NRC FORM 366 (12-81)	LICENSEE EVENT REPORT	APPROVED BY OME 3150-0011 EXPIRES 4-30-82
CONTROL BLOCK:	(PLEASE PRINT OR TYPE ALL F	REQUIRED INFORMATION)
0 1 M D C C N 2 2 0 13	0 - 0 0 0 0 0 - 0 0 3 4 1	1 1 1 1 4 57 CAT 56 5
CON'T  O 1  SOURCE L 60  61  DOG	0 0 3 1 8 7 0 9 2 2 8 3 8 KET NUMBER 69 EVENT DATE 74	0   5   3   0   8   4   9
EVENT DESCRIPTION AND PROBABL  1 During normal operation	econsequences (10) ons, 2-CV-4070 Main Steam Supply Valve	to the
0 3 steam driven Auxiliary	Feedwater (AFW) pumps failed to shut	following
0 4   surveillance testing.	To repair the valve, auto initiate c	apability
	FW pumps was defeated (T.X.3.7.1.2).	
	a normal lineup 13.5 hours after bei	
	driven pump remained operable throug	Hout the
0 8 event. Similar events		80
SYSTEM CAUSE	SUBCODE COMPONENT CODE SUBCO	DE SUBCODE
O 9 C H W X	12 Z 13 V AL V E X 14 F	15) [D] (16)
17 REPORT NUMBER 21 22 23	24 26 27 28 29 30	1 1 12
	ARTHOD HOURS 22 ATTACHMENT PORM SUB.  Z 21 0 0 0 0 Y 23 Y 24  36 37 40 41 41 42	SUPPLIER MANUFACTURER
CAUSE DESCRIPTION AND CORRECT  1 0   The valve actuator was	rIVE ACTIONS (2) s disassembled and the shaft hole in t	he center
of the diaphragm was f	found to be oversized. A new diaphrag	m with
1 2 the correct shaft hole	e size was installed, and the actuator	reinstalled
1 3 on the valve. During	the May, 1984 shutdown all valves wit	h this type
14 actuator were inspecte	ed and diaphragms replaced with ones w	with correct hole sizes
FACILITY STATUS OT STATUS OF POWER OF	1 10101 6	
ACTIVITY CONTENT RELEASE OF RELEASE AMOUNT OF	0 1	OF RELEASE 36
7 6 9 10 11 PERSONNEL EXPOSURES NUMBER TYPE DESCRIPTION	on (39)	30
17 0 0 0 37 Z 38 13	N/A	10
1 8 0 0 0 40 N/A	A	80
LOSS OF OR DAMAGE TO FACILITY 43	N/A	
PUBLICITY ISSUED DESCRIPTION (45)		NRC USE ONLY
20 N 49 N/		(201) 260 4742/4861
8406050226 840520	P. A. Pieringer/W.T. Lyons	(301) 269-4742/4861
PDR ADOCK 05000318		Albertain Probe

LER NO. 83-50/3L
DOCKET NO.: 50-318
LICENSE NO.: DPR 69
EVENT DATE: 09-22-83
REPORT DATE: 10-13-83

ATTACHMENT

## CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (CONT'D)

On September 22, 1983 at 0345, 2-CV-4070 (steam supply from number 22 Steam Generator to the Steam Driven Auxiliary Feedwater Pumps) failed to shut following surveillance testing. The auto initiate capability for the AFW pumps was defeated (T.S.3.7.1.2) and the valve actuator was removed from the valve and repaired. It was reinstalled and the valve was cycled satisfactorily. The system was returned to service at 1708 on September 22, 1983.

Upon investigation it was revealed that while the actuator was being repaired, the maintenance personnel found that the old diaphragm had an oversize shaft hole in it. No other holes or other defects were found in the diaphragm. A new diaphragm with the correct size shaft hole was installed in the actuator and it was reinstalled on the valve. The valve was then cycled several times and the system was returned to service.

Valves with this type actuator were inspected during the May, 1984 shutdown and new diaphragms with the correct size shaft hole were installed. A preventive maintenance program has been initiated to replace the diaphragms on a scheduled basis. All diaphragms in the supply system have the correct size shaft hole.

## BALTIMORE GAS AND ELECTRIC COMPANY

P.O. BOX 1475

BALTIMORE, MARYLAND 21203

NUCLEAR POWER DEPARTMENT CALVERT CLIFFS NUCLEAR POWER PLANT LUSBY, MARYLAND 20657

May 31, 1984

U. S. Nuclear Regulatory Commission Document Control Desk Washington, DC 20555

Docket No. 50-318 License No. DPR 69

Dear Sirs:

The attached revision to LER 83-50/3L is being forwarded to you for your information.

Should you have any questions regarding this report, we would be pleased to discuss them with you.

Very truly yours,

L. B. Russell

Plant Superintendent

LBR:WIL:srm

cc: Dr. Thomas E. Murley

Director, Office of Management Information

and Program Control

Messrs: A. E. Lundvall, Jr.

J. A. Tiernan