UPDATE REPORT-PREVIOUS REPORT

DATE 08/26/82
CONTROL BLOCK:
0 1 G A E I H 1 2 0 0 - 0 0 0 0 - 0 0 3 4 1 1 1 1 1 4 5 6 6 1 CENSE CODE 14 15 LICENSE NUMBER 25 26 LICENSE TYPE 30 57 CAT 58
CON'T
While the unit was operating steady-state at 2392 MWT, the RWCU system
primary containment outboard isolation valve failed to close after
o 4 receiving an isolation signal from the "RWCU System Leak Alarm". This
event is contrary to Tech. Specs. 3.7.A.2.b. and c. The inboard valve was
closed as per the LCO required by Tech. Specs. 3.7.D.2. The health and
safety of the public were not affected. This is a non-repetitive event.
7 8 9
SYSTEM CODE CAUSE CAUSE SUBCODE SUBCOD
17 LER/RO EVENT YEAR SEQUENTIAL REPORT NO. OCCURRENCE CODE TYPE NO. NO
ACTION FUTURE COMPONENT SUBMITTED FORM SUB, PRIME COMP. COMPONENT MANUFACTURER B 18 Z 19 Z 20 Z 21 O O O O O V Y 23 EN COMPONENT MANUFACTURER B 18 Z 19 Z 20 Z 21
CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27) This event was caused by material failure. On 02-06-83 with the unit in
cold shutdown, maintenance personnel determined that the primary
containment valve's operator was at fault. The valve operator was
repaired and returned to service per the "PRIMARY CONTAINMENT ISOLATION
VALVE OPERABILITY" procedure on 02-08-83 (refer to attached narratize).
7 8 9 FACILITY STATUS STATUS STATUS STATUS OTHER STATU
7 8 9 10 12 13 44 45 46 80
RELEASED OF RELEASE AMOUNT OF ACTIVITY (35) NA LOCATION OF PRICEASE (36) NA N
PERSONNEL EXPOSURES NUMBER TYPE DESCRIPTION (39) 1 1 2 0 0 0 0 0 0 0 0 NA
7 8 9 PERSONNEL INJURIES PDR ADOCK 05000321 PDR PDR PDR
1 B O O O O O
TYPE DESCRIPTION NA TYPE DESCRIPTION NA TE27 1 9 Z 42 NA
PUBLICITY ISSUED DESCRIPTION 45 NA NA NA NA
7 8 9 10 68 69 80 5 NAME OF PREPARES S. B. Tipps (912)367-7851

NARRATIVE REPORT FOR LER 50-321/1982-073, REV.1 UPDATE REPORT-PREVIOUS REPORT DATE 08/26/82

LICENSEE : GEORGIA POWER COMPANY

FACILITY NAME : EDWIN I. HATCH

DOCKET NUMBER : 50-321

Tech. Specs. section(s) which requires report:

This 30 day LER is required by Tech. Specs. section 6.9.1.9.b, because it showed that the unit did not meet the requirements of Tech. Specs. sections 3.7.A.2.b and c., and 3.7.D.1.

Plant conditions at the time of the event(s):

The unit was in steady state operation at 2392 MWt (approximately 98% power) when this event occurred.

Detailed description of the event(s):

On O8-08-82, the reactor water cleanup outboard isolation value (1G31-F004) failed to close when it received an isolation signal from the "Clean Up System Leak" alarm. This event is contrary to the requirements of Tech. Specs. sections 3.7.A.2.b and c. and Tech. Specs. section 3.7.D.1.

Consequences of the event(s):

This event did not affect plant operation. The health and safety of the public were not affected by this event.

Status of redundant or backup subsystems and/or systems:

Inboard primary containment isolation valve 1G31-F001, was operable during this event.

Justification for continued operation:

The inboard primary containment isolation valve (1G31-F001) closed when it received the isolation signal. It (1G31-F001) was then isolated in its closed position, and plant operation continued as permitted by Tech. Specs. section 3.7.D.2.

If repetitive, umber of previous LER:

This event is non-resetitive.

Narrative Report for LER 50-321/1982-073, REV. 1 Page Two

Impact to other systems and/or Unit:

This event had no impact upon any other system in Unit 1 or Unit 2.

Cause(s) of the event(s):

Maintenance personnel determined that the valve operator would not close the valve (1G31-F004) because its clutch was disengaged. The clutch would not engage when the valve operator's motor was run. After furthur investigation, it was determined that the valve operator's tripper levers (which engage the clutch) were not functioning because the tripper adjustment arm was out of adjustment.

Immediate Corrective Action:

The reactor water cleanup system's outboard isolation valve (1G31-F0O4) operator's tripper adjustment arm was adjusted to allow the tripper levers to operate and engage the operator's clutch when the valve operator's motor was run.

The valve was then satisfactorily functionally tested per the "PRIMARY CONTAINMENT ISOLATION VALUE OPERABILITY" procedure (HNP-1-3962) and returned to service on O2-08-83.

Supplemental Corrective Action:

No supplemental corrective action is required.

Scheduled (future) corrective action:

N/A

f

Action to prevent recurrence (if different from corrective actions):

N/A



Edwin I. Hatch Nuclear Plant

June 1, 1984 GM-84-405

PLANT E. I. HATCH Licensee Event Report Docket No. 50-321

United States Nuclear Regulatory Commission Document Control Desk Washington, D.C. 20555

ATTENTION: Mr. James P. O'Reilly

Attached is Licensee Event Report No. 50-321/1982-073, Rev. 1. This report is required by Hatch Unit One Technical Specifications section 6.9.1.9.b.

H. C. Mix General Manager

HCN/SBT/ult

XC:

R. J. Kelly

R. E. Conway

J. T. Beckham, Jr.

P. D. Rice

K. M. Gillespie

S. B. Tipps

R. D. Baker

Control Room

Document Control

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