

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Sequoyah, Unit 1	DOCKET NUMBER (2) 0 5 0 0 0 3 2 7	PAGE (3) 1 OF 0 2
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TITLE (4)
Pressurizer Safety Relief Valve Inoperable

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)																																																																																																																											
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)																																																																																																																									
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<table border="1" style="width:100%; border-collapse: collapse;"> <tr> <td style="width:15%;">OPERATING MODE (9) 3</td> <td colspan="11">THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR § (Check one or more of the following) (11)</td> </tr> <tr> <td rowspan="5">POWER LEVEL (10) 0 1 0 0</td> <td>20.402(b)</td> <td>20.406(a)(1)(i)</td> <td>20.406(a)(1)(ii)</td> <td>20.406(a)(1)(iii)</td> <td>20.406(a)(1)(iv)</td> <td>20.406(a)(1)(v)</td> <td>20.406(c)</td> <td>50.38(c)(i)</td> <td>50.38(c)(2)</td> <td>50.73(a)(2)(i)</td> <td>50.73(a)(2)(ii)</td> <td>50.73(a)(2)(iii)</td> <td>50.73(a)(2)(iv)</td> <td>50.73(a)(2)(v)</td> <td>50.73(a)(2)(vii)</td> <td>50.73(a)(2)(viii)(A)</td> <td>50.73(a)(2)(viii)(B)</td> <td>50.73(a)(2)(x)</td> <td>73.71(b)</td> <td>73.71(c)</td> <td>OTHER (Specify in Abstract below and in Text, NRC Form 366A)</td> </tr> <tr> <td></td><td></td><td></td><td></td><td></td><td></td><td></td><td></td><td></td><td></td><td></td><td></td><td></td><td></td><td></td><td></td><td></td><td></td><td></td><td></td><td></td> </tr> <tr> <td></td><td></td><td></td><td></td><td></td><td></td><td></td><td></td><td></td><td></td><td></td><td></td><td></td><td></td><td></td><td></td><td></td><td></td><td></td><td></td><td></td><td></td> </tr> <tr> <td></td><td></td><td></td><td></td><td></td><td></td><td></td><td></td><td></td><td></td><td></td><td></td><td></td><td></td><td></td><td></td><td></td><td></td><td></td><td></td><td></td><td></td> </tr> <tr> <td></td><td></td><td></td><td></td><td></td><td></td><td></td><td></td><td></td><td></td><td></td><td></td><td></td><td></td><td></td><td></td><td></td><td></td><td></td><td></td><td></td><td></td> </tr> </table>												OPERATING MODE (9) 3	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR § (Check one or more of the following) (11)											POWER LEVEL (10) 0 1 0 0	20.402(b)	20.406(a)(1)(i)	20.406(a)(1)(ii)	20.406(a)(1)(iii)	20.406(a)(1)(iv)	20.406(a)(1)(v)	20.406(c)	50.38(c)(i)	50.38(c)(2)	50.73(a)(2)(i)	50.73(a)(2)(ii)	50.73(a)(2)(iii)	50.73(a)(2)(iv)	50.73(a)(2)(v)	50.73(a)(2)(vii)	50.73(a)(2)(viii)(A)	50.73(a)(2)(viii)(B)	50.73(a)(2)(x)	73.71(b)	73.71(c)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)																																																																																							
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LICENSEE CONTACT FOR THIS LER (12)

NAME Glenn B. Kirk, Compliance Section Engineer	TELEPHONE NUMBER 6 1 5 8 7 0 - 6 1 4 6
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COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS
X	A B R	V 1 - -	C 7 1 0	Y					
X	B A L	C V - M	1 2 0	Y					

SUPPLEMENTAL REPORT EXPECTED (14)

YES (If yes, complete EXPECTED SUBMISSION DATE) NO

EXPECTED SUBMISSION DATE (15)

MONTH	DAY	YEAR

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

At 1800 CST on 05/05/84, unit 1 initiated a plant shutdown required by technical specifications due to an inoperable pressurizer safety relief valve. The unit was in mode 3 (547 degrees F, 2235 psig) just prior to initiation of shutdown. On 05/06/84 at 0117 CST with the unit in mode 4 (330 degrees F, 600 psig), a reactor trip occurred on low-low steam generator level in the number 1 steam generator.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

FACILITY NAME (1) Sequoyah, Unit 1	DOCKET NUMBER (2) 0 5 0 0 0 3 2 7 8 4	LER NUMBER (6)			PAGE (3)	
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER		
			0 3 1	0 0	0 2	OF 0 2

TEXT (If more space is required, use additional NRC Form 366A's) (17)

At 1725 CST on 05/05/84 with unit 1 in mode 3 (547 degrees F, 2235 psig), pressurizer safety relief valve 1-SRV-68-565 lifted. The required lift setpoint of the valve is 2485 psig \pm 1%. Plant shutdown to mode 4 as required by technical specification LCO 3.4.3.1 was initiated at 1800 CST.

Later on 05/06/84 at 0117 CST with the unit in mode 4 (330 degrees F, 600 psig), steam generator auxiliary feedwater bypass level control valve 1-LCV-3-164A failed to open sufficiently to maintain level in the number one steam generator resulting in actuation of an automatic reactor trip on low-low level in the number one steam generator.

The premature lifting of the pressurizer safety relief valve has been determined to be due to seat leakage. The spring force calculations to maintain closure of the relief valve are based on cross sectional area of the seat and zero leakage. If leakage occurs, the cross sectional area is effectively increased allowing a lower system pressure to lift the valve. The Crosby valve Model No. GM6-HB-BP-86 was replaced and the failed valve was shipped to Wyle Labs for repair and inspection.

Inspection of the level control valve 1-LCV-3-164A revealed a failed current to pressure converter in the valve control instrumentation. The converter was replaced, and the valve was returned to service.

All other equipment and personnel responded as expected during the event. There was no effect on public health or safety.

For 1984, this was the first occurrence of a pressurizer safety relief valve lifting prematurely. There have been four previous reactor trips on low-low steam generator level, but this is the first occurrence in mode 4 and caused by a failed level control valve.

TENNESSEE VALLEY AUTHORITY

Sequoyah Nuclear Plant
Post Office Box 2000
Soddy Daisy, Tennessee 37379

June 1, 1984

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

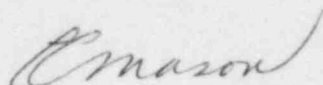
Gentlemen:

TENNESSEE VALLEY AUTHORITY - SEQUOYAH NUCLEAR PLANT UNIT 1 - DOCKET NO.
50-327 - FACILITY OPERATING LICENSE DPR- / - REPORTABLE OCCURRENCE REPORT
SQRO-50-327/84031

The enclosed licensee event report provides details concerning a required plant shutdown following premature lifting of a pressurizer safety relief valve. This event is reported in accordance with 10 CFR 50.73, paragraph a.2.i and a.2.iv.

Very truly yours,

TENNESSEE VALLEY AUTHORITY



C. C. Mason
Power Plant Superintendent

Enclosure
cc (Enclosure):

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NRC Inspector, NUC PR, Sequoyah

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