Ninc Form 306 (9-83)					LIC	LICENSEE EVENT REPORT (LER)					U.S. NUCLEAR REGULATORY COMMISSION APPROVED OMS NO. 3180-0104 EXPIRES: 8/31/86						
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							ICENSEE	CONTACT	FOR THIS	LER (12)			TEL EDW	ONE NUM	AFR		
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	,				COMPLET	E ONE LINE POP	EACH CO	OMPONEN	TFAILURE	DESCRISE	D IN THIS REPO	MT (13)					
CAUSE	SYSTEM			MANUFAC- TURER	REPORTABLE TO NPRDS			CAUSE	SYSTEM	COMPONENT	MANUFAC- TURER		PORTABLE .				
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SUPPLEMENTAL REPORT EXPECTED (14)									EXPEC	EXPECTED MONTH		DAY	YEAR				
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At approximately 0247 hours, April 25, 1984, the reactor tripped from 100% indicated power as a result of a main turbine trip. The trip occurred as the Main Turbine Thrust Bearing Wear Detector was being returned to service following a modification. Following the scram, Feedwater Regulating Valves A and B did not automatically close upon the Reactor Trip coincident with Low Tavg. Feedwater isolation was established by the automatic closure of the Main Feedwater Isolation Valves. The cause of this event was due to personnel error. The technician did not adequately review the system status prior to performing the work. This event was discussed with the individual involved, and the importance of fully understanding a job, no matter how simple, was emphasized.

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ABSTRACT (Limit to 1400 spaces i.e., approximately fifteen single-space typewritten lines) (16)

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NRC Form 366A (9-83) LICENSEE EVENT	LICENSEE EVENT REPORT (LER) TEXT CONTINUATION						
FACILITY NAME (1)	DOCKET NUMBER (2)		LER NUMBER (6)	PAGE (3)			
		YEAR	SEQUENTIAL REVISION				
Virgil C. Summer Nuclear Sta	ation 3 9	5 8 4	0 2 5 0 0	0 2 0 2			

TEXT (If more space is required, use additional NPC Form 366A's) (17)

During the spring outage, a modification was made to the Main Turbine Thrust Bearing Wear Detector circuit. During the subsequent startup of the unit, the thrust bearing turbine trip was defeated by opening the proper terminal board link. Startup and power increase continued with actual rotor movement monitored by Instrument and Controls (I&C) and Operations Groups. The thrust bearing wear detector measurement device was normal with the exception of having a Fluke digital multi-meter across the contacts of the pressure switches. The multi-meter was left installed (power off) during testing to save the trouble of installing and removing the meter every time a measurement was made. On the midshift, April 25, 1984, the decision was made to reinstate the thrust bearing wear detector trip because the plant was at 100% power. The technician performing the closure of the link did not adequately review the system status. When the link was closed, the input impedance (with power off) of the meter was seen in the Electro Hydraulic Control as a closed contact which initiated a turbine trip which caused a reactor trip.

During the trip, the Feedwater Regulating Valves A and B did not automatically close upon Reactor Trip coincident with Low Tayq. Feedwater isolation was accomplished by the automatic closure of the Main Feedwater Isolation Valves. The Feedwater Regulating Valves were subsequently closed remote-manually from the Main Control Board. Inspection of the valves revealed that one (1) of the air bleedoff valves on each valve was improperly adjusted. The valves were re-adjusted and successfully tested to close on an automatic signal.

The cause of this event was personnel error. The technician did not adequately review the system status prior to performing the work. This event was discussed with the individual involved, and the importance of fully understanding a job, no matter how simple, was emphasized.

SOUTH CAROLINA ELECTRIC & GAS COMPANY POST OFFICE 764 COLUMBIA, SOUTH CAROLINA 29218 O. W. DIXON, JR. VICE PRESIDENT NUCLEAR OPERATIONS May 25, 1984

U.S. Nuclear Regulatory Commission Document Control Desk Washington, DC 20555

> SUBJECT: Virgil C. Summer Nuclear Station Docket No. 50/395

Operating License No. NPF-12

LER 84-025

Dear Sir:

Please find attached Licensee Event Report #84-025 for the Virgil C. Summer Nuclear Station. This Report is submitted pursuant to the requirements of 10 CFR 50.73(a)(2)(iv).

Should there be any questions, please call us at your convenience.

Very truly yours

O. W. Dixon, Jr.

RJB:OWD/dwf Attachment

cc: V. C. Summer

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