NRC FORM 366 (12-81)

U.S. NUCLEAR REGULATORY COMMISSION

LICENSEE EVENT REPORT

APPROVED BY OMB 3150-0011 EXPIRES 4-30-82

CONTROL BLOCK: (1) (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

0 1	M D C C N 2 (2) LICENSEE CODE	0 0 - 0 0 0 0 0 - 0 0 (3) LICENSE NUMBER	4 1 1 1 1 (4) LICENSE TYPE	CAT (5)				
CON'T 0 1	REPORT SOURCE L (6)	0 5 0 0 0 3 1 8 (7) DOCKET NUMBER	0 8 1 5 8 3 (8) EVENT DATE	0 5 3 0 8 4 (9) REPORT DATE				

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

0 2 During normal operations, 2-CV-4071 main steam supply valve to the
0 3 steam driven auxiliary feedwater (AFW) pumps failed open causing
0 4 21 AFW pump to start. To repair the valve, the auto initiate capa-
0 5 bility for the steam driven AFW pumps was defeated (T.S. 3.7.1.2).
0 6 The AFW system was returned to a normal line-up 14 hours after
0 7 being declared inoperable. The motor driven pump remained operable
0 8 throughout the event. Similar events: 83-035.

	SYSTEM CODE	CAUSE CODE	CAUSE SUBCODE	COMPONENT CODE	COMP. SUBCODE	VALVE SUBCODE
0 9	C H (11)	X (12)	Z (13)	V A L V E X (14)	F (15)	D (16)

(17) LER/RO REPORT NUMBER	EVENT YEAR		SEQUENTIAL REPORT NO.		OCCURRENCE CODE	REPORT TYPE		REVISION NO.
	8 3	—	0 3 9	/	0 3	L	—	1

ACTION TAKEN	FUTURE ACTION	EFFECT ON PLANT	SHUTDOWN METHOD	HOURS (22)	ATTACHMENT SUBMITTED	NPRD-4 FORM SUB.	PRIME COMP. SUPPLIER	COMPONENT MANUFACTURER (26)
A (18)	Z (19)	Z (20)	Z (21)	0 0 0 0	Y (23)	Y (24)	X (25)	F 1 3 0

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

1 0 The opening of 2-CV-4071 was due to failure of the actuator dia-
1 1 phragm. Diaphragm was replaced on 8-15-83. Further investigation re-
1 2 vealed diaphragm plate nut should be torqued to 400 ft/lb and could
1 3 cause diaphragm failure if improperly torqued. The subject nut torque
1 4 was checked on all type 657NS actuators during May, 1984 shutdown.

	FACILITY STATUS	% POWER	OTHER STATUS (30)	METHOD OF DISCOVERY	DISCOVERY DESCRIPTION (32)
1 5	E (28)	1 0 0 (29)	N/A	A (31)	Operator observation

	ACTIVITY RELEASED	CONTENT OF RELEASE	AMOUNT OF ACTIVITY (35)	LOCATION OF RELEASE (36)
1 6	Z (33)	Z (34)	N/A	N/A

PERSONNEL EXPOSURES

	NUMBER	TYPE	DESCRIPTION (39)
1 7	0 0 0 (37)	Z (38)	N/A

PERSONNEL INJURIES

	NUMBER	DESCRIPTION (41)
1 8	0 0 0 (40)	N/A

LOSS OF OR DAMAGE TO FACILITY (43)

	TYPE	DESCRIPTION
1 9	Z (42)	N/A

8406050273 840530
PDR ADOCK 05000318
S PDR

PUBLICITY

	ISSUED	DESCRIPTION (45)	NRC USE ONLY
2 0	N (44)	N/A	

NAME OF PREPARER P.A. Pieringer/W.T. Lyons PHONE (301) 269-4742/4861

IE22

LER NO.: 83-39/3L
Docket No.: 50-318
License No.: DPR 69
Event Date: 08-15-83
Report Date: 05-30-84
Attachment

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (CONT'D)

On August 15, 1983, 2-CV-4071 (steam supply from number 22 Steam Generator to number 21 Steam Driven Auxiliary Feed Pump) failed open causing number 21 Auxiliary Feedwater Pump to automatically start. (Number 22 Auxiliary Feedwater Pump was in standby.)

The diaphragm was replaced and the valve stroked satisfactorily. Number 21 Auxiliary Feedwater Pump was aligned for automatic initiation at 1750 on August 15, 1983, terminating the event.

Upon further investigation it was revealed that the diaphragm plate nut on 2-CV-4071 was loose when disassembled and when the new diaphragm was installed it was torqued to an unknown value. The subject nut should be torqued to 400 ft./lb. per Fisher Controls.

During the shutdown of both reactors in May 1984 all valves with type 657NS actuators were disassembled, inspected, and reassembled with all nuts being torqued to their correct values under the supervision of the vendor's technical representative.

BALTIMORE GAS AND ELECTRIC COMPANY

P.O. BOX 1475

BALTIMORE, MARYLAND 21203

NUCLEAR POWER DEPARTMENT
CALVERT CLIFFS NUCLEAR POWER PLANT
LUSBY, MARYLAND 20657

May 31, 1984

U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

Docket No. 50-318
License No. DPR 69

Dear Sirs:

The attached revision to LER 83-39/3L is being forwarded to you for your information.

Should you have any questions regarding this report, we would be pleased to discuss them with you.

Very truly yours,

L. B. Russell
Plant Superintendent

LBR:WTL:srm

cc: Dr. Thomas E. Murley
Director, Office of Management Information
and Program Control
Messrs: A. E. Lundvall, Jr.
J. A. Tiernan

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