

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Sequoyah, Unit 1	DOCKET NUMBER (2) 0 5 0 0 0 3 2 7	PAGE (3) 1 OF 0 2
---------------------------------------	--------------------------------------	----------------------

TITLE (4)
Reactor Trip

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)							
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)					
0	5	10	8	4	8	4	0	3	2	0	0	0	5	0	0	0
0	5	10	8	4	8	4	0	3	2	0	0	0	5	0	0	0

OPERATING MODE (9) 1	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)									
POWER LEVEL (10) 0 3 0	<input type="checkbox"/> 20.402(b)	<input type="checkbox"/> 20.405(c)	<input checked="" type="checkbox"/> 50.73(a)(2)(iv)	<input type="checkbox"/> 73.71(b)						
	<input type="checkbox"/> 20.405(a)(1)(i)	<input type="checkbox"/> 50.36(c)(1)	<input type="checkbox"/> 50.73(a)(2)(v)	<input type="checkbox"/> 73.71(e)						
	<input type="checkbox"/> 20.405(a)(1)(ii)	<input type="checkbox"/> 50.36(c)(2)	<input type="checkbox"/> 50.73(a)(2)(vii)	OTHER (Specify in Abstract below and in Text, NRC Form 365A)						
	<input type="checkbox"/> 20.405(a)(1)(iii)	<input type="checkbox"/> 50.73(a)(2)(i)	<input type="checkbox"/> 50.73(a)(2)(viii)(A)							
	<input type="checkbox"/> 20.405(a)(1)(iv)	<input type="checkbox"/> 50.73(a)(2)(ii)	<input type="checkbox"/> 50.73(a)(2)(viii)(B)							
<input type="checkbox"/> 20.405(a)(1)(v)	<input type="checkbox"/> 50.73(a)(2)(iii)	<input type="checkbox"/> 50.73(a)(2)(ix)								

LICENSEE CONTACT FOR THIS LER (12)		TELEPHONE NUMBER	
NAME Glenn B. Kirk, Compliance Section Engineer		AREA CODE 6 1 5	
		8 7 0 - 6 1 4 6	

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)										
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPROS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPROS	

SUPPLEMENTAL REPORT EXPECTED (14)			EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
<input type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE)	<input checked="" type="checkbox"/> NO					

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On 05/10/84 at 2322 CST, with unit 1 in mode 1 (553 degrees F, 2240 psig) at 30% reactor power, a reactor trip occurred on low feedwater flow coincident with low steam generator level in the number 2 steam generator.

8406050386 840601
PDR ADOCK 05000327
S PDR

IE 22
//

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

FACILITY NAME (1) Sequoyah, Unit 1	DOCKET NUMBER (2) 0 5 0 0 0 3 2 7 8 4 -	LER NUMBER (6)			PAGE (3)	
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER		
		0 3 2	-	0 0	0 2	OF 0 2

TEXT (If more space is required, use additional NRC Form 366A's) (17)

On 05/10/84 at 2225 CST, the number 2 steam generator feedwater flow transmitter, 1-FT-3-48A, was declared inoperable and its associated bistables were tripped at 2309 CST as required by technical specifications. The low pressure feedwater heaters later isolated due to high level resulting in low pressure suction to the 1A main feedwater pump. With the bistables to 1-FT-3-48A tripped, one-half of the automatic trip logic for low feedwater flow coincident with low steam generator level was already made up on the number 2 steam generator. With unit 1 in mode 1 at 30% reactor power (553 degrees F, 2240 psig), the reactor tripped on low feedwater flow coincident with low steam generator level at 2322 CST when the number 2 steam generator level fell to 25%. All other equipment and personnel responded as expected during the event.

Investigation of the low pressure feedwater heaters high level isolations revealed a leaking air line supplying the number 7 heater drain tank level control valve, 1-LCV-6-109A. Investigation of the feedwater flow transmitter 1-FT-3-48A failure revealed dirty contacts in the loop square root converter. The air line to 1-LCV-6-109A was repaired, and the square root converter contacts were cleaned.

There was no effect on public health or safety. This has been the first reactor trip on low feedwater flow coincident with low steam generator level for 1984.

TENNESSEE VALLEY AUTHORITY

Sequoyah Nuclear Plant
Post Office Box 2000
Soddy Daisy, Tennessee 37379

June 1, 1984

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

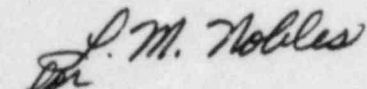
Gentlemen:

TENNESSEE VALLEY AUTHORITY - SEQUOYAH NUCLEAR PLANT UNIT 1 - DOCKET NO.
50-327 - FACILITY OPERATING LICENSE DPR-77 - REPORTABLE OCCURRENCE REPORT
SQRO-50-327/84032

The enclosed licensee event report provides details concerning a reactor trip from 30% reactor power. This event is reported in accordance with 10 CFR 50.73, paragraph a.2.iv.

Very truly yours,

TENNESSEE VALLEY AUTHORITY


C. C. Mason

Power Plant Superintendent

Enclosure
cc (Enclosure):

James P. O'Reilly, Director
U.S. Nuclear Regulatory Commission
Suite 2900
101 Marietta Street, NW
Atlanta, Georgia 30303

Records Center
Institute of Nuclear Power Operations
Suite 1500
1100 Circle 75 Parkway
Atlanta, Georgia 30339

NRC Inspector, NUC PR, Sequoyah

IE22
/