Update Report - Previous Report Date 12/20/83

(12-81) 10 CFR 50		LICENSEE EVENT REPORT	3150-0011
	ONTROL BLOCK:	(PLEASE PRINT OR TYPE	ALL REQUIRED INFORMATION)
	LICENSEE CODE 14	0 - 0 0 0 0 0 - 0 0 3 4	1 1 1 1 1 30 (1) 57 CAT 58 (5)
CON T	REPORT L 6 0 5 0	0 0 2 5 9 0 1 2 1 0 8 3	8 0 5 2 5 8 4 9 75 REPORT DATE 80
the second se	On 12/10/83, with unit	LE CONSEQUENCES (10) 1 in a refueling outage, an opera	ator observed RHR
0 3 LF	oump 1C motor was spar	king and smoking. The motor was s	shut down and subsequently
04	estarted. After rest	arting, the motor tripped on over	current. With RHR Loop II
050	out-of-service, only R	HR pump 1A, Loop I, was operable.	This exceeded limit
0 6 0	of LCO, Tech Spec 3.5.	B.9. Limits were exceeded for two	o hours. There was no
076	effect on public healt	h and safety. Reactor vessel temp	perature remained within
	imits required by Tec	h Specs 3.6.A.3 and .5.	80
09		SUBCODE COMPONENT CODE S 12 Z 13 M O T O R X 14	Z 15 Z 16
, .	9 10 11		19 20 REPORT REVISION TYPE NO. Y 1
		24 26 27 28 29 HUTDOWN METHOD HOURS 22 ATTACHMENT SUBMITTED NPRD FORM S Z 21 0 0 0 Y 23 N 40 36 37 40 41 42	SUB. SUPPLIER MANUFACTURER
	AUSE DESCRIPTION AND CORREC The GE model number 5K	TIVE ACTIONS (27) 6348XC23A, 2000 HP, 3 phase motor	is believed to have
11 t	ripped due to winding	failure. RHR Loop II was placed	back in-service and
1 2 R	HR pump 1C motor was	replaced. All relevant data has b	een collected and a
13 [f	ailure report is bein	g finalized. Failure mode and rec	urrence control will be
14 a	ddressed in a follow-	up report to this LER by 7/16/84.	80
	H 20 0 0 029		overy description (32) Observation
	TUITY CONTENT EASED OF RELEASE AMOUNT OF	-	ATION OF RELEASE 36
	10 11 PERSONNEL EXPOSURES NUMBER TYPE DESCRIPT 0 0 0 0 0 37 2 38	44 45	80
7 8 9	PERSONNEL INJURIES NUMBER DESCRIPTION (4)		80
1 8	0 0 0 0	NA	
19	S OF OR DAMAGE TO FACILITY (43) DESCRIPTION	NA	1822
7 8 9			NRC USE ONLY
20	NI@L	NA	
	NAME OF PREPARER	Walter T. Christopher PHO	«* « ** ** ** ONE 205-729-0800
060402	24 840525 K 05000259 PDR		

Tennessee Valley Authorit Browns Ferry Nuclear Plant

Form BF 17 BF 15.2 2/12/82

LER SUPPLEMENTAL INFORMATION

BFRO-50-259 / 83068R1 Technical Specification Involved _______3.5.B.9 Reported Under Technical Specification _____6.7.2.a(2)* Date Due NRC __12/24/83

Event Narrative:

Unit 1 was in a refueling outage with irradiated fuel in the vessel and the vessel at atmospheric pressure; unit 2 was operating normally at 100 percent power; unit 3 was in a refueling outage. Unit 1 was the only unit affected by this event. On December 10, 1983, an operator observed that RHR pump 1C motor was sparking and smoking. The motor was shut down and subsequently restarted. After being restarted and running for a few minutes the motor tripped on overcurrent at 2045 hours. A and C phase and ground instantaneous overcurrent relays actuated. RHR Loop II was out-of-service. Therefore, RHR pump 1A, Loop I, was the only RHR pump operable. This exceeded the limiting condition of operation as defined by Technical Specification (T.S.) 3.5.B.9. All unit 1 and 2 diesel generators were operable. RHR Loop II was returned to service at 2243 hours. T.S. limits 3.5.B.9 were therefore exceeded for 2 hours. RHR pump 1C motor replacement was completed on 12/16/83. The GE model number 5K6348XC23A, 2000 HP, 3 phase motor is believed to have tripped due to winding failure. There was no effect on public health and safety. Reactor vessel temperature was maintained within limits as required by T.S. 3.6.A.3 and .5 during this event. All relevant data concerning the motor failure has been collected and a failure report is being finalized at this time. After issue of the failure report, the failure mode and recurrence control will be addressed in a follow-up to this LER by 7/16/84.

* Previous Similar Events: BFR0-50-296/81034

*Revision:

Retention: Period - Lifetime; Responsibility - Document Control Supervisor

TENNESSEE VALLEY AUTHORITY Browns Ferry Nuclear Plant P. 0. Box 2000 Decatur, Alabama 35602

May 25, 1984

Mr. James P. O'Reilly, Regional Administrator U. S. Nuclear Regulatory Commission Region II 101 Marietta Street, Suite 3100 Atlanta, Georgia 30303

Dear Mr. O'Reilly:

TENNESSEE VALLEY AUTHORITY - BROWNS FERRY NUCLEAR PLANT UNIT 1 - DOCKET NO. 50-259 - FACILITY OPERATING LICENSE DPR-33 - REPORTABLE OCCURRENCE REPORT BFR0-50-259/83068 R1

Reference: H. J. Green's memorandum to you dated December 20, 1983.

The enclosed report provides followup information concerning failure of residual heat removal system pump motor. This report is submitted in accordance with Browns Ferry Unit 1 Technical Specification 6.7.2.a.(2).

Very truly yours,

TENNESSEE VALLEY AUTHORITY

G. T. Jones Power Plant Superintendent Browns Ferry Nuclear Plant

Enclosure cc (Enclosure): U. S. Nuclear Regulatory Commission Document Control Desk Washington D.C. 20555

NRC Inspector, Browns Ferry Nuclear Plant