

OPERATING DATA REPORT

DOCKET No. 50-289
 DATE 1/13/92
 COMPLETED BY W G HEYSEK
 TELEPHONE (717) 948-8191

OPERATING STATUS

- 1. UNIT NAME: THREE MILE ISLAND UNIT 1
- 2. REPORTING PERIOD: DECEMBER 1991
- 3. LICENSED THERMAL POWER: 2568
- 4. NAMEPLATE RATING (GROSS MWe): 871
- 5. DESIGN ELECTRICAL RATING (NET MWe): 819
- 6. MAXIMUM DEPENDABLE CAPACITY (GROSS MWe): 856
- 7. MAXIMUM DEPENDABLE CAPACITY (NET MWe): 808

NOTES:

8. IF CHANGES OCCUR IN (ITEMS 3-7) SINCE LAST REPORT, GIVE REASONS:

- 9. POWER LEVEL TO WHICH RESTRICTED, IF ANY (NET MWe):
- 10. REASONS FOR RESTRICTIONS, IF ANY:

		THIS MONTH	YR-TO-DATE	CUMMULATIVE
11. HOURS IN REPORTING PERIOD	(HRS)	744.0	8760.0	151921.0
12. NUMBER OF HOURS REACTOR WAS CRITICAL	(HRS)	744.0	7566.8	76731.0
13. REACTOR RESERVE SHUTDOWN HOURS	(HRS)	0.0	-0.0	2245.6
14. HOURS GENERATOR ON-LINE	(HRS)	744.0	7536.5	75656.2
15. UNIT RESERVE SHUTDOWN HOURS	(HRS)	0.0	0.0	0.0
16. GROSS THERMAL ENERGY GENERATED	(MWH)	52658	1739871.4	183969045
17. GROSS ELECTRICAL ENERGY GENERATED	(MWH)	624088	6033355	61994259
18. NET ELECTRICAL ENERGY GENERATED	(MWH)	588817	5667789	58157325
19. UNIT SERVICE FACTOR	(%)	100.0	86.0	49.8
20. UNIT AVAILABILITY FACTOR	(%)	100.0	86.0	49.8
21. UNIT CAPACITY FACTOR (USING MDC NET)		97.9	80.1	48.7
22. UNIT CAPACITY FACTOR (USING DER NET)		96.6	79.0	46.7
23. UNIT FORCED OUTAGE RATE	(%)	0.0	0.7	44.5
UNIT FORCED OUTAGE HOURS	(HRS)	0.0	51.9	60648.7
24. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE AND DURATION OF EACH):				

25. IF SHUT DOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP:

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-289
 UNIT TMI-1
 DATE 1/13/92
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 TELEPHONE (717) 948-8191

MONTH: DECEMBER

DAY	AVERAGE DAILY POWER LEVEL (MWe-NET)	DAY	AVERAGE DAILY POWER LEVEL (MWe-NET)
1	807	17	821
2	814	18	820
3	814	19	821
4	817	20	821
5	780	21	817
6	818	22	817
7	817	23	818
8	815	24	820
9	808	25	820
10	815	26	822
11	815	27	818
12	389	28	815
13	507	29	813
14	809	30	813
15	817	31	816
16	820		

UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH December 1991

DOCKET NO. 50-289
 UNIT NAME TMI-1
 DATE 1/13/92
 COMPLETED BY W. G. Heysek
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No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	Licensee Event Report#	System Code * & *	Component Code * & *	Cause & Corrective Action to Prevent Recurrence
91-06	911212	F	0	A	NA	NA			Following a dropped rod during surveillance testing, power was reduced to 45% of full power for approximately 40 hours. The plant was returned to full power on completion of repairs to the control rod drive mechanism/cable electrical connection.

1
 F Forced
 S Scheduled

2
 Reason
 A - Equipment Failure (Explain)
 B - Maintenance or Test
 C - Refueling
 D - Regulatory Restriction
 E - Operator Training & Licensing Examination
 F - Administrative
 G - Operational Error (Explain)
 H - Other (Explain)

3
 Method
 1 - Manual
 2 - Manual Scram
 3 - Automatic Scram
 4 - Other (Explain)

4
 Exhibit G - Instructions for
 preparation of Data Entry Sheets
 for Licensee Event Report (LER)
 File (NUREG-0161)

5 Exhibit 1 same source

6 Actually used exhibits F & H NUREG 0161

REFUELING INFORMATION REQUEST

1. Name of Facility: Three Mile Island Nuclear Station, Unit 1
2. Scheduled date for next refueling shutdown: September 17, 1993 (10R)
3. Scheduled date for restart following current refueling: NA
4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment? NA

If answer is yes, in general, what will these be?

If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload (Ref. 10 CFR Section 50.59)?

If no such review has taken place, when is it scheduled?

5. Scheduled date(s) for submitting proposed licensing action and supporting information:

None planned.

6. Important licensing considerations associated with refueling; e.g. new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures:

GPU Nuclear has installed four Westinghouse Lead Test Assemblies during the reload of the TMI-1 core for cycle 9 operation. Westinghouse fuel technology will be utilized to the extent possible.

7. The number of fuel assemblies (a) in the core, and (b) in the spent fuel storage pool: (a) 177 (b) 521

8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies:

The present licensed capacity is 752. Planning to increase licensed capacity through fuel pool reracking is in progress.

9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity:

The 9R (1991) refueling discharge is the last which allows full core off-load capacity (177 fuel assemblies).