRS GENERATOR ON-LINE	0.0	0.0	82693.8
16. UT RESERVE SHUTDOWN HRS	0.0	0.0	0.0
17. GROSS THERM ENER (MWH)	0.0	0.0	136224729
18. GROSS ELEC ENER (MWH)	0.0	0.0	46056905
19. NET ELEC ENER (MWH)	-1900	-7616	44278067
20. UT SERVICE FACTOR	0.0	0.0	65.7
21. UT AVAIL FACTOR	0.0	0.0	65.7
22. UT CAP FACTOR (MDC NET)	0.0	-0.4	56.8
23. UT CAP FACTOR (DER NET)	0.0	-0.4	54.1
24. UT FORCED OUTAGE RATE	0.0	0.0	9.7
25. FORCED OUTAGE HRS	0.0	0.0	8916.8
26. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, DURATION):	N/A		
27. IF CURRENTLY SHUTDOWN ESTIMATED STARTUP TIME:	6/11/84		

AVERAGE DAILY POWER LEVEL
NET MWe

DOCKET #50-219
UNITOyster Creek #1
REPORT DATEMAY 03, 1984
COMPILED BYDONALD V. NOTIGAN
TELEPHONE #609-971-4695

MONTH APRIL, 1984

<u>DAY</u>	<u>MW</u>	<u>DAY</u>	<u>MW</u>
1.	0	16.	0
2.	0	17.	0
3.	0	18.	0
4.	0	19.	0
5.	0	20.	0
6.	0	21.	0
7.	0	22.	0
8.	0	23.	0
9.	0	24.	0
10.	0	25.	0
11.	0	26.	0
12.	0	27.	0
13.	0	28.	0
14.	0	29.	0
15.	0	30.	0

REFUELING INFORMATION - April, 1984

Name of Facility: Oyster Creek Station #1

Scheduled date for next refueling shutdown: Presently shutdown for Refueling

Scheduled date for restart following refueling: June 11, 1984

Will refueling or resumption of operation thereafter require a Technical Specification change or other license amendment?

NONE

Scheduled date(s) for submitting proposed licensing action and supporting information:

1. Amendment #7 to NEDO-24195: G.E. Reload Analysis incorporating LOCA analysis. (5/30/84)
2. Technical Specification Change Request 119 for Scram Discharge Instrument Volume System Modification. (5/15/84)

Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures:

1. General Electric Fuel Assemblies - fuel design and performance analysis methods have been approved by the NRC. New operating procedures, if necessary, will be submitted at a later date.
2. Exxon Fuel Assemblies - no major changes have been made nor are there any anticipated.

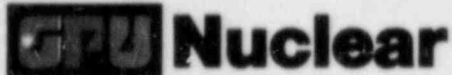
The number of fuel assemblies (a) in the core = 150
(b) in the spent fuel storage pool = 1313

The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies:

Present: 1,300 Planned: 2,600

The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity:

Full core offload capability will be lost after the 1985 outage. Batch discharge capability will be lost after the 1987 outage. Expanded spent fuel pool rack capacity (2,600) is scheduled for 1984.



GPU Nuclear Corporation
Post Office Box 388
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Forked River, New Jersey 08731-0388
609 971-4000
Writer's Direct Dial Number:

May 14, 1984

Director
Office of Management Information
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Dear Sir:

Subject: Oyster Creek Nuclear Generating Station
Docket No. 50-219
Monthly Operating Report

In accordance with the Oyster Creek Nuclear Generating Station Operating License No. DPR-16, Appendix A, Section 6.9.1.C, enclosed are two (2) copies of the Monthly Operating Data (gray book information) for the Oyster Creek Nuclear Generating Station.

If you should have any questions, please contact Mr. Drew Holland at (609) 971-4643.

Very truly yours,

Peter B. Fiedler
Vice President and Director
Oyster Creek

PBF:dam
Enclosures

cc: Director (10)
Office of Inspection and Enforcement
U.S. Nuclear Regulatory Commission
Washington, DC 20555

Dr. Thomas E. Murley, Administrator
Region I
U.S. Nuclear Regulatory Commission
631 Park Avenue
King of Prussia, PA 19406

NRC Resident Inspector
Oyster Creek Nuclear Generating Station
Forked River, NJ 08731

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