		CONTROL BLOCK:IIIII (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)
		A R A N 0 2 2 2 0 0 - 0 0 0 0 0 0 0
		REPORT L 16 10 10 10 13 16 8 17 10 5 11 6 8 2 8 10 15 12 17 18 14 19 SOURCE 60 61 DOCKET NUMBER 68 69 EVENT DATE 74 75 REPORT DATE 80
	10121	EVENT DESCRIPTION AND PROBABLE CONSEQUENCES 10
	10131	IOn 5/16/82 while in Mode 1 at 100% full power, the breaker for 2CV-1075, discharged control valve from
	10141	Emergency Feedwater Pump (EFP) 2P-7B to the 'B' Steam Generator 2E-24B, was found to be tripped. The Valve was closed at the time of the failure and could not be opened remotely if emergency feedwater
	10151	lactuation were required through this flow path. However, the equipment required for the redundant flow
	1016.1	Ipath was proven operable. This occurrence is reportable per T.S. 6.9.1.9.b. Similar occurrences were
	10171	reported in LER's 50-368/79-054 and 78-009.
	10181	1
1		9 SYSTEM CODE CAUSE CODE CAUSE CODE CAUSE SUBCODE CAUSE SUBCODE COMPONENT CODE SUBCODE COMP SUBCODE VALVE SUBCODE 80 1 1 1 1 1 1 1 1 1 1 1 80 1 1 1 1 1 1 1 1 1 1 1 80 1
1	4 1 3	NUMBER 1_{21} 22 23 24 24 26 27 180 31 11 $12-1$ 11 12 ACTIONFUTUREEFFECTSK'ITDOWN METHODATTACHMENTNPRD-4 SUBMITTEDPRIME COMP. SUBPLIERCOMPONENT MANUFACTURERA18 12 19 12 12 10 10 10 122 1 14 123 14 125 1 112 17 126 33 34 35 36 37 10 10 10 122 1 123 14 125 1 112 17 126 CAUSE DESCRIPTION AND CORRECTIVE ACTIONS 27
		The occurrence was caused when the hydraulic pump motor bearings associated with the valve operator failed
		I due to high ambient temperature in the room where 2CV-1075 is located. The hydraulic pump motor was
1	$\frac{1}{1}$	Ireplaced. The valve was proven operable and returned to service. Damper adjustments were made to assist
		Icooling in the room. This Weston-Hydraulic valve actuator was replaced with a Limitorque SMB-000-2 460V A.C. I
	7 8	Imotor operator during the 2R3 refueling outage. I 9 FACILITY METHOD OF 80 STATUS % POWER OTHER STATUS DISCOVERY DISCOVERY DESCRIPTION 1 1 0 1 0 1 1 9 10 12 13 44 45 46 ACTIVITY CONTENT
	$\frac{1}{7}$	RELEASE OF RELEASE AMOUNT OF ACTIVITY LOCATION OF RELEASE 1 135 1 NA 9 10 11 44 45 9 10 12 80
1		NUMBER TYPE DESCRIPTION 9 1
	8 P	ERSONNEL INJURIES
17		NUMBER DESCRIPTION I 0 0 0 141 9 11 12 141 LOSS OF OR DAMAGE TO FACILITY 80
1	1191	TYPE DESCRIPTION 1 Z 142 1 NA [43
17	<u>2</u> 1 <u>0</u> 1 8	PUBLICITY NRC USE ONLY ISSUED DESCRIPTION I NRC USE ONLY 9 10 68 69 69 69
		NAME OF PREPARER:Patrick RogersPHONE: (501) 364-3100
		В406050494 В40525 PDR ADDCK 05000368 S PDR



ARKANSAS POWER & LIGHT COMPANY POST OFFICE BOX 551 LITTLE ROCK, ARKANSAS 72203 (501) 371-4000

May 25, 1984

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U. S. Nuclear Regulatory Commission Document Control Desk Washington, D.C. 20555

> Subject: Arkansas Nuclear One - Unit 2 Docket No. 50-368 License No. NPF-6 Licensee Event Report No. 82-018/03X-1

Gentlemen:

In accordance with Arkansas Nuclear One - Unit 2 Technical Specification 6.9.1.9.b, attached is the subject report concerning a failure of the emergency feedwater pump discharge control valve breaker.

Very truly yours,

John R. Marshall Manager, Licensing

IE-22

11,

JRM: RJS: ac

Attachment

cc: Mr. Richard P. Denise, Director Division of Resident Reactor Projects and Engineering Programs U. S. Nuclear Regulatory Commission Region IV 611 Ryan Plaza Drive, Suite 1000 Arlington, TX 76011