NRC Form 366 (9.83)					LICENSEE EVENT REPORT (LER)							U.S. NUCLEAR REQULATORY COMMISSION APPROVED OMB NO. 3150-0104 EXPIRES 8/31/86												
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At 2345 on March 16, 1984, with the reactor at 2% power (main generator off line), the turbine overspeed trip test was begun.

Low electro-hydraulic (EHC) oil pressure, due to an oil leak on turbine control valves #3 and #4, caused the start of the second EHC pump. Manual isolation of the leak resulted in an EHC pressure spike causing the rapid opening of #4 turbine control valve. The increased steam demand caused steam generator 1B level to swell to the hi-hi setpoint coincident with P-7 (at power trip permissive) enabling due to hi turbine impulse pressure. This resulted in a turbine trip/reactor trip.

Immediate operator actions for a turbine trip/reactor trip were taken and systems verified stable.

There was no impact on public health or safety.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO. 3150-0104 EXPIRES: 8/31/85

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6) PAGE (3)
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TEXT (If more space is required, use additional NAC Form 366A's) (17)

Mid-day on March 16, 1984, a load backdown for the scheduled annual refueling outage was initiated. At 2329 the main turbine generator (TG) was taken off line and reactor (RCT) power reduced to 2% (hot standby condition) in preparation for surveillance procedure SP 54-064, "Turbine Overspeed Trip Test".

At 2345, while performing this surveillance procedure, an electro-hydraulic control (TG) fluid leak was indicated by control operator observation of decreasing TG oil pressure and subsequent start of second TG pump. A nonlicensed operator was dispatched to the high pressure turbine (TRB) house to investigate. TG oil was observed to be flowing from #3 and #4 turbine control valves (FCV) rupture disc (RPD) drain (DRN) line. Manual action was taken to isolate the DRN line, and TG oil pressure quickly recovered. The TG demand signal (for overspeed trip testing) was still active causing #4 turbine control valve to open rapidly admitting steam to the high pressure TRB. High TRB impulse pressure due to the increased steam flow enabled permissive P-7 (10%), "turbine at-power trips". Coincident with this, the sudden steam demand caused steam generator (SG) 1B level to "swell" effecting a turbine trip with resultant reactor trip.

The turbine/reactor trip was followed up by the immediate operator actions of emergency operating procedure E-O-O4, "Turbine and Reactor Trip". At the time of this event, individual rod position indication for rod (ROD) E-11 was out of service. Therefore, the operators also performed the immediate operator actions of emergency procedure E-CVC-35, "Emergency Boration", requiring emergency boration. Stable plant conditions were verified. There was no adverse affect on public health or safety.

Investigative/corrective maintenance revealed that the TG oil leak was due to RPD's on TRB FCV #3 and #4 rupturing.

Follow-up investigation by plant and turbine generator (TG) vendor personnel failed to positively identify the root cause of the RPD failures. However, lower than recommended nitrogen charge pressure on the accumulators (ACC) on the FCV actuators is believed to be a contributing factor.

Correspondence with the TG vendor indicates that the RPD in the TG system drain lines was intended to supplement the Single Channel Overspeed Trip system (S.C.O.T.S.). Since the S.C.O.T.S is no longer a functioning option on the Kewaunee turbine (it has been replaced by the redundant overspeed trip system), the need for RPD's in the TG system drain lines is precluded. Therefore, on the basis of the vendor's recommendation, the RPD's have been removed, and the lines plugged.

No further investigation or corrective action is planned.

WISCONSIN PUBLIC SERVICE CORPORATION



P.O. Box 1200, Green Bay, Wisconsin 54305

June 1, 1984

U. S. Nuclear Regulatory Commission Document Control Desk Washington, D.C. 20555

Gentlemen:

Docket 50-305 Operating License DPR-43 Kewaunee Nuclear Power Plant Reportable Occurrence 84-002-01

In accordance with the requirements of 10 CFR 50.73 "Licensee Event Report System", the attached Licensee Event Report for reportable occurrence 84-002-01 is being submitted.

Very truly yours,

C. W. Giesler

Vice President - Power Production

JGT/jks

Attach.

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