



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

VIRGINIA ELECTRIC AND POWER COMPANY

OLD DOMINION ELECTRIC COOPERATIVE

DOCKET NO. 50-338

NORTH ANNA POWER STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 150  
License No. NPF-4

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Virginia Electric and Power Company et al., (the licensee) dated August 29, 1991, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public;  
and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

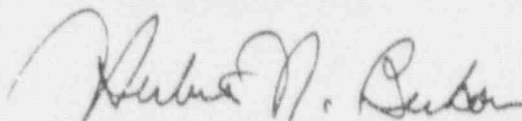
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.D.(2) of Facility Operating License No. NPF-4 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 150, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of issuance and shall be implemented prior to restart after the next refueling outage.

FOR THE NUCLEAR REGULATORY COMMISSION



Herbert N. Berkow, Director  
Project Directorate II-2  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: November 29, 1991

ATTACHMENT TO LICENSE AMENDMENT NO. 150

FACILITY OPERATING LICENSE NO. NPF-4

DOCKET NO. 50-338

Replace the following page of the Appendix "A" Technical Specifications with the enclosed page as indicated. The revised page is identified by amendment number and contains vertical lines indicating the area of change.

Page  
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TABLE 3.3-4 (continued)  
ENGINEERED SAFETY FEATURE ACTUATION SYSTEM  
INSTRUMENTATION TRIP SETPOINTS

<u>FUNCTIONAL UNIT</u>	<u>TRIP SETPOINT</u>	<u>ALLOWABLE VALUES</u>
6. AUXILIARY FEEDWATER PUMP START		
a. Manual	N. A.	N. A.
b. Automatic Actuation Logic	N. A.	N. A.
c. Steam Generator Water Level Low-Low	≥18% of narrow range instrument span each steam generator	≥17% of narrow range instrument span each steam generator
d. S. I.	See 1 above (all S.I. Setpoints)	
e. Station Blackout	≥2392 volts on Transfer Bus	≥2184 volts on Transfer Bus
f. Trip of Main Feed Pump	N. A.	N. A.
7. LOSS OF POWER		
a. 4160 Volt Emergency Bus Undervoltage (Loss of Voltage)	3080 ±13 volts with a time delay of 2.0 ±0.5 seconds	≥2989 volts with a time delay of ≤3.0 seconds
b. 4160 Volt Emergency Bus Undervoltage (Degraded Voltage)	3746 ±7 volts with a time delay of 56 ±6 seconds	≥3688 volts with a time delay of ≤63 seconds



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VIRGINIA ELECTRIC AND POWER COMPANY

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DOCKET NO. 50-339

NORTH ANNA POWER STATION, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 134  
License No. NPF-7

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Virginia Electric and Power Company, et al., (the licensee) dated August 29, 1991, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.



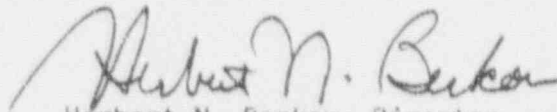
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-7 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 134, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of issuance and shall be implemented prior to restart after the next refueling outage.

FOR THE NUCLEAR REGULATORY COMMISSION



Herbert N. Berkow, Director  
Project Directorate II-2  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: November 29, 1991

ATTACHMENT TO LICENSE AMENDMENT NO. 134

TC FACILITY OPERATING LICENSE NO. NPF-7

DOCKET NO. 50-339

Replace the following page of the Appendix "A" Technical Specifications with the enclosed page as indicated. The revised page is identified by amendment number and contains vertical lines indicating the area of change. The corresponding overleaf page is also provided to maintain document completeness.

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TABLE 3.3-4 (Continued)

ENGINEERED SAFETY FEATURE ACTUATION SYSTEM INSTRUMENTATION TRIP SETPOINT

<u>FUNCTIONAL UNIT</u>	<u>TRIP SETPOINT</u>	<u>ALLOWABLE VALUES</u>
4. STEAM LINE ISOLATION		
a. Manual	Not Applicable	Not Applicable
b. Automatic Actuation Logic	Not Applicable	Not Applicable
c. Containment Pressure--Intermediate High-High	$\leq 17.8$ psia	$\leq 19.3$ psia
d. Steam Flow in Two Steam lines-- High Coincident with $T_{avg}$ --Low-Low Or Steam Line Pressure--Low	<p><math>&lt;</math> A function defined as follows: a <math>\Delta p</math> corresponding to 40% of full steam flow between 0% and 20% load and then a <math>\Delta p</math> increasing linearly to a <math>\Delta p</math> corresponding to 110% of full steam flow at full load.</p> <p><math>T_{avg} \geq 543^{\circ}\text{F}</math>  <math>\geq 600</math> psig steam line pressure</p>	<p><math>&lt;</math> A function defined as follows: a <math>\Delta p</math> corresponding to 44% of full steam flow between 0% and 20% load and then a <math>\Delta p</math> increasing linearly to a <math>\Delta p</math> corresponding to 111.5% of full steam flow at full load.</p> <p><math>T_{avg} \geq 542^{\circ}\text{F}</math>  <math>\geq 585</math> psig steam line pressure</p>
5. TURBINE TRIP AND FEEDWATER ISOLATION		
a. Steam Generator Water level-- High-High	$< 75\%$ of narrow range instrument span each steam generator	$< 76\%$ of narrow range instrument span each steam generator



TABLE 3.3-4 (continued)  
ENGINEERED SAFETY FEATURE ACTUATION SYSTEM  
INSTRUMENTATION TRIP SETPOINTS

FUNCTIONAL UNIT	TRIP SETPOINT	ALLOWABLE VALUES
6. AUXILIARY FEEDWATER PUMP START		
a. Manual	Not Applicable	Not Applicable
b. Automatic Actuation Logic	Not Applicable	Not Applicable
c. Steam Generator Water Level Low-Low	≥18% of narrow range instrument span each steam generator	≥17% of narrow range instrument span each steam generator
d. S. I.	See 1 above (all S.I. Setpoints)	
e. Station Blackout	≥2392 volts on Transfer Bus	≥2184 volts on Transfer Bus
f. Trip of Main Feed Pump	N. A.	N. A.
7. LOSS OF POWER		
a. 4160 Volt Emergency Bus Undervoltage (Loss of Voltage)	3080 ±13 volts with a time delay of 2.0 ±0.5 seconds	≥2989 volts with a time delay of ≤3.0 seconds
b. 4160 Volt Emergency Bus Undervoltage (Degraded Voltage)	3746 ±7 volts with a time delay of 56 ±6 seconds	≥3688 volts with a time delay of ≤63 seconds