



A Centennial Energy Company

EDISON PLAZA
300 MADISON AVENUE
TOLEDO, OHIO 43652-0001

January 10, 1992
KB92-0079

Docket No. 50-346
License No. NPF-3

Document Control Desk
U.S. Nuclear Regulatory Commission
7920 Norfolk Avenue
Bethesda, MD 20814

Gentlemen:

Monthly Operating Report, December, 1991
Davis-Besse Nuclear Power Station Unit 1

Enclosed are ten copies of the Monthly Operating Report for Davis-Besse Nuclear Power Station Unit No. 1 for the month of December, 1991.

If you have any questions, please contact Bilal Sarsour at (419) 321-7384.

Very truly yours,

Louis F. Storz
Plant Manager
Davis-Besse Nuclear Power Station

BMS/tld

Enclosures

cc: Mr. A. Bert Davis
Regional Administrator, Region III

Mr. J. B. Hopkins
NRC Senior Project Manager

Mr. William Levis
NRC Resident Inspector

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AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-346

UNIT Davis-Besse #1

DATE January 10, 1992

COMPLETED BY Bilal Sarsour

TELEPHONE (419)321-7384

MONTH December, 1991

| DAY | AVERAGE DAILY POWER LEVEL (MWe-Net) | DAY | AVERAGE DAILY POWER LEVEL (MWe-Net) |
|-----|--|-----|--|
| 1 | <u>877</u> | 17 | <u>885</u> |
| 2 | <u>879</u> | 18 | <u>886</u> |
| 3 | <u>881</u> | 19 | <u>887</u> |
| 4 | <u>879</u> | 20 | <u>887</u> |
| 5 | <u>880</u> | 21 | <u>885</u> |
| 6 | <u>325</u> | 22 | <u>885</u> |
| 7 | <u>0</u> | 23 | <u>885</u> |
| 8 | <u>0</u> | 24 | <u>886</u> |
| 9 | <u>0</u> | 25 | <u>885</u> |
| 10 | <u>0</u> | 26 | <u>885</u> |
| 11 | <u>361</u> | 27 | <u>885</u> |
| 12 | <u>732</u> | 28 | <u>885</u> |
| 13 | <u>871</u> | 29 | <u>884</u> |
| 14 | <u>855</u> | 30 | <u>884</u> |
| 15 | <u>886</u> | 31 | <u>886</u> |
| 16 | <u>886</u> | | |

INSTRUCTIONS

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

OPERATING DATA REPORT

DOCKET NO. 50-346
 DATE January 10, 1992
 COMPLETED BY Bilal Sarsour
 TELEPHONE (419) 321-7384

OPERATING STATUS

1. Unit Name: Davis-Besse Unit #1
2. Reporting Period: December, 1991
3. Licensed Thermal Power (MWt): 2772
4. Nameplate Rating (Gross MWe): 925
5. Design Electrical Rating (Net MWe): 906
6. Maximum Dependable Capacity (Gross MWe): 918
7. Maximum Dependable Capacity (Net MWe): 874

| |
|-------|
| Notes |
|-------|

8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:

9. Power Level To Which Restricted, If Any (Net MWe):

10. Reasons For Restrictions, If Any:

| | This Month | Yr.-to-Date | Cumulative |
|--|------------|-------------|-------------|
| 11. Hours In Reporting Period | 744.0 | 8,760.0 | 117,649 |
| 12. Number of Hours Reactor Was Critical | 630.5 | 7,054.6 | 66,175.8 |
| 13. Reactor Reserve Shutdown Hours | 113.5 | 113.5 | 5,507.2 |
| 14. Hours Generator On-Line | 613.7 | 6,963.8 | 64,033.1 |
| 15. Unit Reserve Shutdown Hours | 0.0 | 0.0 | 1,732.5 |
| 16. Gross Thermal Energy Generated (MWH) | 1,681,828 | 18,482,806 | 156,609,293 |
| 17. Gross Electrical Energy Generated (MWH) | 562,258 | 6,163,360 | 51,893,377 |
| 18. Net Electrical Energy Generated (MWH) | 530,063 | 5,843,860 | 48,792,488 |
| 19. Unit Service Factor | 82.5 | 79.5 | 54.4 |
| 20. Unit Availability Factor | 82.5 | 79.5 | 55.9 |
| 21. Unit Capacity Factor (Using MDC Net) | 81.5 | 76.3 | 47.5 |
| 22. Unit Capacity Factor (Using DER Net) | 78.6 | 73.6 | 45.8 |
| 23. Unit Forced Outage Rate | 17.5 | 2.1 | 25.2 |
| 24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): | | | |

25. If Shut Down At End Of Report Period, Estimated Date of Startup:

26. Units In Test Status (Prior to Commercial Operation):

INITIAL CRITICALITY
 INITIAL ELECTRICITY
 COMMERCIAL OPERATION

Forecast

Achieved

UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-346UNIT NAME Davis-Besse #1DATE January 10, 1992COMPLETED BY Bilal SarsourTELEPHONE (419) 321-7384REPORT MONTH December, 1991

| No. | Date | Type ¹ | Duration (Hours) | Reason ² | Method of Shutting Down Reactor ³ | Licensee Event Report # | System Code ⁴ | Component Code ⁵ | Cause & Corrective Action to Prevent Recurrence |
|-----|----------|-------------------|---------------------|---------------------|--|-------------------------------|-----------------------------|--------------------------------|--|
| 5 | 91-12-06 | F | 130.3 | A | 1 | 91-007 | EK | SCO | The turbine-generator was taken off line due to exceeding the 72 hour time limit for Emergency Diesel Generator (EDG) inoperability per Technical Specification 3.8.1.1. See Operational Summary for further details. |

¹ F: Forced
S: Scheduled

² Reason:
A-Equipment Failure (Explain)
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error (Explain)
H-Other (Explain)

³ Method:
1-Manual
2-Manual Scram
3-Automatic Scram
4-Continuation from
Previous Month
5-Load Reduction
9-Other (Explain)

⁴ Exhibit G - Instructions for Preparation of Data
Entry Sheets for Licensee Event Report (LER)
File (NUREG-0161)

⁵ Exhibit I - Same Source
*Report challenges to Power Operated Relief Valves
(PORVs) and Pressurizer Code Safety Valves (PCSVs)

Operational Summary
December, 1991

Reactor power was maintained at approximately 100 percent full power until 0805 hours on December 6, 1991, when a manual power reduction was initiated to take the turbine off line. The turbine was taken off line at 1034 hours on December 6, 1991, and the reactor was shutdown to comply with Technical Specification 3.8.1.1, which requires both Emergency Diesel Generators (EDG) to be operable while at power. EDG Number 2 was declared inoperable on December 3, 1991, when the speed switch failed.

The reactor was critical at 0430 hours on December 11, 1991, and the Turbine-Generator was synchronized on line at 2057 hours on December 11, 1991.

Reactor power was slowly increased to approximately 75 percent of full power, which was achieved at 0400 hours on December 12, 1991. Reactor power was maintained at this power level for approximately four hours for conditioning of the fuel.

Reactor power escalation continued. Reactor power was slowly increased to approximately 100 percent full power, which was achieved at approximately 1700 hours on December 12, 1991, and maintained at this power level until 1859 hours on December 13, 1991, when a manual power reduction to approximately 95 percent was initiated to isolate Feedwater Heater 1-4 in order to repair a steam leak on instrument line.

After the completion of Feedwater Heater 1-4 steam leak repair, reactor power was slowly increased to approximately 100 percent full power, which was achieved at 1418 hours on December 14, 1991. Reactor power was maintained at 100 percent for the remainder of the month.

