

CONTROL BLOCK: [ ] [ ] [ ] [ ] [ ] [ ] [ ] [ ] [ ] [ ] (1) (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

0 1 M D C C N 2 2 0 0 - 0 0 0 0 0 0 - 0 0 0 3 4 1 1 1 1 4 [ ] [ ] 5  
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40

CON'T

0 1 REPORT SOURCE L 6 0 5 0 0 0 0 3 1 8 7 0 8 1 5 8 3 8 0 5 3 0 8 4 9  
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

0 2 During normal operations, 2-CV-4071 main steam supply valve to the  
0 3 steam driven auxiliary feedwater (AFW) pumps failed open causing  
0 4 21 AFW pump to start. To repair the valve, the auto initiate capa-  
0 5 bility for the steam driven AFW pumps was defeated (T.S. 3.7.1.2).  
0 6 The AFW system was returned to a normal line-up 14 hours after  
0 7 being declared inoperable. The motor driven pump remained operable  
0 8 throughout the event. Similar events: 83-035.

0 9 SYSTEM CODE [ C ] [ H ] (11) CAUSE CODE [ X ] (12) CAUSE SUBCODE [ Z ] (13) COMPONENT CODE [ V ] [ A ] [ L ] [ V ] [ E ] [ X ] (14) COMP. SUBCODE [ F ] (15) VALVE SUBCODE [ D ] (16)  
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40  
17 LER/RO REPORT NUMBER [ R ] [ 3 ] (21) SEQUENTIAL REPORT NO. [ 0 ] [ 3 ] [ 9 ] (24) OCCURRENCE CODE [ / ] (27) REPORT TYPE [ I ] (30) REVISION NO. [ ] (32)  
18 ACTION TAKEN [ A ] (18) FUTURE ACTION [ Z ] (19) EFFECT ON PLANT [ Z ] (20) SHUTDOWN METHOD [ Z ] (21) HOURS [ 0 ] [ 0 ] [ 0 ] [ 0 ] (22) ATTACHMENT SUBMITTED [ Y ] (23) NFRD-4 FORM SUB. [ Y ] (24) PRIME COMP. SUPPLIER [ X ] (25) COMPONENT MANUFACTURER [ F ] [ 1 ] [ 3 ] [ 0 ] (26)

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

1 0 The opening of 2-CV-4071 was due to failure of the actuator dia-  
1 1 phragm. Diaphragm was replaced on 8-15-83. Further investigation re-  
1 2 vealed diaphragm plate nut should be torqued to 400 ft/lb and could  
1 3 cause diaphragm failure if improperly torqued. The subject nut torque  
1 4 was checked on all type 657NS actuators during May, 1984 shutdown.

1 5 FACILITY STATUS [ E ] (28) % POWER [ 1 ] [ 0 ] [ 0 ] (29) OTHER STATUS [ N/A ] (30) METHOD OF DISCOVERY [ A ] (31) DISCOVERY DESCRIPTION [ Operator observation ] (32)

1 6 ACTIVITY CONTENT RELEASED OF RELEASE [ Z ] (33) AMOUNT OF ACTIVITY [ N/A ] (35) LOCATION OF RELEASE [ N/A ] (36)

1 7 PERSONNEL EXPOSURES NUMBER [ 0 ] [ 0 ] [ 0 ] (37) TYPE [ Z ] (38) DESCRIPTION [ N/A ] (39)

1 8 PERSONNEL INJURIES NUMBER [ 0 ] [ 0 ] [ 0 ] (40) DESCRIPTION [ N/A ] (41)

1 9 LOSS OF OR DAMAGE TO FACILITY TYPE [ Z ] (42) DESCRIPTION [ N/A ] (43)

2 0 PUBLICITY ISSUED DESCRIPTION [ N ] (44) DESCRIPTION [ N/A ] (45)

8406050273 840530  
PDR ADOCK 05000318  
S PDR

NAME OF PREPARER P.A. Pieringer/W.T. Lyons PHONE: (301) 269-4742/4861

IE22

LER NO.: 83-39/3L  
Docket No.: 50-318  
License No.: DPR 69  
Event Date: 08-15-83  
Report Date: 05-30-84  
Attachment

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (CONT'D)

On August 15, 1983, 2-CV-4071 (steam supply from number 22 Steam Generator to number 21 Steam Driven Auxiliary Feed Pump) failed open causing number 21 Auxiliary Feedwater Pump to automatically start. (Number 22 Auxiliary Feedwater Pump was in standby.)

The diaphragm was replaced and the valve stroked satisfactorily. Number 21 Auxiliary Feedwater Pump was aligned for automatic initiation at 1750 on August 15, 1983, terminating the event.

Upon further investigation it was revealed that the diaphragm plate nut on 2-CV-4071 was loose when disassembled and when the new diaphragm was installed it was torqued to an unknown value. The subject nut should be torqued to 400 ft./lb. per Fisher Controls.

During the shutdown of both reactors in May 1984 all valves with type 657NS actuators were disassembled, inspected, and reassembled with all nuts being torqued to their correct values under the supervision of the vendor's technical representative.

# BALTIMORE GAS AND ELECTRIC COMPANY

P.O. BOX 1475  
BALTIMORE, MARYLAND 21203

NUCLEAR POWER DEPARTMENT  
CALVERT CLIFFS NUCLEAR POWER PLANT  
LUSBY, MARYLAND 20657

May 31, 1984

U. S. Nuclear Regulatory Commission  
Document Control Desk  
Washington, DC 20555

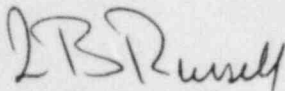
Docket No. 50-318  
License No. DPR 69

Dear Sirs:

The attached revision to LER 83-39/3L is being forwarded to you for your information.

Should you have any questions regarding this report, we would be pleased to discuss them with you.

Very truly yours,



L. B. Russell  
Plant Superintendent

~~WFL~~  
LBR:WFL:srm

cc: Dr. Thomas E. Murley  
Director, Office of Management Information  
and Program Control  
Messrs: A. E. Lundvall, Jr.  
J. A. Tiernan

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